

## Nuclear energy fundamentals

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**Abstract** The field has been evolving at an incredible pace, with the clean energy movement in support of nuclear energy and the increased presence of start-ups and international fusion projects like ITER, W7-X, etc. The current state of science and technology allows for the assessment of key problems that will eventually make a reality of the idea that fusion and fission power plants can be the primary sources of energy. There are lots of resources in introductory plasma and nuclear physics available online, yet their advantage goes harder for the writer, who is actively studying during the writing process than for any young PhD starting his research. We aim to develop a basic theory on fission nuclear reactor physics as well as plasma and tokamak/stellarator theory. Special attention will be given to difficulties with the theory, safety issues, and historical development for both cases.

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## 1 Historical Background

- Eddington on the energy production in stars
    - Hydrogen bomb
    - 1950 Toroidal magnetic confinement (stellarator and tokamak)
- The following quote comes from Homi J. Bhabha (1955).

Its well known that atomic energy can be obtained by a fusion process as in the H bomb[...]. The technical problems are formidable, but one should remember that its not yet 15 years since atomic energy was released in an atomic pile for the first time by Fermi. I venture to predict that a method will be found for liberating fusion energy in a controlled manner within the next two decades. When that happens the energy problems of the world will truly have been solved forever for the fuel will be as plentiful as the heavy hydrogen in the oceans.

Spitzer stellarator

Tokamak studies in russia

ITER, WX-7

There is also inertial confinement, z-pinch and theta-pinch

## 2 Plasma physics basics

### 2.1 Definition of plasma

**Definition 1** *A plasma is a quasineutral gas of charged and neutral particles that exhibits collective behavior*

Collective behavior refers to the long-range coulombic interactions driving the motion of the particles forming the plasma, causing it to depend not on local conditions but on the properties of the whole system.

### 2.2 Debye shielding

Quasineutrality arises as a consequence of the distribution and interaction of charged particles in the plasma. At low temperatures, thermal motion becomes negligible, so ions and positively charged particles get surrounded by a cloud of electrons completely shielding the potential, so no electric field will be perceived in the exterior of the electron cloud. With temperature  $T$  the cloud expands to the edge of the ion to a distance where the potential matches the thermal energy partially shielding the charge, causing a weak electric field in the plasma.

The scale of the cloud is of the order,

$$\lambda_D = \left( \frac{k_B T}{4\pi n e^2} \right)^{1/2} \quad (1)$$

so that the potential decreases with radius as,

$$\phi(r) = \phi_0 e^{-r/\lambda_D} \quad (2)$$

This concept is valid for plasmas with numerous particles in the system. The order of magnitude of particles in the plasma will prove insufficient to refer to it as a plasma. This density of particles has to be a lot greater than the order of unity in a volume of radius debye length.

### 2.3 Plasma frequency

A neutral plasma in thermal equilibrium, ions (given their significant mass in comparison with electrons) are considered static and uniformly distributed. Still, a large enough group of electrons in motion can perturb the ions causing by coulomb interaction a pull back, as a restoring force. This yield oscillations in the plasma density. The oscillation angular frequency of the plasma is

$$\omega_e = \left( \frac{n_e e^2}{\epsilon_0 m_e} \right)^{1/2} \quad (3)$$

and similarly for ions,

$$\omega_i = \left( \frac{n_i q_i^2}{\epsilon_0 m_i} \right)^{1/2} \quad (4)$$

There is a different definition for cases when the oscillation wavelength is larger than the debye length. So far the order of magnitude is  $\omega_e = 56.4 n_e^{1/2} s^{-1}$  for a specific species density.

### 2.4 Single particle motion

A charged particle moving in an electromagnetic field is subject to the Lorentz force, given by:

$$\mathbf{F}_L = q(\mathbf{E} + \mathbf{v} \times \mathbf{B}). \quad (5)$$

If the magnetic field is uniform and aligned with the  $z$ -axis, i.e.,  $\mathbf{B} = B_z \mathbf{e}_z$ , then the equation of motion simplifies to:

$$m \frac{d\mathbf{v}}{dt} = q\mathbf{v} \times \mathbf{B}. \quad (6)$$

This interaction causes the particle to undergo helical motion, where its velocity components evolve as:

$$\mathbf{v} = \begin{pmatrix} v_{\perp} \cos(\omega t) \\ v_{\perp} \sin(\omega t) \\ v_{\parallel} \end{pmatrix}.$$

Here,  $v_{\perp}$  represents motion in the plane perpendicular to the field, while  $v_{\parallel}$  describes free movement along  $\mathbf{B}$ . The cyclotron frequency, which determines the rate of circular motion, is:

$$\omega = \frac{qB_z}{m}. \quad (7)$$

The radius of this gyration, known as the *Larmor radius*, is:

$$r_L = \frac{v_{\perp}}{\omega} = \frac{mv_{\perp}}{qB_z}. \quad (8)$$

This circular motion plays a crucial role in plasma physics, confinement systems, and space environments. Additionally, the charge's tendency to drift in response to external forces leads to *diamagnetic effects*, where the motion effectively opposes changes in the magnetic field.

## 2.5 Magnetic mirror

We start talking about the first adiabatic invariant, which is of major importance in the description of particles in the local magnetic fields.

$$\oint p ds = \mu_m = \frac{\frac{1}{2} m \vec{v}_{\perp}^2}{B} \quad (9)$$

By substituting  $v_{\perp}^2 = v^2 \sin(\alpha)$  where  $\alpha$  is the angle between magnetic field and velocity vector. If this is a conserved quantity (no scattering) along the motion, then it is bounded by a previous value of the magnetic field and the angle  $\alpha_i$  at that moment.

$$\frac{\sin^2(\alpha)}{B(s)} = \frac{\sin^2(\alpha_i)}{B_i} \quad (10)$$

Pitch angles evolve as  $\sin(\alpha(s)) = \sqrt{\frac{B(s)}{B_i}} \sin(\alpha_i)$ . This marks the initial pitch angle  $\alpha_i$  that gets mirrored at  $s$  if the inequality is not fulfilled.

$$\sin(\alpha_i) < \sqrt{B_i/B(s)} \quad (11)$$

## 2.6 Velocity drift

The drift velocity of a charged particle due to an external force and magnetic field variations is given by:

– **External Force Drift:**

$$\mathbf{v}_F = \frac{\mathbf{F} \times \mathbf{B}}{qB^2}$$

where  $q$  is the particle charge, and  $B = |\mathbf{B}|$  is the magnetic field magnitude.

– **Gradient- $B$  Drift:**

$$\mathbf{v}_{\nabla B} = \frac{mv_{\perp}^2}{2qB^2}(\mathbf{B} \times \nabla B)$$

where  $m$  is the particle mass,  $v_{\perp}$  is the velocity component perpendicular to  $\mathbf{B}$ , and  $\nabla B$  represents the magnetic field gradient.

– **Curvature Drift:**

$$\mathbf{v}_c = \frac{mv_{\parallel}^2}{qB^2}(\mathbf{B} \times \mathbf{R}_c)$$

where  $v_{\parallel}$  is the velocity parallel to  $\mathbf{B}$ , and  $\mathbf{R}_c$  is the radius of curvature of the magnetic field lines.

– **Diamagnetic Drift:**

$$\mathbf{v}_D = \frac{\nabla P \times \mathbf{B}}{qnB^2}$$

where  $P$  is the plasma pressure, and  $n$  is the particle number density. The diamagnetic drift arises due to the pressure gradient in a magnetized plasma.

## 2.7 Collision vs collisionless plasma

## 2.8 MHD and stability

Fundamental MHD equations (diff between ideal and non ideal) Frozen-in flux in ideal MHD types of plasma instabilities (e.g., Rayleigh-Taylor, kink, tearing mode) and their impact on confinement magnetic reconnection instability classical, neoclassical, and turbulent transport in plasmas magnetic shear in plasma stability zonal flows in suppressing turbulence in fusion plasmas Alfvén waves and magnetosonic waves Landau damping and its role in wave-particle interactions main heating methods used in fusion plasmas (e.g., ohmic heating, neutral beam injection, electron and ion cyclotron resonance heating)

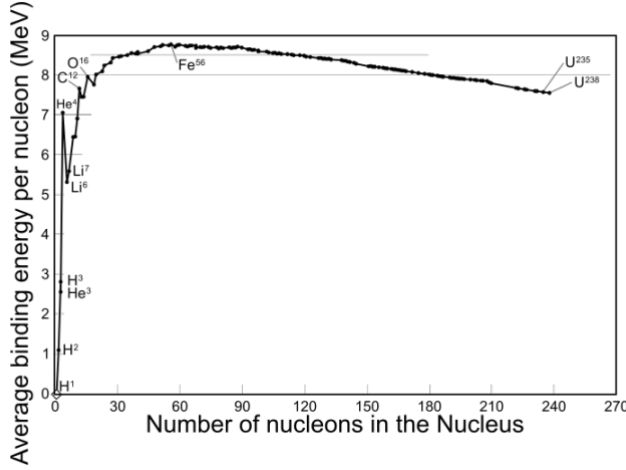
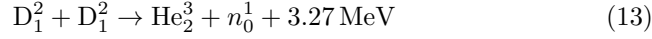
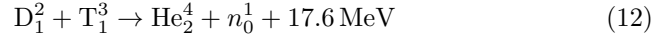


Fig. 1: Binding energy of the elements

### 3 Thermonuclear fusion

#### 3.1 Fusion reactions



Its easy to prove, just by adding the masses of the reactants and compare with the products, we find that there is some mass missing, this mass corresponds with the energy mass relation  $\Delta E = \Delta mc^2 = 17.6 \text{ MeV}$  describing the release of energy has to be in relation with the excess mass  $\Delta m$  of each side of the equation. Said released energy is turned into net kinetic energy. Where the preferred reaction is that of Deuterium and Tritium due to their high cross section at lower energies compared with the second reaction and their derivatives. There is also a reaction between deuterium and helium-3 but its cross section is smaller than the previous.

This cross sections peak at energies that correspond with very high temperatures, this causes the gas to be fully ionized therefore resulting in neutral plasma. Material confinement cannot contain this temperatures, therefore another types of confinement must be used.

#### 3.2 Reaction rate

Given the cross sections and a specific density, we should be able to tell how often this fusion events will take place. We calculate this. Assuming velocity

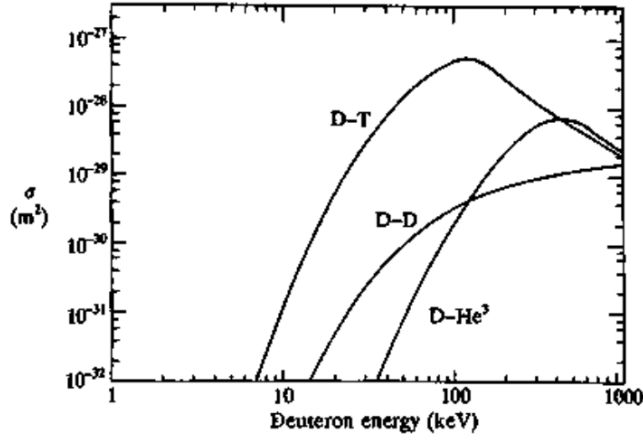


Fig. 2: Deuteron cross section with  $D_1^2$ ,  $T_1^3$  and  $He_2^3$

distribution to be Maxwellian,  $\vec{v}' = \vec{v}_1 - \vec{v}_2$  and the rate of reaction per unit volume is,

$$\mathcal{R} = \int \int \sigma(v') v' f_1(\vec{v}_1) f_2(\vec{v}_2) d^3 v_1 d^3 v_2 \quad (14)$$

Having that the reaction cross section has to be proportional to both the cross section and the relative velocity, hence  $\sigma(v')v'$ . Making sense of the equation, calculates the overlap of the distribution function across all possible velocities in the distribution and weights by the reaction cross section.

Calculate this explicitly

From this, the power generated will be proportional to both deuterium and tritium densities  $n_T, n_D$ , mean energy liberated per reaction  $\epsilon$  and the reaction rate  $\langle \sigma v \rangle$ .

$$P_{gen} = n_T n_D \langle \sigma v \rangle \epsilon \quad (15)$$

which can be rewritten in term of total density

$$P_{gen} = n_D (n - n_D) \langle \sigma v \rangle \epsilon \quad (16)$$

which is maximized if the density  $n_D \approx \frac{1}{2}n$

$$P_{gen} = \frac{1}{4} n^2 \langle \sigma v \rangle \epsilon \quad (17)$$

### 3.2.1 Ignition and confinement time

The total thermal energy will be consistent with the thermal energy of the electrons and ions, and since from previous results we now that we can maximize the power generated by having a plasma with  $n_e = n_i = n$

$$W = \int dV \left( \frac{3}{2} k_B T n_i \right) + \int dV \left( \frac{3}{2} k_B T n_e \right) \quad (18)$$

$$W = \int dV k_B T n = 3k_B \langle nT \rangle$$

with  $\langle nT \rangle$  being the average value of the product

We define the energy lost in a period  $\tau$  or energy confinement time as

$$P_L = \frac{W}{\tau} \rightarrow \tau = \frac{W}{P_L} \quad (19)$$

Where, to sustain the reaction the energy lost is replenished such that  $P_H = P_L$

$$\tau = \frac{W}{P_H} \quad (20)$$

where  $\sim \frac{4}{5}$  of  $P_{gen}$  are carried by the neutron and the rest by the  $\alpha$ -particle, given that it's almost 4 times easier to accelerate a neutron than a helium nucleus. Neutrons don't interact with the plasma, but alpha particles propagate following magnetic field lines, they are confined. This causes the  $\alpha$ -particle to have its own collision events at 3.5 MeV with the plasma, heating it. The measure of this heating in a volume  $V$  is,

$$P_\alpha = \frac{1}{4} \int n^2 \langle \sigma v \rangle \epsilon dV \quad (21)$$

$$P_\alpha = \frac{1}{4} \overline{n_\alpha^2 \langle \sigma v \rangle} \epsilon_\alpha V \quad (22)$$

This allows us to compare the power generated by fusion, the  $\alpha$ -particle heating and the thermal energy.

$$P_H + P_\alpha = W \quad (23)$$

The energy supplied to balance the energy lost by external heating and the  $\alpha$ -heating has to be equal to the total thermal energy of the plasma in a confinement time basis.

$$P_H + \frac{1}{4} \overline{n_\alpha^2 \langle \sigma v \rangle} \epsilon_\alpha V = \frac{3k_B \langle nT \rangle}{\tau} V \quad (24)$$

This external heat source can be achieved in many ways. Three main methods are used: neutral beam injection, ion cyclotron heating, and electron cyclotron heating.

The  $\alpha$ -particle collisions increasingly heat the confinement of D-T plasma. If the adequate confinement conditions are reached, the heating by  $\alpha$ -particles can overcome the energy losses and



$$P_H = \left( \frac{3k_B \langle nT \rangle}{\tau} - \frac{1}{4} n_\alpha^2 \langle \sigma v \rangle \epsilon_\alpha \right) V \quad (25)$$

The condition for a self-sustaining reaction or "Ignition", is that  $P_H = 0$  (which means no external sources of heat).

$$n_\alpha \tau > \frac{12k_B \langle nT \rangle}{n_\alpha \langle \sigma v \rangle \epsilon_\alpha} \quad (26)$$

For the densities being in the same order of magnitude, so we end up with the ignition condition

$$n_\alpha \tau > 1.5 \times 10^{20} \text{ m}^{-3} \text{ s} \quad (27)$$

But, instead of taking the value of temperature as a given for and substitute the reaction rate  $\langle \sigma v \rangle \propto T^2$  we get a more precise relation now with the triple product  $n_\alpha \tau T$ . We change notation to fit the book but lets take  $k_B T \rightarrow T'$  and is now in keV.

$$n \tau T' > 3 \times 10^{21} \text{ m}^{-3} \text{ keV s} \quad (28)$$

This is a very good criterion to obtain ignition. Some values that would be required to fulfill this condition are  $n = 10^{20} \text{ m}^{-3}$ ,  $T' = 10 \text{ keV}$  and  $\tau = 3 \text{ s}$ . Which is still difficult to obtain due to several instability, wall interactions and neutron feeding.

$T' = k_B T$	$\tau$	$n$
10 keV	3 sec	$\sim 10^{20}$

Important to remember that this calculation or condition, comes from taking into account  $\alpha$ -particle heating as one big contributor to keeping the reaction sustained. In the first versions of this condition, the Lawson criterion would only consider external heating sources and neglect alpha heating. He also considered bremsstrahlung which is a weak loss of energy in a tokamak plasma.



The thermal energy  $E_T$  can be expressed similarly to the plasma thermal energy  $E_T = 3nk_B T$

$$3nk_B \frac{dT}{dt} = \frac{P_H}{V} + \frac{1}{4}n_\alpha^2 \langle \sigma v \rangle \epsilon_\alpha - \frac{3k_B \langle nT \rangle}{\tau(n, T)} \quad (32)$$

Then we consider the value of temperature and confinement time needed to achieve stability.

$$3 \frac{k_B T}{\tau} = 3 \frac{T'}{\tau} = \frac{1}{4}n_\alpha \langle \sigma v \rangle \epsilon_\alpha \quad (33)$$

We assume that this stability condition is independent of energy supplied, since we just balanced energy difference between thermal energy of plasma and  $\alpha$ -heating.

But we are interested in a more precise condition for the bounds of  $T, \tau$  that represent equilibrium. We assume a small change in temperature  $\Delta T$  (assumed to be small) and expand  $\langle \sigma v \rangle$  and  $\tau$  to first order in terms of temperature. We adopt  $T \rightarrow k_B T$  to follow the book *Tokamaks by J. Wesson*. Where the subscript *eq* means evaluated at equilibrium position  $T$ .

$$\langle \sigma v \rangle \approx \langle \sigma v \rangle_{eq} + \Delta T \frac{d\langle \sigma v \rangle}{dT} \quad (34)$$

$$\tau \approx \tau_{eq} + \Delta T \frac{d\tau}{dT} \quad (35)$$

We will need to consider the relaxation of the system, on other words, the heat evolution of the system.

$$\frac{dT}{dt} = -\frac{T}{\tau} \quad (36)$$

If we perturb the system, will allows to simplify the expansion of  $\tau$ .

$$\frac{d(T + \Delta T)}{dt} = -\frac{T + \Delta T}{\tau_{eq}} \left( 1 + \frac{\Delta T}{\tau_{eq}} \frac{d\tau}{dT} \right)^{-1} \quad (37)$$

$$\approx -\frac{T + \Delta T}{\tau_{eq}} \left( 1 - \frac{\Delta T}{\tau_{eq}} \frac{d\tau}{dT} \right) \quad (38)$$

$$\approx -\frac{T + \Delta T}{\tau_{eq}} + \frac{T + \Delta T}{\tau_{eq}} \frac{\Delta T}{\tau_{eq}} \frac{d\tau}{dT} \quad (39)$$

$$\approx -\frac{T + \Delta T}{\tau_{eq}} + \frac{T \Delta T}{\tau_{eq}^2} \frac{d\tau}{dT} \quad (40)$$

We notice that by getting  $\frac{d\delta T}{dt}$  many terms will cancel and we will get

$$3n \frac{d\Delta T}{dt} = \left[ -3n \left( \frac{1}{\tau_{eq}} - \frac{T}{\tau_{eq}^2} \frac{d\tau_{eq}}{dT} \right) + \frac{1}{4}n^2 \frac{d\langle \sigma v \rangle}{dT} \epsilon_\alpha \right] \Delta T \quad (41)$$

and substituting (33) in it for

$$3n \frac{d\Delta T}{dt} = \frac{1}{4} \pi^2 \langle \sigma v \rangle \frac{\xi_\alpha}{T} \left( -1 + \frac{T}{\tau_{eq}} \frac{d\tau_{eq}}{dT} + \frac{T}{\langle \sigma v \rangle} \frac{d\langle \sigma v \rangle}{dT} \right) \Delta T \quad (42)$$

To have a failure at ignition due to temperature instabilities, then the right side of (42) has to be negative.

$$\frac{T}{\tau_{eq}} \frac{d\tau_{eq}}{dT} < 1 - \frac{T}{\langle \sigma v \rangle} \frac{d\langle \sigma v \rangle}{dT} \quad (43)$$

which is the strong form of the equilibrium condition

## 4 Tokamaks

A tokamak is a toroidal magnetic confinement system. The fusion devices using this systems were built in Russia in the 1950's. The proposal to use controlled thermonuclear fusion for industrial purposes and a specific scheme using thermal insulation of high-temperature plasma by an electric field was first formulated by the Soviet physicist Oleg Lavrentiev in a mid-1950 paper. In 1951, Andrei Sakharov and Igor Tamm modified the scheme by proposing a theoretical basis for a thermonuclear reactor, where the plasma would have the shape of a torus and be held by a magnetic field.

### 4.1 Magnetic confinement

The main magnetic field in the system is  $B_\phi$  which points in the in the polar angle, here called *Toroidal* by coils with external current.

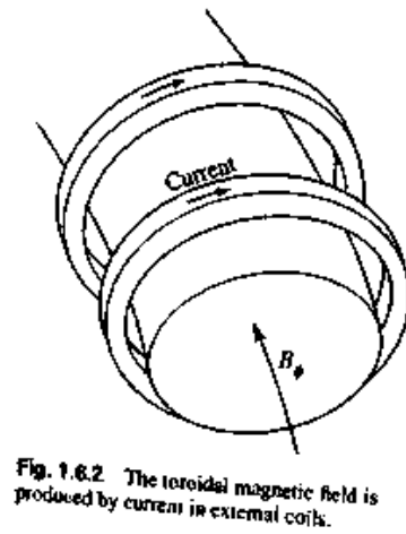


Fig. 4: Toroidal field (TF) coils (*figure from Tokamak by J. Wesson*)

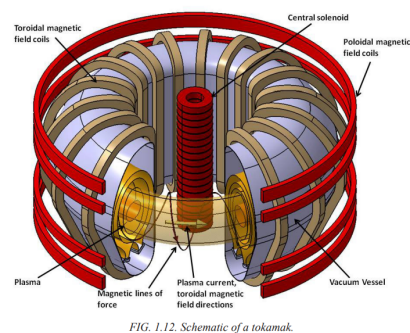


Fig. 5: Caption

## 4.2 Challenges

Sputtering and tritium breeding MHD instabilities tungsten wall sputtering and photon emission (line emission cooling) berilyum wall

## 4.3 Instabilities

Vertical displacements events (VDE)

## **5 Stellarator**

### 5.1 Magnetic confinement

### 5.2 Challenges

### 5.3 Instabilities

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## References

1. Wesson, J. and Campbell, D. Tokamaks. International series of monographs (OUP Oxford, 2011). ISBN 978-0-19-959223-4.
2. Chen, F. F. Introduction to Plasma Physics and Controlled Fusion (Springer International Publishing, Cham, 2016). ISBN 978-3-319-22308-7 978-3-319-22309-4.
3. Schwarz, N. (2020). Vertical Displacement Events in ASDEX Upgrade (IPP 2020-14). Max Planck Institute for Plasma Physics (IPP Garching).