# msre design

## the design and dimensions of the molten salt reactor experiment is detailed in several

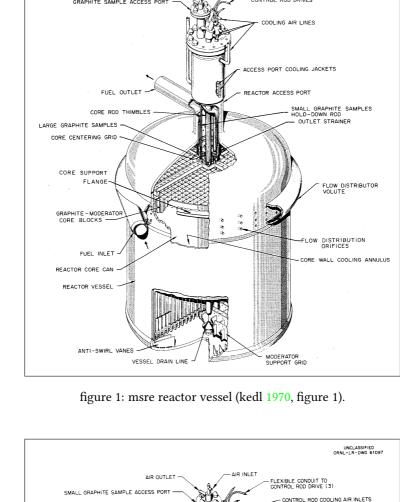
1

introduction

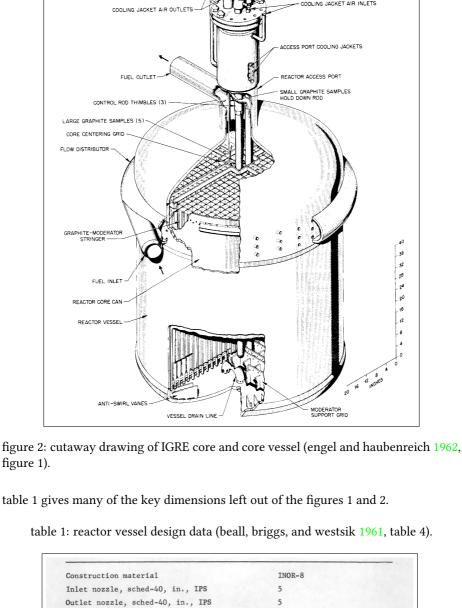
ornl reports. this document aims at providing a comprehensive overview of the core design dimensions and materials, for the purpose of creating an accurate cad model of the reactor. care is taken to give proper references to where data comes from or how its extrapolated from the available information. 2 core design

### the msre core design can be seen in figure 1 and figure 2, note that they differ slight, e.g. the vessel drain line and around the control rod.

ORNL-LR-DWG 61097R1A



CONTROL ROD COOLING AIR OUTLETS



OD, in. 59-1/8 (60 in. max) ID, in. 9/16 Wall thickness, in. Over-all height, in. (to ¢ of 5 in. nozzle) 100-3/4

50

1300

Reactor vessel

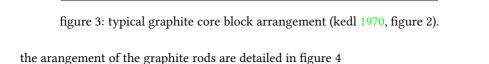
Head thickness, in. Design pressure, psi

Design temperature, °F

Fuel inlet temperature, °F

1225 Fuel outlet temperature, °F Inlet Constant-area distributor Annulus ID, in. Annulus OD, in. 58 55-1/4 Diameter, in. Core block section, in. 2 x 2 Number of fuel channels 1064 1.2 x 0.4 (rounded corners) Fuel channel size, in. Effective core length, in. ~63 Effective core volume, ft<sup>3</sup> ~88 Fractional fuel volume 0.225 Core container ID, in. 55-1/2 1/4 Wall thickness, in. Height, in. 68 graphite rods the graphite moderator rod dimension, given in figure 3, are missing the angle of the spike, the dimensions of the pole, and disk at the bottom. ORNL-LR-DWG 56874 R PLAN VIEW TYPICAL MODERATOR STRINGERS SAMPLE PIECE

NOTE: NOT TO SCALE



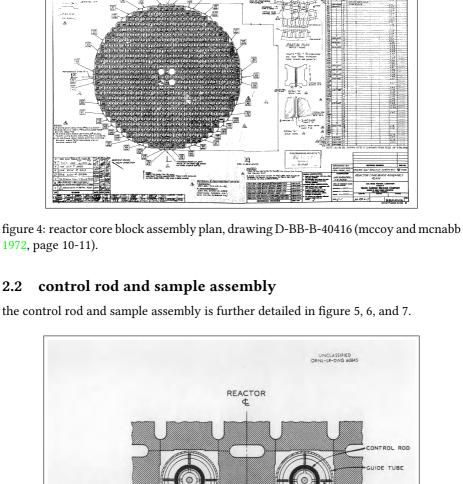


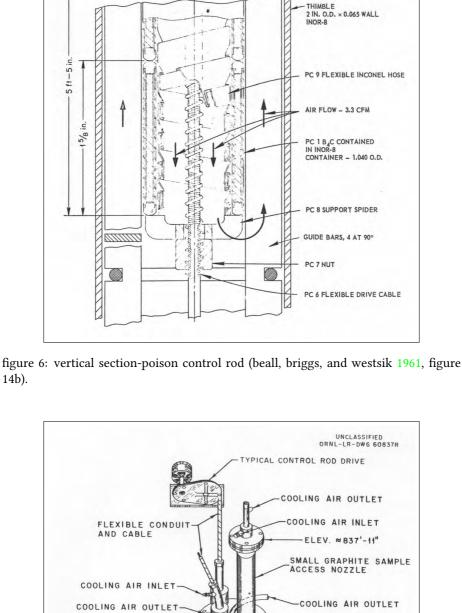
figure 5: msre control rod arrangement (beall, briggs, and westsik 1961, figure 14a).

UNCLASSIFIED ORNL-LR-DWG 61152

C 8 SUPPORT SPIDER

GUIDE TUBE 1½ IN. O.D. × 0.065 WALL

PC 10 SEAL RING



ELEV. ≈837'-11"

REACTOR OUTLET -ELEV. ≈835'-3"

ELEV. ≈833'-8"

ELEV. ≈828'-3"

GRAPHITE CORE BLOCKS

SMALL GRAPHITE SAMPLES

beall, s. e., dr. b. briggs, and j. h. westsik (1961). "addendum to ornl cf-61-2-46, moltensalt reactor experiment preliminary hazards report". In: oak ridge national laboratory ad-cf-61-2-46.

engel, j. r. and p. n. haubenreich (1962). "temperatures in the msre core during steadystate power operation". In: oak ridge national laboratory ornl-tm-0378. kedl, r. j. (1970). "fluid dynamic studies of the molten-salt reactor experiment (msre)

mccoy, h. e. and b. mcnabb (1972). "postirradiation examination of materials from the

REACTOR TOP HEAD

CONTROL ROD THIMBLE-

SEGMENTED CONTROL ROD

CONTROL ROD GUIDE TUBE

figure 7: msre control system (beall, briggs, and westsik 1961, figure 14c). References

core". In: oak ridge national laboratory ornl-tm-3229.

msre". In: oak ridge national laboratory ornl-tm-4174.