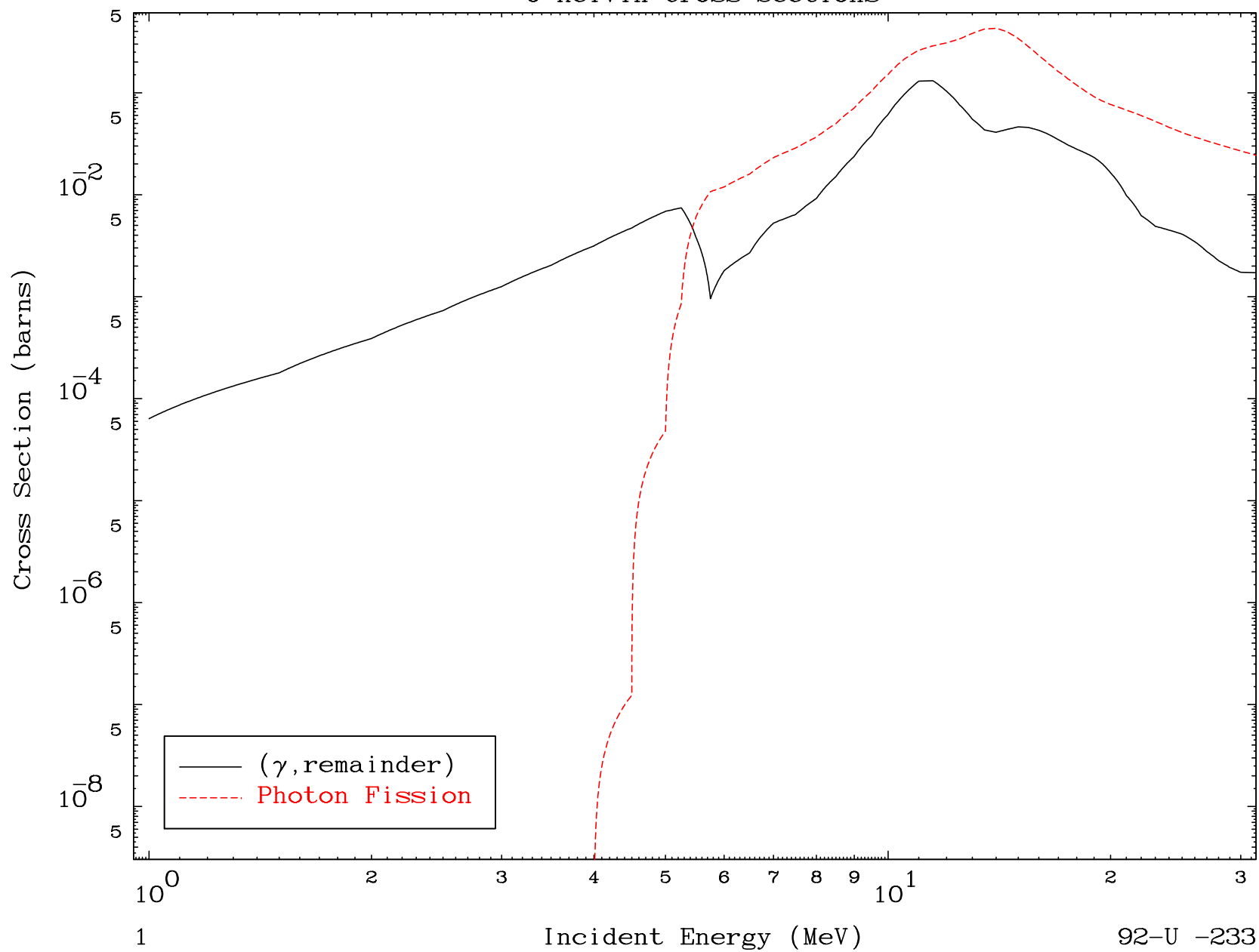


MAT 9222

Photon Major
0 Kelvin Cross Sections

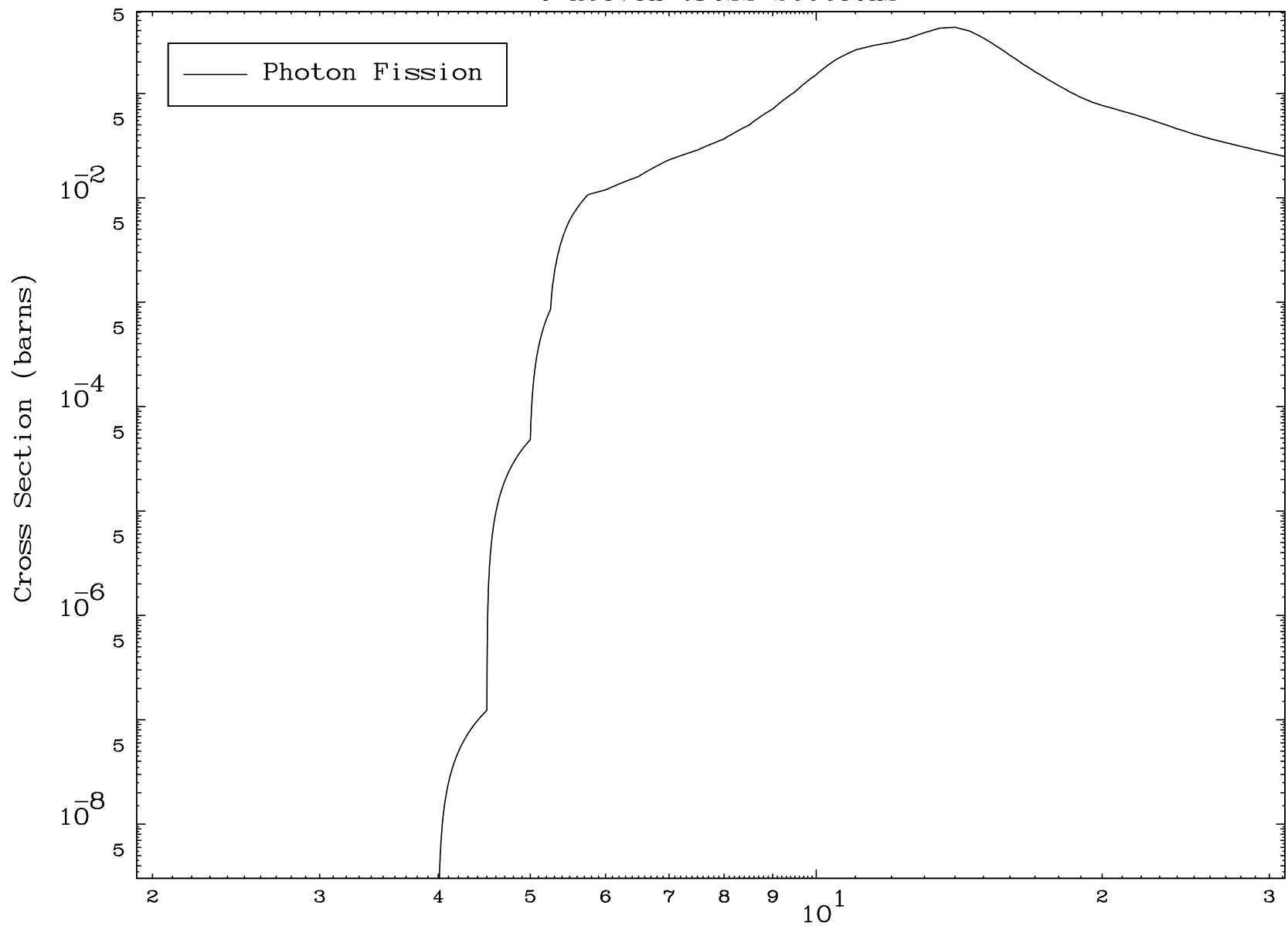
92-U -233



MAT 9222

Photon Fission
0 Kelvin Cross Sections

92-U -233



2

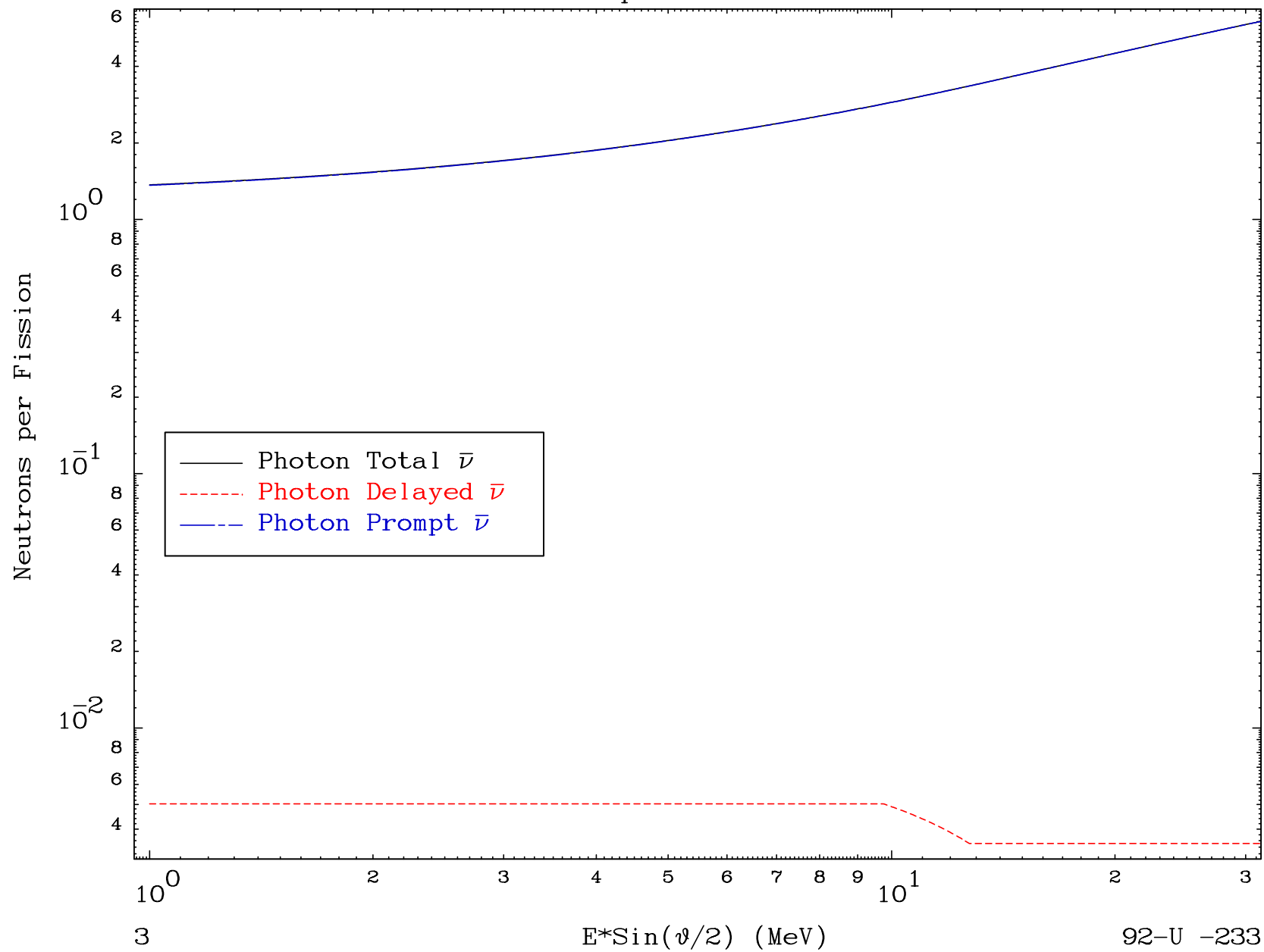
Incident Energy (MeV)

92-U -233

MAT 9222

Photon Neutrons
per Fission

92-U -233

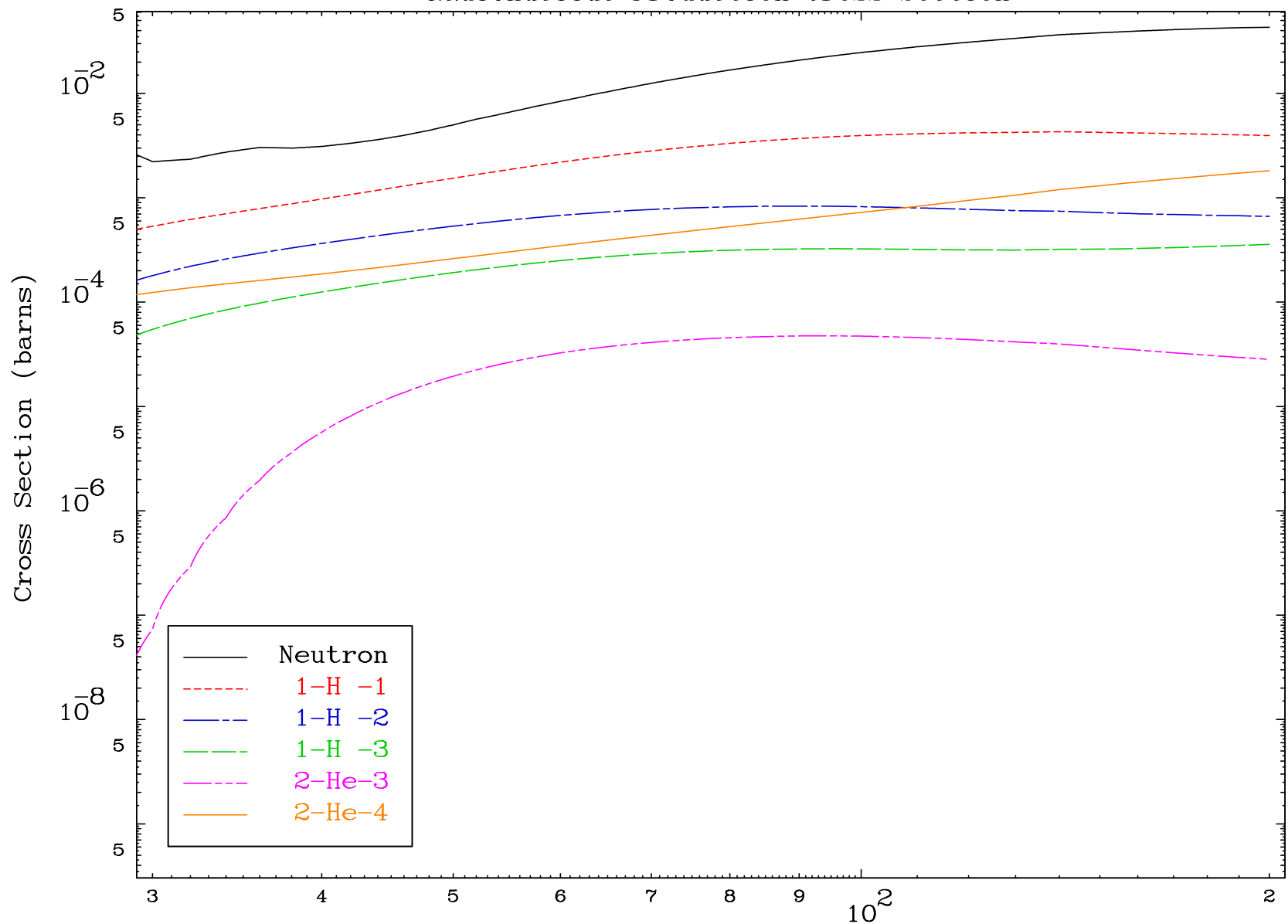


MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



4

Incident Energy (MeV)

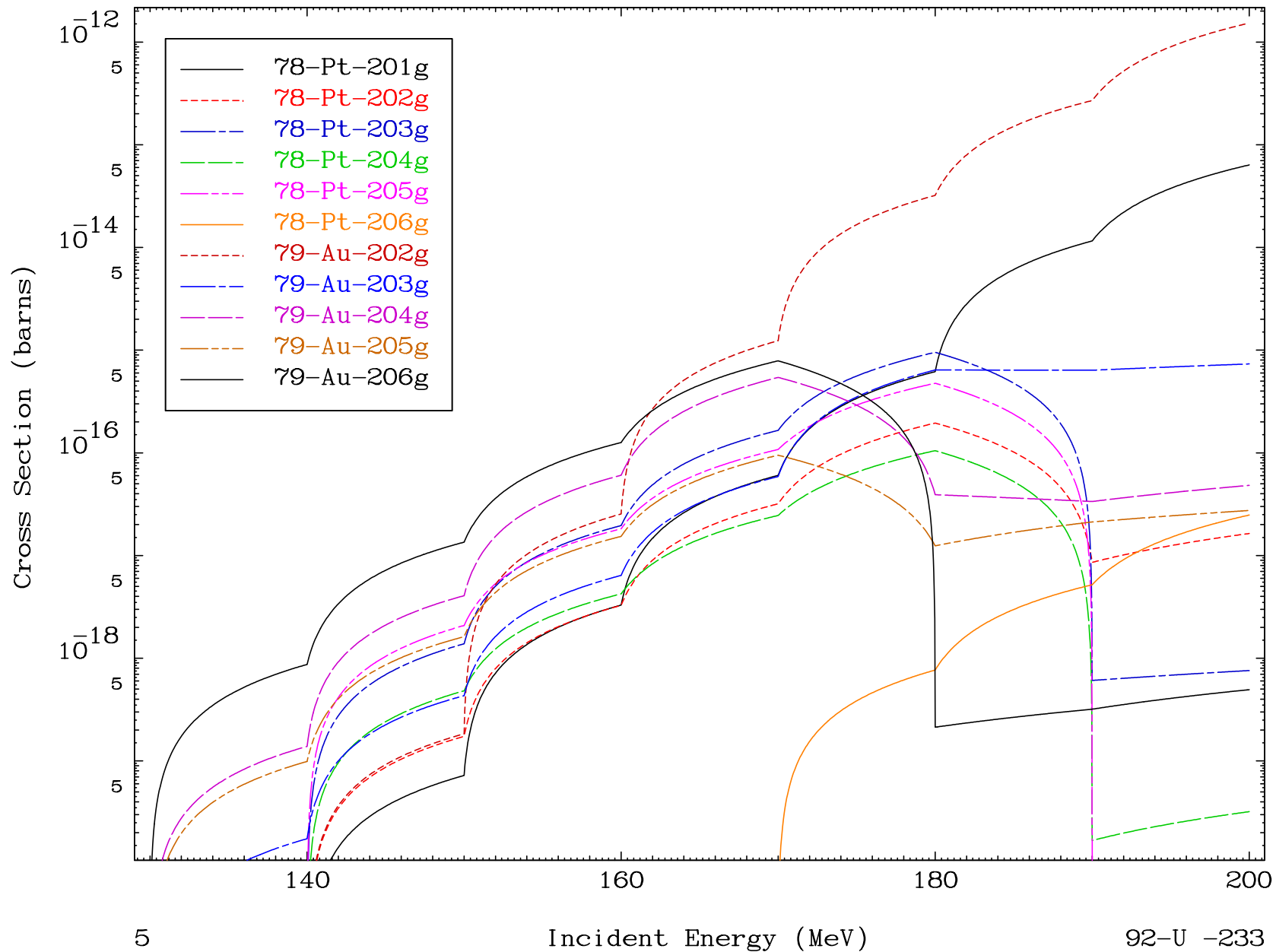
92-U -233

MAT 9222

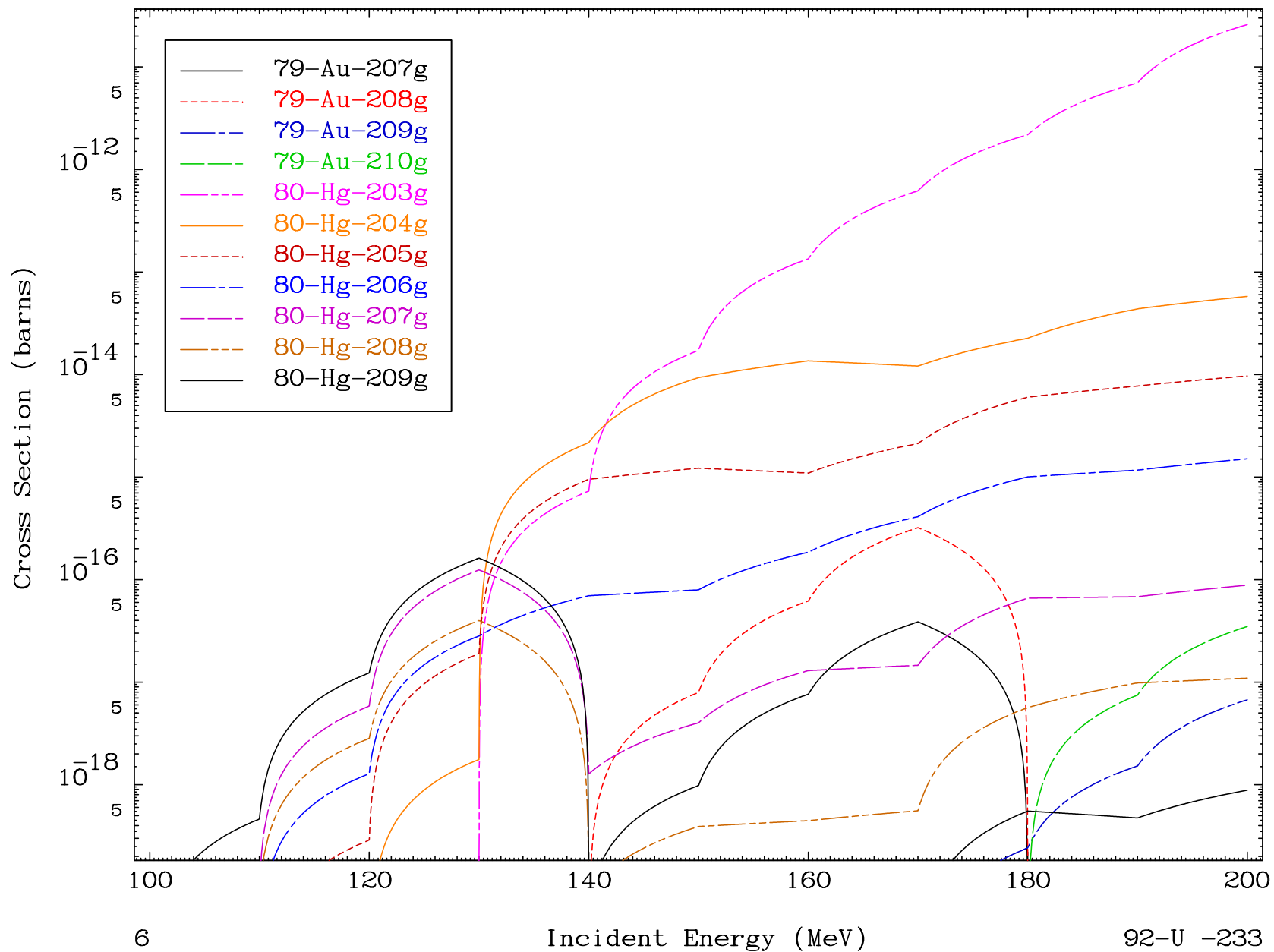
 $(\gamma, \text{remainder})$

92-U -233

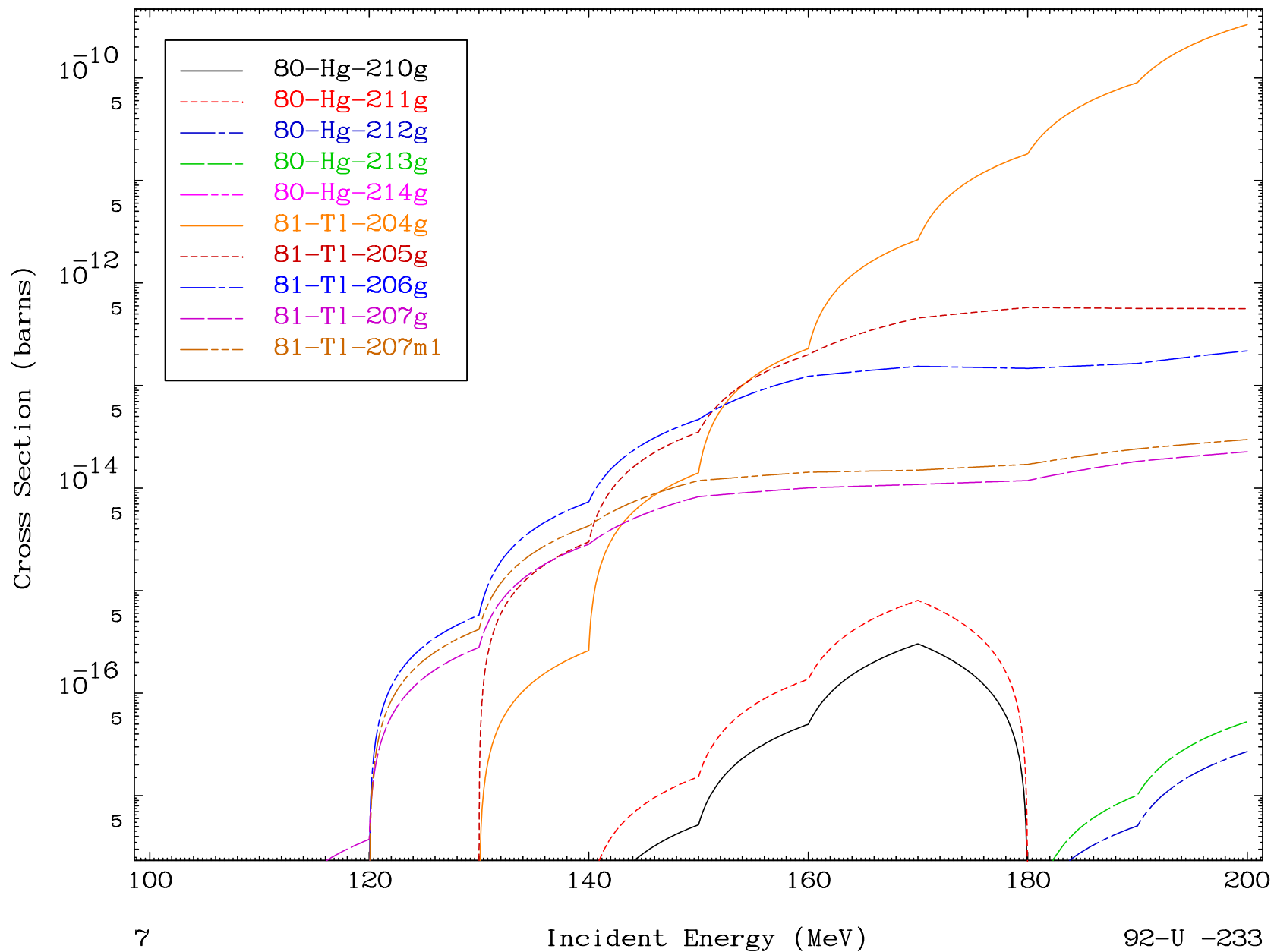
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

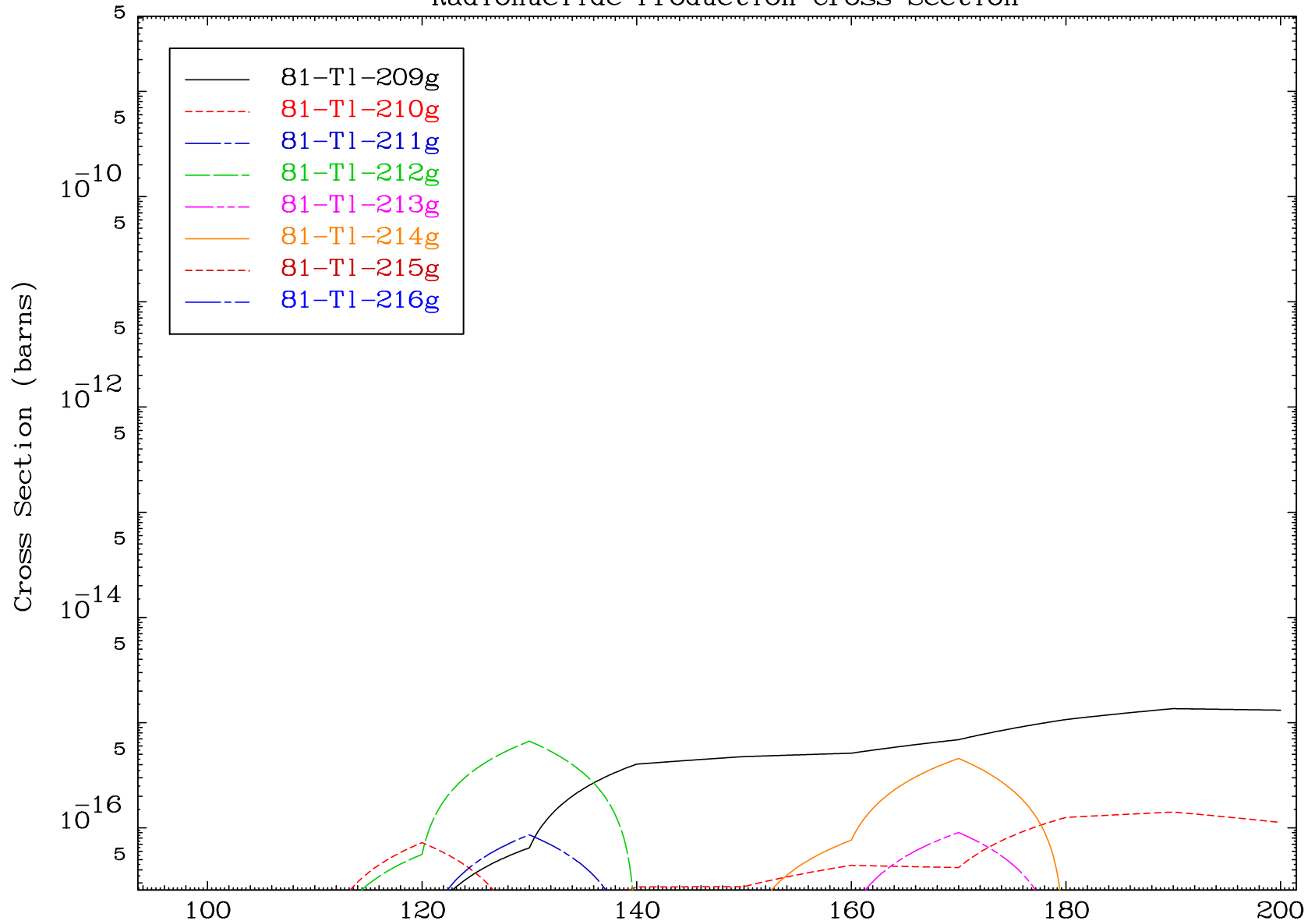


MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section

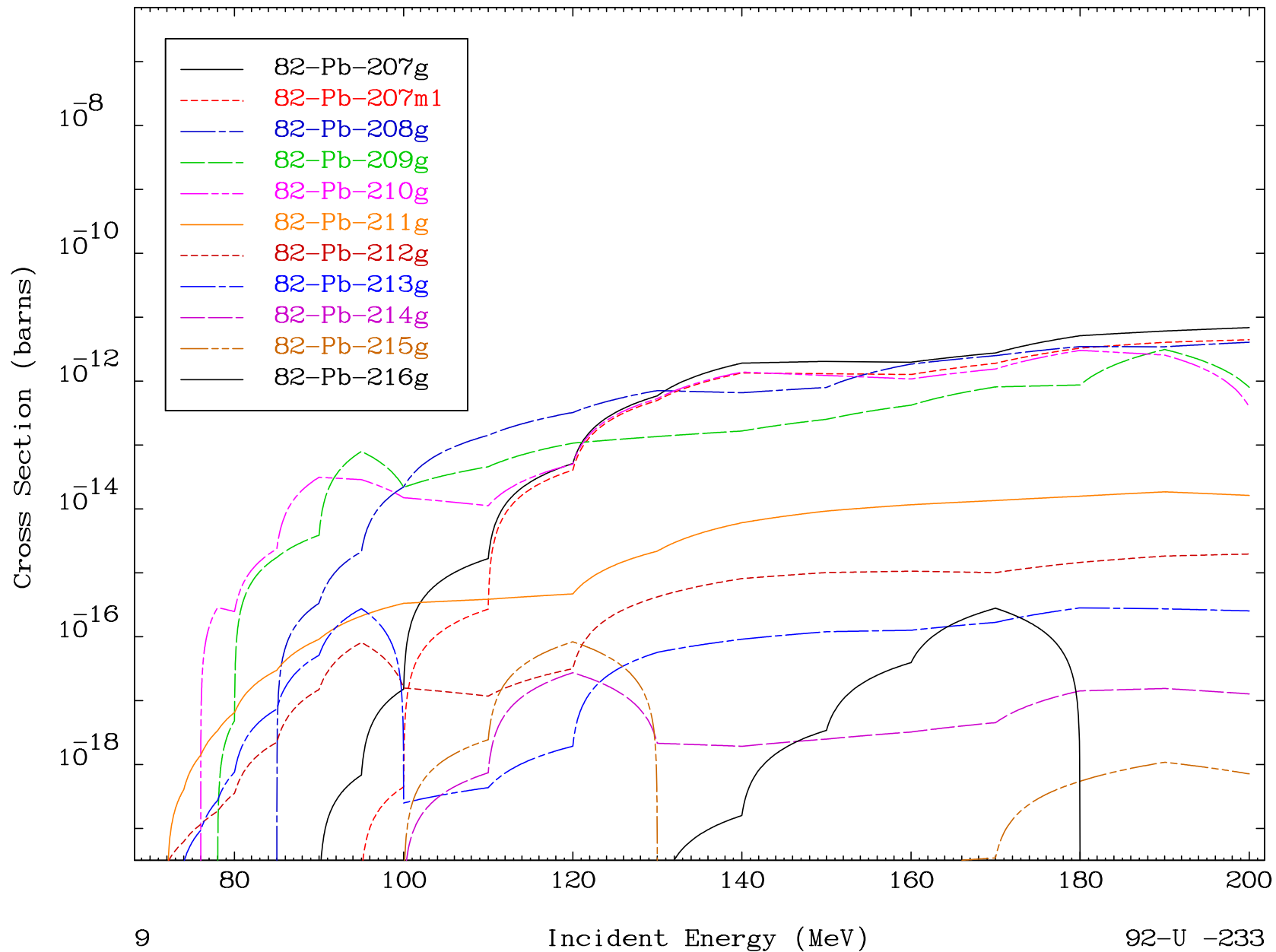


8

Incident Energy (MeV)

92-U -233

Radionuclide Production Cross Section

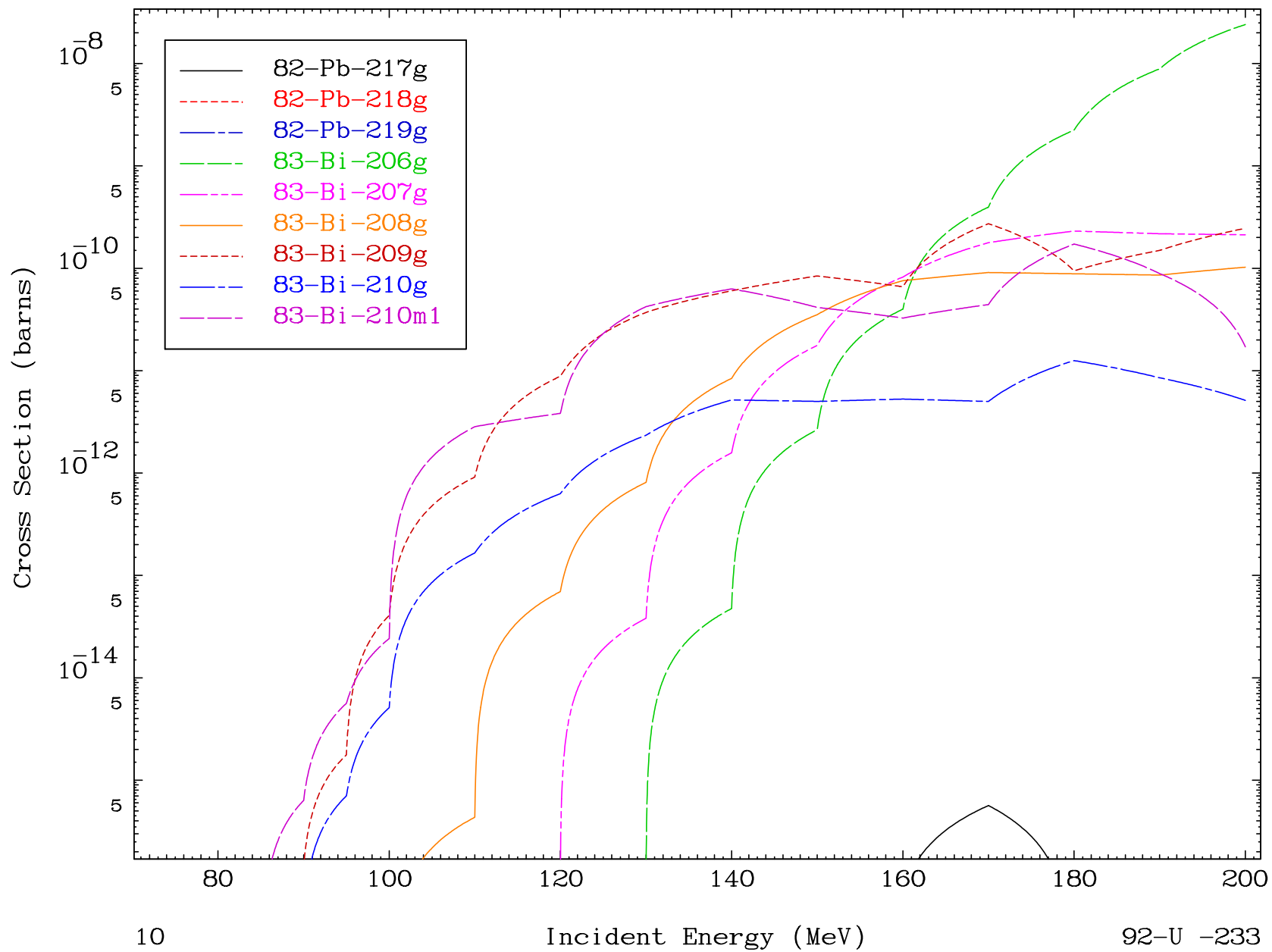


MAT 9222

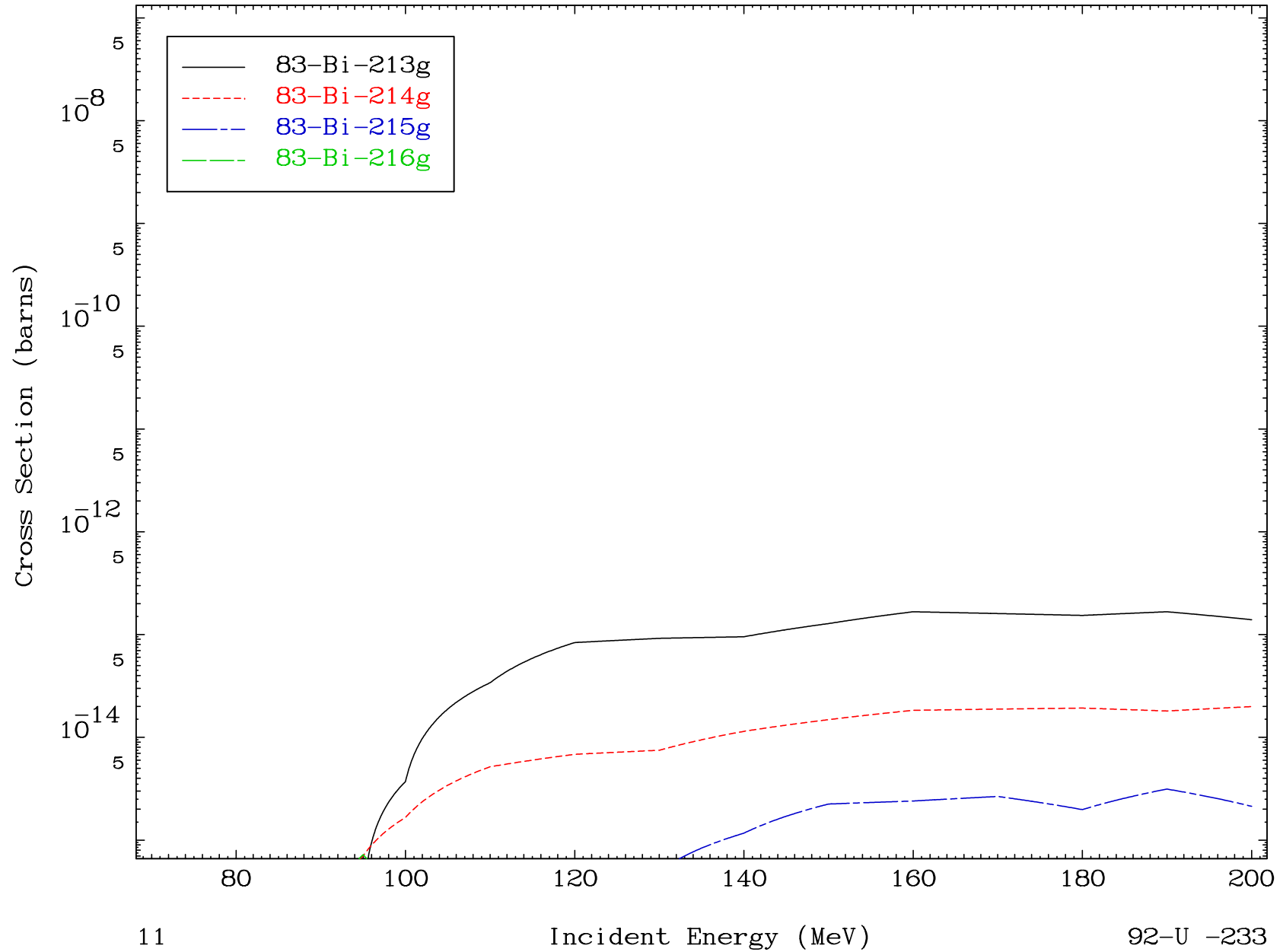
 $(\gamma, \text{remainder})$

92-U -233

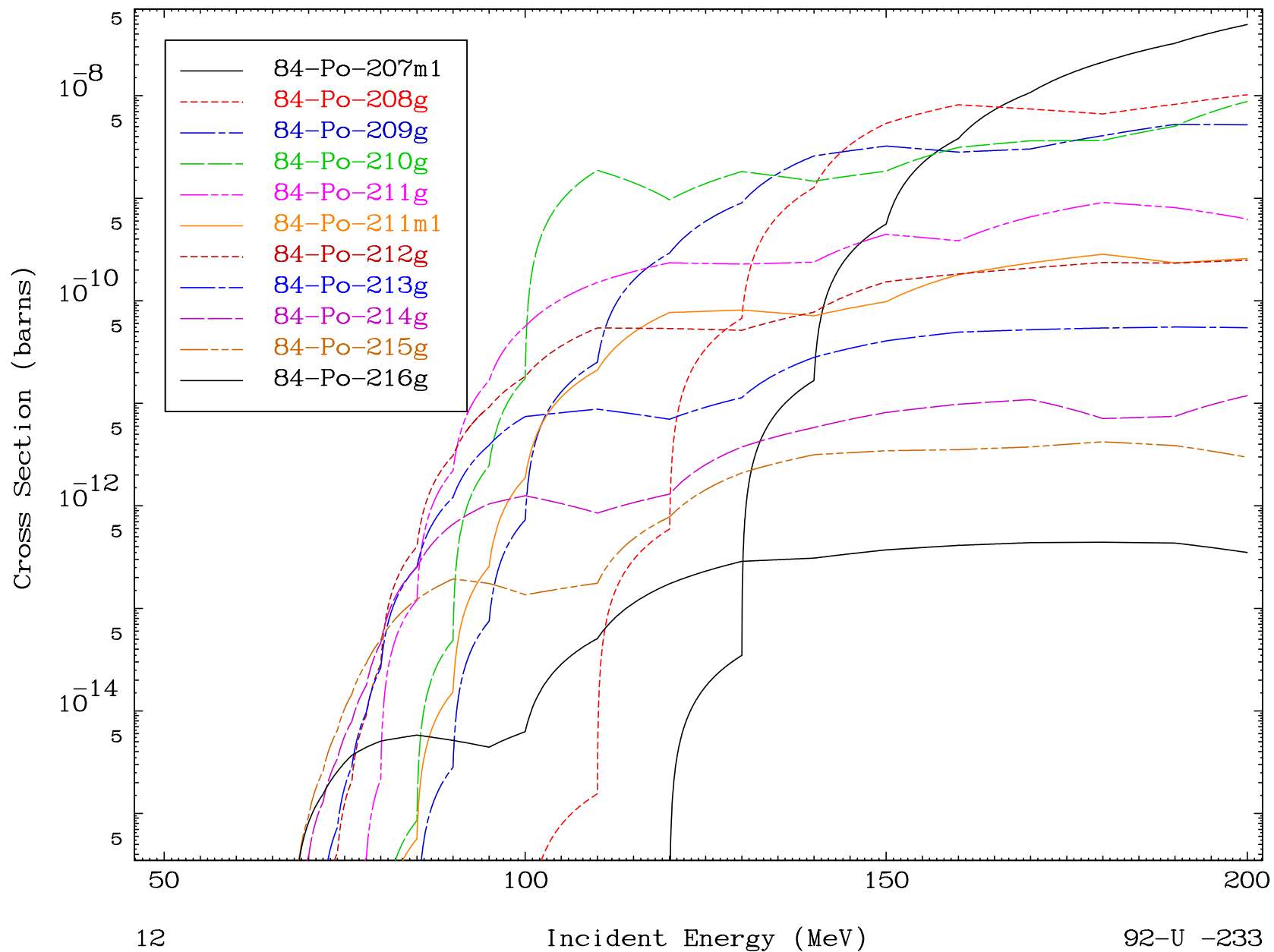
Radionuclide Production Cross Section



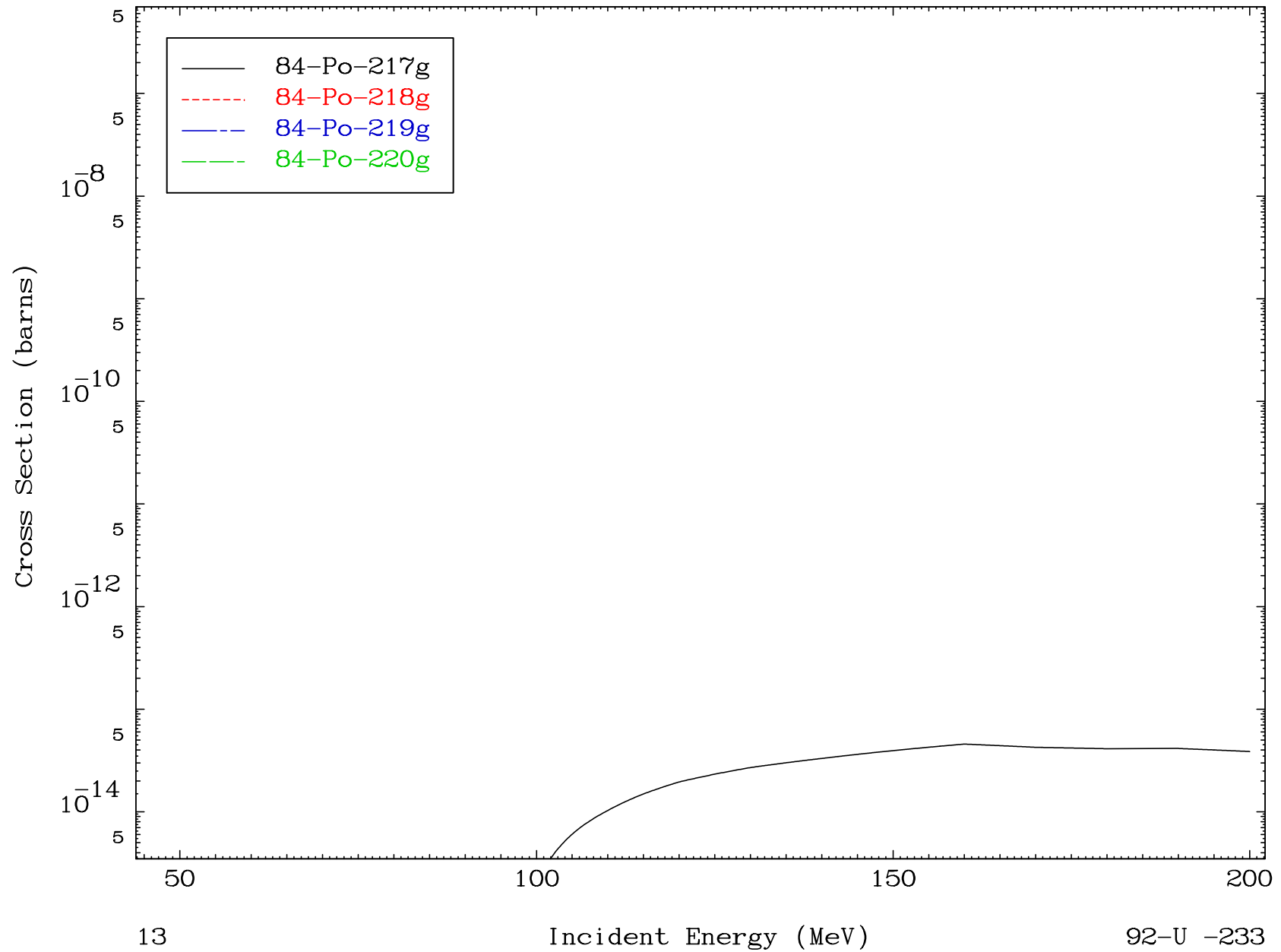
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

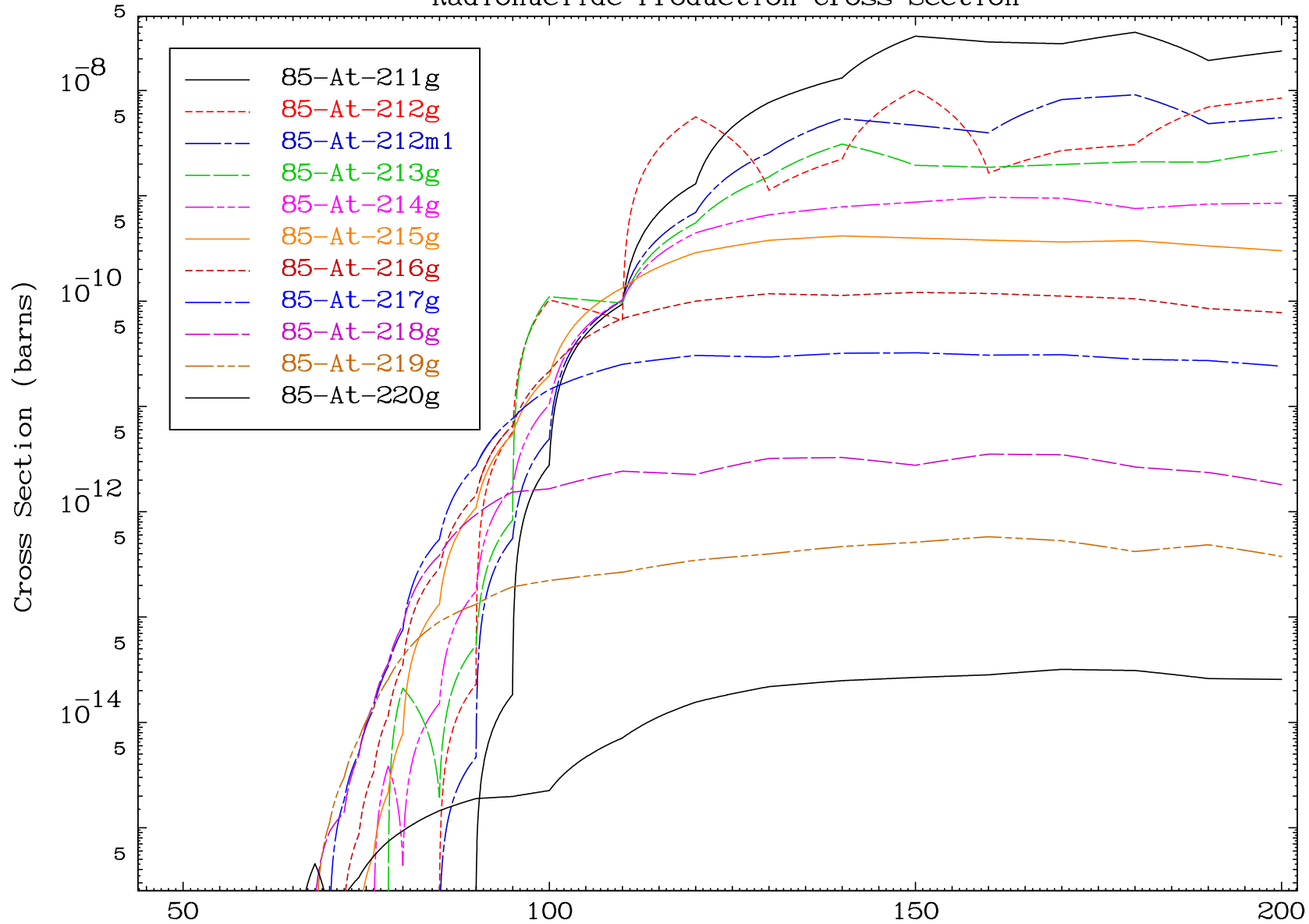


MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



14

Incident Energy (MeV)

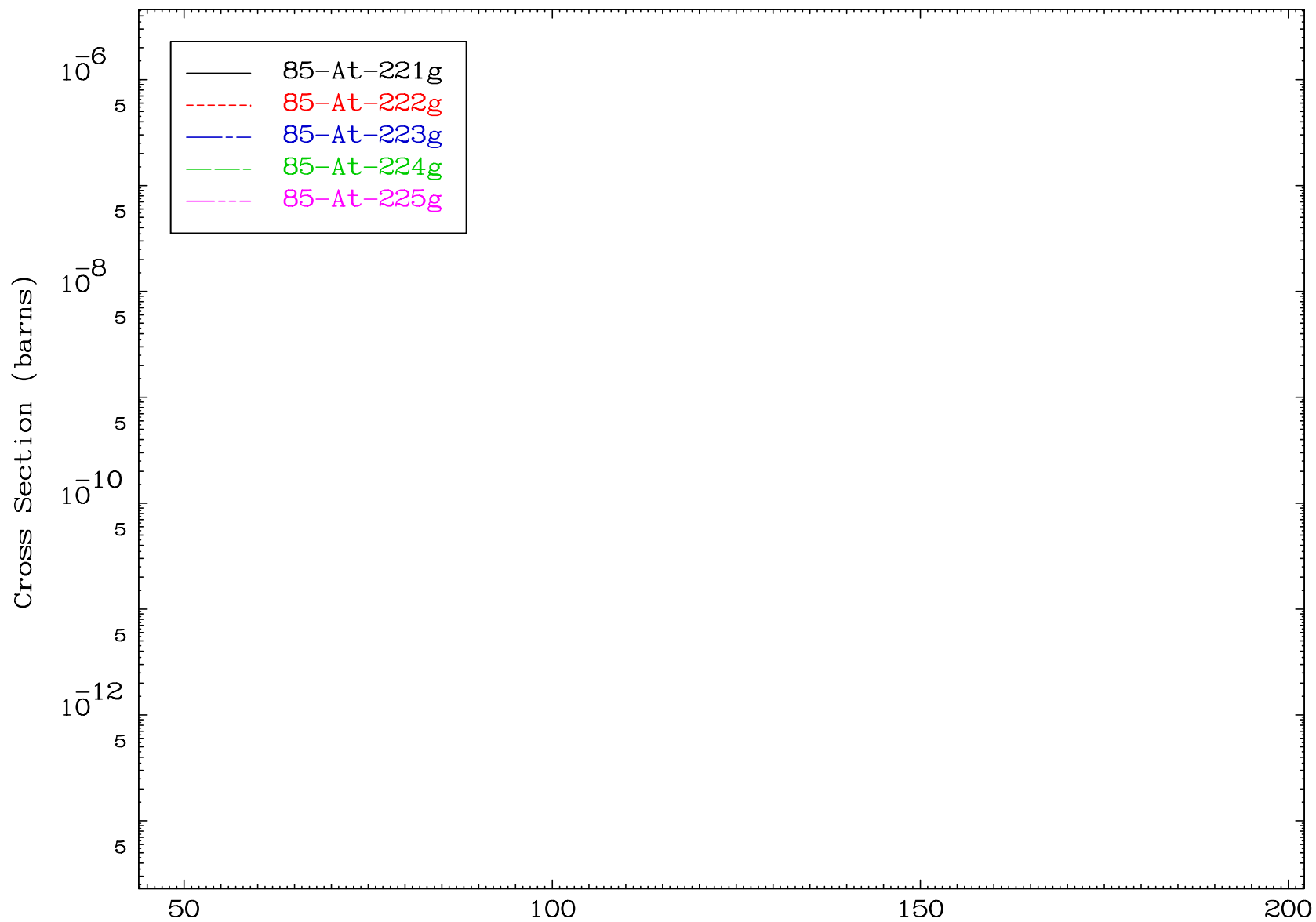
92-U -233

MAT 9222

(γ , remainder)

92-U -233

Radionuclide Production Cross Section



15

Incident Energy (MeV)

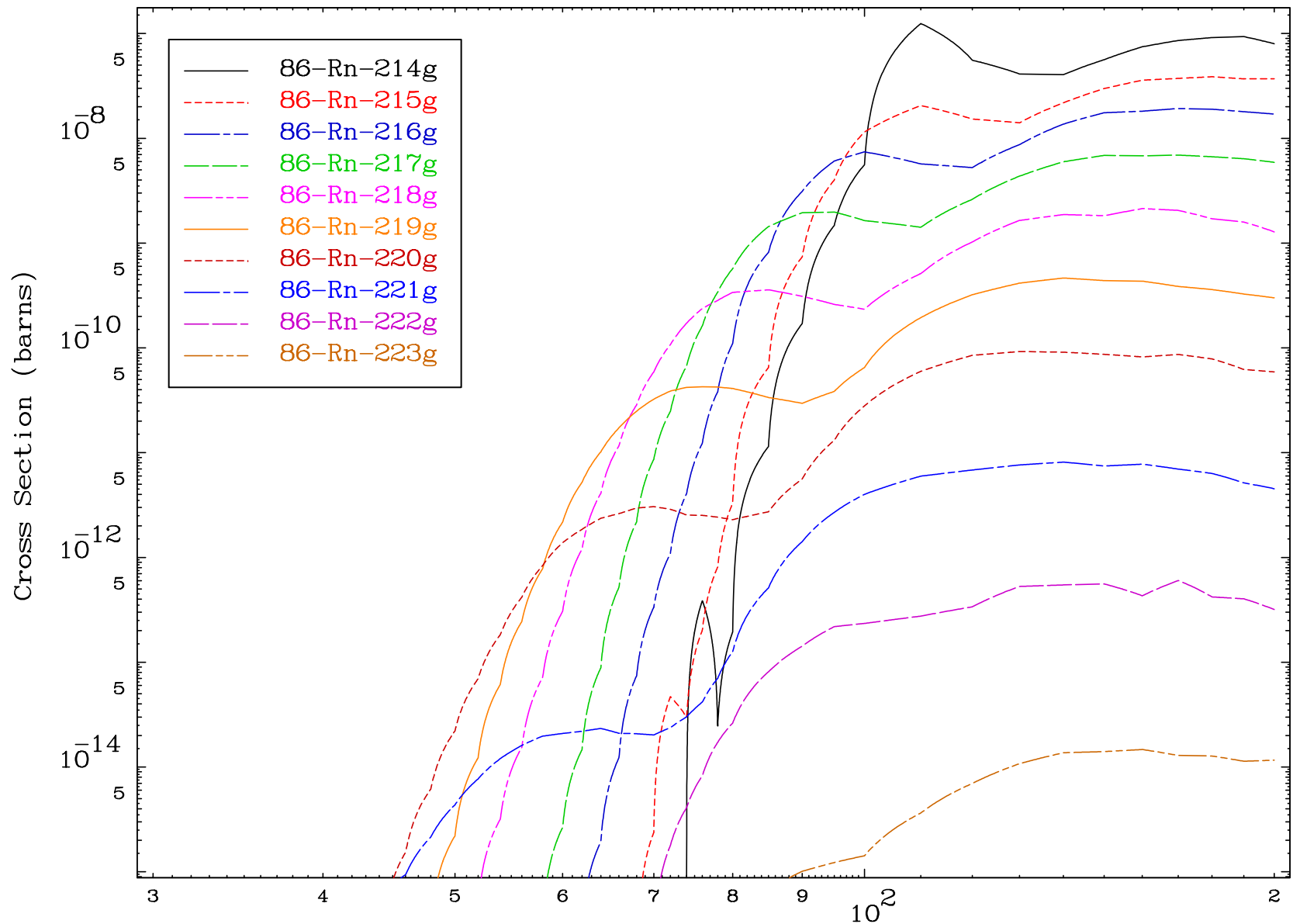
92-U -233

MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section

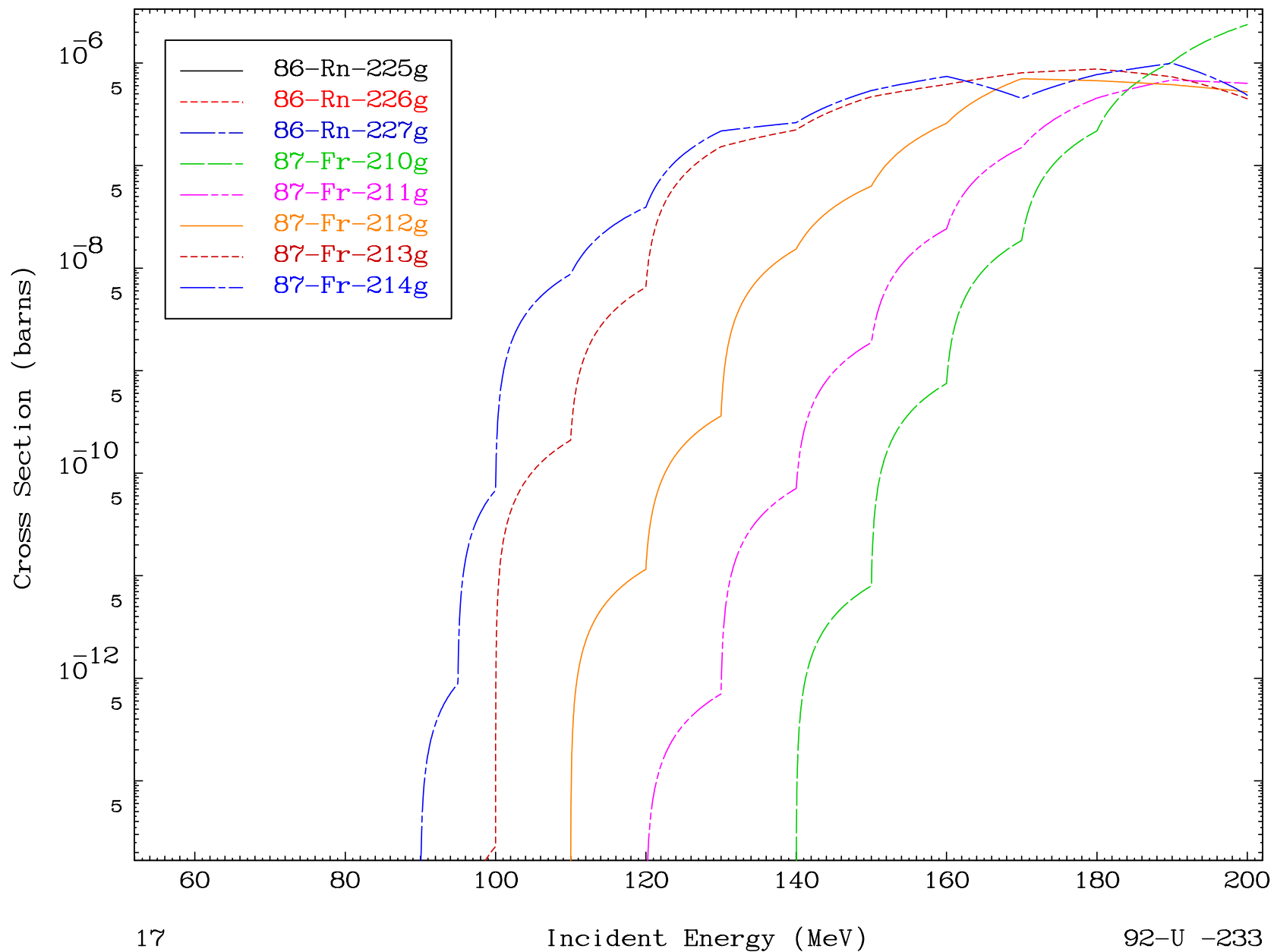


16

Incident Energy (MeV)

92-U -233

Radionuclide Production Cross Section

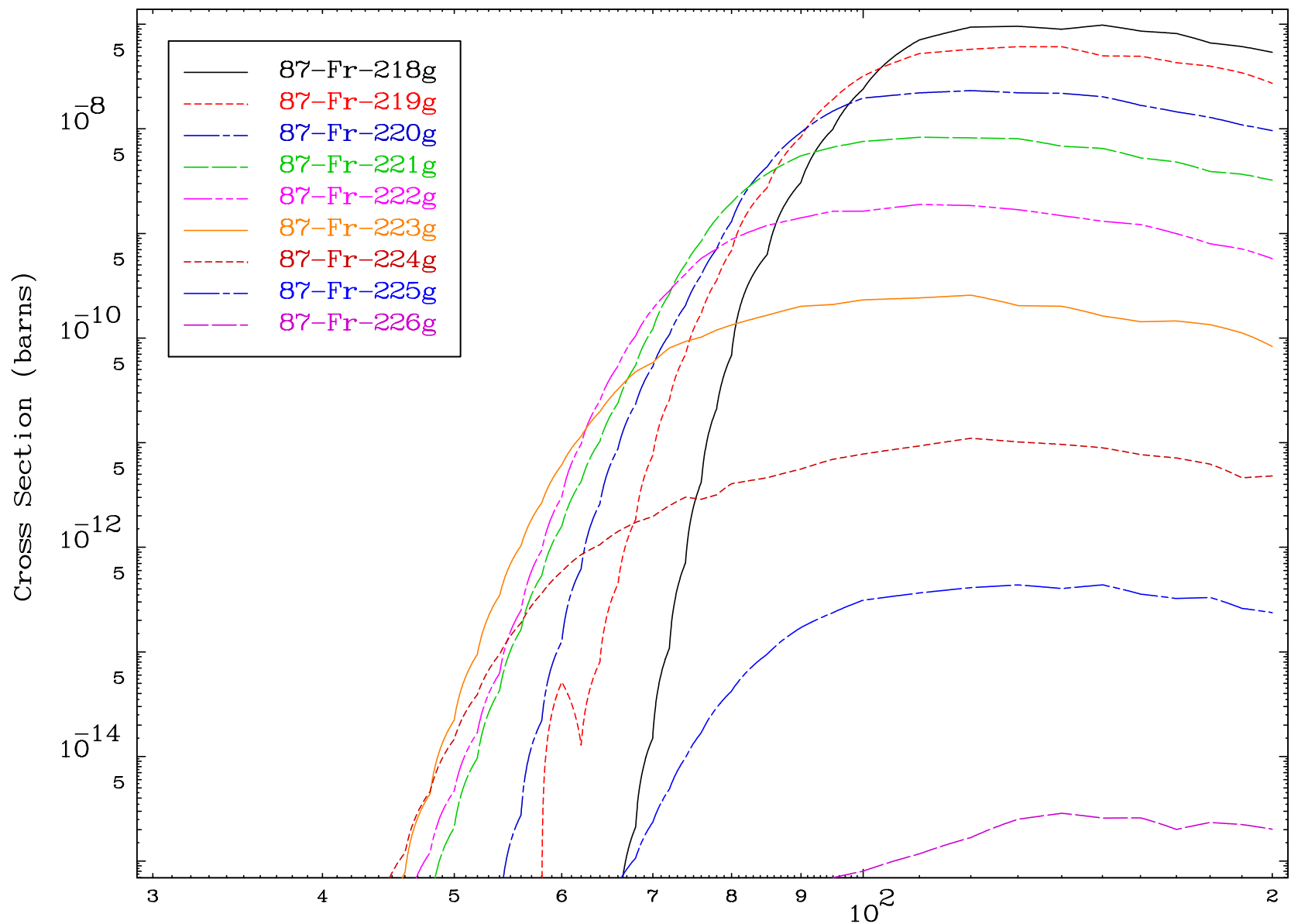


MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



18

Incident Energy (MeV)

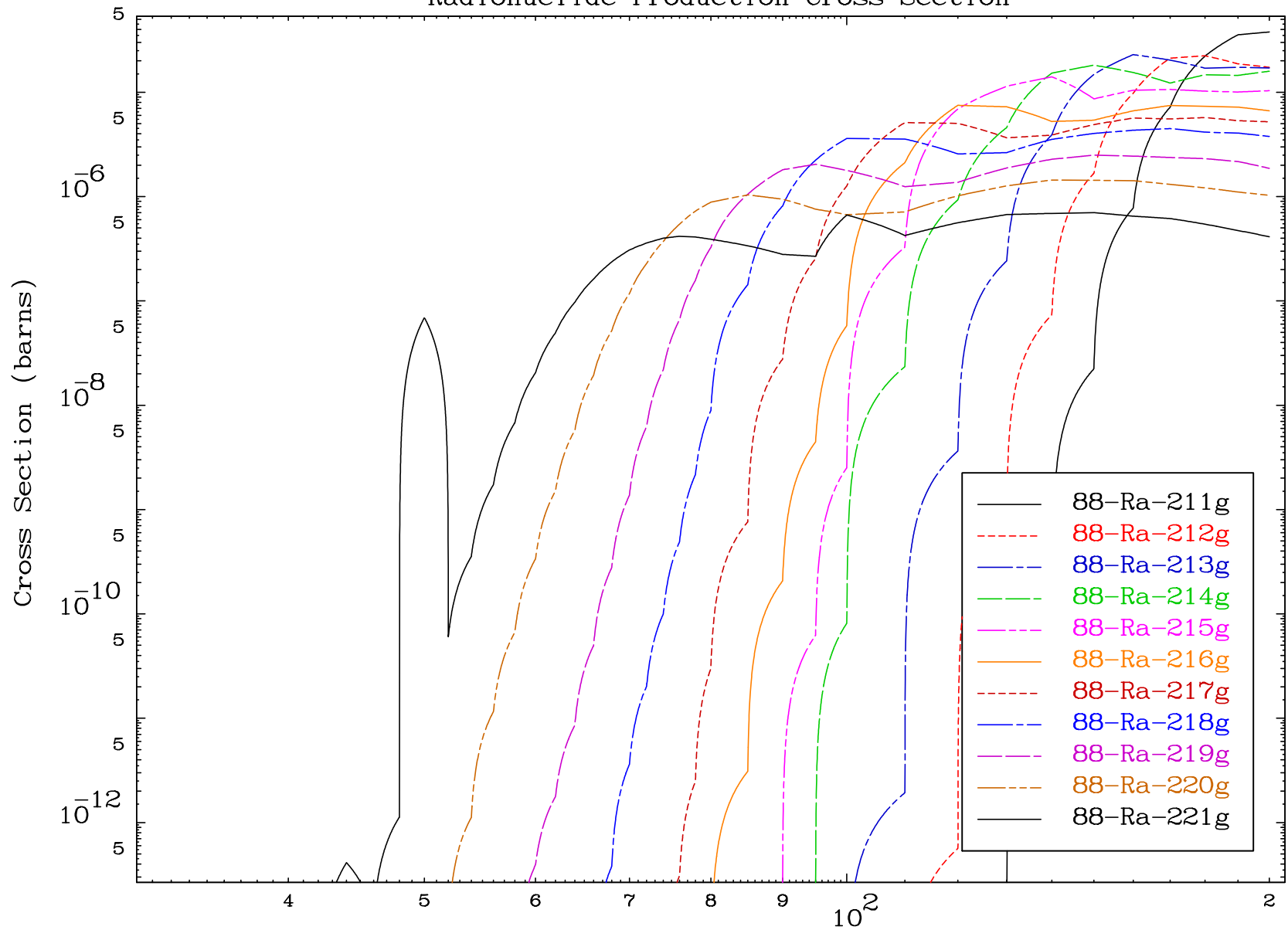
92-U -233

MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



19

Incident Energy (MeV)

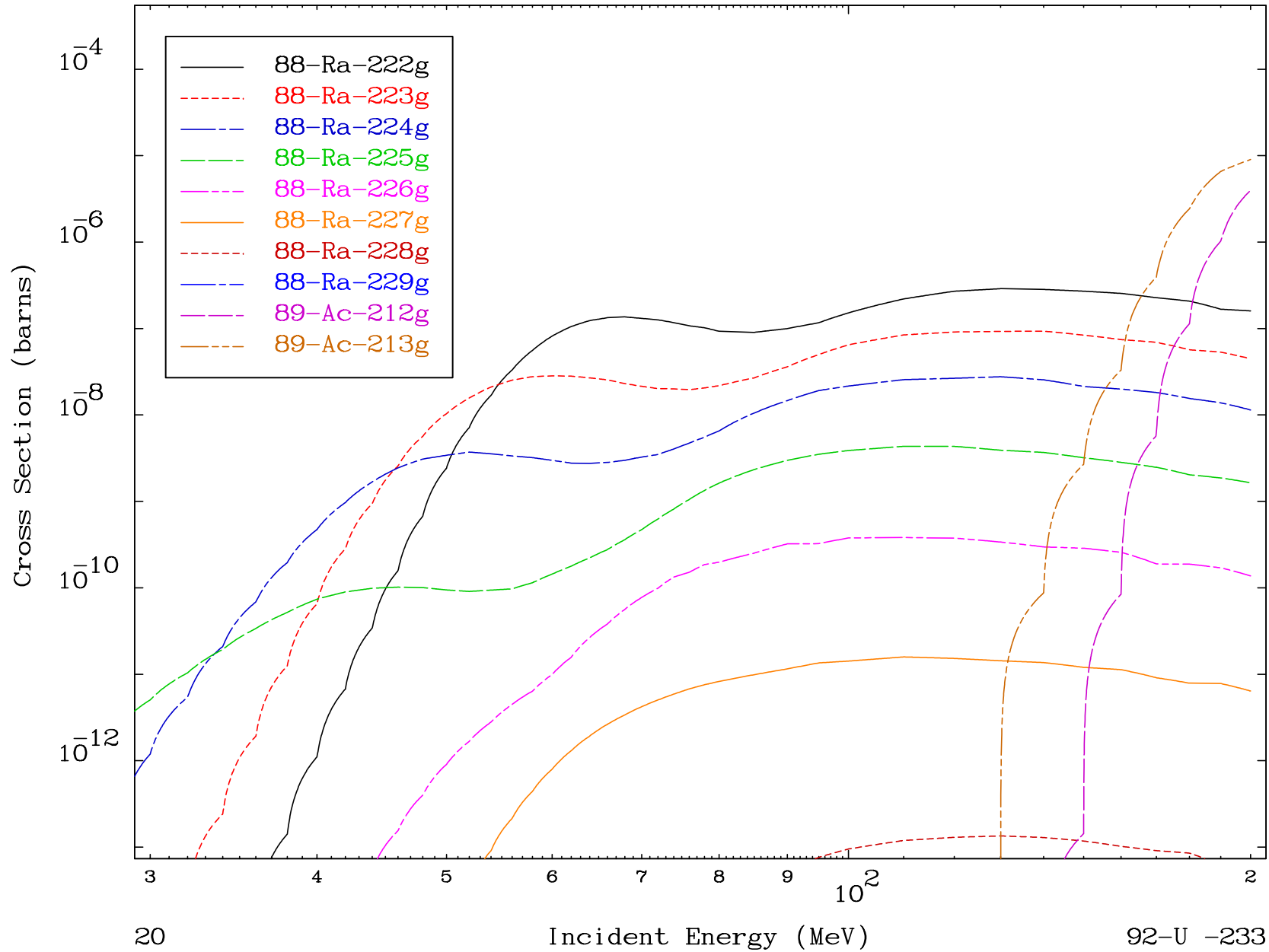
92-U -233

MAT 9222

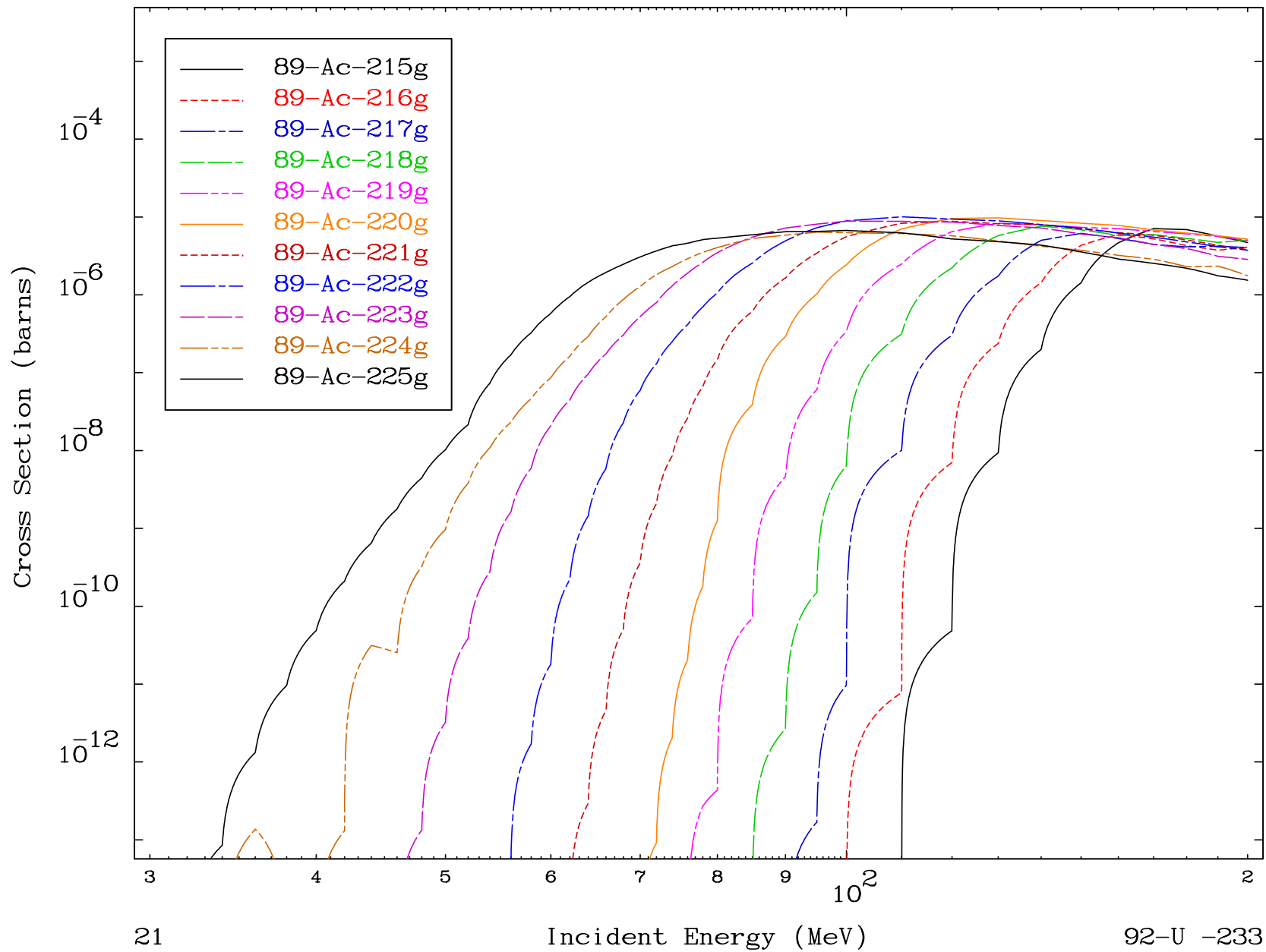
 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



Radionuclide Production Cross Section

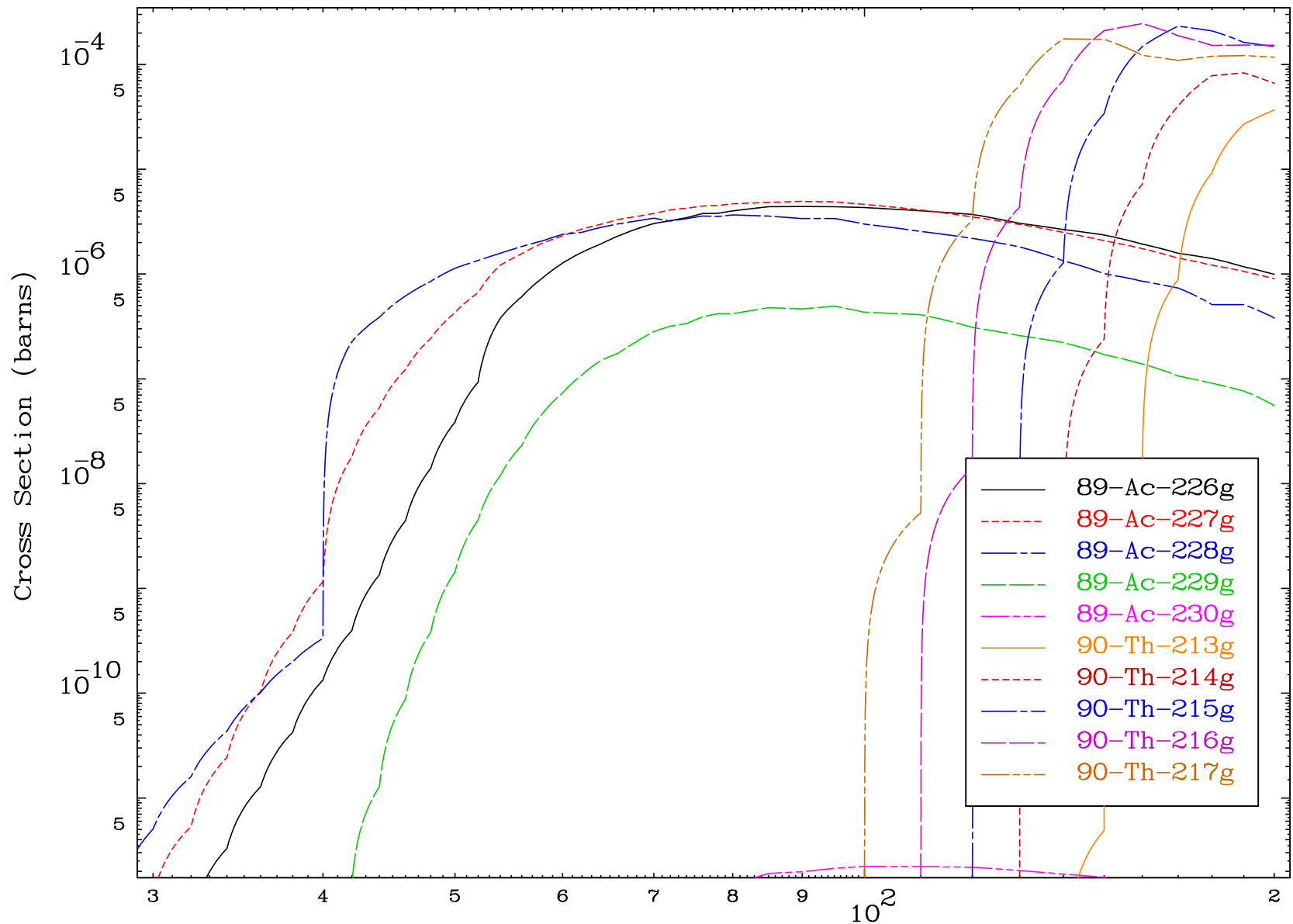


MAT 9222

 $(\gamma, \text{remainder})$

92-U -233

Radionuclide Production Cross Section



22

Incident Energy (MeV)

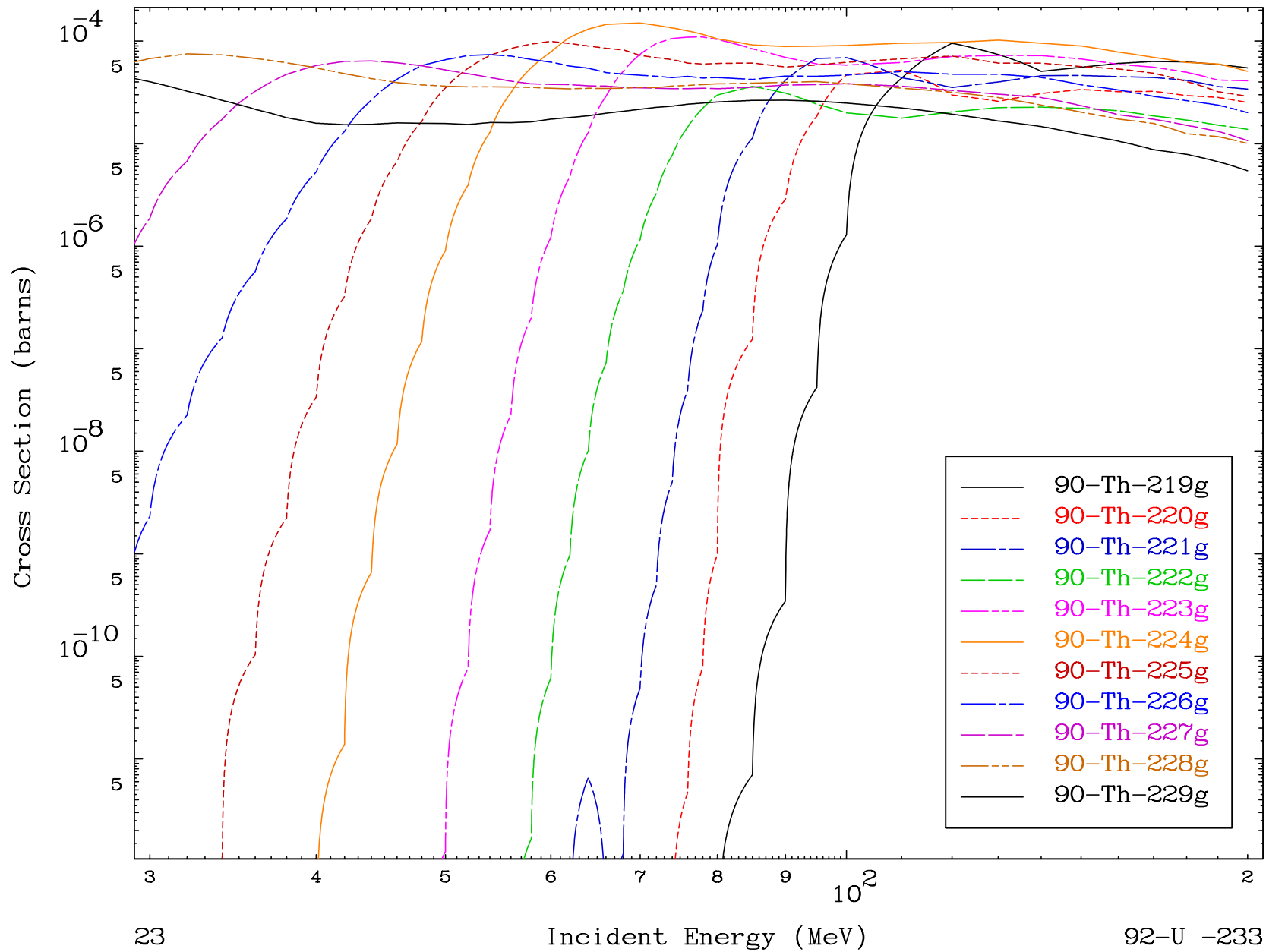
92-U -233

MAT 9222

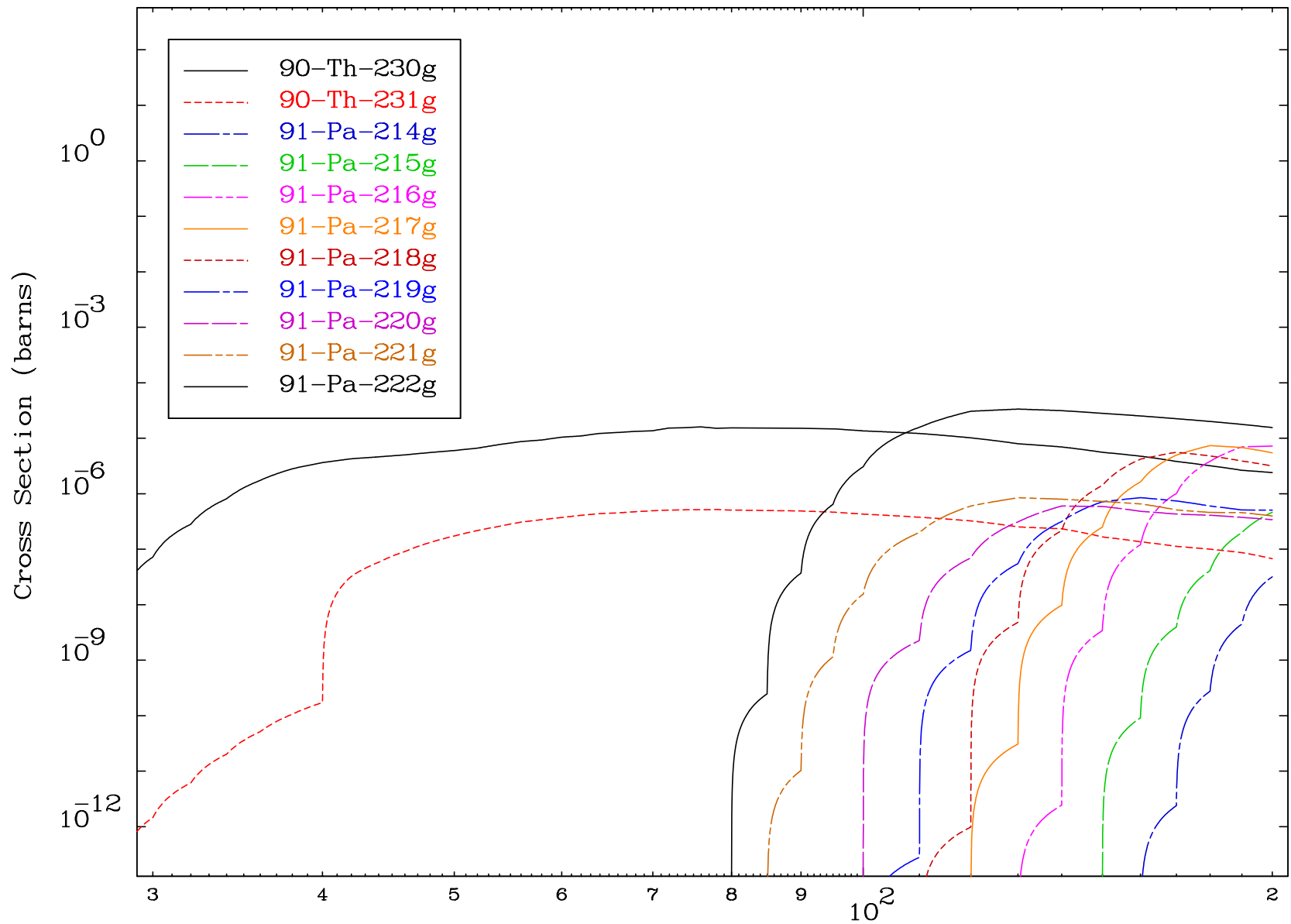
 $(\gamma, \text{remainder})$

92-U -233

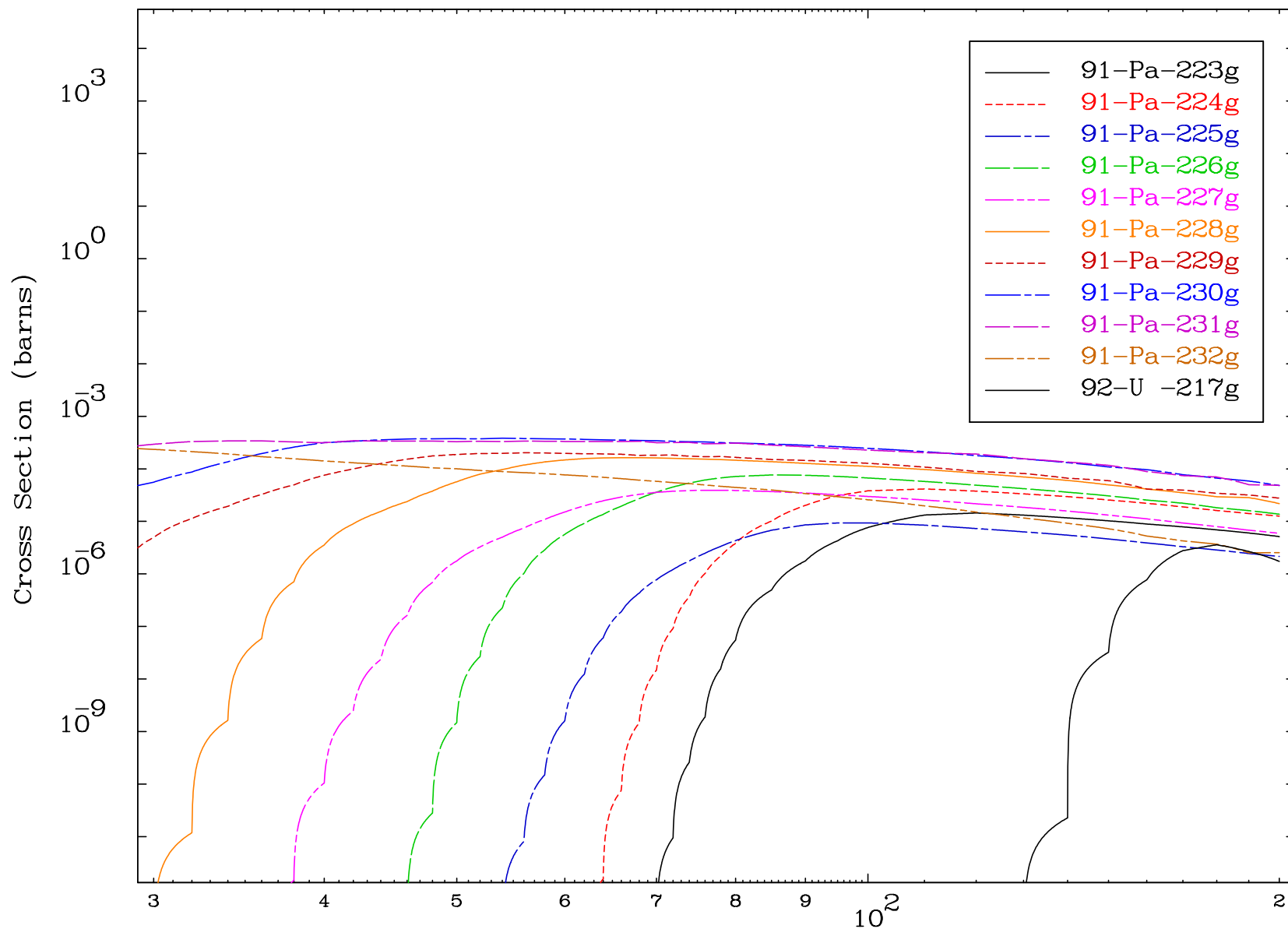
Radionuclide Production Cross Section



Radionuclide Production Cross Section



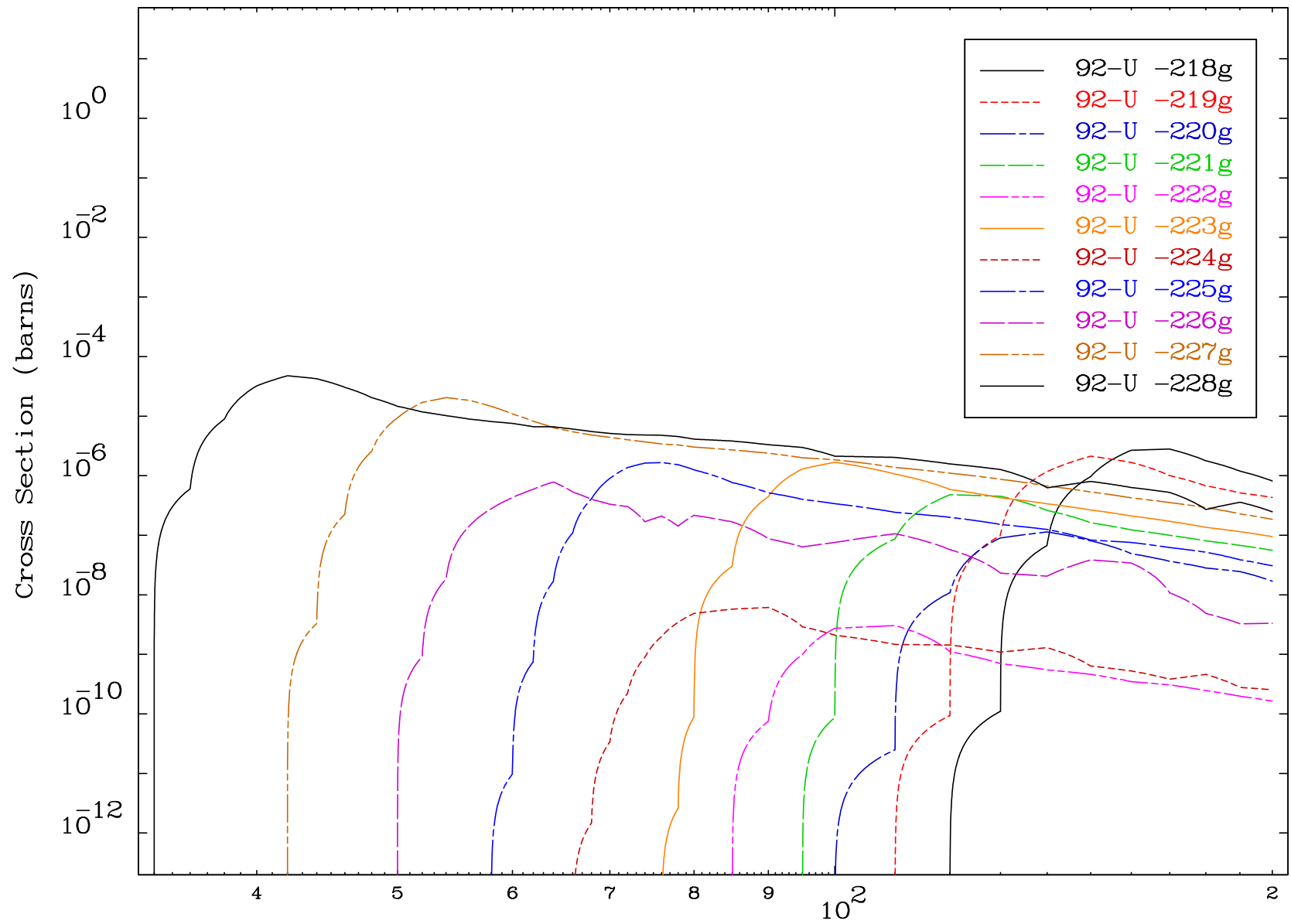
Radionuclide Production Cross Section



MAT 9222

(γ , remainder)
Radionuclide Production Cross Section

92-U -233



26

Incident Energy (MeV)

92-U -233

