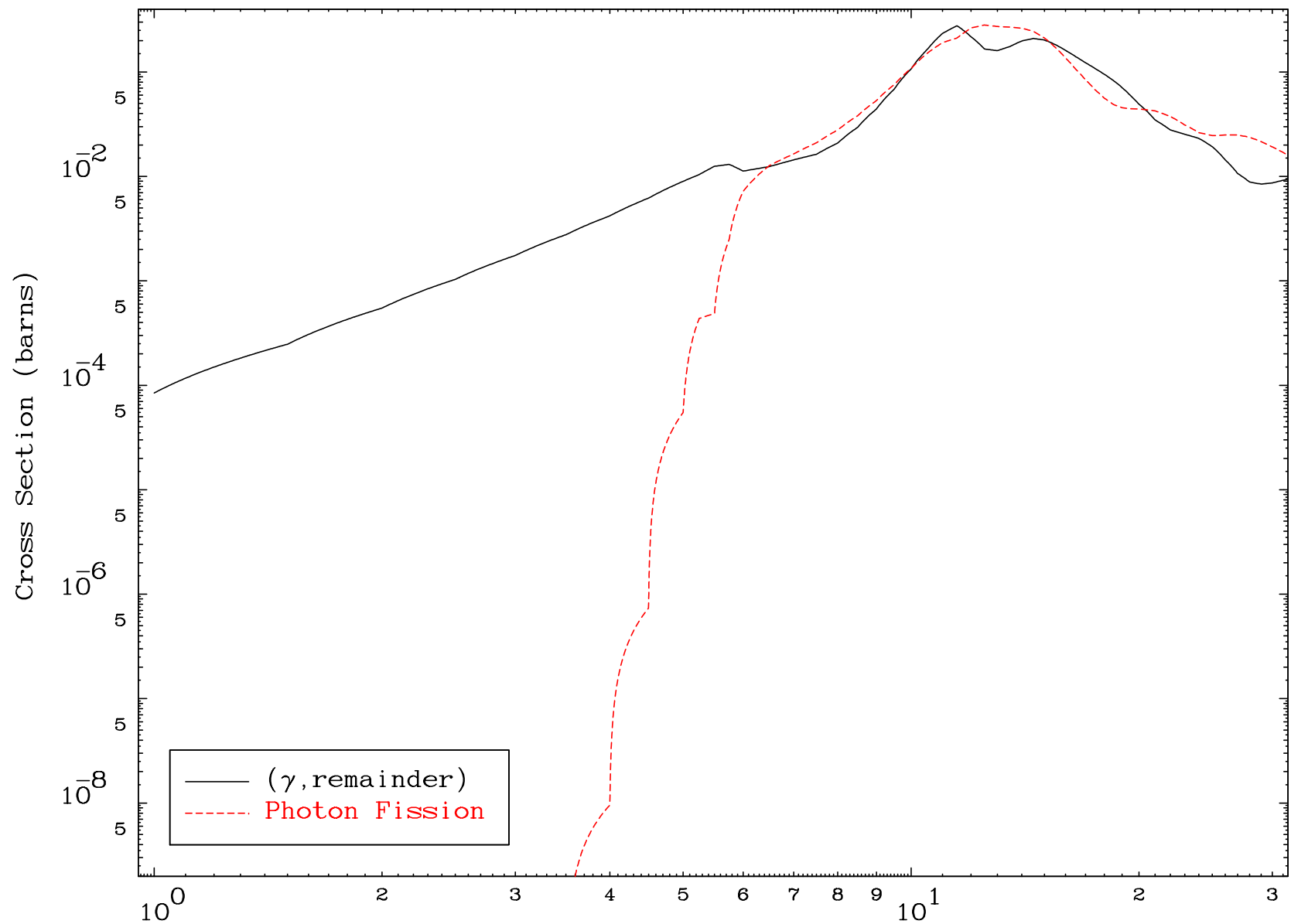
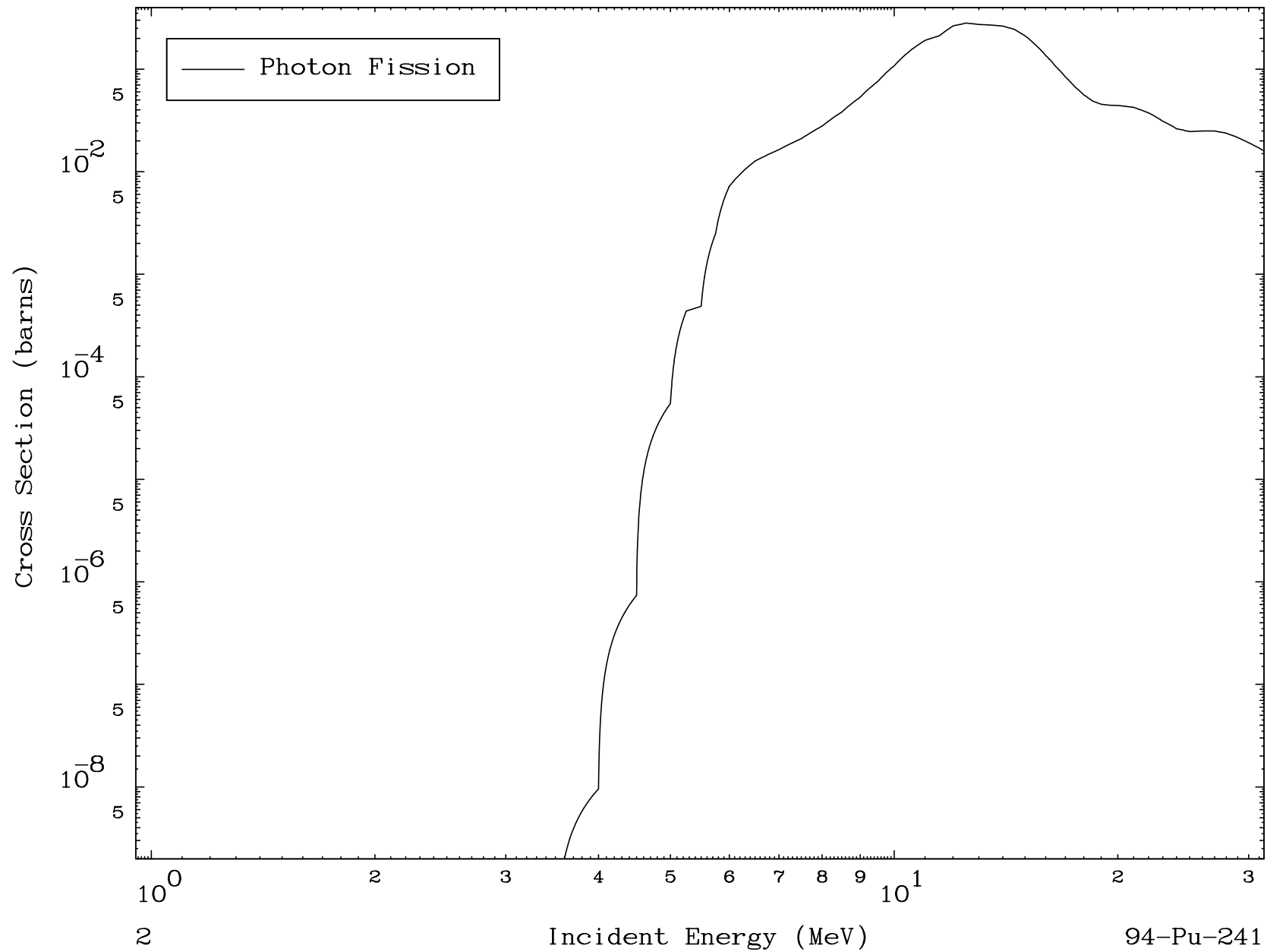


MAT 9443

Photon Major  
0 Kelvin Cross Sections

94-Pu-241

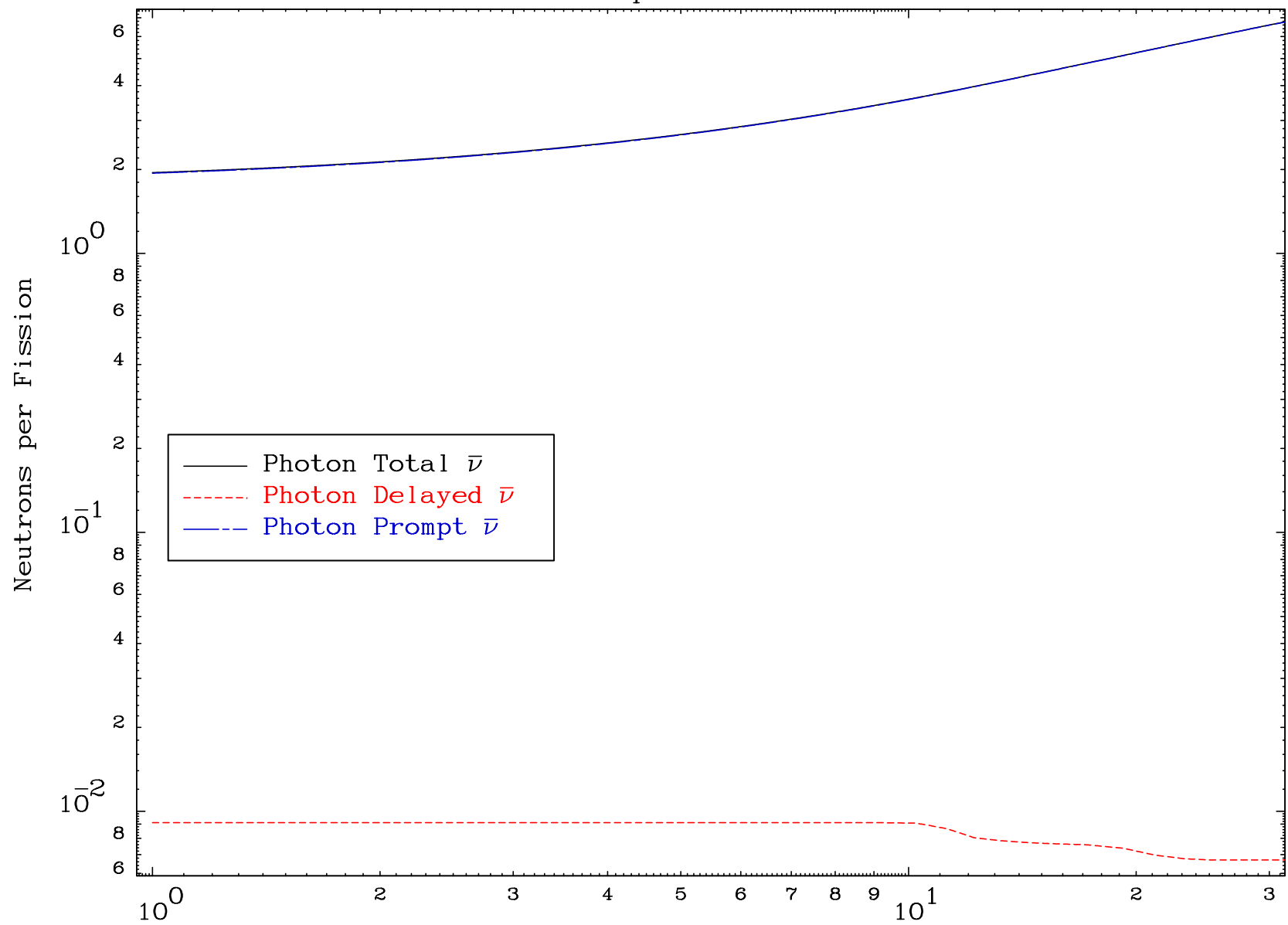




MAT 9443

Photon Neutrons  
per Fission

94-Pu-241



3

 $E \cdot \sin(\vartheta/2) \text{ (MeV)}$ 

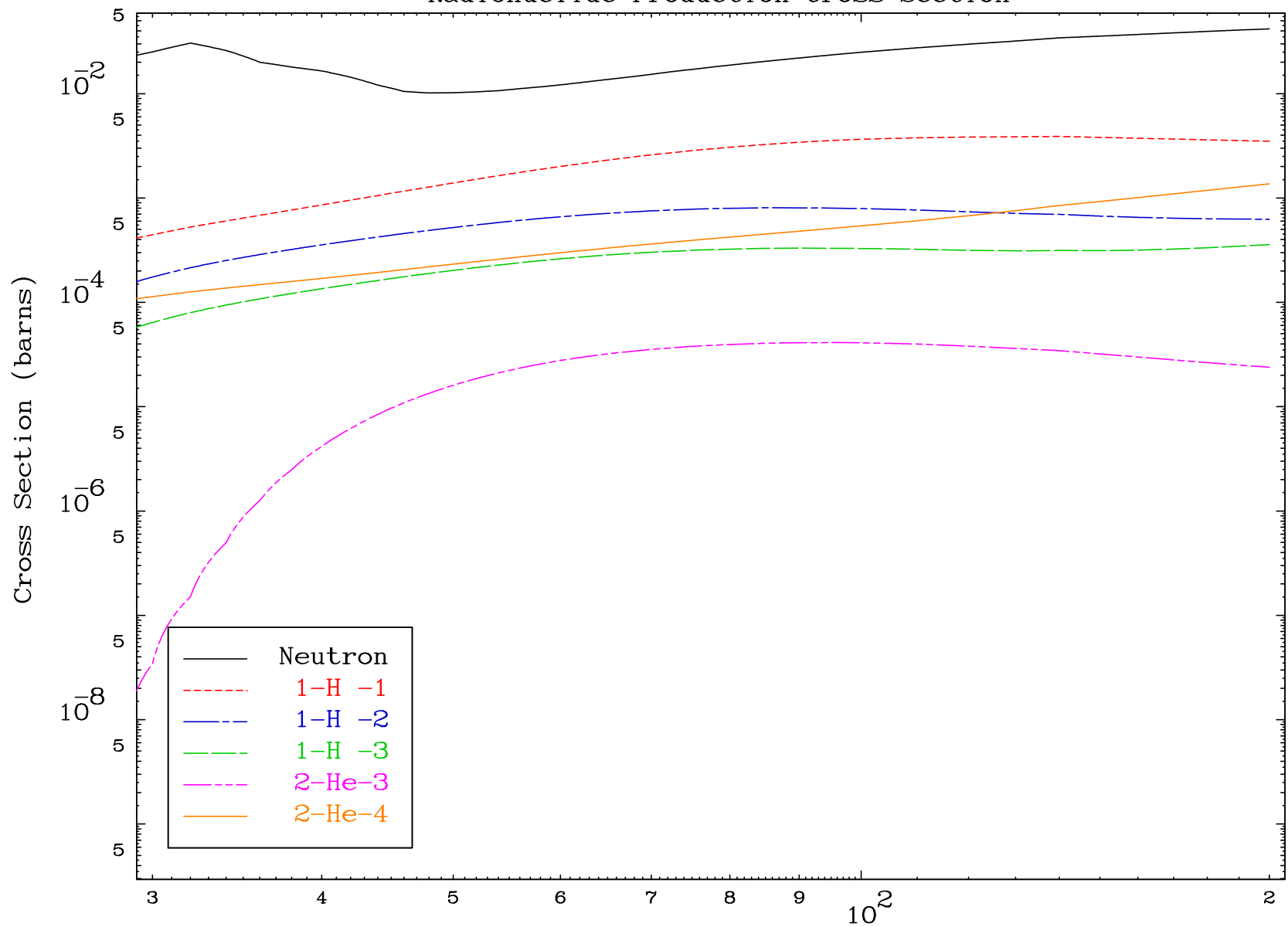
94-Pu-241

MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

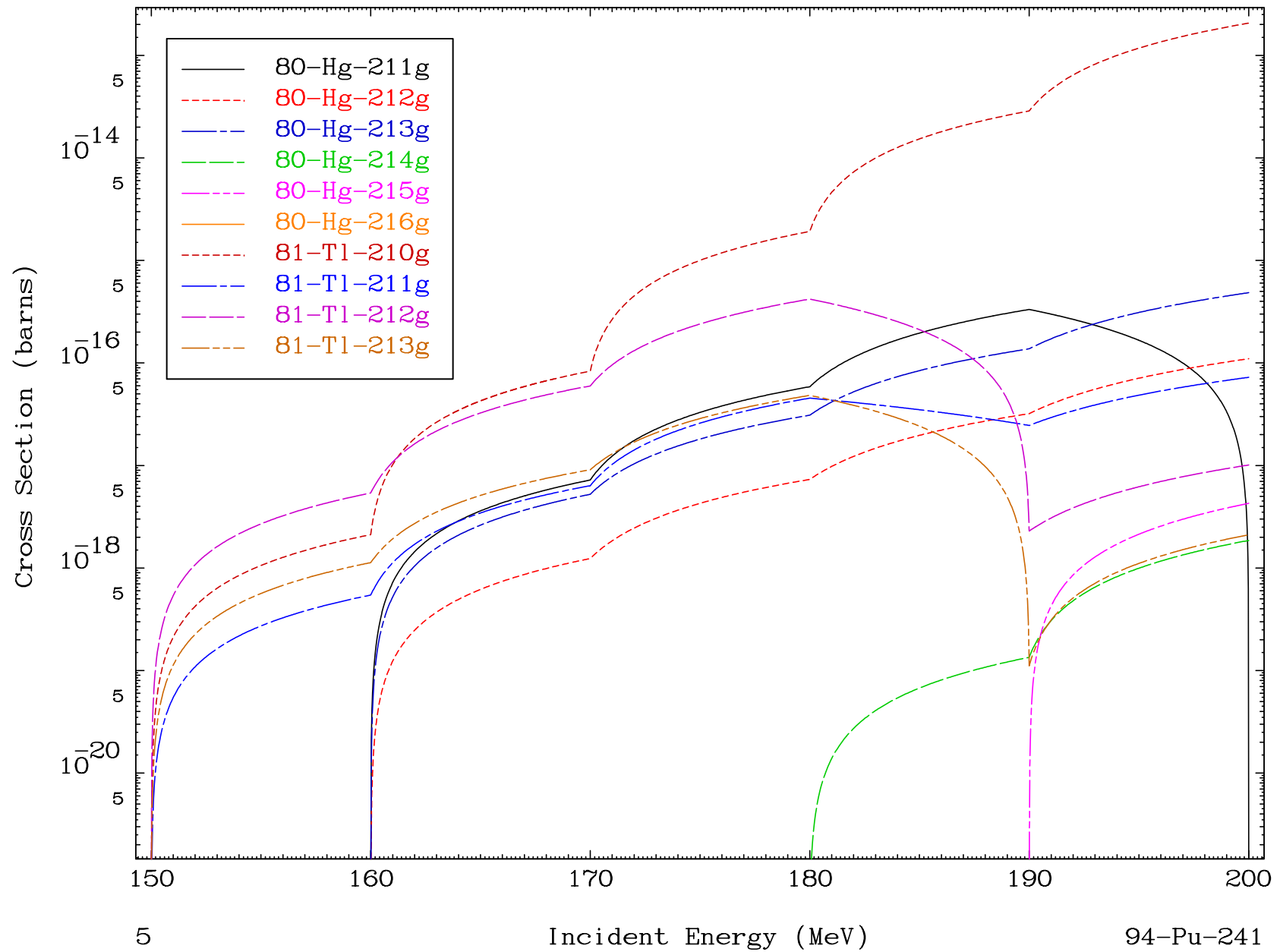
## Radionuclide Production Cross Section

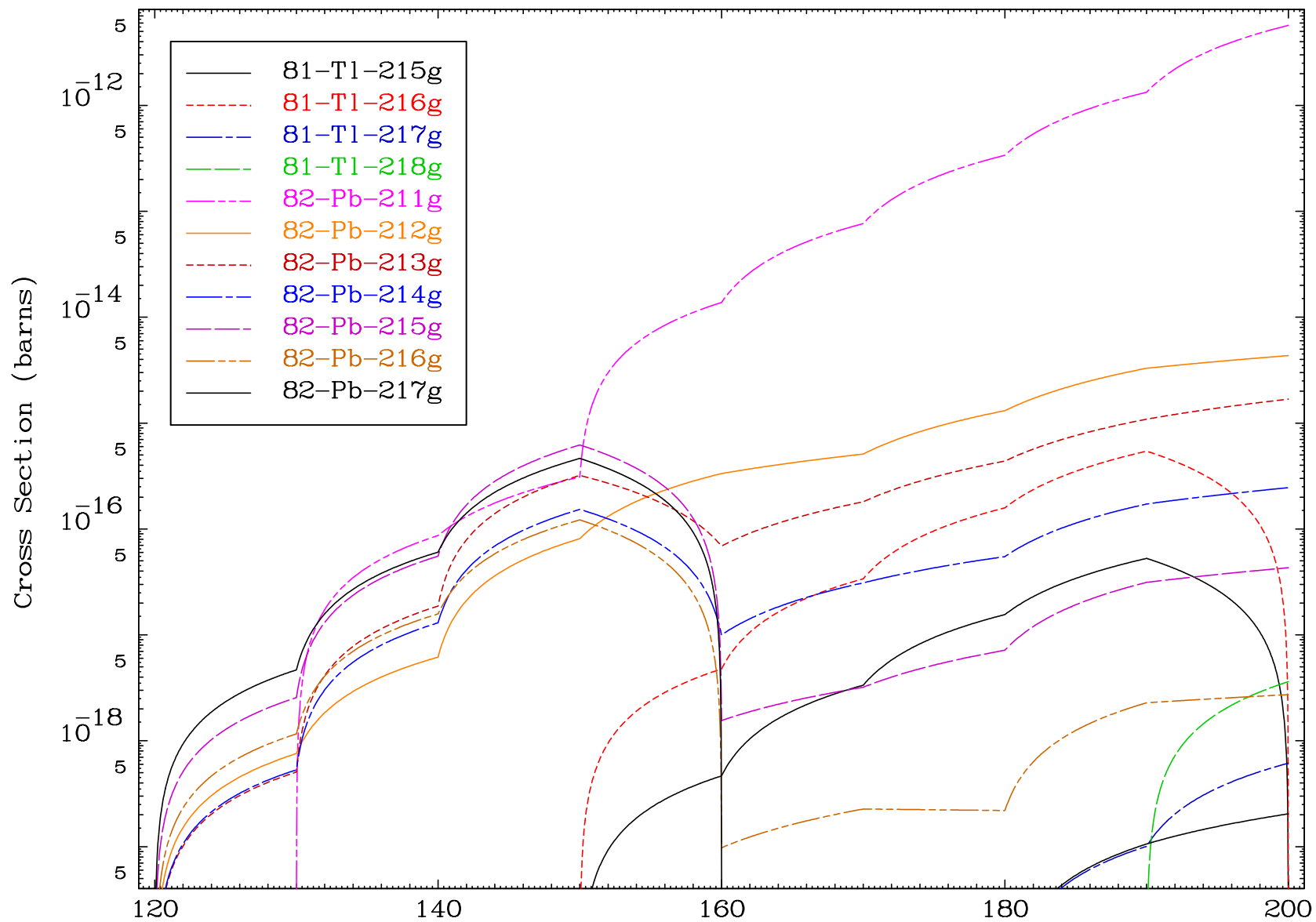


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Incident Energy (MeV)

94-Pu-241



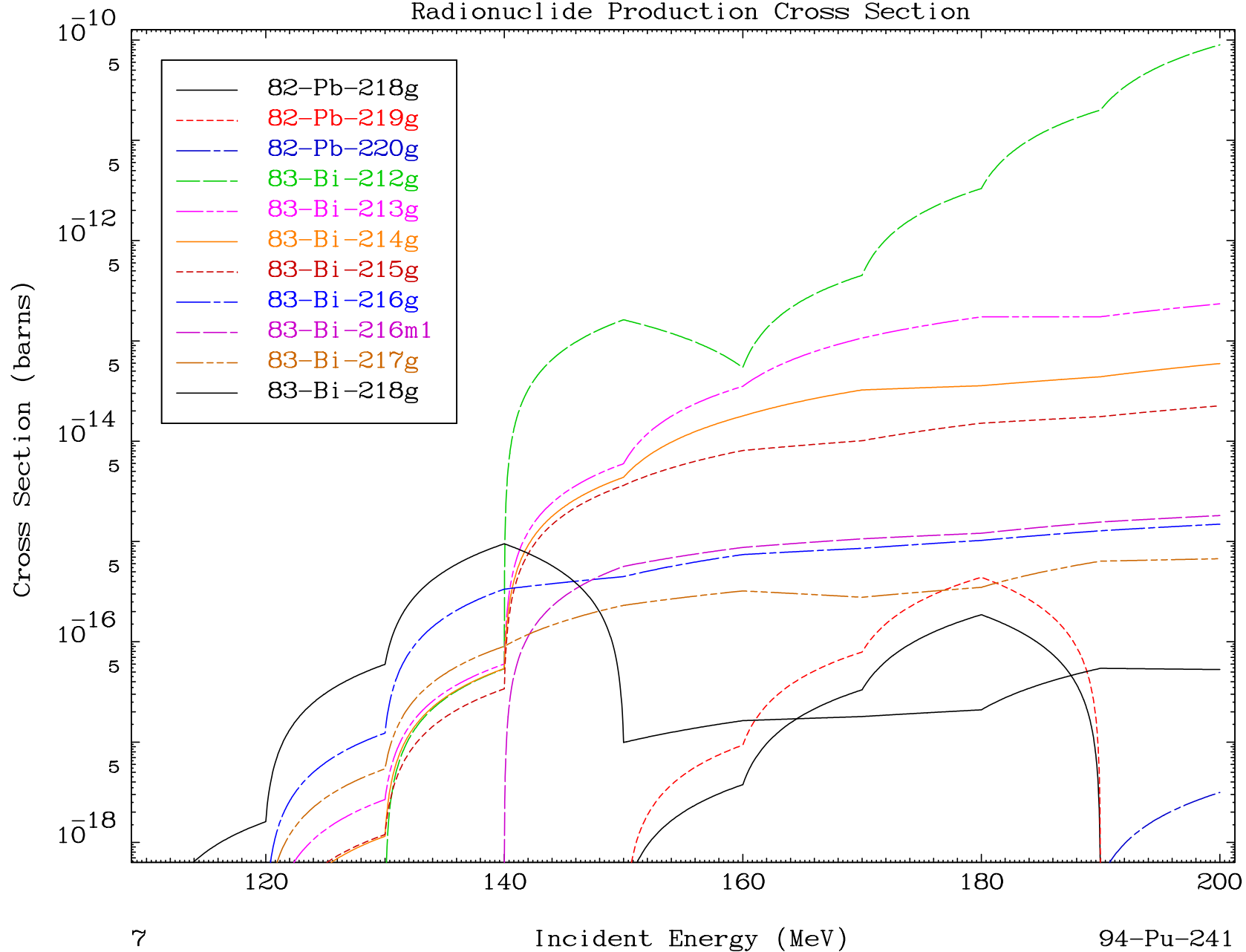


MAT 9443

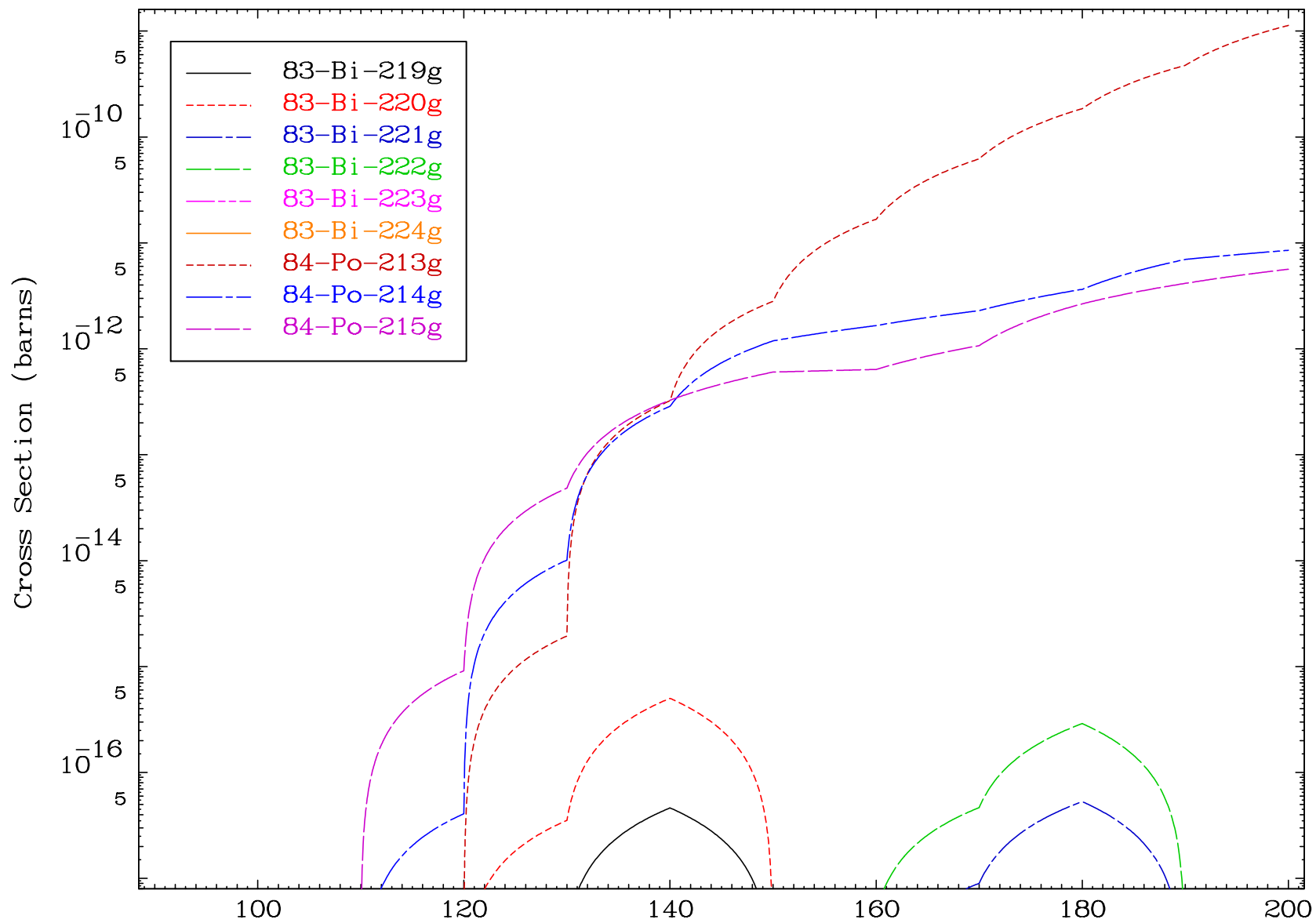
 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



## Radionuclide Production Cross Section



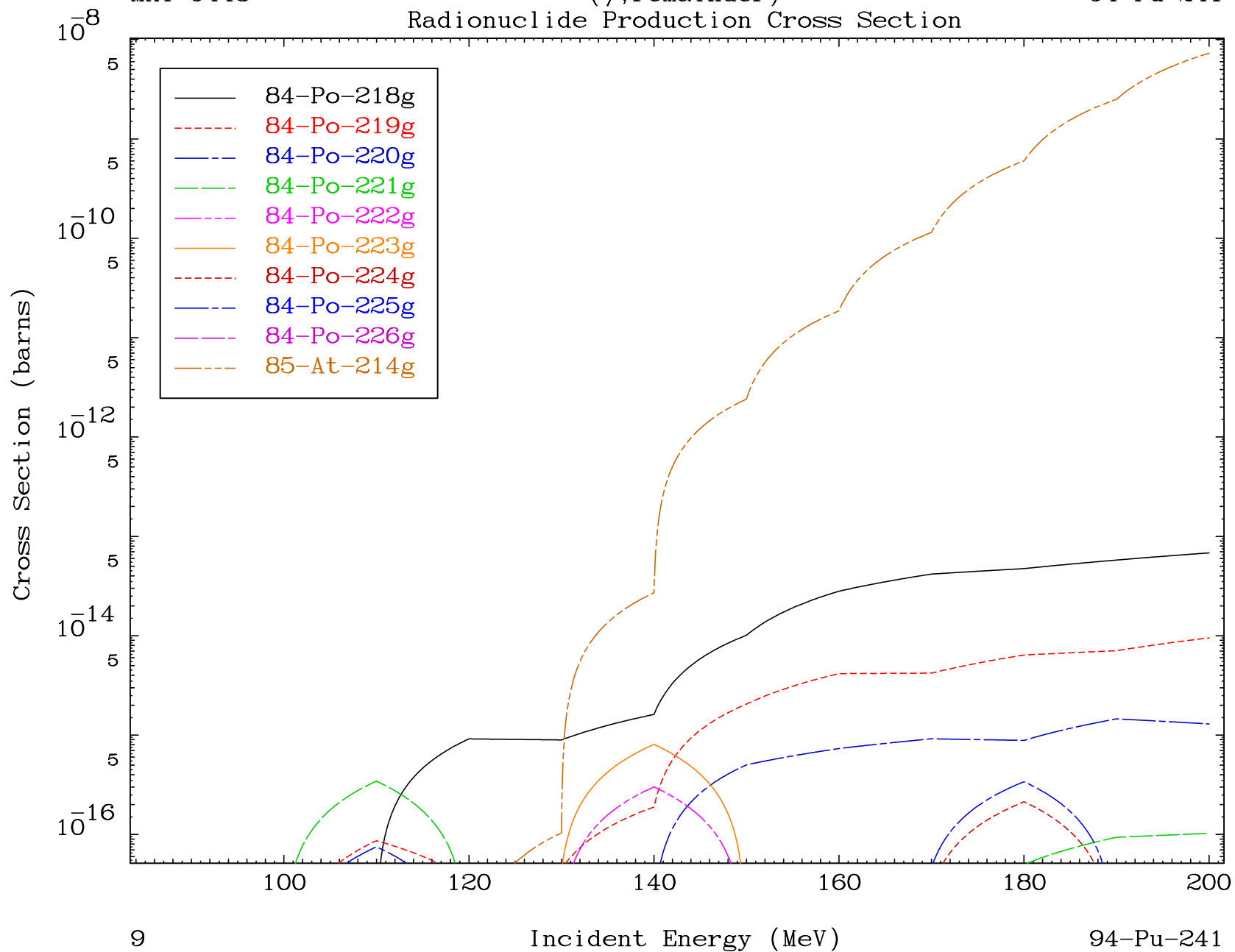


MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section

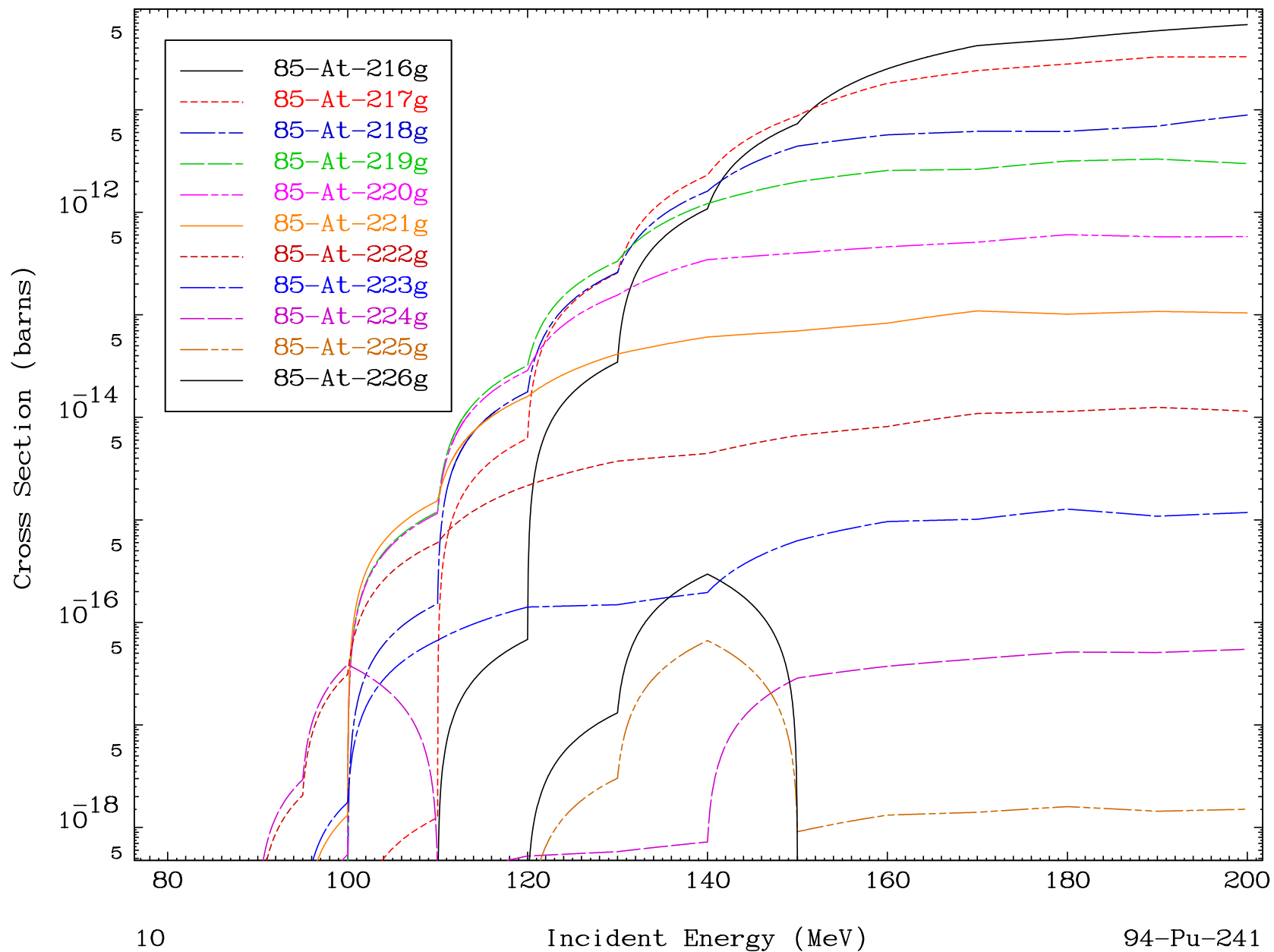


MAT 9443

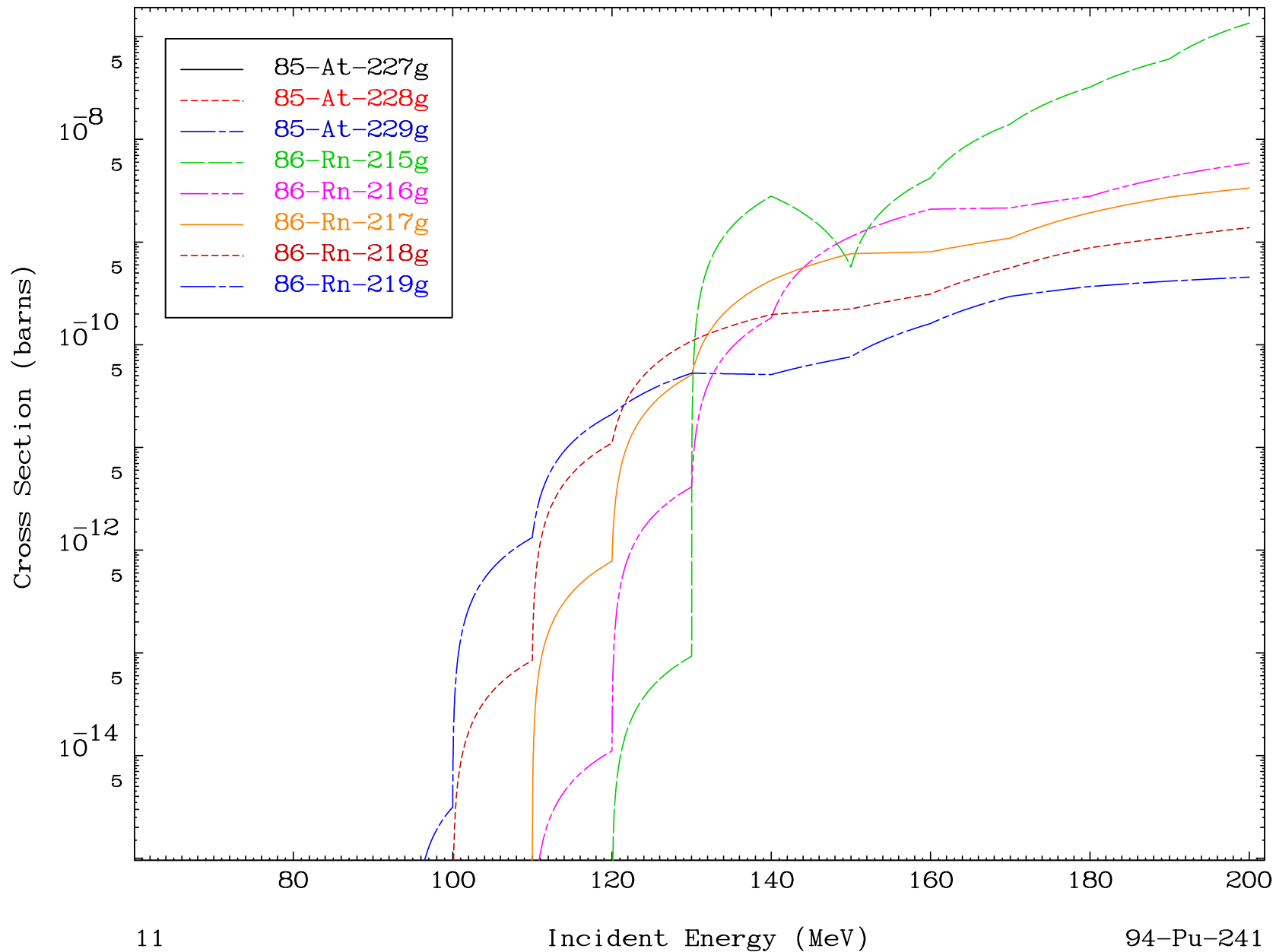
 $(\gamma, \text{remainder})$ 

94-Pu-241

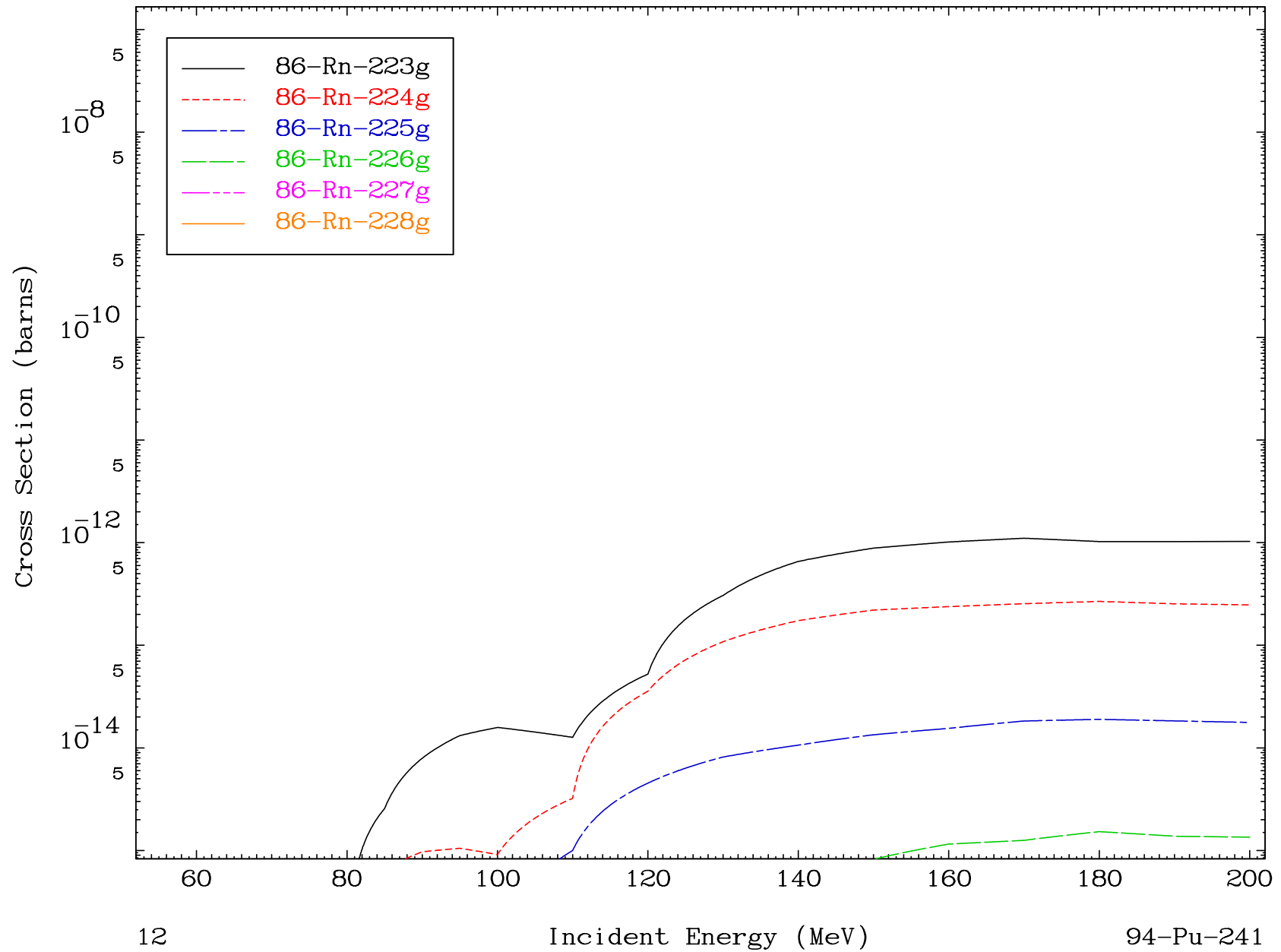
## Radionuclide Production Cross Section



## Radionuclide Production Cross Section



## Radionuclide Production Cross Section

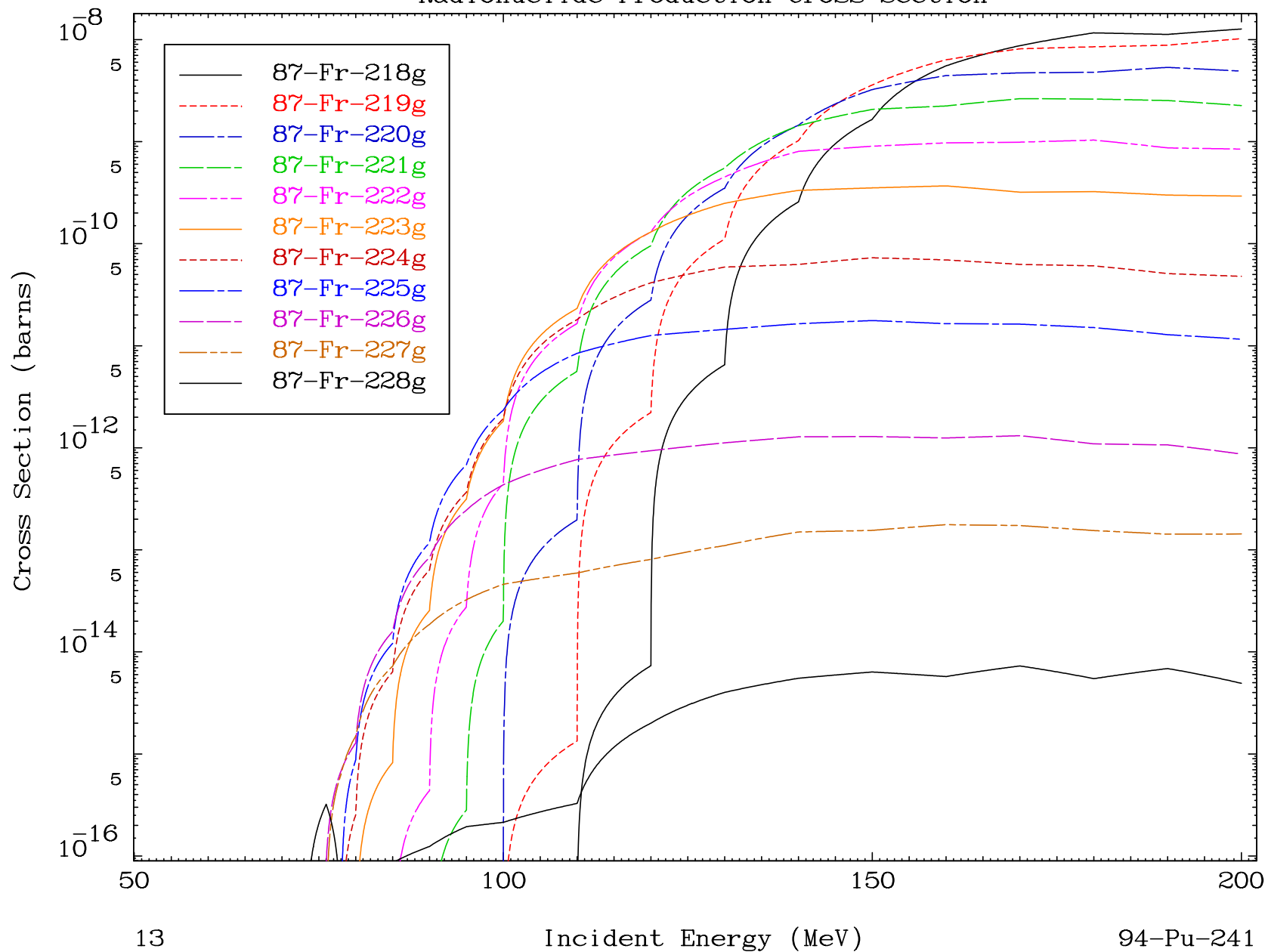


MAT 9443

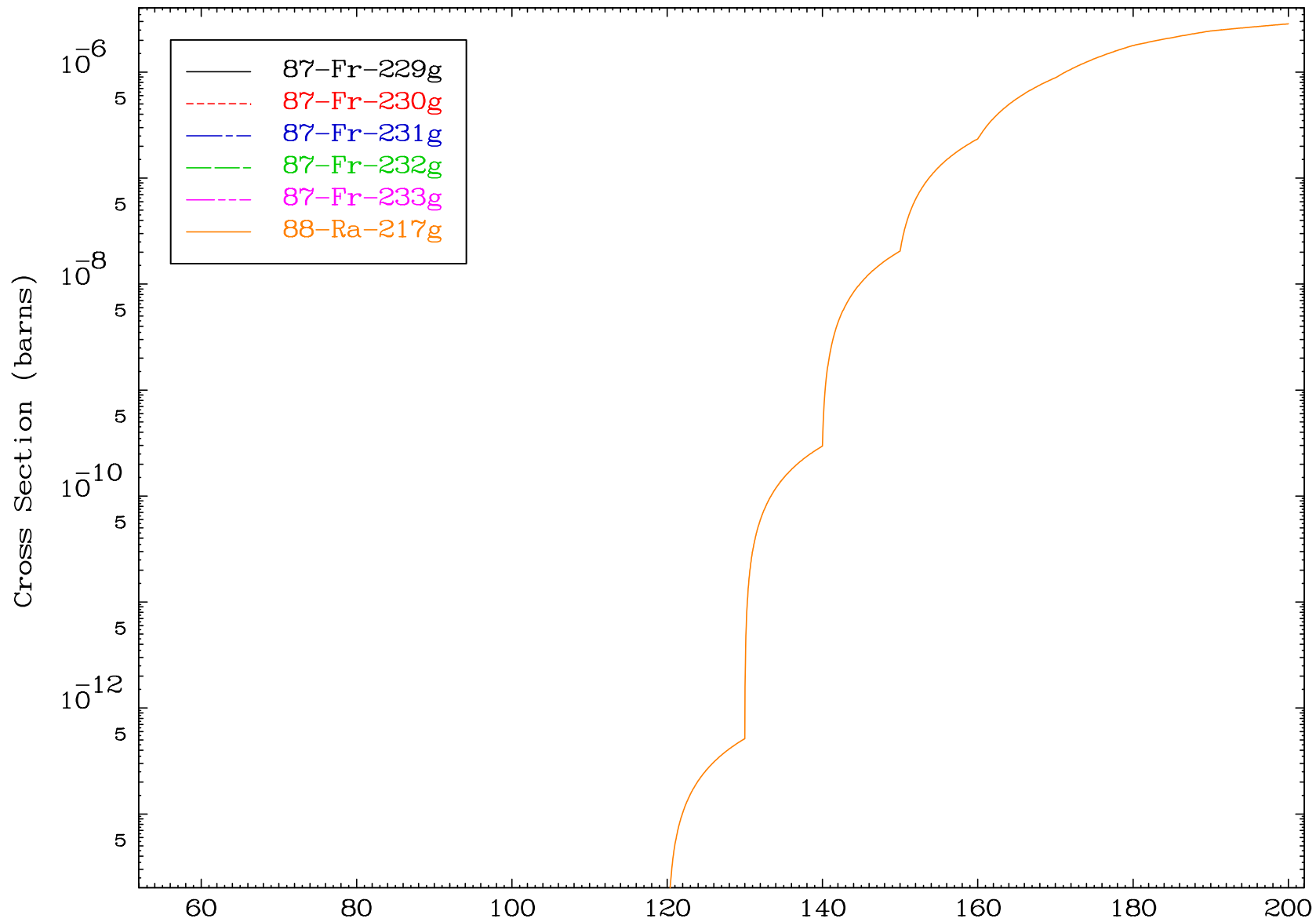
 $(\gamma, \text{remainder})$ 

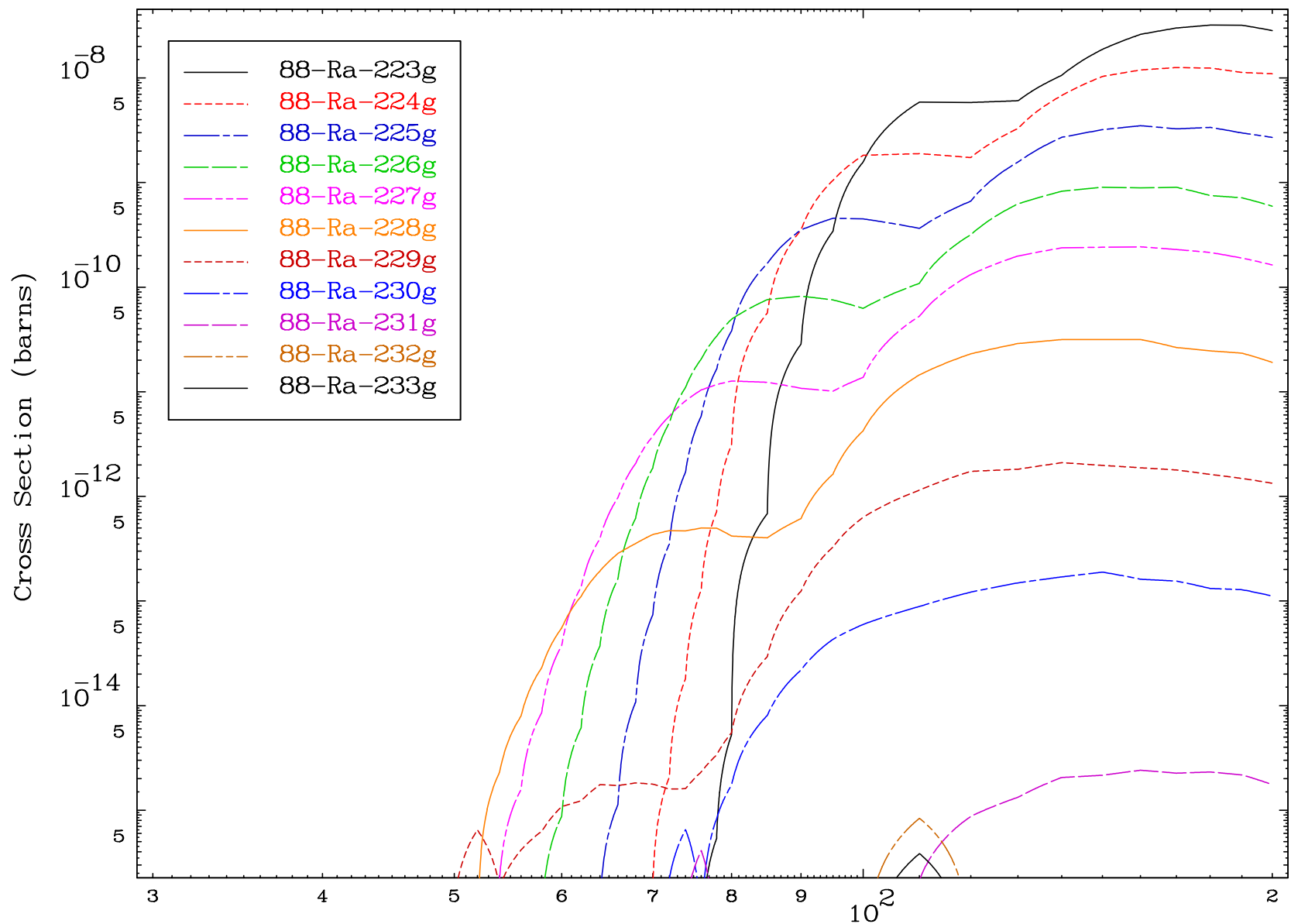
94-Pu-241

## Radionuclide Production Cross Section



## Radionuclide Production Cross Section



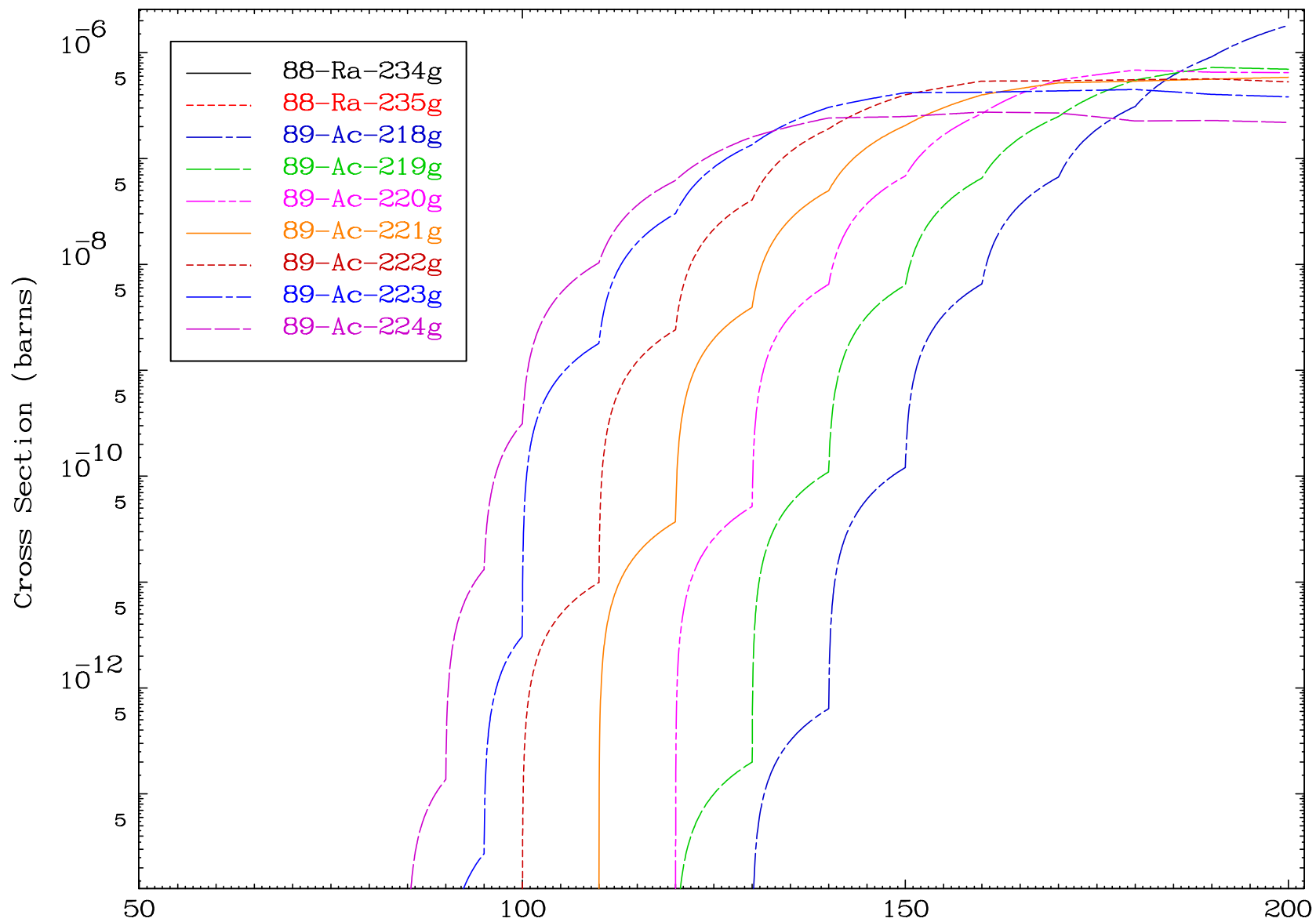


MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section

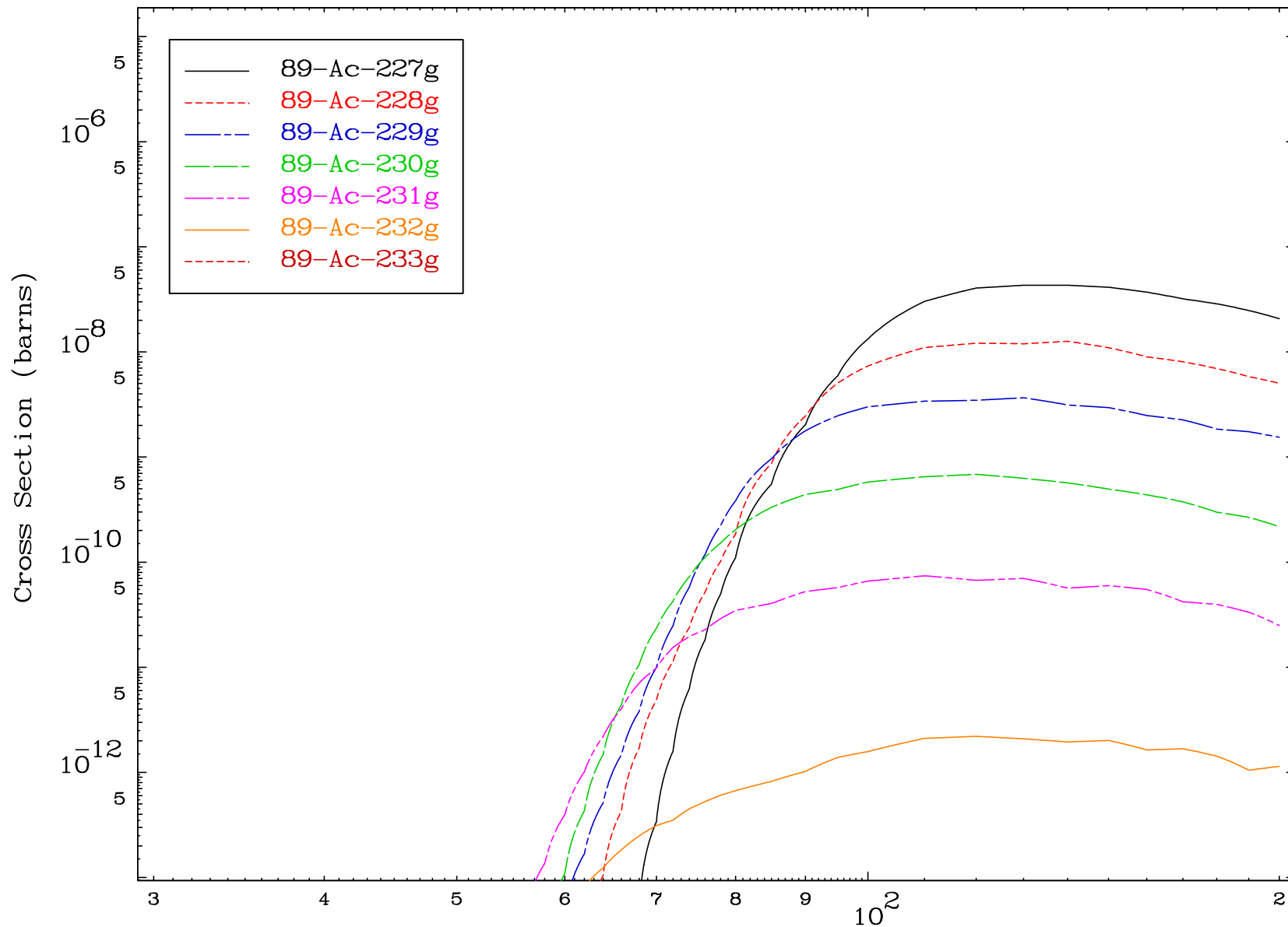


16

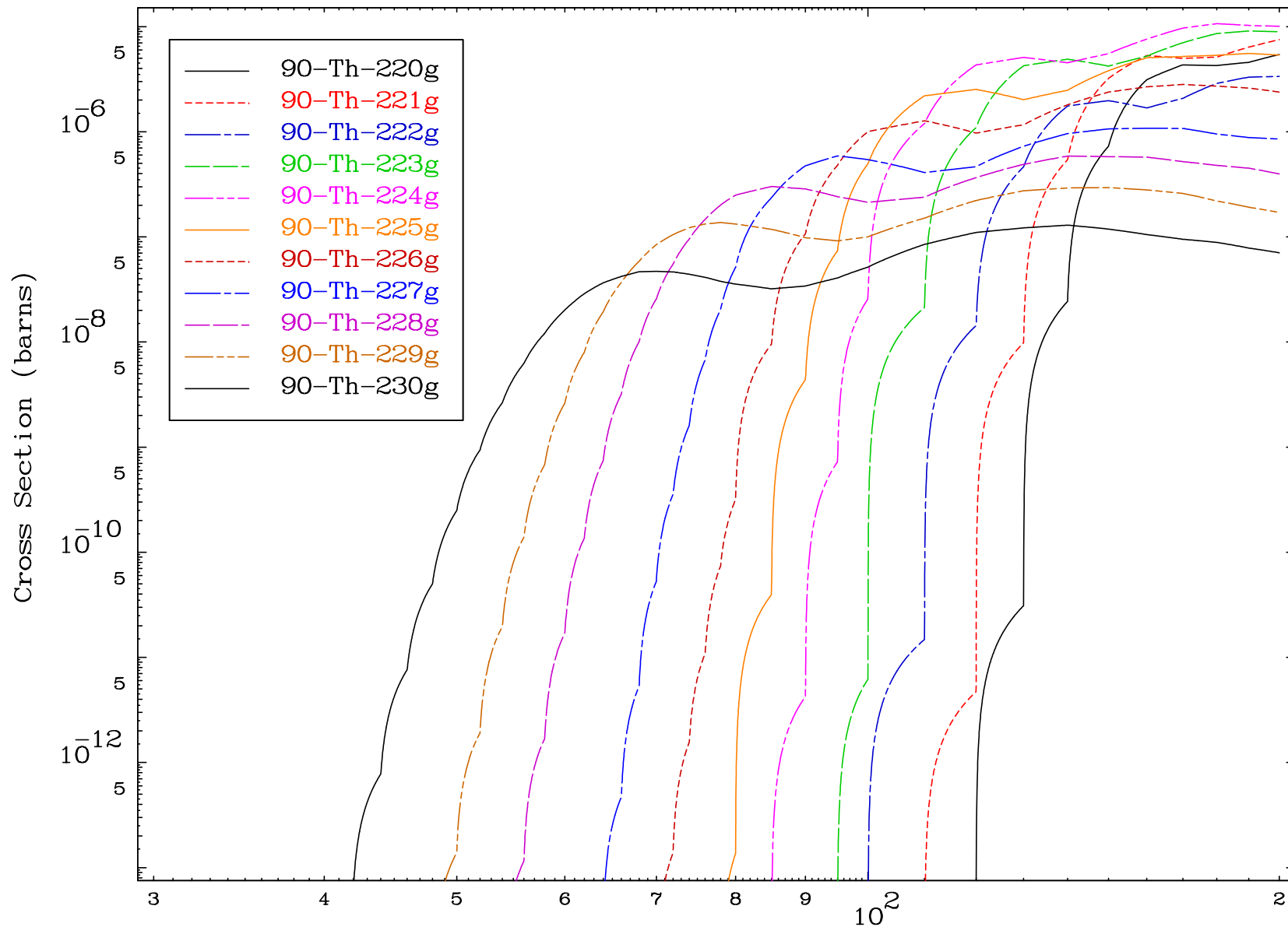
Incident Energy (MeV)

94-Pu-241





## Radionuclide Production Cross Section

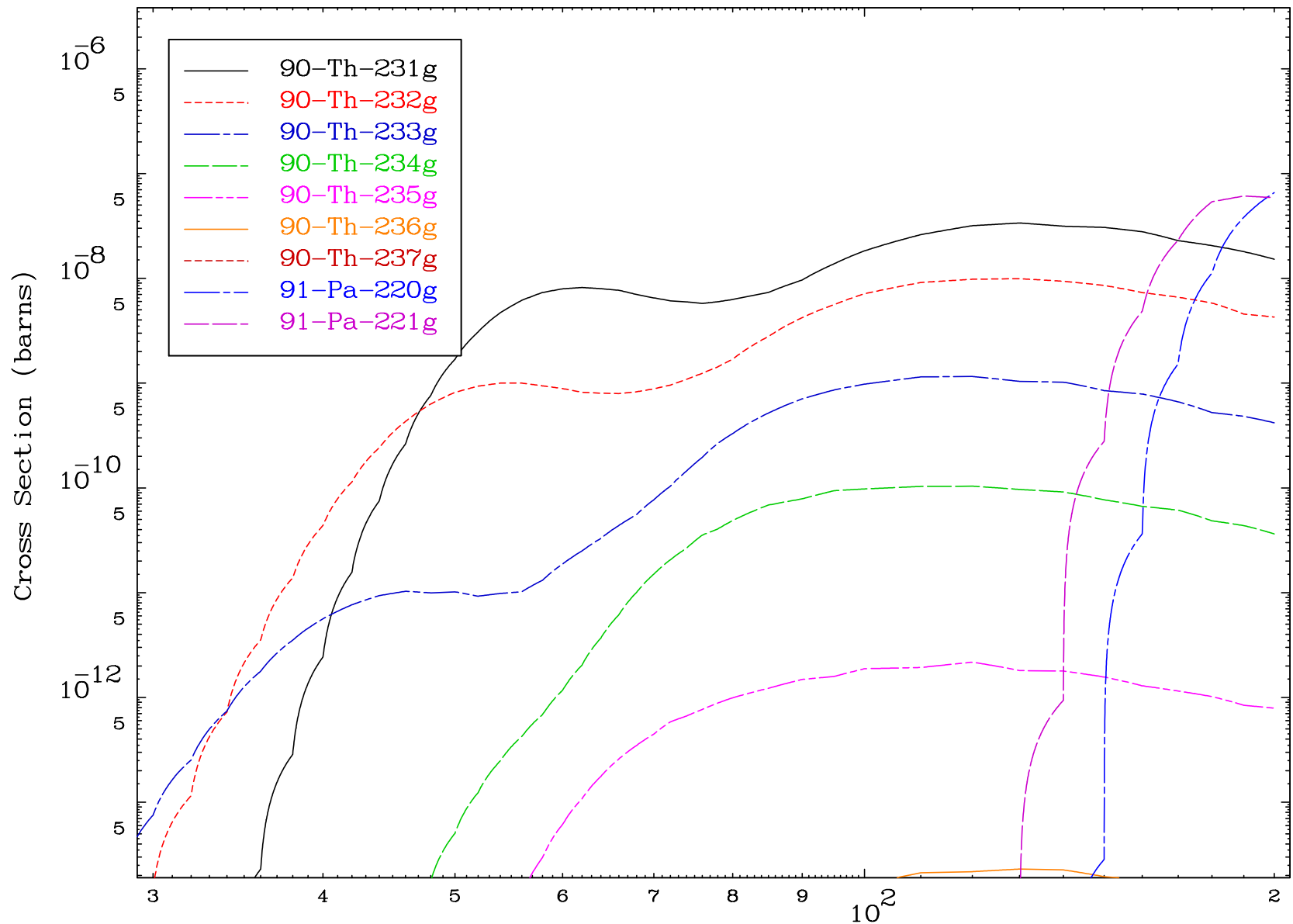


MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



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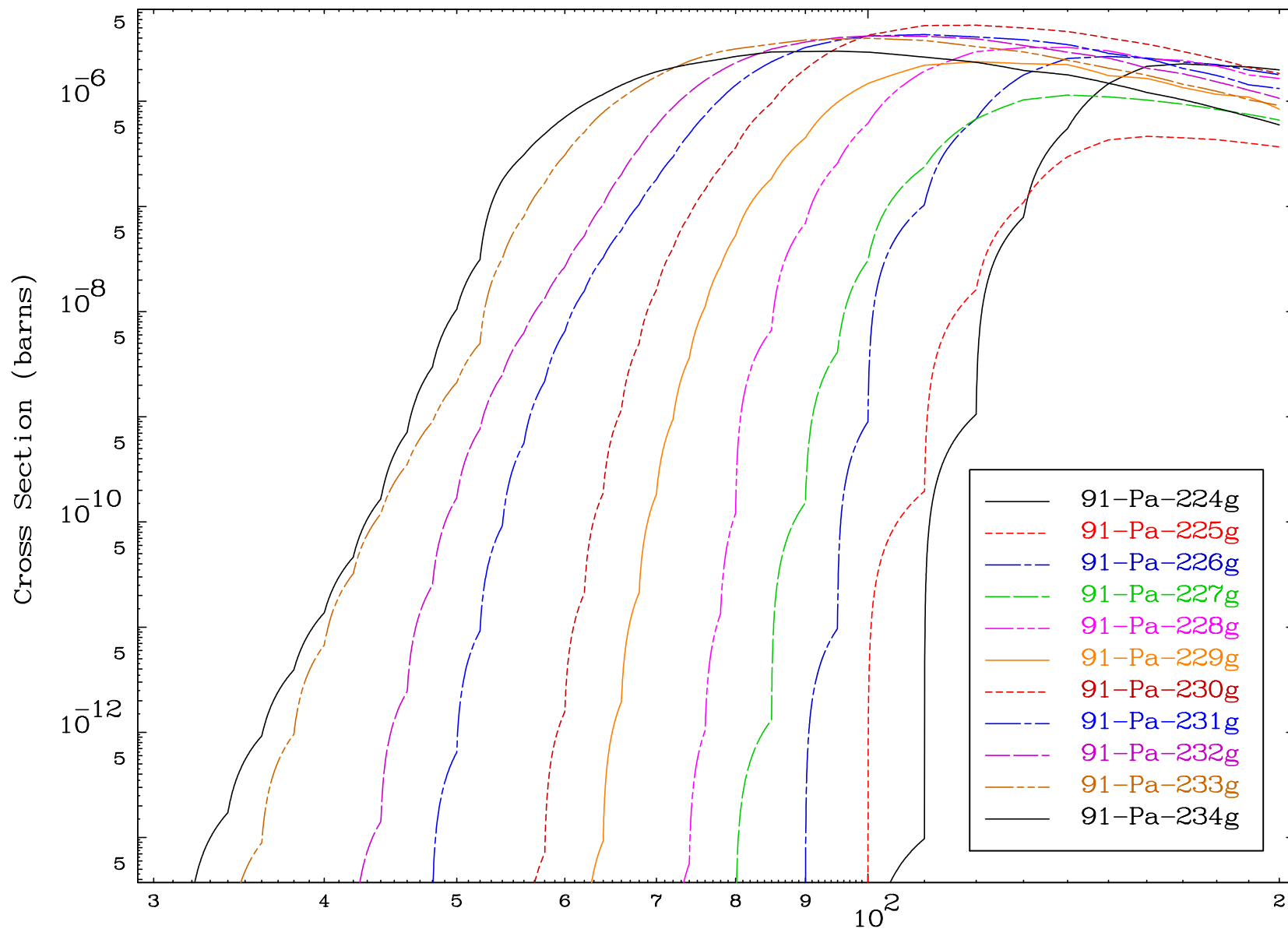
Incident Energy (MeV)

94-Pu-241

MAT 9443

( $\gamma$ , remainder)  
Radionuclide Production Cross Section

94-Pu-241

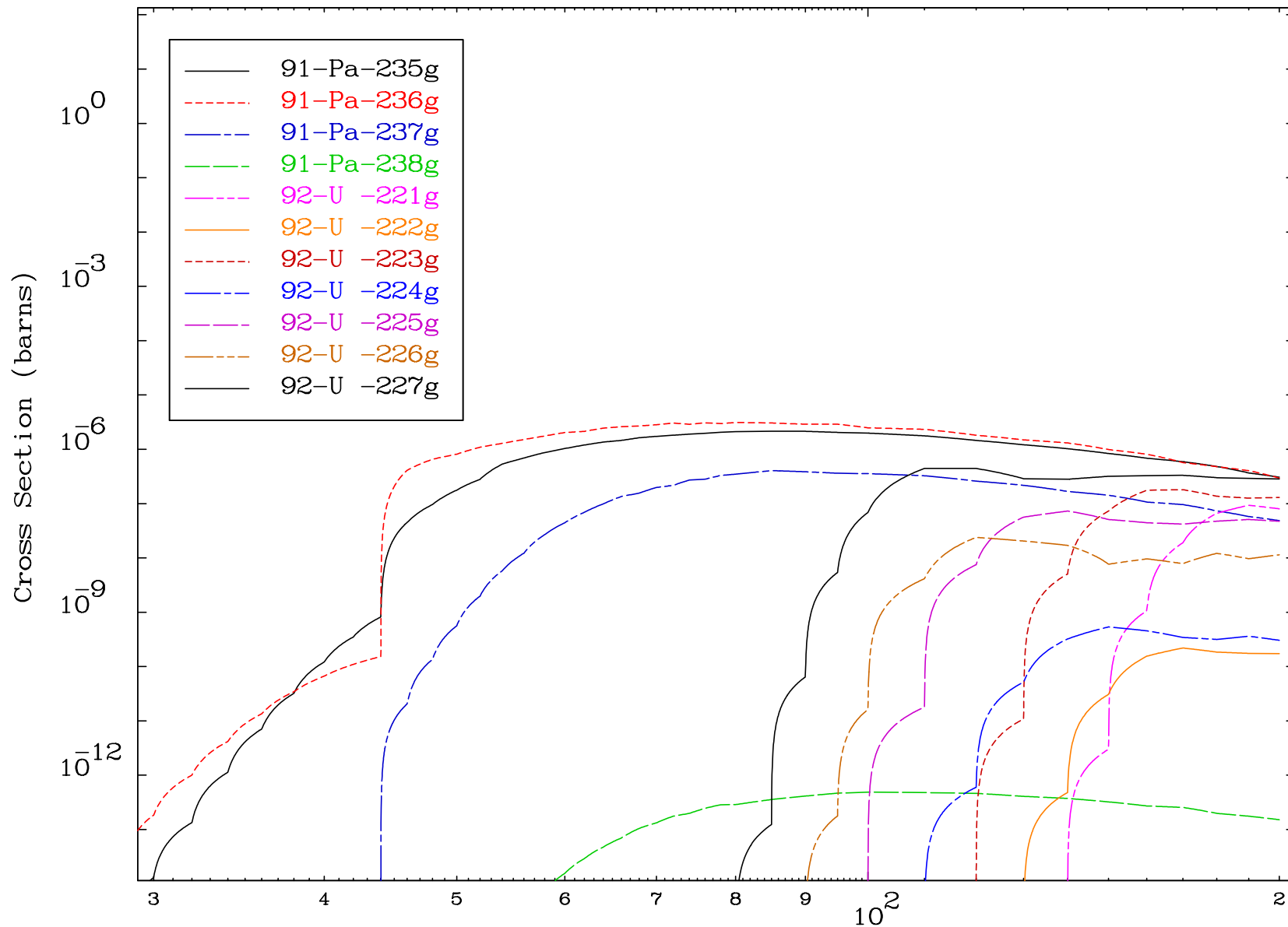


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Incident Energy (MeV)

94-Pu-241

## Radionuclide Production Cross Section

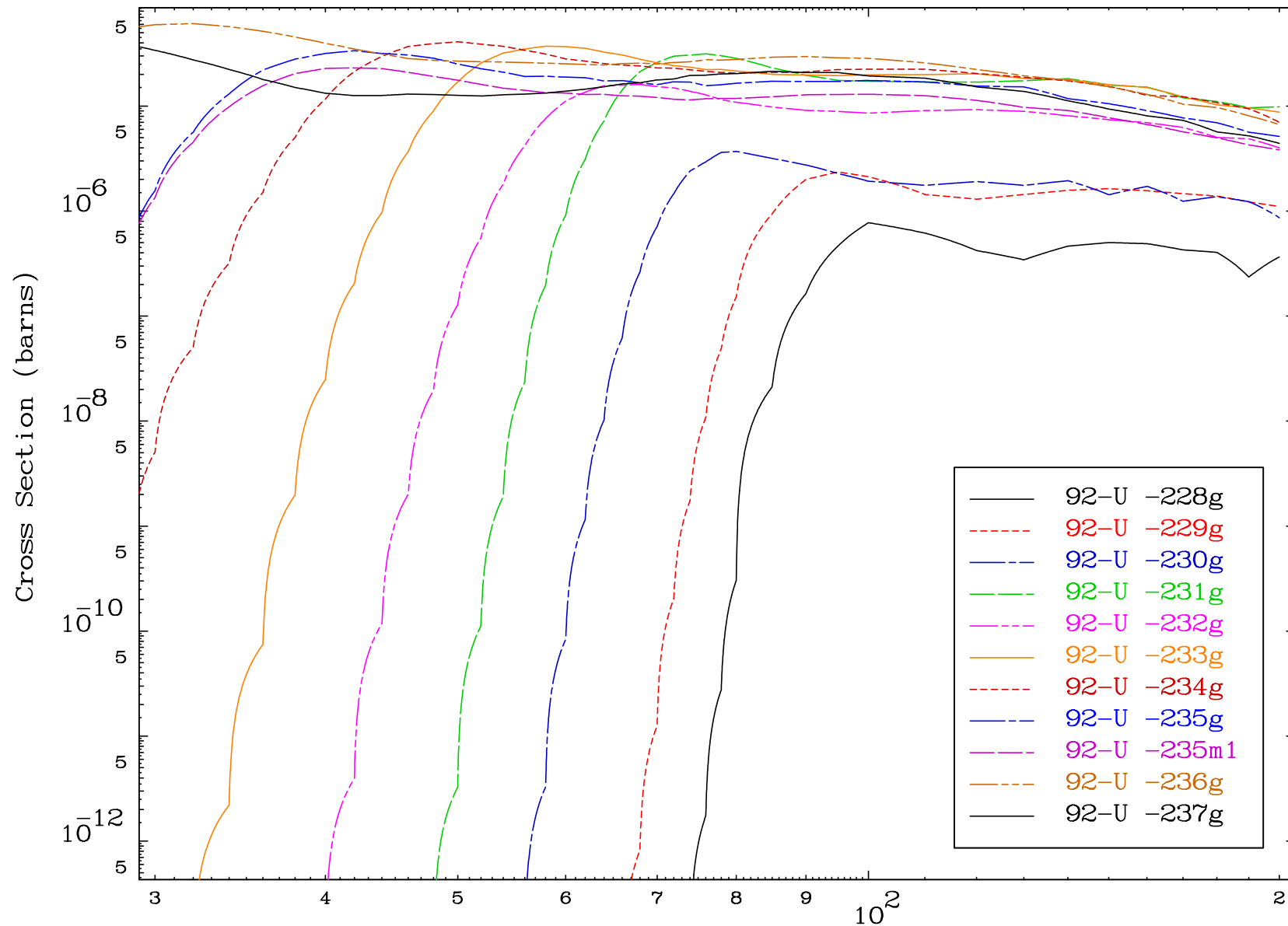


MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



22

Incident Energy (MeV)

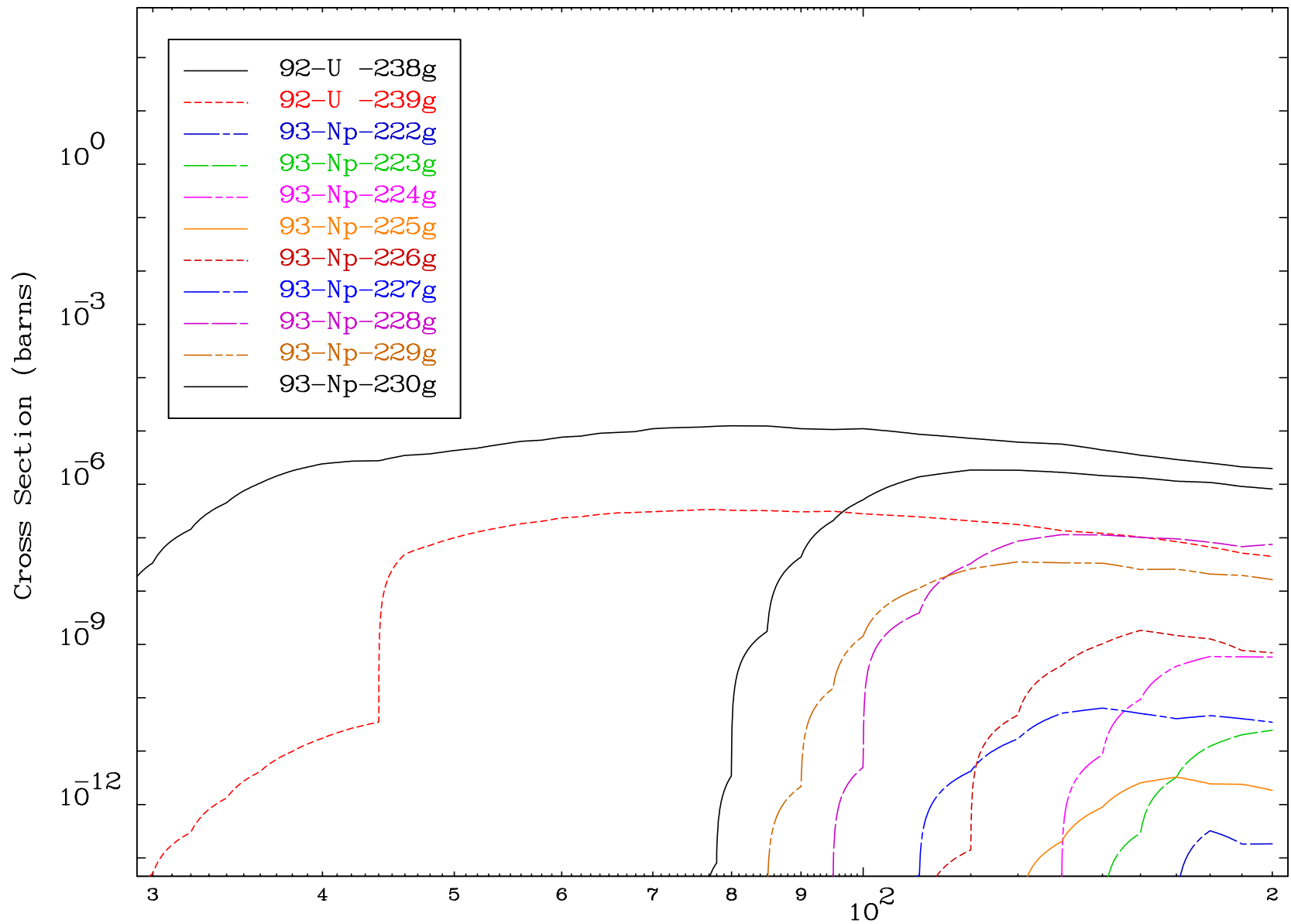
94-Pu-241

MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



23

Incident Energy (MeV)

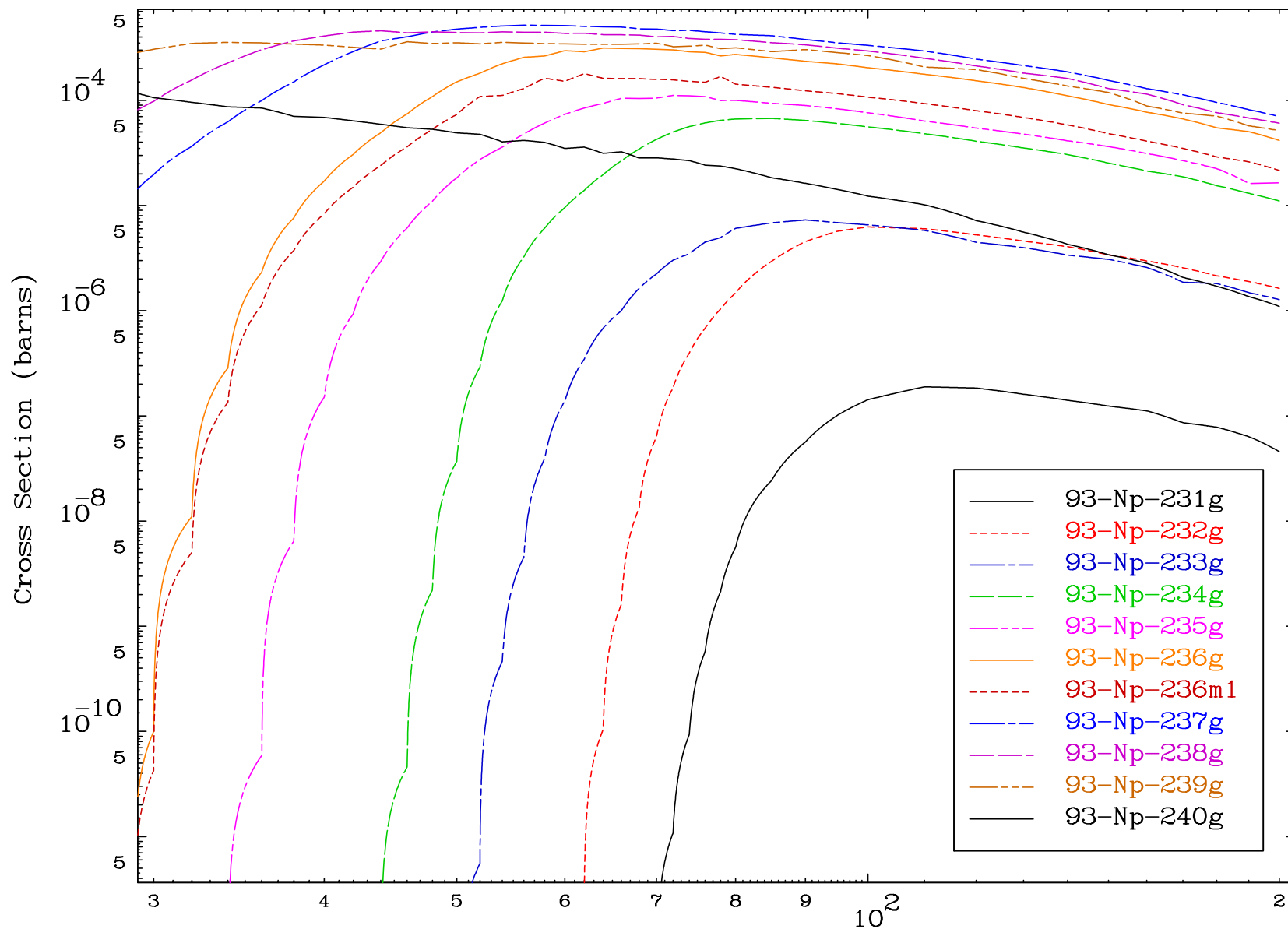
94-Pu-241

MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



24

Incident Energy (MeV)

94-Pu-241

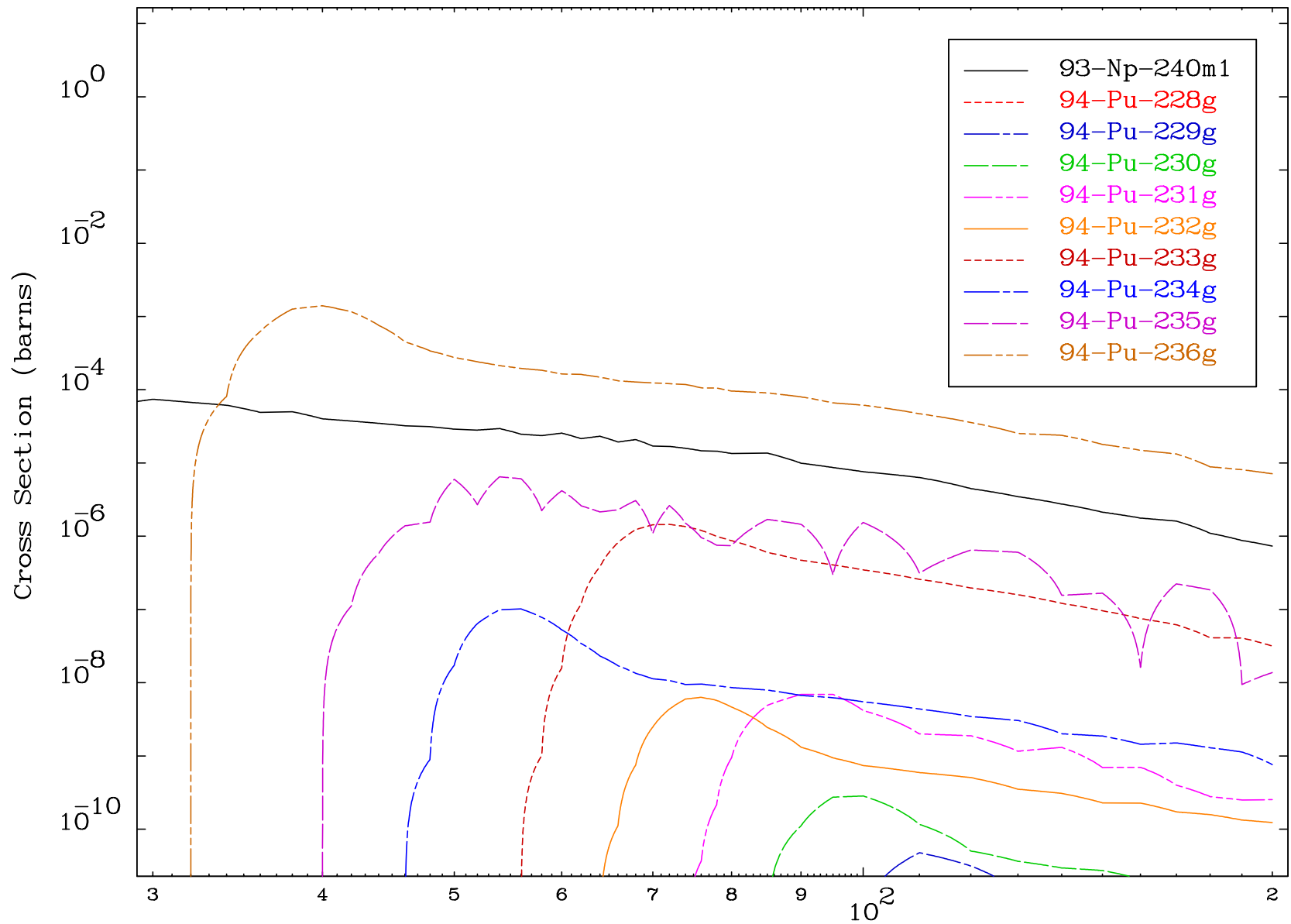


MAT 9443

 $(\gamma, \text{remainder})$ 

94-Pu-241

## Radionuclide Production Cross Section



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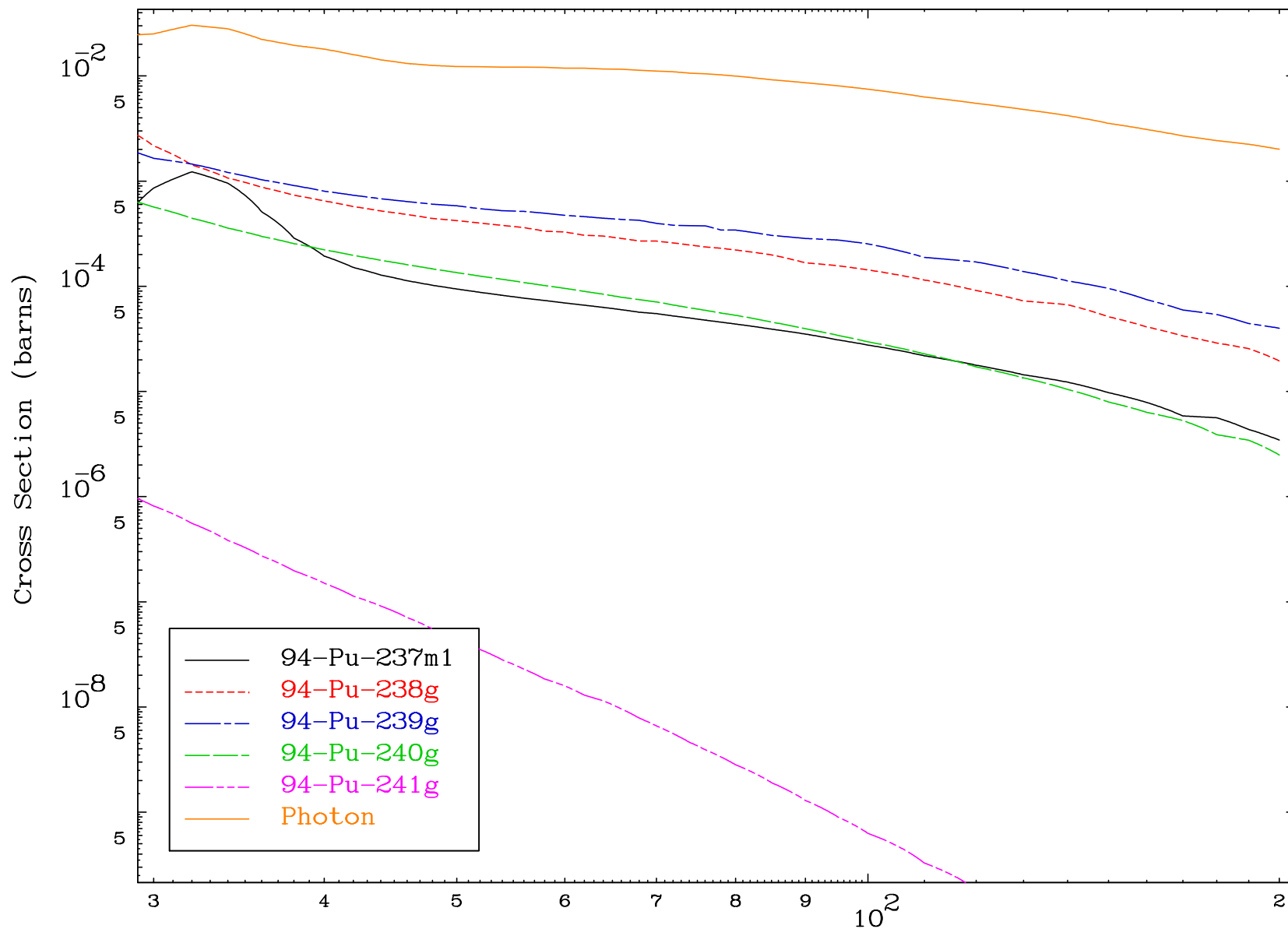
Incident Energy (MeV)

94-Pu-241

MAT 9443

 $(\gamma, \text{remainder})$   
Radionuclide Production Cross Section

94-Pu-241



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Incident Energy (MeV)

94-Pu-241