

Turbulence and Transport in Fusion Plasmas

Part VIII



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Tuesday Recap

Yesterday, we went over

- saturation of linear instabilities:
 - zonal flows & stable modes
 - ITG vs. ∇T -TEM vs. ∇n -TEM vs. KBM saturation
 - loss of saturation in the non-zonal transition
- quasilinear transport modeling using linear gyrokinetics
- improved modeling with the τ correction

Now, let's finish the quasilinear tasks with our Vlasov code . . .

How to evaluate the ion temperature fluctuation?

Definitions for density & pressure fluctuations (normalized):

$$n_i = n_{i0} \pi B_0 \int f_i dv_{\parallel} d\mu$$

$$p_i = n_{i0} T_{i0} \pi B_0 \int v^2 f_i dv_{\parallel} d\mu$$

Further noting

$$\delta p = \delta(nT) = n_0 \delta T + T_0 \delta n \text{ and } v^2 = v_{\parallel}^2 + v_{\perp}^2$$

we can write a definition for T_i based on n_i and p_i

Group Work: Scans & Quasilinear

2 hours group work

Using your drift-kinetic Vlasov code,

- 1 conduct a 2D parameter scan, over either $k_y\text{-}\omega_{Ti}$ or $k_y\text{-}\omega_n$ at some fixed $k_{||}$ (suggested range $0.1 \leq k_y \leq 0.8$)
- 2 based thereupon, construct a simple quasilinear model

$$Q_i^{\text{QL}} = \omega_{Ti} \sum_{k_y} \frac{\gamma(k_y)}{\langle k_y^2 \rangle}, \quad \langle k_y^2 \rangle = \frac{\int k_y^2 |\Phi(k_y)|^2 dz}{\int |\Phi(k_y)|^2 dz} = k_y^2$$

to gauge heat flux scalings of slab ITG turbulence

- 3 extract the $\Phi \times T_i$ phase and include it in the model, normalized so $\alpha_{\Phi \times T} = \pi/2$ yields 100% Q_i^{QL}

Group Work: Fluid Quasilinear

2 hours group work:

Returning to the (linear) Horton-Holland model solution,

$$\omega_{c1,2} = \frac{k_y}{2 + 2k_{\perp}^2} \pm ik_y \left(\frac{(1 + \eta)\epsilon}{1 + k_{\perp}^2} \right)^{1/2} - \frac{i\nu k_{\perp}^2}{2 + 2k_{\perp}^2}$$

- 1 construct a quasilinear $Q_{QL} = \eta\epsilon \sum_{k_y} C(k_y)\gamma/k_{\perp}^2$ with $k_x = 0$
- 2 evaluate this model for scans over η and ν , assuming the base case $Q_{NL}(k_y) \leftrightarrow C(k_y)$ to be an inverse parabola peaked at $k_y = 0.5$, with no flux below/above $k_y = 0.1/0.9$
- 3 add in the τ correction, assuming $\omega_{ZF} = 0$ and $k_x^{ZF} = 0.1$
- 4 repeat the scans and compare the two models

For those who are interested: P.-Y. Li *et al.*, Phys. Plasmas **28**, 102507 (2021) has a Horton-Holland comparison NL-QL(τ)

Discussion: Benchmark Point

Let us have a short discussion:

What would go into an “ideal” benchmark parameter point?

Benchmark: comparison of different simulation codes

- Why would we want to do this?
- Where in our vast parameter space should we do this?

Questions & Discussion

Any questions about anything nonlinear? Or linear?

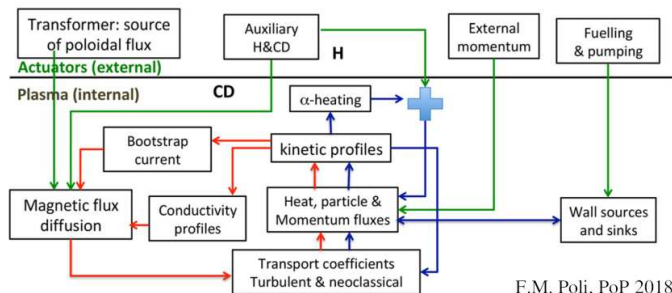
Any feedback for the fella doin' so much talkin'?

Integrated Modeling

Now that we understand turbulence: **integrated modeling**

Fusion reactor: **multi-component, multi-physics** system

⇒ need to use many models, account for complex interactions



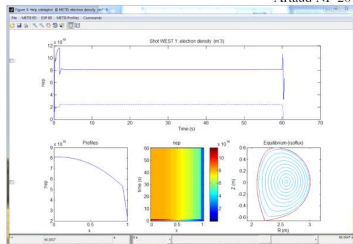
Whole-Device Modeling: a few frameworks, e.g. Romanelli 2014

Shout-out DIFFER expert: Jonathan Citrin, head of IMT group

Aside from fundamental understanding, goal: flight simulator



Artaud NF 2018



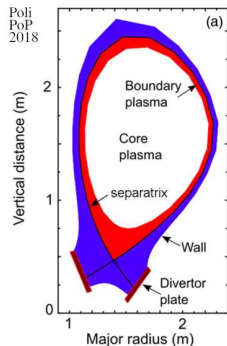
- simulate plasma evolution ahead of experiment
- ensure good performance, no disruptions
- determine good settings for heating, fueling, etc.
- *ideally modular*: many code coupling choices

See, e.g., Felici PPCF 2012, Artaud NF 2018, Janky FED 2021

Need for speed: **cannot use full-physics models**

Core vs. Edge

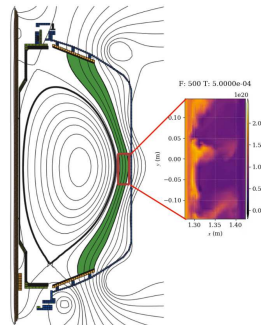
So far, this class has focused on **core plasmas**



Currently hot
research topic:
plasma edge

- pedestal
- separatrix
- scrape-off layer (SOL)

Edge gyrokinetics



(e.g., Shi JPP 2017)

For our purposes, will ignore scrape-off layer—2D transport!
(plus: open field lines, sheath boundary, full- f , neutrals, . . .)

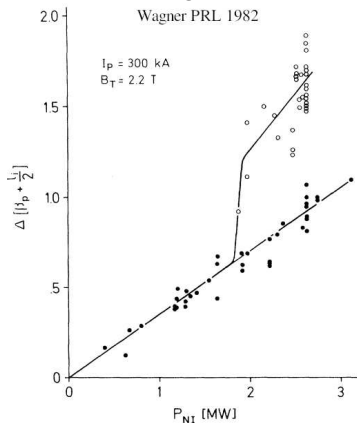
H-Mode & Pedestal

ASDEX: transition **Low- → High-confinement mode**

Heating power boost

⇒ sudden increase in Φ_0

⇒ lower Q , higher τ_E



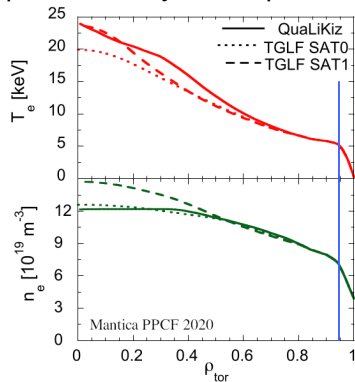
caused by ion orbit losses?

MHD pedestal models:

EPED (Snyder NF 2011)

IMEP (Luda NF 2020)

However, common to just put boundary inside pedestal:



Sources

Fusion reactors: need **heat, particle**, possibly momentum **sources** \Rightarrow compensate losses, refuel D-T

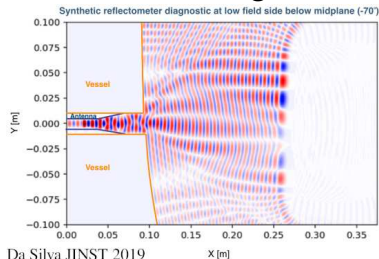
Heat sources:

- ICRH
- ECRH
- NBI

Particle sources:

- pellets
- gas puffing
- *wall erosion*

ECRH: rays; ICRH: need **full-wave modeling**



NBI: localized heat/particle/momentum source \Rightarrow **current drive**

Transport Equations: Particles

How to describe transport and profile evolution mathematically?

$$\text{Vlasov: } \left[\frac{\partial}{\partial t} + \mathbf{v} \cdot \frac{\partial}{\partial \mathbf{x}} + \frac{q_j}{m_j} \left(\mathbf{E} + \frac{\mathbf{v}}{c} \times \mathbf{B} \right) \cdot \frac{\partial}{\partial \mathbf{v}} \right] f_j(\mathbf{x}, \mathbf{v}, t) = \mathcal{C}[f_j]$$

Term by term, take 0th moment:

$$\int d\mathbf{v} \frac{\partial f_j}{\partial t} = \frac{\partial n_j}{\partial t} \quad \int d\mathbf{v} \mathcal{C}[f_j] = 0 \text{ (particle conservation)}$$

$$\int d\mathbf{v} \mathbf{v} \cdot \frac{\partial f_j}{\partial \mathbf{x}} = \nabla \cdot (n_j \mathbf{u}_j) \equiv \nabla \cdot \mathbf{\Gamma}_j \quad (\mathbf{\Gamma}: \text{particle flux})$$

$$\int d\mathbf{v} \left(E_a + \frac{v_b}{c} B_c \right) \frac{\partial f_j}{\partial v_a} = \underbrace{\left[\left(E_a + \frac{v_b}{c} B_c \right) f_j \right]_{-\infty}^{\infty}}_{=0 \text{ } [f(v \rightarrow \infty) = 0]} - \underbrace{\int d\mathbf{v} \frac{\partial}{\partial v_a} \left(E_a + \frac{v_b}{c} B_c \right) f_j}_{=0 \text{ } (\partial_v E = 0 \text{ \& } \partial_{v_a} / \partial_{v_b} = 0)}$$

$$\Rightarrow \text{particle balance } \frac{\partial n_j}{\partial t} + \nabla \cdot \mathbf{\Gamma}_j = S_j \text{ } (S_j: \text{external particle sources})$$

Transport Equations: Heat I

Similar to particle balance: **heat/energy balance**, $\frac{m}{2} \int v^2 d\mathbf{v}$

$$\text{Vlasov: } \left[\frac{\partial}{\partial t} + \mathbf{v} \cdot \frac{\partial}{\partial \mathbf{x}} + \frac{q_j}{m_j} \left(\mathbf{E} + \frac{\mathbf{v}}{c} \times \mathbf{B} \right) \cdot \frac{\partial}{\partial \mathbf{v}} \right] f_j(\mathbf{x}, \mathbf{v}, t) = \mathcal{C}[f_j]$$

$$\frac{m_j}{2} \int d\mathbf{v} \frac{\partial v^2 f_j}{\partial t} = \frac{3}{2} \frac{\partial p_j}{\partial t}$$

$$\frac{m_j}{2} \int d\mathbf{v} \mathbf{v}^3 \cdot \frac{\partial f_j}{\partial \mathbf{x}} = \nabla \cdot \mathbf{Q}_j \quad (\mathbf{Q}: \text{energy/heat flux})$$

$$\frac{q_j}{2} \int d\mathbf{v} v^2 \mathbf{E} \cdot \frac{\partial f_j}{\partial \mathbf{v}} = \underbrace{\frac{q_j}{2} [v^2 E f_j]_{-\infty}^{\infty}}_{=0 [v^2 f(v \rightarrow \infty) = 0]} - q_j \mathbf{E} \cdot \int d\mathbf{v} \mathbf{v} f_j = -q_j n_j \mathbf{E} \cdot \mathbf{u}_j$$

$$\frac{m}{2c} \int d\mathbf{v} v^2 \mathbf{v} \times \mathbf{B} \cdot \frac{\partial f_j}{\partial \mathbf{v}} = \dots = 0 \quad (\text{the magnetic field does no work})$$

However, the collision term does not vanish this time . . .

Transport Equations: Heat II

$$\frac{m_j}{2} \int d\mathbf{v} v^2 \mathcal{C}[f_j] = \frac{m_j}{2} \int d\mathbf{v} (\mathbf{v} - \mathbf{u}_j)^2 \mathcal{C}[f_j] + m_j \mathbf{u}_j \cdot \int d\mathbf{v} \mathbf{v} \mathcal{C}[f_j] \equiv Q_{ei} + \mathbf{u}_j \cdot \mathbf{F}_j$$

with F : collisional friction force; Q_{ei} : collisional energy exchange

$$\Rightarrow \text{energy balance } \frac{3}{2} \frac{\partial p_j}{\partial t} + \nabla \cdot \mathbf{Q}_j = Q_{ei} + Q_{j,\text{fus}} + Q_{j,\text{ext}} + \mathbf{u}_j \cdot (\mathbf{F} + q_j n_j \mathbf{E})$$

(gray terms: negligible for purely **radial transport**)

$Q_{j,\text{fus}}$: fusion power source (j : fusion alphas)

$Q_{j,\text{ext}}$: external heating: ECRH, ICRH, NBI

\mathbf{Q}, Γ : **turbulent** (or neoclassical) **fluxes**

However, still in Cartesian coordinates

Need to **account for** $dV = dV(r) = A(r)dr$ **dependence!**

Group Work: Transport Equations

30 minutes group work:

- 1 Perform volume integration combined with radial derivative on particle balance

$$\frac{\partial n_j}{\partial t} + \nabla \cdot \mathbf{\Gamma}_j = S_j$$

Hint 1: use Gauss divergence theorem for flux term

Hint 2: can insert unity factors like $\frac{dr}{dr}$ and $\frac{\int dA dr}{\int dA dr}$

- 2 Express particle balance in terms of flux-surface averages

$$\langle \xi \rangle \equiv \frac{\int_A \int_0^r \xi \, dA dr}{\int_A \int_0^r dA dr}, \quad \text{noting that } V' \equiv \frac{dV}{dr}$$

(each resultant term should have a $\langle \cdot \rangle$ and a V')

Group Work: Solution

$$\frac{\partial n_j}{\partial t} + \nabla \cdot \mathbf{\Gamma}_j = S_j$$

$$\frac{\partial}{\partial r} \int_A \int_0^r \frac{\partial n_j}{\partial t} dA dr = \frac{\partial}{\partial t} \int_A n_j dA = \frac{\partial}{\partial t} \int_A n_j dA \frac{\frac{dr}{dr} \frac{\int_A \int_0^r dA dr}{\int_A \int_0^r dA dr}} = \frac{\partial V' \langle n_j \rangle}{\partial t}$$

$$\frac{\partial}{\partial r} \int_A \int_0^r S_j dA dr = V' \langle S_j \rangle$$

$$\frac{\partial}{\partial r} \int_A \int_0^r \nabla \cdot \mathbf{\Gamma}_j dA dr = \frac{\partial}{\partial r} \int_A \mathbf{\Gamma}_j \cdot \hat{\mathbf{n}} dA = \frac{\partial}{\partial r} \int_A \Gamma_{j,r} dA \frac{\frac{dr}{dr} \frac{\int_A \int_0^r dA dr}{\int_A \int_0^r dA dr}} = \frac{\partial V' \langle \Gamma_{j,r} \rangle}{\partial r}$$

Typical notation: brackets $\langle \cdot \rangle$ are dropped, $\Gamma_j \equiv \Gamma_{j,r}$

$$\textbf{Overall : } \frac{\partial V' n_j}{\partial t} + \frac{\partial V' \Gamma_j}{\partial r} = V' S_j$$

Transport Equations: Summary

For two-component plasma, ignoring current diffusion:

$$\frac{\partial V' n_i}{\partial t} + \frac{\partial V' \Gamma_i}{\partial r} = V' S_i$$

$$n_e = n_i \quad \Gamma_e = \Gamma_i \quad P_{ei} = -P_{ie}$$

$$\frac{3}{2} \frac{\partial V' n_i T_i}{\partial t} + \frac{\partial V' Q_i}{\partial r} = V' (P_{ie} + P_{i,\text{fus}} + P_{i,\text{ext}})$$

$$\frac{3}{2} \frac{\partial V' n_e T_e}{\partial t} + \frac{\partial V' Q_e}{\partial r} = V' (-P_{ie} + P_{e,\text{ext}})$$

Circular flux surfaces: $V = 2\pi^2 r^2 R \Rightarrow V' = 4\pi^2 r R$,
thus, can replace $V' \rightarrow r$ in above expressions

Typically, need to evaluate $10^3 - 10^5$ times
 \Rightarrow require fast models for Q, Γ

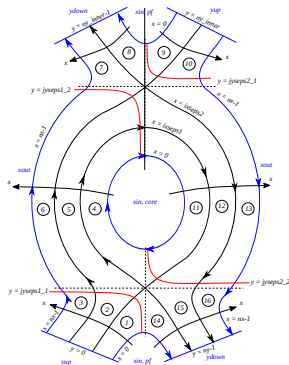
MHD & Neoclassical

Recall: Grad-Shafranov

$$R^2 \nabla \cdot \frac{\nabla \psi}{R^2} = -R^2 \frac{\partial p}{\partial \psi} - F \frac{\partial F}{\partial \psi}$$

used to get B_0 equilibrium

Transport modeling:
prescribe **shape of last closed flux surface (LCFS)**
or (magnetic/inductive)
coil currents



Complement Grad-Shafranov solver with **MHD stability code**

Also need code to **evaluate neoclassical fluxes**,
especially when looking at stellarators (*any idea why?*)

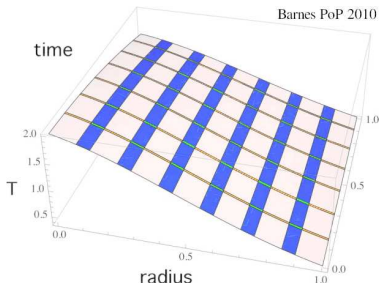
Turbulent Transport

Turbulent transport: **trade-off** between **speed** and **accuracy**

Coupling multiple nonlinear simulations at different r/a :

Can get turbulent Q , Γ from

- nonlinear gyrokinetics
- quasilinear gyrokinetics
- reduced QL models (QuaLiKiz, TGLF)
- simple transport coefficients and critical-gradient models

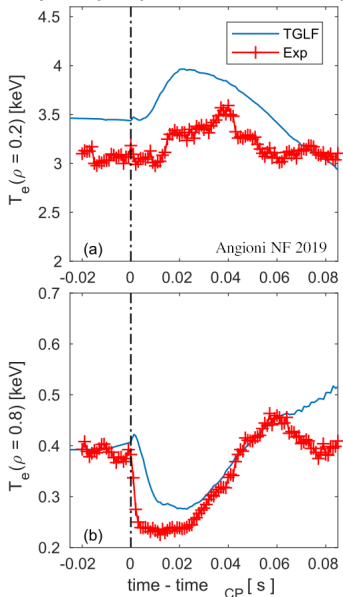


Trinity, Tango frameworks

For real-time control, flight simulators, need **machine learning/neural networks** (can be based off gyrokinetics and/or reduced QL models; e.g., van de Plassche PoP 2020)

Application I

Impurity injection: lowering edge T , expect inward propagation



Experiments (AUG/CMOD/DIII-D):
edge “**cold pulse**” surprisingly
causes T increase in core

Note: low n_e discharge,
 ∇n -TEM turbulence regime

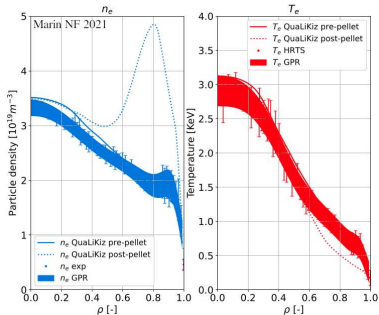
Poorly understood until recently
(Angioni NF 2019):

- 1 impurities decrease T_{edge} ,
increase $n_{e,\text{edge}}$
- 2 impurities, electrons
move inward
- 3 n_e boost flattens ∇n_e
- 4 TEM reduced \Rightarrow lower Q_e
 \Rightarrow core T_e increases

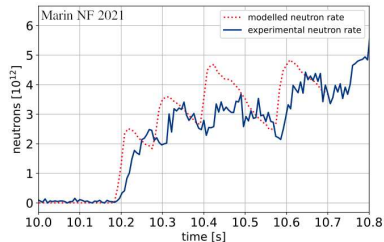
Application II

Key assumption for fusion reactors: deuterium & tritium from pellets need to reach the plasma center

Marin NF 2021: use JINTRAC+QuaLiKiz to analyze D pellet in H plasma \Rightarrow *does D move inward?*



ITG causes D pinch, boosts core n_D & D-D reactions
 \Rightarrow **more fusion neutrons**



Post-injection: massive $\pm \nabla n_i$
 $r/a > 0.8$: TEM; $r/a < 0.8$: ITG

Caveat/future work: JINTRAC requires $T_H = T_D$

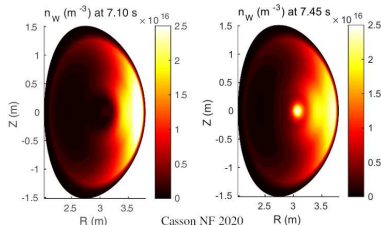
Application III

Recall: tritium retention means no C but W wall needed,
but W in core leads to radiative collapse of T_e

Casson NF 2020: study tungsten accumulation in JET

$\Gamma_W < 0$ due to
neoclassical convection

\Rightarrow fast core accumulation

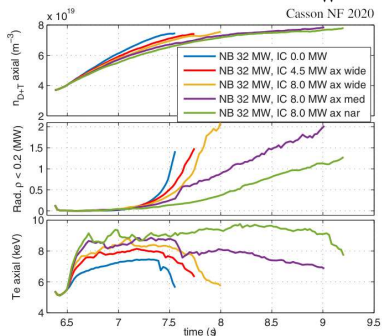


ICRH known to help \Rightarrow why?

ICRH against W not as important in ITER (AUG prefers ECRH)

Rather, the study shows we **can explain/predict the experiment**

By reducing main ion ∇n ,
boost ITG \Rightarrow turbulent $\Gamma_W > 0$



Questions & Discussion

Any questions about integrated modeling? Other comments?