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Error Propagation for Fuel Cycle Calculation

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Principal Investigator: Paul P.H. Wilson, Professor, University of Wisconsin-Madison

Co-PI: Dr. Baptiste Mouginot, University of Wisconsin-Madison

Time Frame: 3 years

Estimated Cost: \$800,000

1 Proposed Scope/Context Description

The fuel cycle simulation tool can have a large scope of application, from the study of the behavior of some type of fuel or reactor inside an existing nuclear fleet to the prospective analysis of a complete nuclear transition. Beside the study of fleet evolution it can also be used to assess the possibility of material hijacking in the context of non-proliferation study or policy...

Each application field of the nuclear fuel cycle calculation requires a specific level of confidence, which are up to now very un-precisely assessed, if assessed. There is a real need of validation the those kind of calculation, which can hardly be reached. Indeed, the only existing way to validate any fuel cycle calculation/tool, is the benchmark with other similar tools, or other existing data... When the first is generally conclude with a list of why the different softwares end up with different results (without concluding on the precision of any), the second allow only the validation on existing concept and have no impact on calculation implying the use of new concept. The aim of this project is to add error propagation capability to the CYCLUS fuel cycle simulator [1]. By their usage for predicting the evolution of a large industrial enterprise in an uncertain future, nuclear fuel cycle simulations are generally based on approximations and uncertain input data. Since validation is largely considered to be impractical, such simulations are seen as indications of future behavior rather than predictions of that behavior.

Nevertheless, it would be valuable to be able to place some confidence bounds on those indications, both to assess the robustness of conclusions that derive from those indications and

to provide information about the sensitivity of those conclusions to the uncertain data and algorithms.

Having a broad distribution for each metric calculated in a fuel cycle simulation instead of unique values will allow a better comparison between different fuel cycle scenarios.

Moreover for some critical analysis such as retrospective non-proliferation analysis, it could be extremely valuable to add some degree of confidence on the simulation performed. This could allow at least to confirm or invalidate the possibility to use those calculation tools for such purpose.

This project will extend the Cyclus concept of resources to include error information and then develop a number of archetypes that can perform operations to propagate that error in a rigorous fashion. Ultimate calculation of fuel cycle performance metrics will also need to be updated in order to represent final results as distributions rather than single values.

2 Logical Path to Work Accomplishment

The goal of this project is to add optional extensions to Cyclus that will allow an assessment of the error as it propagates through a fuel cycle.

2.1 Add uncertainty to ressources

2.2 Archetypes uncertainty management

In a fuel cycle calculation, there are three different uncertainty/error sources :

- the material uncertainty, δ , which is the input and the output of each archetypes,
- the "tolerance", τ , on the input parameter, which corresponds the possible variation range of a physics parameter,
- the error, ϵ , introduce by the archetypes modeling/operation on the material.

I'll add a schema to illustrate this..

The work we are proposing to do on this project, is to allows all archetypes of CYCLUS, to combine all those sources and computes the resulting uncertainties on the output materials. In the following parts, one will tried to detail the work which need to be done on the different archetypes, we are proposing to update.

2.2.1 Enrichment facility

The enrichment facility is probably the easiest facility to model, since the only error come from the the enrichment variation on the final product, the tails and the feed stream, and the enrichment process can be linearly models. *Is it true with any enrichment technic ?*

This can be easily analytically combine and propagate.

do we want the exact error propagation formula ?

2.2.2 Separation

As the enrichment process, the separation should be very easy to model including uncertainty. Indeed, the only parameter which can introduce extra variance is the separation efficiency. *do we want the exact error propagation formula ?*

2.2.3 Reactor

We are envisaging two version of the reactor, which will be both capable to handle uncertainty. The first one which is a very simple one, will be built as an upgrade of the existing CYCAMORE reactor, making it error aware. The CYCAMORE reactor is a recipe base reactor, in addition of all the classical reactor parameter (batch number, cycle length, power, capacity factor,...) the user provides the input and output fuel recipes. The only requirement is to force user to provide the output recipe with the according uncertainty.

The second version of reactor will be able to calculate the evolution of the fuel provided by the fuel fabrication (see §2.2.4). To do so, we propose investigating two ways, both using pre-trained models, allowing the prediction of key physics parameters needed to compute the evolution of a fuel during the irradiation. It has been proven that from pre-trained neural network models, one can predict the evolution of the one group cross section during the irradiation of the fuel from its initial isotopic composition, and is working for a various range of reactor, from LWR to SFR [?, ?].

The first application using the neural network predictive models, is to train a model to directly predict the composition evolution as the function of the burnup. This application might not work, since the usage of neural network has been proven to predict one group macroscopic cross section.

If the neural network model fail to directly predict the isotopics evolution, one have to consider the second option, which imply to predict the 1 group cross section, then integrate the Bateman equation.

The main reason leading the try of direct prediction of the isotopic evolution, is the error/uncertainty propagation. The determination and the propagation of all uncertainty source is, for this kind of reactor, a complicated matter. Indeed, in addition of the uncertainty of the input fuel material, we need to consider tolerance variation on the input parameter, and the error coming from the computation of the evolution.

The input parameter allows to fluctuate are the power, the capacity factor and the discharge burnup. Since those three parameters will have an impact on the discharge time of the fuel, which will be very complicated to deal, it has been choose to consider them as constant. Nevertheless, a brute force study (also sometimes called "Total Monte Carlo method") could allow the determine their impact, running many time the same simulation choosing randomly the parameter value at each cycle of the reactor, the distribution of the results will provides a precise measurement of their role.

The error due to the computation of the depletion calculation are wild:

- the error of the neural network predictor, ϵ_i^{nn} , with $i \in [0..N]$, N the number of predicted parameter,
- the error on the data sets used to train the neural network,
- the convolution of the uncertainty on the material composition with the neural network prediction.

Part on the error, we should consider, come from the training sample used to train the different models. Indeed, generally those training sample are populated using few thousand of depletion calculation.

The depletion calculation take as input, the nuclear data. Those nuclear data are interpolated/extrapolated from many different experimental measurement using many different models. Therefore the nuclear data contain uncertainty... Those uncertainty are extremely difficult to propagate properly through a full depletion calculation because of the coupling between neutron transport and depletion calculation: the composition of the fuel impact the shape of the neutron spectrum, which impact the reaction rate on the nuclei...

The depletion calculation are also subject to a more "complicated" uncertainty. Indeed the modeling uncertainty come from the different approximation necessary to simulate a full depletion calculation on the geometry (there is a lot of difficulty using Monte Carlo technique on a PWR full core calculation due to source convergence issue) or on the different reactor parameter, such as boron concentration, rod control management, charge factor evolution, neutron leakage... The study and the propagation of the modeling uncertainty, such as the modeling simplification and the nuclear data uncertainty is way beyond the scope of this project... This require a full dedicated research project (and probably more).

Those error will then be removed of the equation on the first version of this work. This might need to be reconsidered when the depletion calculation error propagation capacity will have done important progress.

Hopefully there might be some breakthrough during the realization of this work. If not, one have to roughly estimate it using the state of the art on this field.

2.2.4 Fabrication

2.3 Problems/Applications/Validations

2.4 Brief Project Description

Different step have to be followed to accomplish it. The first step is to extend the standard resources to contain their own error information and develop methods to propagate the errors as agents operate on those resources. With such an infrastructure in place, specific archetypes will be developed to perform operations that depend on this error information, with a focus on recycled fuel fabrication, advanced fast reactors that consume recycled

fuel, and separations facilities. When used as part of an actinide recycle scheme, these facilities will trade in approximations to ideal material compositions and hence introduce errors. Analysis tools that can express the outcomes of this error propagation on fuel cycle performance metrics will also be developed. Demonstration of error capability will begin with once through fuel cycles, followed by a simple MOX-recycle system, and culminating with multi-recycle fast reactor systems.

2.4.1 Update CYCLUS

Expressing uncertainty in resources will require a fundamental change to the Cyclus kernel to extend the resource objects to include support for their own error estimates. Prior examples of this type of capability [e.g. 2] will serve as a guide for implementation in the Cyclus context. When complete, this capability should allow every resource to carry a measure of its uncertainty. In addition, all operations that are available to modify resources will be extended to also modify error estimates appropriately. This work will focus on analytical approaches to error propagation, that can be implemented in a way that reduces to the current behavior in the absence of uncertainty information. This will ensure backward compatibility with existing Cyclus archetypes that may not be aware of uncertainty propagation.

2.4.2 Models development

The recipe-based fuel management of Cycamore is not well suited to perform precise error estimation and propagation: by definition, the output fuel composition is precisely defined by the input fuel composition. By contrast, when exploring recycling in nuclear fuel cycles, it is necessary to introduce archetypes that seek to achieve an integrated performance goal by blending streams of material. One such algorithm has been developed as part of the French nuclear fuel cycle tool, CLASS [3]. Using either a polynomial regression [4] or a neural network approach [5], two material streams are mixed to achieve a particular neutronics performance, e.g. burnup of PWR MOX fuel.

Both model, we are proposing using, are based on the neural network prediction. those neural network are priorly trained using on a set of data composed by many differents fuel isotopic composition (between thousand and few thousands depletion calculation). Different exemple have shown the great potential of neural network predicting different neutron spectrum integrated parameter (macroscopic cross section, multiplication neutron factor...) for many types of reactor (PWR, Sodium cooled - FBR, lead cooled ADS) on many types of fuel, MOX, Pu-MA ADS fuel, UOX...

To elaborate such kind of model, one need to build the training sample used to feed the different models, used for fuel fabrication and macroscopic cross section prediction. This training sample are composed by few thousand different couple of input and output parameters. In our case, the input parameter are the initial fuel composition, and the output parameter are the direct (i.e. macroscopic cross section, k_{eff} ...) or indirect ($< k_{eff} >$, maximal burnup accessible...) results of depletion calculation. In any case, one need to use a

depletion calculation tool, to populate the training sample. To generate our training sample data we are envisaging using the SERPENT tool as it seems to be the actual best depletion calculation tool available (in term of precision and speed).

Moreover the quality of the the model will depend on the geometrical repartition and density of the data points in the training sample. It seems that using a latin hyper square repartition [] is the best to train a neural network model when a square repartition may probably netter for a polynomial models

Since all the output parameters have been computed, one can train the model. Two different kind of model have to be generated:

- the fabrication model: allowing to mix N different material streams to build a fuel which will have the correct neutronic/physics properties,
- cross section model/depletion model: allowing to determine the isotopic evolution of the fuel during irradiation.

Fuel fabrication Model

The fabrication model is generally generally the combinaison of a predictor which estimate some neutronics/physics parameters from the initial fuel composition and a mixing algorithm used to find the best material stream mix matching the neutronics/physics requirement.

On application of those applied to PWR MOX fuel, use reactivity criteria to calculate the maximal achievable burnup for a set fuel. Then a model is train (either a polynomial regression [?] , or a neural network [?]) to predict this achievable burnup from the initial composition of a fuel. Applications on FBR MOX fuel allow the estimation the initial k_{keff} [?]...

Cross Section/Depletion Model

Cross section model are used to predict the one group cross section evolution as a function of the burnup (or the time) for all reaction on any nuclei that might appear in the fuel during the irradiation. Using them it is possible to solve the Bateman equation and then determine the fuel composition evolution as the function of the burnup (or the time).

Instead prediction each different cross sections, it might be able to predict directly, using properly trained neural network, the isotopic fuel composition.

Since both kind of model should be trained on complete depletion calculation, containing both one group cross section evolution and the isotopic composition evolution, it could be very interesting to try both, keeping only the most interesting solution.

2.5 Uncertainty

The following project aims to introduce uncertainty capability to material metrics, or material related metrics such as separation tails, burnup... into CYCLUS fuel cycle simulator. Moreover, it could be consider as a base work for the management and the propagation of any metrics in CYCLUS tool.

2.6 Estimation

2.6.1 input parameter

The CYCLUS archetypes will be updated to allow them deal with the uncertainty on metrics related to the flow process itself (separation efficiency, enrichment tails, burnup achievement...). Those uncertainty should be fixed by the user to the appropriate values. Some default values could also be set...

Even if those uncertainty should have a moderate impact on the full fuel cycle calculation, it should be quite simple to implement it... and it could be used as a validation test on the overall uncertainty propagation in the different archetypes comparing the new build-in CYCLUS capacity and some brute force calculation...

Although, those uncertainty should have a limited impact on the fuel cycle calculation, it is still required to measure it. To do so, it will be interesting to perform a sensitivity analysis on those.

2.6.2 Depletion : Nuclear Data / Monte Carlo / Modeling

As explained before, part of the uncertainty, we should consider, comes from the training sample used to train the different models. Indeed, generally those training samples are populated using few thousand of depletion calculation.

The depletion calculation takes as input, the nuclear data. Those nuclear data are interpolated/extrapolated from many different experimental measurements using many different models. Therefore the nuclear data contain uncertainty... Those uncertainty are extremely difficult to propagate properly through a full depletion calculation because of the coupling between neutron transport and depletion calculation: the composition of the fuel impacts the shape of the neutron spectrum, which impacts the reaction rate on the nuclei...

The depletion calculation is also subject to a more "complicated" uncertainty. Indeed the modeling uncertainty comes from the different approximations necessary to simulate a full depletion calculation on the geometry (there is a lot of difficulty using Monte Carlo technique on a PWR full core calculation due to source convergence issues) or on the different reactor parameters, such as boron concentration, rod control management, charge factor evolution, neutron leakage...

The study and the propagation of the modeling uncertainty, such as the modeling simplification and the nuclear data uncertainty, is way beyond the scope of this project... This requires a full dedicated research project (and probably more). Hopefully there might be some breakthrough during the realization of this work. If not, one has to roughly estimate it using the state of the art on this field.

In addition of nuclear data uncertainty and modeling uncertainty, because we are considering using Monte Carlo based depletion calculation tools, one also needs to consider statistical uncertainty. Hopefully this could be quickly assessed in the context of this project by recalculating many times (few hundred) some well chosen fuel compositions.

2.6.3 Models

Those models, as explained previously, are trained on a large amount of pre-calculated depletion calculation. Some study have been done to determine the optimal density of a training sample to train the model [B. Leniau private com?]. This could be quickly completed in the context of this project, allowing to determine the precision of the model intrinsic precision as a function of the training sample density.

One also need to perform some bibliography to determine which neural network library will be the most suitable for the use in a fuel cycle simulation tool and/or if other statistical predictive method might be interesting as well.

Finally, one need to assess the precision of the models. This have two main different components: the one directly connected to the prediction capability of the used model, and the uncertainty of the the computed data used as training data for the models.

The computed data correspond to a sample of few thousand of different depletion calculation. Each of the depletion calculation have 3 uncertainty components, a statistical, a nuclear data and a modeling one.

In addition of the model uncertainty, one need to convolute, the isotopic uncertainty (on the isotopic fuel composition) with the model predictions. A solution could be to use a brutal method such as the Total Monte Carlo, realizing many different predictions using as input many different small variation of the isotopic composition, the dispersion on the prediction provides an correct estimation of the corresponding uncertainty. One will also investigate more elegant solution, using analytic uncertainty combination.

2.7 Propagation

The main difficulty in this project remain the propagation of the different uncertainty along the fuel cycle calculation.

2.7.1 input parameter

The new CYCLUS archetypes developed in a first times be capable to handle error on materials flow, allowing the propagation of the uncertainty on the input materials flow through all the facility process, delivering an material output flow(s) with the corresponding uncertainty.

2.7.2 Models

It should be possible use a simplified version the "Global Perturbation Method" apply on the depletion calculation developed by [Williams, Takeda, Salvatores], allowing the propagation of the different uncertainty across the integration of the Bateman equation.

One solution that could be investigate, is simply to use a TMC method [TMC] to evaluate the sensitivity to the cross section and initial inventory uncertainty to the final composition. Unfortunately those sensitivity should depends on the composition (for a PWR reactor, the

macroscopic cross section accordingly to the shape of the neutron spectrum shape highly depending on the isotopic composition depending of the initial composition). In consequence, using TMC to determine the different sensitivity imply a very high preliminary CPU cost, but should improve the speed on the fuel cycle simulation versus a perturbative propagation of the different uncertainty.

3 Final Phase :

We imagine 3 step in the final phase. First we would like to step up a testing mechanism, based on simply brute force the uncertainty propagation in the fuel cycle calculation by using some kind of Total monte Carlo, allowing all parameters to fluctuate accordingly to their respective uncertainty on many integration of the same calculation. The result distribution can be read as a direct measure of the uncertainty on the final parameter and will be compared to the direct uncertainty propagation calculation as a validation of all the method previously presented. This method will (turning on the uncertainty on each parameter one by one) to validate the uncertainty propagation on each parameter...

One need nevertheless keep in mind that everything is never perfect, and one should always try to improve. To do so it is required to identify the most problematic issue and solve it. In this particular case, one need to identify what is the uncertainty goal depending on the application of the fuel calculation, of course the precision goal is very different when one doing a prospective calculation versus a non-proliferation retrospective analysis... Knowing it, one very important of this project is it could allow through sensitivity analysis to identify the biggest uncertainty source, and help to focus to improve it.

That is why, in addition of the TMC test tool, I would like to develop a tool/method to systematically allows user to perform sensitivity analysis applied to their calculation allowing us to improve future models dedicated to some precise use of the CYCLUS tool.

Finally, we would like Third step : application to simple case : PWR, transition from PWR to FBR...

4 Milestone Task Listing

PHASE 1: CYCLUS update for uncertainty awareness and propagation

- Task 1: update material classes to handle uncertainty (+ composition ?)
- Task 2: upgrade/develop archetype capable to interact with material uncertainty
- Task 3: develop archetype capable to introduce material related uncertainty
- Task 4: develop/write/implement a Bateman numerical solver in CYCLUS
- Task 5: update post-analysis traitement to deal with uncertainty

PHASE 2: Developing fuel fabrication model cross section prediction models and uncertainty capabilities. This has to be done for each couple of fuel/reactor: PWR / UOX - MOX (- U Blanket?), FBR / MOX - U-Blanket, (CANDU?)...

- Task 1: training Sample generation, generation of set of evolution allowing to train our how models. To do so, one need for each fuel to define the isotopic space we need to sample.
- Task 2: develop and train the models.
- Task 3: uncertainty assessment in the calculation
- Task 4: uncertainty prediction capabilities for the models
- Task 5: coupling isotopic uncertainty with models uncertainty
- Task 6: propagation uncertainties along depletion calculation (*task 4 again ?*)

PHASE 3: test & application

- Task 1: simple transition calculation, PWR MOX to FBR
- Task 2: brute force validation of the uncertainty propagation : piece by piece validation overall validation
- Task 3: sensitivity analysis
- Task X: comparison with other physic modeling capabilities such as Bright-Lite ?