# Thermomechanical Properties of W-Re Alloys & Initial Survey of Molten Tin Corrosion Data

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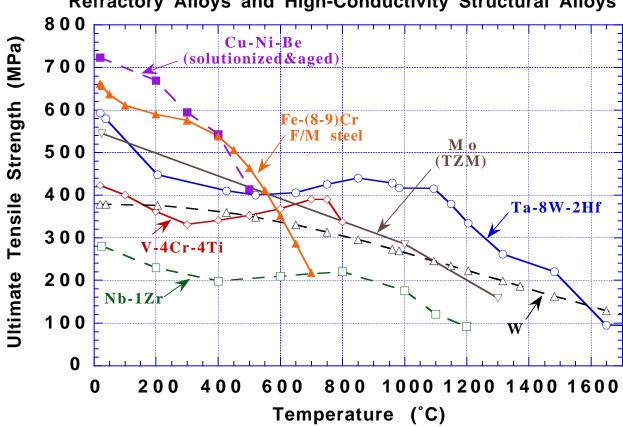
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## **Motivation for Studying W-Re Alloys**

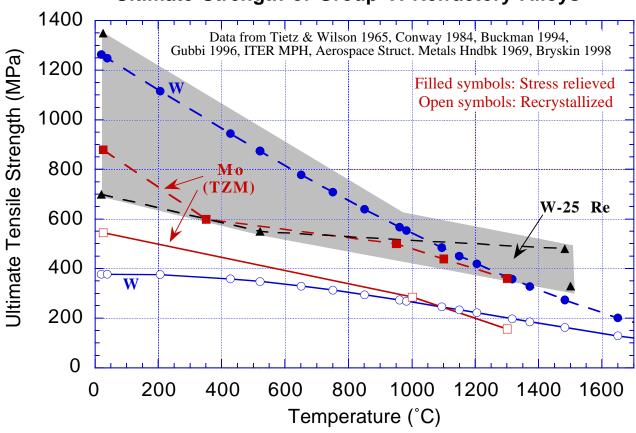
- Tensile strength of recrystallized W is relatively low compared to other refractory alloys (TZM, Ta-8W-2Hf) for temperatures below 1200-1500°C
  - recrystallized tungsten UTS=380 MPa at 20°C
- Pure tungsten has poor fabricability
- W-(5-25%)Re alloys offer potential for improved low temperature fabricability
  - Low temperature (<1200°C) strength is not necessarily higher than that of pure W (depends on thermomechanical processing; "solution softening" usually occurs in W-Re alloys, cf. Klopp 1975)
- Hafnium and carbon are typically added to tungsten alloys (~0.5 at.% Hf, C) in order to improve high temperature creep strength
  - W-Re (Hf,C) alloys offer possibility of improved creep resistance at high temperatures (>1200°C) compared to pure W

Data from Tietz & Wilson (1965), Conway (1984), Buckman (1994), Zinkle et al (1998), ITER MPH, and Aerospace Structural Metals Handbook (1969)





#### **Ultimate Strength of Group VI Refractory Alloys**



# **Summary of Recrystallized Tungsten Properties (from IMPH)**

### **Ultimate Tensile Strength (unirradiated)**

$$\sigma_{\text{UTS}}(\text{MPa}) = 377.9 + 0.03207 * \text{T} - 1.955 \times 10^{-4} * \text{T}^2 + 5.129 \times 10^{-8} * \text{T}^3$$
 (T in °C)

#### **Yield Strength (Unirradiated)**

$$\sigma_{\rm Y}({\rm MPa}) = 94.2 - 0.0214 * {\rm T} - 2.12 \times 10^{-4} * {\rm T}^2 - 7.48 \times 10^{-10} * {\rm T}^3$$
 (T in °C)

#### **Elongation**

$$e_{tot}(\%) = 20.8 + 0.053*T - 2.18x10^{-5}*T^2$$
 (T>500°C)

#### Elastic constants

$$E_{\rm Y}$$
 (GPa) =398 - 0.00231\*T - 2.72x10<sup>-5</sup> T<sup>2</sup> (T in °C)  
v=0.279 + 1.09x10<sup>-5</sup> T (T in °C)

#### Thermophysical properties

$$\alpha_{m} (10^{-6})^{\circ} \text{C}) = 3.922 + 5.835 \times 10^{-5} \times \text{T} + 5.705 \times 10^{-11} \times \text{T}^{2} - 2.046 \times 10^{-14} \times \text{T}^{3}$$
 (T in °C) 
$$C_{p} (J/kg-K) = 128.3 + 0.0328 \times \text{T} - 3.41 \times 10^{-6} \times \text{T}^{2}$$
 (T in °C) 
$$K_{rh} (W/m-K) = 174.9 - 0.107 \text{ T} + 5.01 \times 10^{-5} \text{ T}^{2} - 7.835 \times 10^{-9} \times \text{T}^{3}$$
 (T in °C)

#### Recommended operating temperature limits (structural applications)

Tmin =  $800^{\circ}$ C (due to rad.-induced increase in DBTT at low  $T_{irr}$ ) Tmax =  $1400^{\circ}$ C (Li, Pb-Li corrosion/chemical compatibility and thermal creep)

## **Summary of Recrystallized W-(5-10%) Re Properties**

#### **Ultimate Tensile Strength (unirradiated)**

 $\sigma_{\text{UTS}}(\text{MPa}) = 377.9 + 0.03207 * \text{T} - 1.955 \times 10^{-4} * \text{T}^2 + 5.129 \times 10^{-8} * \text{T}^3$  (T in °C) –use pure W values

#### **Yield Strength (Unirradiated)**

 $\sigma_{\rm Y}({\rm MPa}) = 94.2 - 0.0214 * {\rm T} - 2.12 \times 10^{-4} * {\rm T}^2 - 7.48 \times 10^{-10} * {\rm T}^3$  (T in °C) –use pure W values

#### **Elongation**

 $e_{tot}(\%) = 20.8 + 0.053*T - 2.18x10^{-5}*T^2$  (T>500°C) --use pure W values

#### Elastic constants

 $E_{\rm Y}$  (GPa) =398 - 0.00231\*T - 2.72x10<sup>-5</sup> T<sup>2</sup> (T in °C) --pure W values; W-25Re E(20°C)=410 GPa v=0.279 + 1.09x10<sup>-5</sup> T (T in °C) W-25Re v(20°C)=0.30, G(20°C)=159 GPa

#### Thermophysical properties

 $\alpha_{m} (10^{-6}/^{\circ}C) = 3.9 + 5.8 \times 10^{-5} \times T + 5.7 \times 10^{-11} \times T^{2} - 2.0 \times 10^{-14} \times T^{3} \qquad (T \text{ in } ^{\circ}C) \text{ --use pure W values}$   $C_{p} (J/kg-K) = 128 + 0.033 \times T - 3.4 \times 10^{-6} \times T^{2} \quad ??? \quad (T \text{ in } ^{\circ}C) \quad \text{--use pure W values}$   $K_{th} (W/m-K) \sim 85 \text{ W/m-K} \quad (1000-2400 ^{\circ}C) \qquad \text{--conductivity decreases with increasing Re content}$ 

#### Recommended operating temperature limits (structural applications)

Tmin =  $800^{\circ}$ C (due to rad.-induced increase in DBTT at low  $T_{irr}$ )
Tmax =  $1400^{\circ}$ C (Li, Pb-Li corrosion/chemical compatibility and thermal creep)

# Maximum temperatures of structural alloys (bare walls) in contact with high-purity liquid coolants, based on a 5 $\mu$ m/yr corrosion limit

	Li	Pb-17 Li	Flibe
F/M steel	550-600°C [1,2,3]	450°C [1,2,9 ]	700°C ? 304/316 st. steel [13]
V alloy	600-700°C [1,4,5]	~650°C [1,10]	?
Nb alloy	>1300°C [6,7]	>600°C [10] (>1000°C in Pb) [11]	>800°C [14]
Ta alloy	>1370°C [6,7]	>600°C [10] (>1000°C in Pb) [11]	?
Mo	>1370°C [6,7]	>600°C [10]	>1100°C? [15,16]
W	>1370°C [6,7]	>600°C [10]	>900°C? [15]
SiC	~550°C ? [8]	>800°C ? [12]	?

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#### Chemical Compatibility of Structural Materials with Molten Tin (static tests)

Nb: no corrosion observed at ~600°C chemical attack occurred at 800°C [1] and 1000°C [2,3]

Ta: chemical attack observed at both 600-630 [1,4] and 800°C [1] intergranular penetration observed at 1000°C [2,3,5]

Mo: minimal corrosion observed below ~600°C [4] chemical attack observed at both 630 and 800°C [1] significant corrosion (predominantly intergranular) observed at 1000°C [2,3-5,6] -1.7% weight loss after 340 h at 1000°C [4,6]

W: good chemical resistance at 630°C; moderate attack at 800°C [1] Very little corrosion (10 ppm weight loss) observed after 40 h at 1000°C [6] moderate corrosion (<5 μm) observed after 100 h at 1000°C [3]

Austenitic, Ferritic stainless steels: rapid attack at temperatures above 400-500°C [7]

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Summary of maximum temperatures of structural alloys (bare walls) in contact with high-purity liquid or gaseous coolants, based on a 5  $\mu$ m/yr corrosion limit

	Li	Pb-17 Li	Sn-20 Li (pure Sn)	Flibe	He*
F/M steel	550-600°C	450°C	~400°C	700°C ? 304/316 st. steel	
V alloy	600-700°C	~650°C	?	?	~600°C?§
Nb alloy	>1300°C	>600°C (>1000°C in Pb)	800-850°C	>800°C	?
Ta alloy	>1370°C	>600°C (>1000°C in Pb)	>600°C (>900°C/Sze)	?	?
Мо	>1370°C	>600°C	>700°C (1000°C/Sze)	>1100°C?	~1100°C **
W	>1370°C	>600°C	~1000°C	>900°C?	~1100°C **
SiC	~550°C ?	>800°C ?	?	?	•

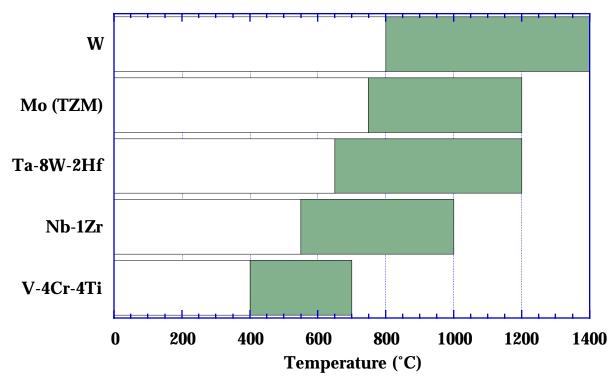
<sup>\*</sup> assumes 1 appm O in 50 MPa He gas

§ the temperature limit for vanadium and other Group V metals in helium will be determined by oxide dissolution and oxygen absorption kinetics; recent work (e.g., B.A. Pint et al. 1998) suggests that the temperature limit for V-4Cr-4Ti may be ~600°C due to interstitial oxygen hardening/embrittlement effects

dashed line (--) indicates that the corrosion-based temperature limit is higher than the structural temperature limit

<sup>\*\*</sup> see accompanying APEX presentation by N.M. Ghoniem

# **Estimated Operating Temperature Limits for Refractory Alloys in Fusion Reactors**



- § Lower temperature limit based on radiation hardening/ fracture toughness embrittlement ( $K_{1C}$ <30 MPa-m<sup>1/2</sup>)
- § Upper temperature limit based on 100 MPa creep rupture strength; chemical compatibility considerations may cause further decreases in the max operating temp.