

# PIRS: Python-based Framework for coupled MC-TH Reactor Calculations

**SNA-MC 2013** 



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#### What is PIRS



### PIRS: Python Interfaces for Reactor Simulations

A set of packages for Python programming language, to facilitate interaction with reactor calculation codes.

#### Python

- www.python.org
- Free

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- Interpreted
- OS-independent
- Big community
- Lot of packages

#### Interaction with code

- Model description
- Generation of Input file(s)
- Job submission
- Reading of calculation results

#### What for?



- Routine preparation of input files
- Framework for coupled calculations
- ...



#### Geometry:

```
from hpmc import Box, Cylinder
b = Box(X=1.2, Y=1.2, Z=110)
c = Cylinder(R=0.5, Z=100)
b.insert(0, c)
b.material = 'water'
c.material = 'fuel'
b.dens.set_grid([1, 1])
b.dens.set_values(1.)
c.temp.set_grid([1]*3)
c.temp.set_values([300, 500, 350])
c.heat.set_grid([1]*10)
```



#### Neutronics:

```
from hpmc import McnpInterface
from mcnp import Material
m = McnpInterface(b)
u = Material((92235, 0.5, 2),
             (92238, 95.5, 2))
o = Material('0')
h = Material('H')
f = u + 2*o
w = h*2 + o
w.thermal = 'lwtr'
```

```
f.sdict[8018] = 8016
w.sdict[8018] = 8016
m.materials['fuel'] = f
m.materials['water'] = w
m.bc['radial'] = '*'
m.adc.append('ksrc 0 0 0')
m.adc.append('kcode 500 1. 20 100')
m. run ('P')
r = m.run(R)
```

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#### Generated MCNP input file

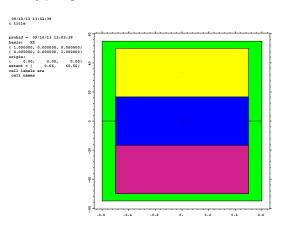
```
c title
1 0 -3 4 -5 6 -2 1 fill=1 imp:n=1
2 0 -7 fill=2 imp:n=1 u=1
3 1 -1.0 -8 imp:n=1 tmp=2.585203e-08 u=2
4 2 -1.0 8 -9 imp:n=1 tmp=4.308671e-08 u=2
5 3 -1.0 9 imp:n=1 tmp=3.016070e-08 u=2
6 4 -1.0 -10 7 imp:n=1 tmp=2.585203e-08 u=1
7 4 -1.0 10 7 imp:n=1 tmp=2.585203e-08 u=1
8 0 11 (3:-4:5:-6:2:-1) imp:n=0 tmp=2.585203e-08
c surfaces
1 pz -55.0
2 pz 55.0
*3 px 0.6
*4 px -0.6
*5 pv 0.6
*6 pv -0.6
7 rcc 0.0 0.0 -50.0 0.0 0.0 100.0 0.5
8 pz -16.666666667
9 pz 16.66666667
10 pz 0.0
11 pz -1055.01817881
c data cards
c materials
                       $ mixture U-O at 300 K
m1
     92235 31c 5 0000000e=01
     92238.31c 9.5500000e+01
      8016 31c 1 9951400e+00
       8017 31c 7 6000000e=04
       8016.31c 4.1000000e-03
```

```
$ mixture II-0 at 500 K
     92235.31c 3.9962042e-01 92235.40c 1.0037958e-01
     92238 31c 7 6327500e+01 92238 40c 1 9172500e+01
       8016 31c 1 5945974e+00 8016 40c 4 0054263e=01
      8017.31c 6.0742304e-04 8017.40c 1.5257696e-04
      8016 31c 3 2768874e=03 8016 40c 8 2311256e=04
                       $ mixture U-O at 350 K
     92235.31c 5.0000000e-01
     92238 31c 9 5500000e+01
      8016.31c 1.9951400e+00
      8017 31c 7 6000000e=04
      8016 31c 4 1000000e=03
                       $ mixture H-O at 300.0 K
      1001.31c 1.9997700e+00
      1002.31c 2.3000000e-04
      8016.31c 9.9757000e-01
       8017 31c 3 8000000e=04
      8016.31c 2.0500000e-03
mt4 lutr01 31t
                       $ thermal data at 293 606K
c tallies
fmesh14:n
                       $ heat in ('/', 0)
    geom=cvl
    origin=0.0 0.0 -50.0
     axs=0.0 0.0 1.0
     vec=1 0 0 0 0 0
    imesh= 0.5
     jmesh= 10.0 20.0 30.0 40.0 50.0 60.0
           70 0 80 0 90 0 100 0
     kmesh= 1.0
fm14 -1 0 -6 -8
prdmp i i 1
                        $ write mctal file
                                      4 P + 4 E + 40 Q C
ksrc 0 0 0
```

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#### Geometry plot generated with MCNP



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### Concept



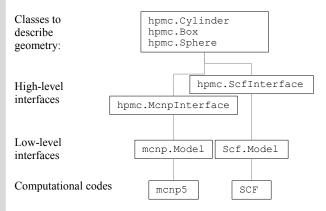
### Class types

- To describe calculation geometry: Solids (Cylinder, Box) can be inserted into each other and positioned with respect to container.
- Low-level interfaces:
  - Assure correct syntax of input file,
  - "Know" command line parameters of the code,
  - provide functions to read output.
- High-level interfaces:
  - Conversion: geometry ⇒ low-level interfaces,
  - Interface to specify code-specific parameters

## Concept



#### Interaction between classes



reactor calculations

### Geometry classes

model dimensions, axial meshes for system variables (temperature, density, heat)

High-level interfaces code-specific data, e.g. path to xsdir, isotopic material compositions for MCNP

## Current development status



Governed by HPMC project: Monte-Carlo neutronics and sub-channel TH modelling of PWR assembly.

#### Interface to MCNP

- handles any geometry represented by boxes and cylinders
- Repeated structure can be modelled as lattice
- Description of materials
  - Convenient definition of composition
  - Automatic choice of suffixes and interpolation of XS
- Reading of meshtal

### Means to set up geometry

- Cylinder: vertical cylinder of finite height
- Box: rectangular parallelepiped with facets perpendicular to axes

### Interface to SCF

- Only for PWR-like geometries
- Reading of output.txt

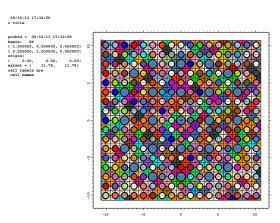


Coupled MCNP - SCF calculations, organized with PIRS

- Model: PWR assembly 17x17,
- moderator channels.
- two different types of fuel pins,
- boron in coolant.
- Coolant-centered subchannels.
- non-uniform axial mesh.
- Calculations conducted on a linux desktop
- relaxation scheme for power axial distribution with varying relaxation parameter and increasing statistical precision

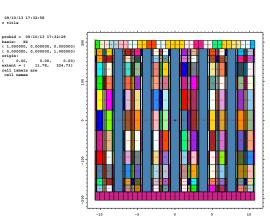


#### Horizontal cross-section of MCNP model:



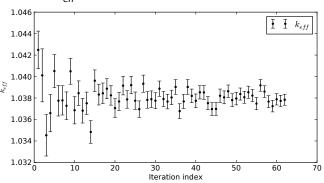


#### Vertical cross-section of MCNP model:



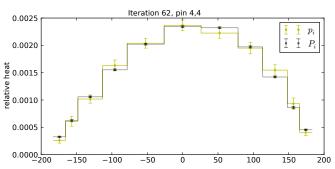


#### Behaviour of $k_{eff}$ with N–TH iterations





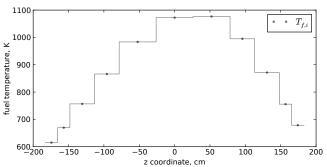
Axial distribution of heat deposition in one fuel pin



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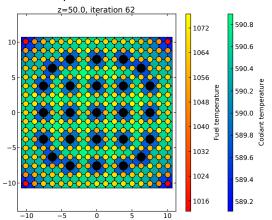
Axial distribution of fuel temperature in the same pin



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#### Temperature map



## Ongoing work



#### Curent

- Interface to SERPENT-2
- Interaction with job submission system at Juelich Supercomputer
- Writing documentation
- Extension of SCF interface to represent cluster of assemblies, calculation of a minicore

#### Plans

- New interface to SCF
- Geometry plotter
- Interface to NMC, parts of KANEXT

## **Appendix**



#### Relaxation scheme



Based on J.Dufek, W.Gudowski, "Stochastic Approximation for Monte Carlo Calculation of Steady-State Conditions in Thermal Reactors", *Nucl. Sci. Eng.*, Vol. 152, 2006, pp. 274.

#### Declared quantities

The following quantities are declared:

- i the index of an iteration step of the coupling scheme,
- s<sub>i</sub> number of neutron histories simulated by a Monte Carlo criticality code in ith iteration step.
- $S_i$  combined number of neutron histories simulated in all iteration steps  $1,\dots,i.$
- $\tilde{\rho}^{(i)}$  distribution of the coolant density computed according to relaxed power distribution  $\tilde{P}^{(i)}$  (see below),
- $\vec{p}^{(i)}$  power distribution computed by the MC criticality code in *i*th step
  - $\alpha$  stepsize for the relaxation scheme,
- $\vec{P}^{(i)}$  relaxed power distribution, computed in ith step,
- $c_i$  the number of active cycles (generations) simulated by the MC code at  $i{\rm th}$  iteration step.

#### Description of the coupling scheme

## Algorithm 1 Basic scheme for coupled NK-TH calculations input: s<sub>1</sub>, b, $\tilde{\rho}^{(0)}$

```
S_0 \leftarrow 0
```

for  $i \leftarrow 1, 2, ... do$  $s_i \leftarrow (s_1 + \sqrt{s_1^2 + 4s_1S_{i-1}})/2$ 

 $s_i \leftarrow (s_1 + \sqrt{s_1^2 + 4s_1}S_i - s_i)$ 

 $c_i \leftarrow integer(s_i/b)$  $\vec{p}^{(i)} \leftarrow power distribution in a system with coolant density <math>\vec{\rho}^{(i-1)}$ 

 $\rho$  bower distribution in a system with coolain density  $\rho$  distribution based on MC crit. simulation with  $c_i$  active cycles

 $S_i \leftarrow S_{i-1} + s_i$  $\alpha_i = s_i/S_i$ 

 $\vec{P}^{(i)} \leftarrow (1 - \alpha_i)\vec{P}^{(i-1)} + \alpha_i \vec{p}^{(i)}$ 

remormalize  $\vec{\vec{P}}^{(i)}$  so that the system gives required power

 $\vec{\rho}^{(i)} \leftarrow$  calculation of the steady-state coolant density distribution based on power distribution  $\vec{P}^{(i)}$ 

end for