Status of Recent Activities by the APEX Materials Group

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Chemical Compatibility and Radiation Effects Issues in High Temperature Refractory Metals

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Possible Structural Materials for High Wall Loading Concepts

• Low-activation materials

Vanadium alloys Ferritic/martensitic (8-9%Cr) steels, ODS steels SiC/SiC composites

• Refractory alloys

Nb-1Zr Nb-18W-8Hf T-111 (Ta-8W-2Hf) TZM (Mo-0.5Ti-0.1Zr-0.02C) Mo-Re W-25Re

Composites

Fe₃A1

TiAl

Intermetallics

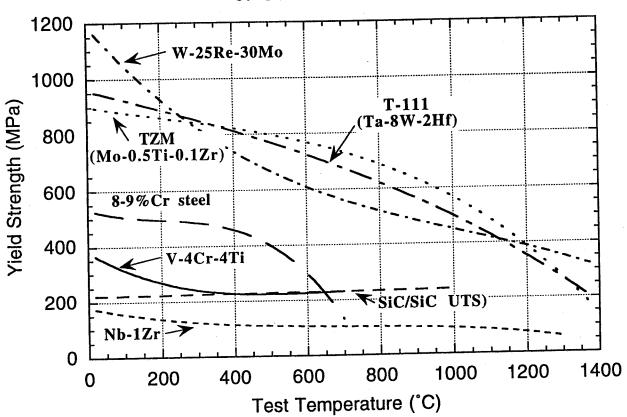
C/C
metal matrix composites
Cu-graphite
Ti₃SiC₂ composites

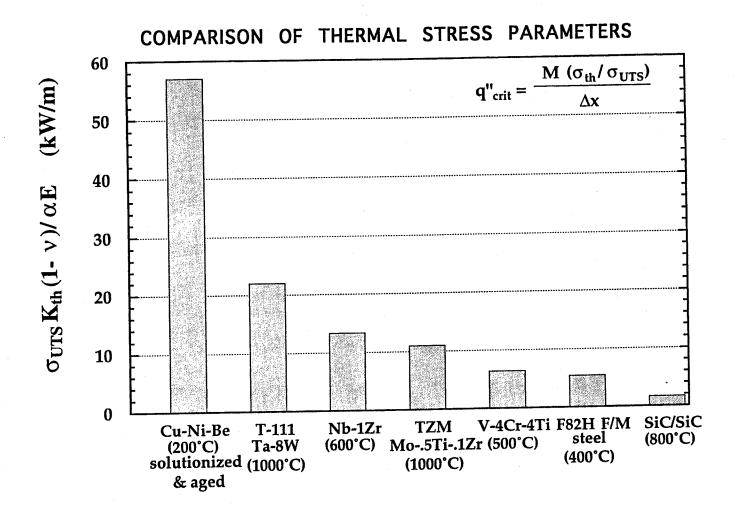
- Ni-based superalloys
- Porous-matrix metals and ceramics

Factors Affecting Selection of Structural Materials

- Unirradiatiated mechanical and thermophysical properties
- Chemical compatibility/corrosion effects
- Materials availability / fabricability / joining technology
- Radiation effects
- Safety aspects (decay heat, induced radioactivity, etc.)

Comparison of the Yield Strengths of Several Materials





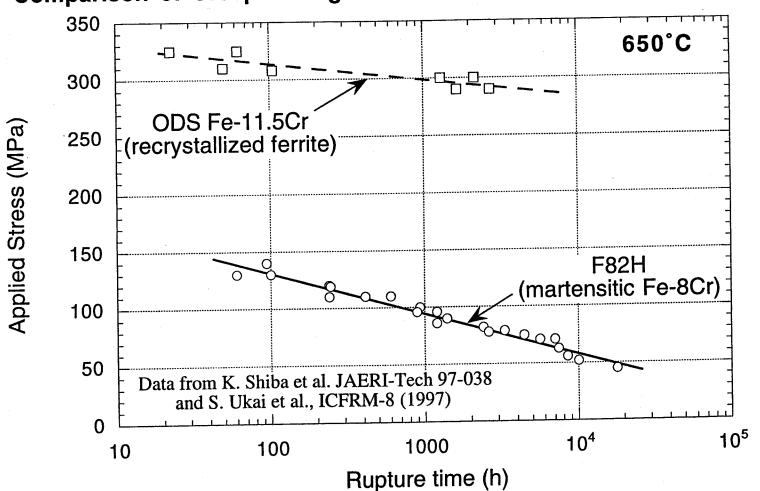
Resources for Structural Materials Database

- Fusion Materials Properties Handbook / ITER Materials Properties Handbook, ed. J.W. Davis (Boeing/St Louis)
 - V alloy chapter is incomplete in present version of IMPH (pub. 4); updated version to be included in IMPH pub. 5, expected summer 1998
 - nothing yet compiled for F/M steels (to be initiated in pub. 5), SiC/SiC
- Aerospace Structural Metals Handbook (1963-1988), ed. W.F. Brown, Jr.
 mechanical and thermophysical properties of refractory alloys vs.
 temperature
- Proc. Conf. on Refractory Alloys for Space Nuclear Power Applications, eds. R.H. Cooper, Jr. and E.E. Hoffman, CONF-8308130 (1984)
- Original research publications

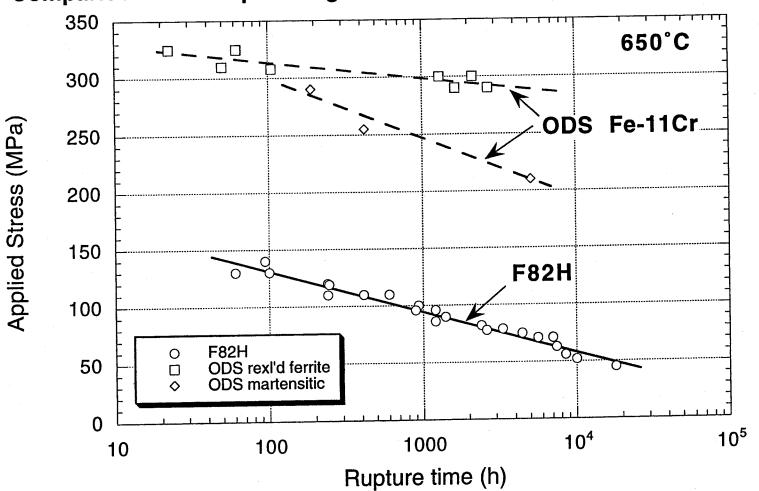
Recent Activities by the APEX Materials Group

- Completed assessment reports on properties of unirradiated and irradiated low-activation structural materials
 - V-4Cr-4Ti
 - Fe-8Cr ferritic/martensitic steel
 - SiC/SiC composites (irradiated thermal conductivity is a key issue)
- Information on oxide dispersion strengthened ferritic and martensitic steels is being compiled
 - operation of an ODS ferritic steel up to 650°C or higher appears feasible, based on thermal creep data
- Assessment reports on high temperature refractory alloys are in progress
 - draft report completed for T-111 (Ta-8W-2Hf)
- Chemical compatibility of refractory alloys with liquid metals is being compiled
- Limited database on radiation effects on refractory alloys has been assembled
 - no fracture toughness or Charpy impact tests

Comparison of Creep Strength of F82H and ODS Fe-11.5Cr steels



Comparison of Creep Strength of F82H and ODS Fe-11.5Cr steels



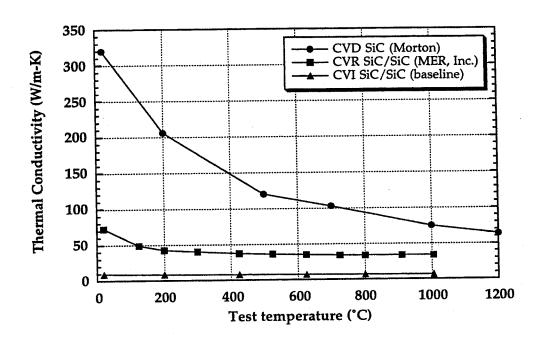


Fig. 1. Comparison of the transverse thermal conductivity of monolithic CVD SiC and two grades of SiC/SiC composites [11,18].

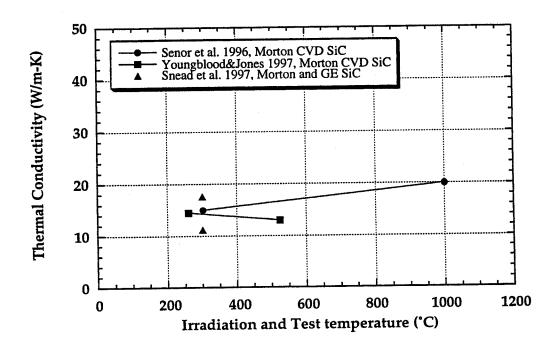
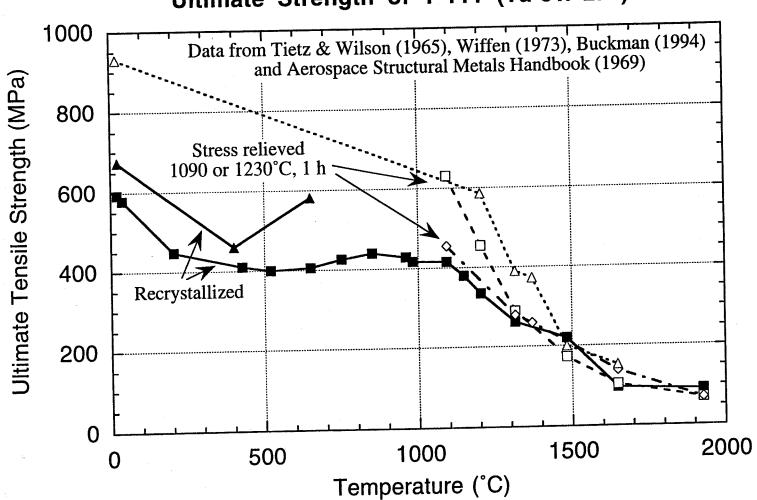
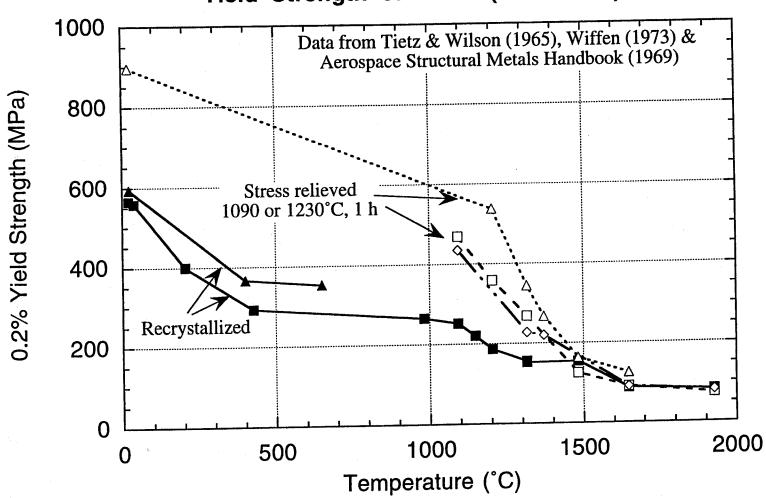


Fig. 2. Effect of low-temperature neutron irradiation on the thermal conductivity of SiC [9,19,20]. The studies by Senor et al. and Youngblood and Jones were performed on samples irradiated to 33-43 dpa, whereas the data by Snead et al. were obtained on samples irradiated to 0.1 dpa.

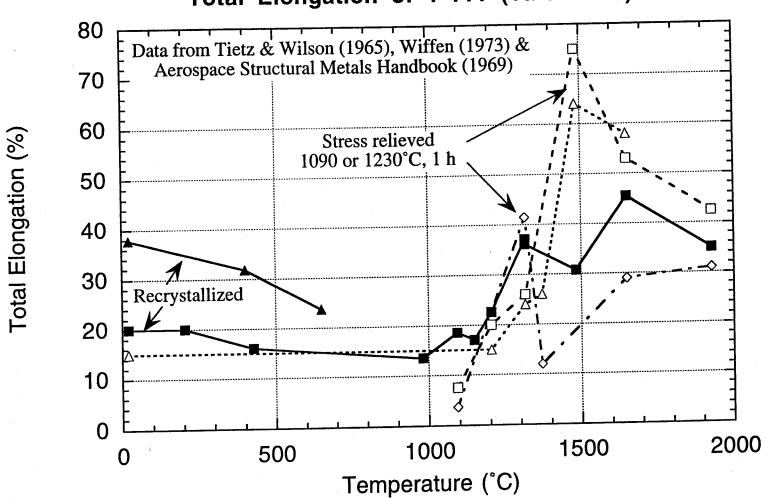
Ultimate Strength of T-111 (Ta-8W-2Hf)



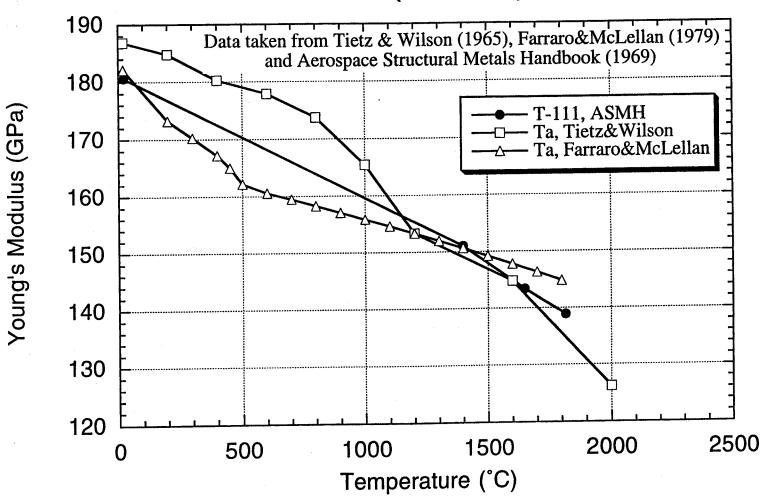
Yield Strength of T-111 (Ta-8W-2Hf)



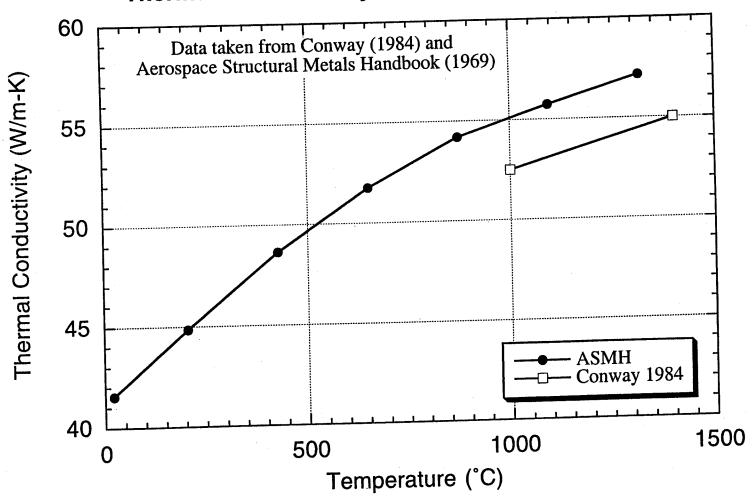
Total Elongation of T-111 (Ta-8W-2Hf)



Elastic Modulus of T-111 (Ta-8W-2Hf) and Tantulum







Chemical Compatibility of High Temperature Refractory Alloys with Liquid Metals

• In general, the refractory alloys have very good compatibility with the liquid metals of interest for fusion applications

- impurity pickup is the key engineering issue

• Li chemical compatibility data base: T-111 (Ta-8W-2Hf) data up to 1370°C (good compatibility; static and circulating loops)

Nb-1Zr data up to 1000°C (good compatibility; static and circulating loops)

W alloys up to 1370°C (attack observed at ≥1540°C)

Mo alloys (TZM) up to 1370°C (attack observed at ≥1540°C)

Overview of Radiation Effects in Refractory Metals

• Void swelling is not anticipated to be a lifetime-limiting issue due to the BCC structure of the high-temperature refractory alloys

- existing fission reactor data base indicate low swelling (<2%) for doses up to 50 dpa or higher

- effects of fusion-relevant He generation on swelling is uncertain

- swelling regimes are ~600 to 1000°C for all 4 classes of refractory alloys
- The Group Va alloys (Nb, Ta) exhibit better ductility before and after irradiation
 - very limited mechanical properties data base on irradiated Nb, Ta alloys
 - extensive mechanical properties data base on irradiated Mo, W alloys
- Very limited or no fracture toughness/Charpy impact data on irradiated high temperature refractory alloys
 - "tensile DBTT" of Mo, W alloys increases to very high values even for low dose irradiations at moderate temperatures (e.g., 600°C after ~1 dpa irradiation at 300 C for W, W-10Re)
- Refractory alloys are generally designed for use in stress-relieved condition (rather than recrystallized) in order to achieve of higher strength
 - radiation-enhanced recrystallization and/or radiation creep effects need to be investigated (designs should use recrystallized strengths to be conservative)