

# Monte Carlo calculation of the flight distance to the first collision

Alan L. Arsen

28 November 2019

Monte Carlo Methods and Simulations in Nuclear Technology (SH2704) KTH Royal Institute of Technology



#### Problem definition

Using a Monte Carlo simulation, calculate the mean distance that the fission neutrons fly until their first collision. Use an infinite system composed of a single fissile nuclide at a reasonable mass density.

Perform a simple sampling simulation to evaluate the aforementioned parameter. Collect at least several thousands of samples and compute the mean distance and the variance of the mean value.



## Watt Fission Spectrum

The analytical expression of the Watt Fission Spectrum is:

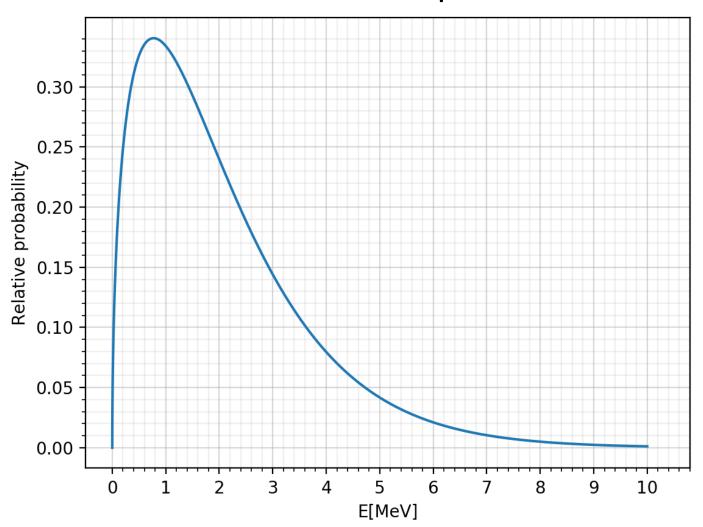
$$\chi(E) = C e^{-E/a} \sinh(\sqrt{bE}),$$

Where a and b are constants that depend on the nuclide and C is the normalization constant. This study was performed using the constants corresponding to U-233 for thermal fissions which are[1]:

$$a = 0.977$$
;  $b = 2.546$ .



## Watt Fission Spectrum





## Solution approach

The system was assumed to be made of U-233 with an atomic density given by:

$$N \approx \frac{19.1 \frac{g}{cm^3} N_a}{233},$$

where  $N_a$  is the Avogadro number.

The energy was sampled using acceptance rejection method, sampling the values in a box from 0 to 20 MeV in the x-axis and 0 to  $\max[\chi(E)]$  in the y-axis.



## Solution approach

The cross section data was obtained from ENDF/B-VIII.0 library[2]. The cross section for a given energy was obtained using a linear interpolation.

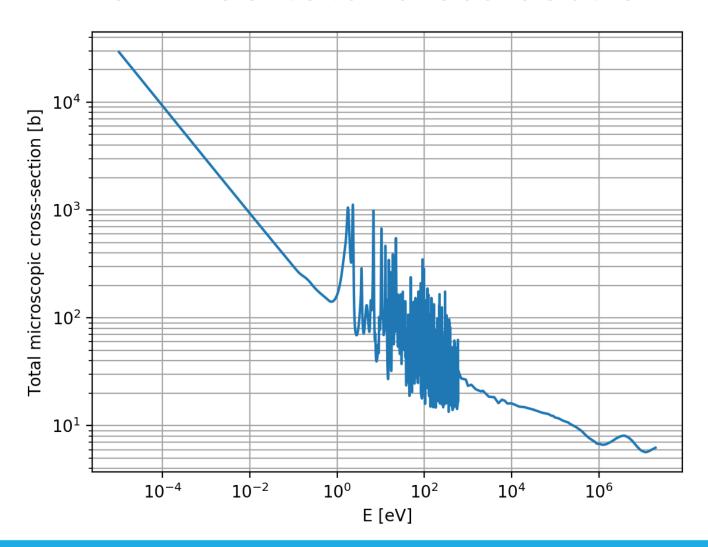
The distance between collisions was sampled using the inverse transform method knowing that the CDF for this process is given by:

$$F_{S}=1-e^{-\Sigma_{t}S},$$

where  $\Sigma_t$  is the total cross section and s the parameter of interest.



## U - 233 total cross-section





# Solution implementation: coding

```
# Functions definition
watt = lambda var, paramA, paramB, paramC: paramC * np.exp(-var/paramA) * np.sinh(np.sqrt(paramB*var))
      = lambda var: var
Х
x2
      = lambda var: var**2
funOp = lambda var,paramA,paramB,paramC,fun1,fun2: fun1(var,paramA,paramB,paramC) * fun2(var)
normF = lambda paramA,paramB,paramC,fun: paramC/(quad(fun, 0, np.inf, args=(paramA,paramB,paramC))[0])
probF = lambda delta, sigma: erf(delta/(sigma * np.sqrt(2)))
def findXS(argE, argDataXS):
    i = 0
    while argE > argDataXS[i,0]:
        i += 1
    interp = interp1d(data[i-1:i+1,0], data[i-1:i+1,1])
    return interp(argE)
sortS = lambda argXS, numDensU: -(1/(argXS*numDensU)) * np.log(random.random())
```



## Solution implementation: coding

```
def singleSim(argSimNum, argEnergyScalling, argProbScalling, argDataXS, numDensU):
   i = 0
   fcount = 0
   fSumE = 0
   fSumE2 = 0
   fSumS = 0
   fSumS2 = 0
   while i<int(argSimNum):</pre>
        fcount = fcount + 1
        E = argEnergyScalling * random.random()
        P = argProbScalling * random.random()
        if watt(E,a,b,c) >= P:
            i = i + 1
            s = sortS(findXS(E*1e6, argDataXS)*1e-24, numDensU)
            fSumE += E
            fSumE2 += E**2
            fSumS += s
            fSumS2 += s**2
   return fSumE, fSumE2, fcount, fSumS, fSumS2
```



## Solution implementation: coding

```
# Main
simNum = 1e5
energyScalling = 20 # MeV
probScalling = watt(maxEdet,a,b,c)
sumE, sumE2, count, sumS, sumS2 = singleSim(simNum, energyScalling, probScalling, data, numDens)
meanE = sumE/simNum
stDvMeanE = np.sgrt((sumE2/simNum - meanE**2)/simNum)
deltaE = 2 * stDvMeanE
pE = probF(deltaE,stDvMeanE) # Confidence interval
meanS = sumS/simNum
stDvMeanS = np.sqrt((sumS2/simNum - meanS**2)/simNum)
deltaS = 2 * stDvMeanS
pS = probF(deltaS,stDvMeanS) # Confidence interval
eff = simNum/count
```



#### Results

Parameter	Monte Carlo*
Simulation result	2.70 ± 0.02 cm
Confidence interval	0.954
Efficiency of the sampling method**	14.62%
Variance	7E-5 cm <sup>2</sup>

<sup>\*</sup>The simulation was performed using 1E5 accepted samples

<sup>\*\*</sup> Defined as the ratio of accepted samples over the total number of samples



## References

[1] X-5 Monte Carlo Team. "MCNP — A General Monte CarloN-Particle Transport Code, Version 5". Volume I: Overview and Theory. 2003.

[2] https://www.oecd-nea.org/janisweb/book/neutrons/U233/MT1/renderer/12



Thank you for your attention

Tack för er uppmärksamhet