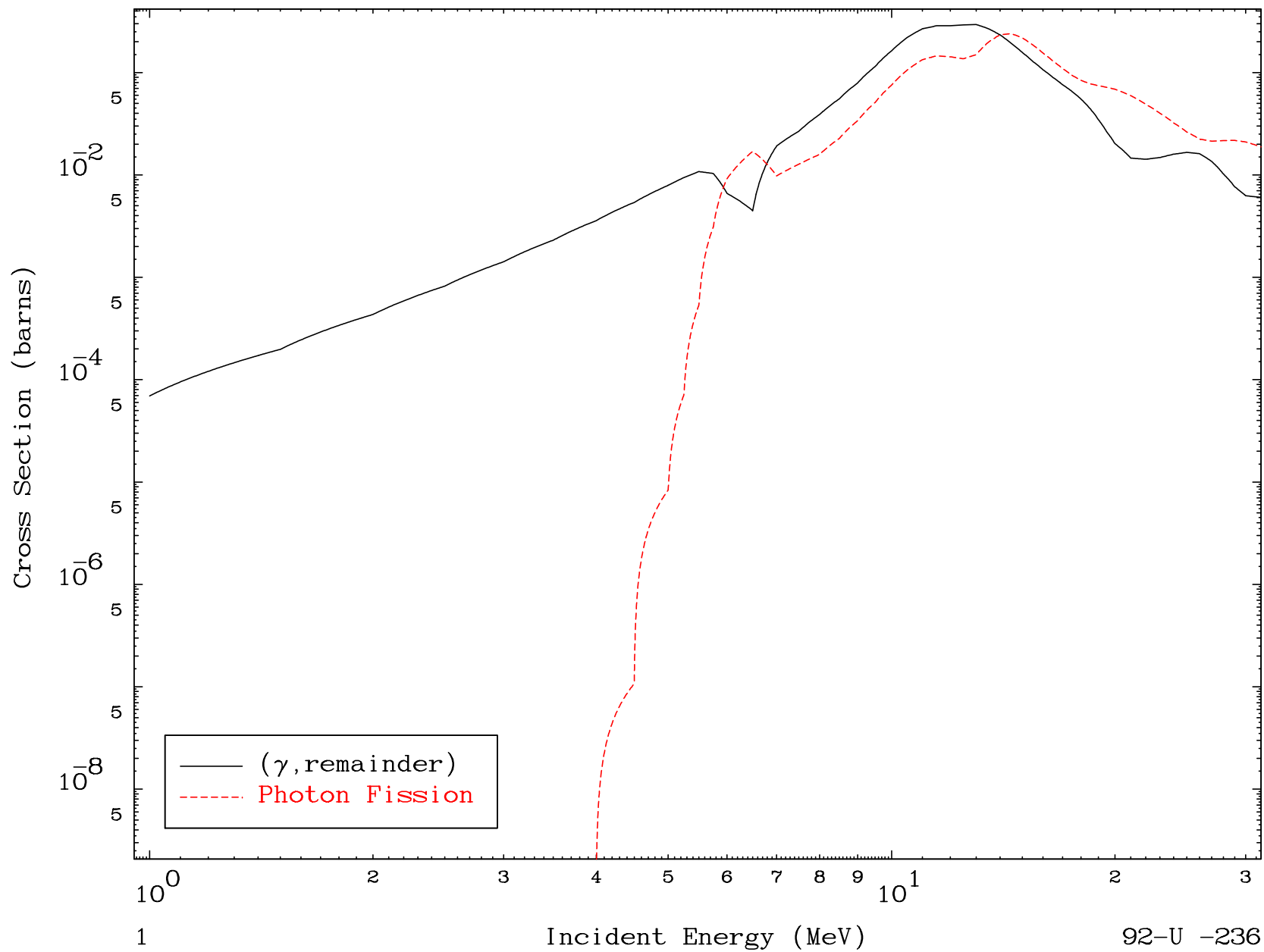
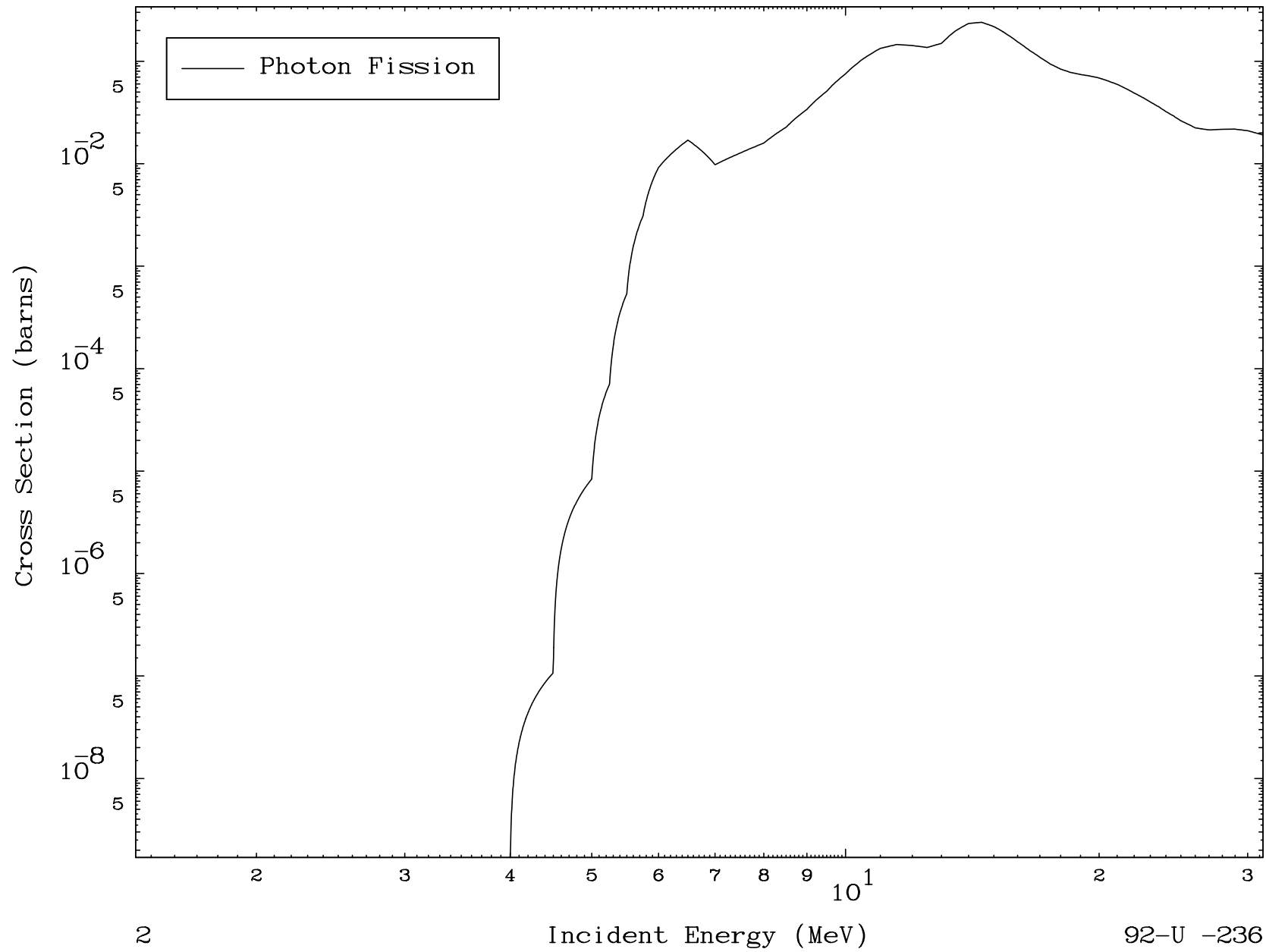


MAT 9231

Photon Major  
0 Kelvin Cross Sections

92-U -236

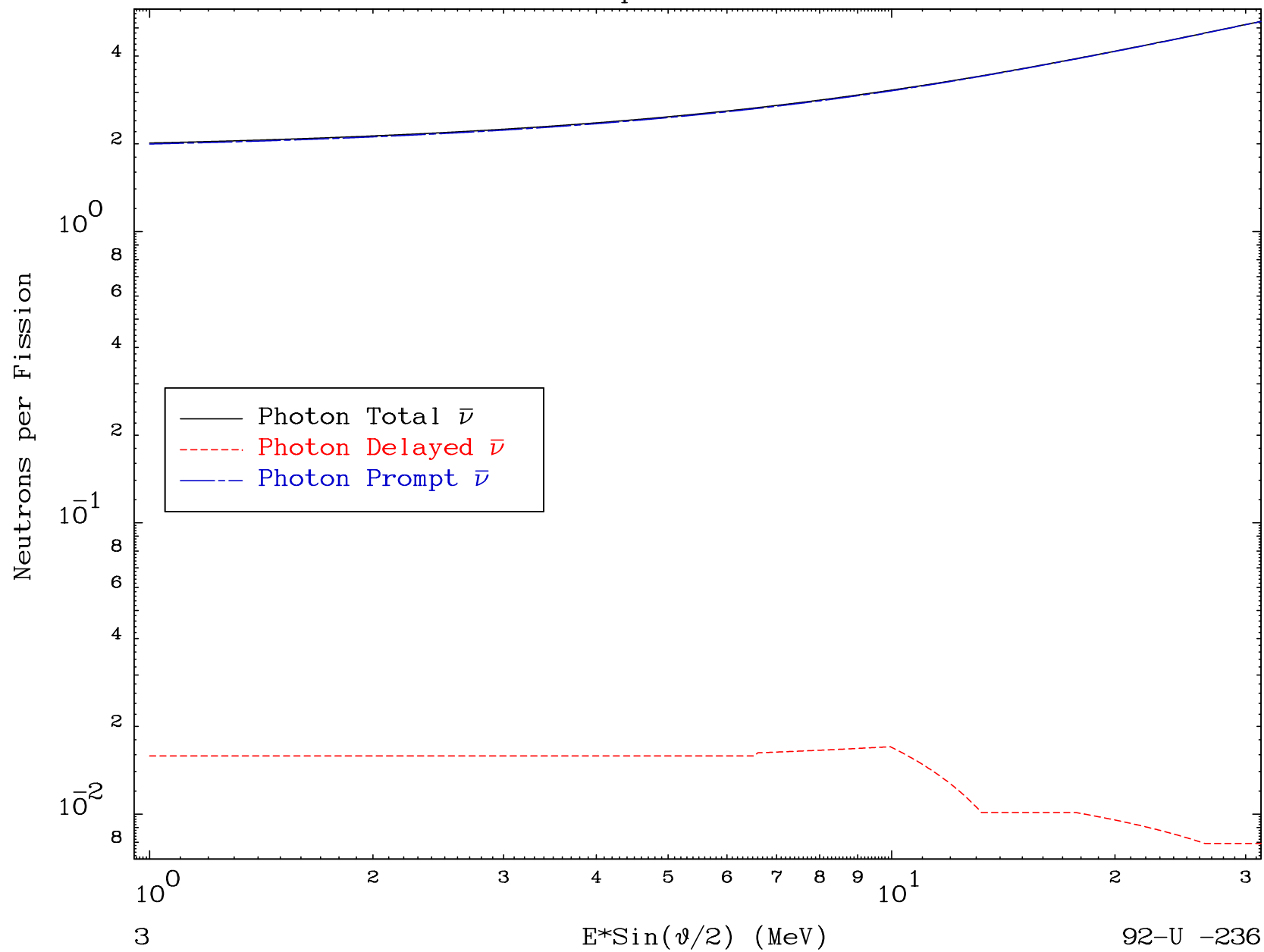




MAT 9231

Photon Neutrons  
per Fission

92-U -236

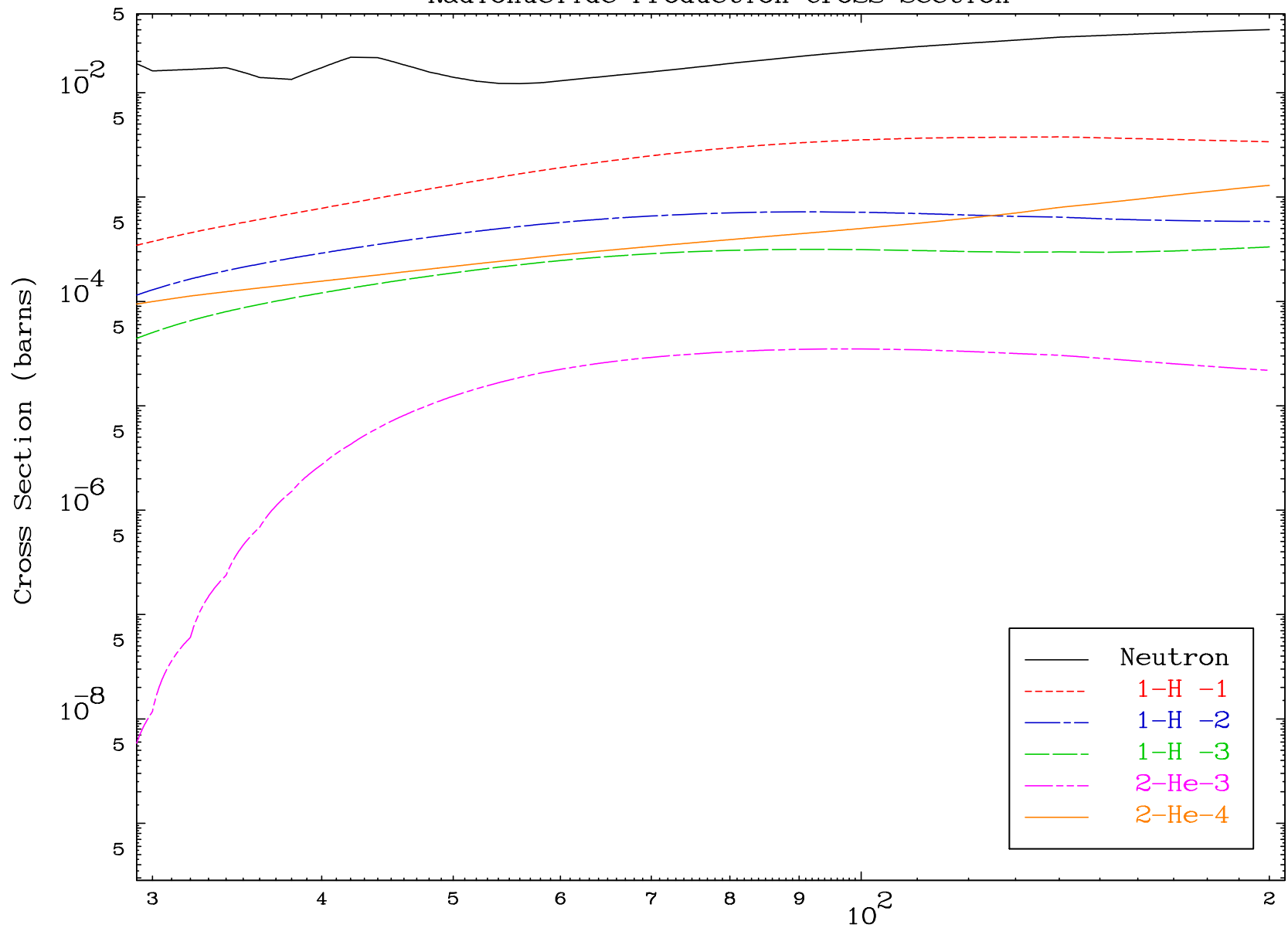


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section

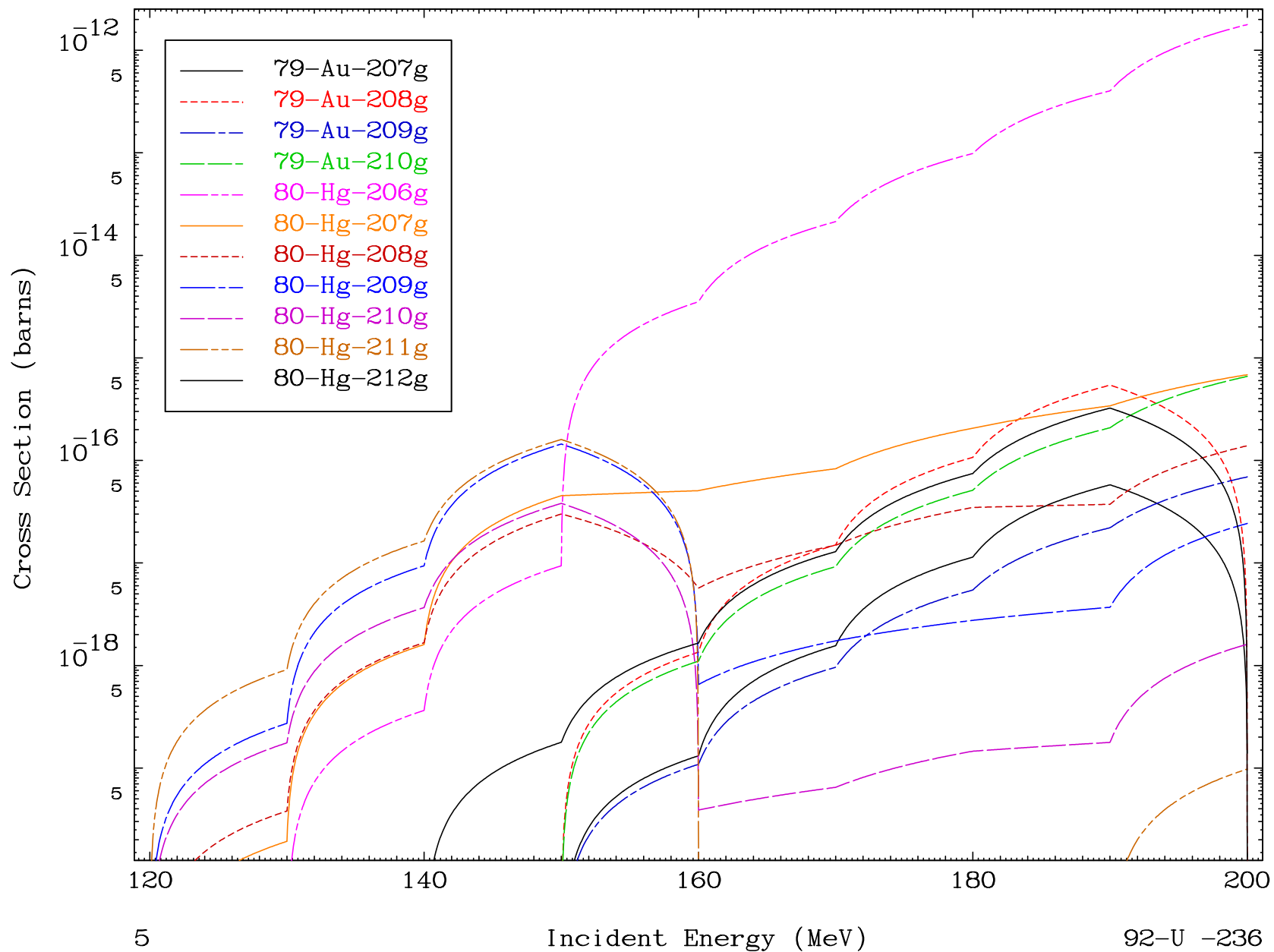


4

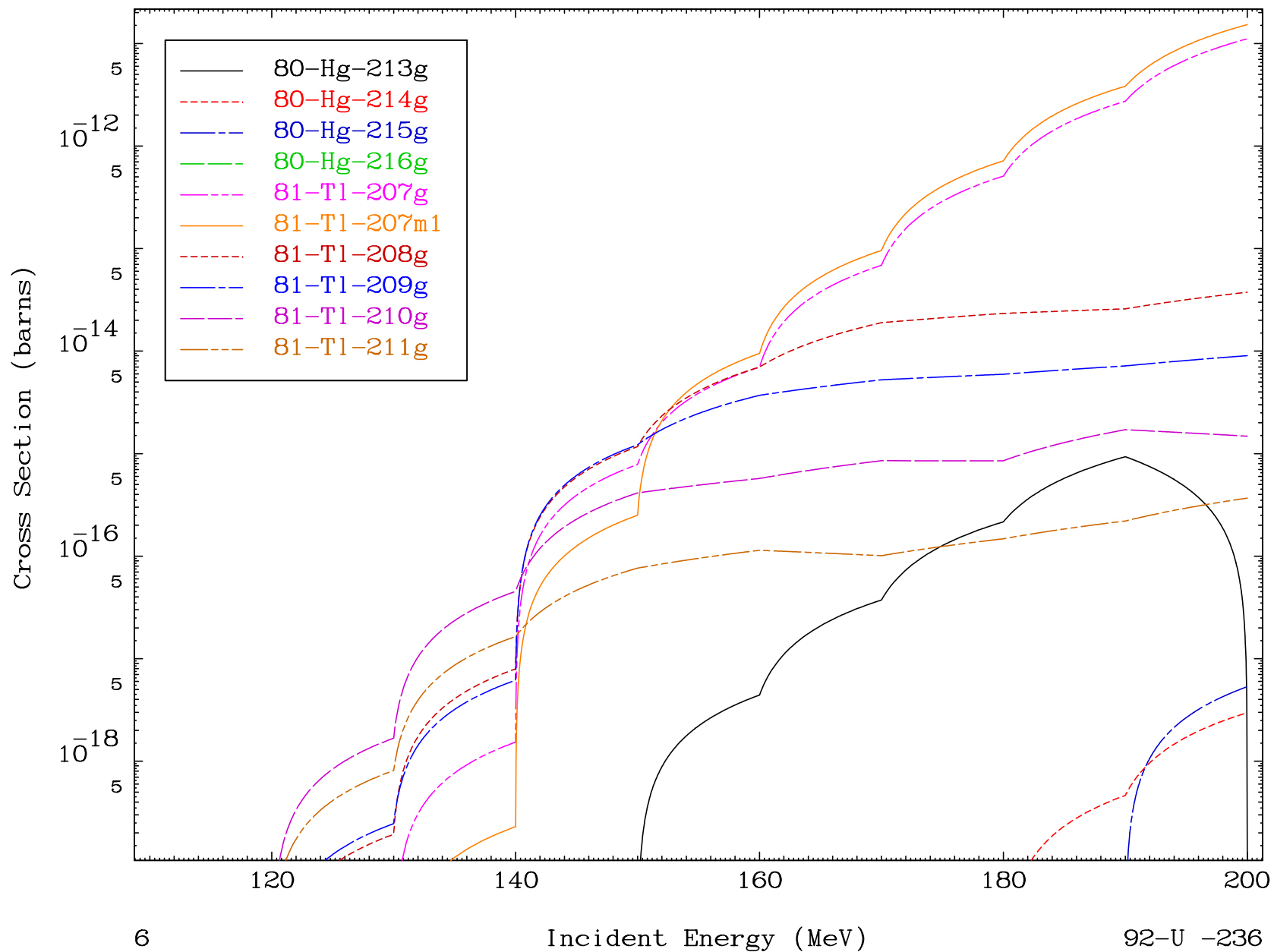
Incident Energy (MeV)

92-U -236

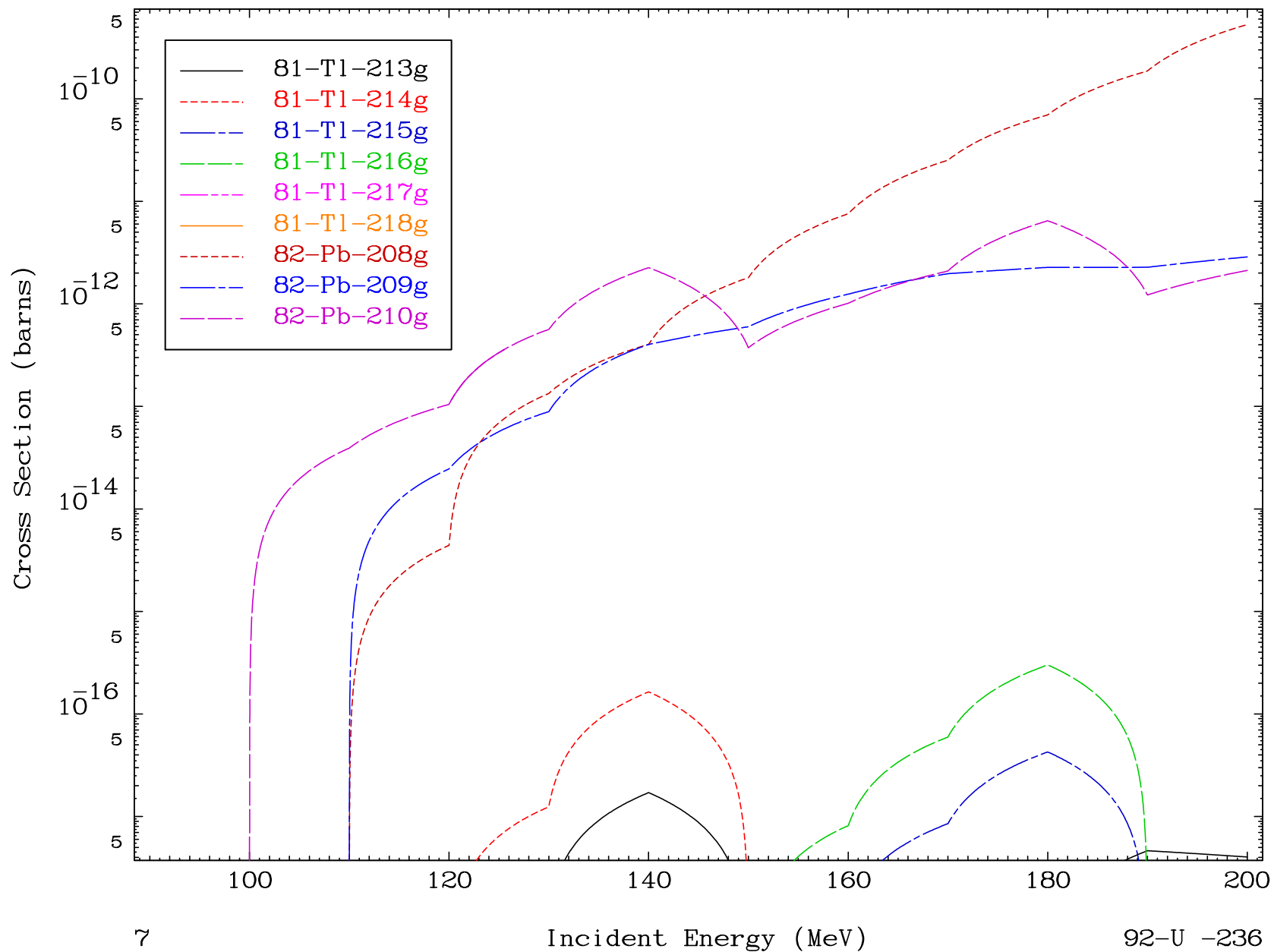
## Radionuclide Production Cross Section



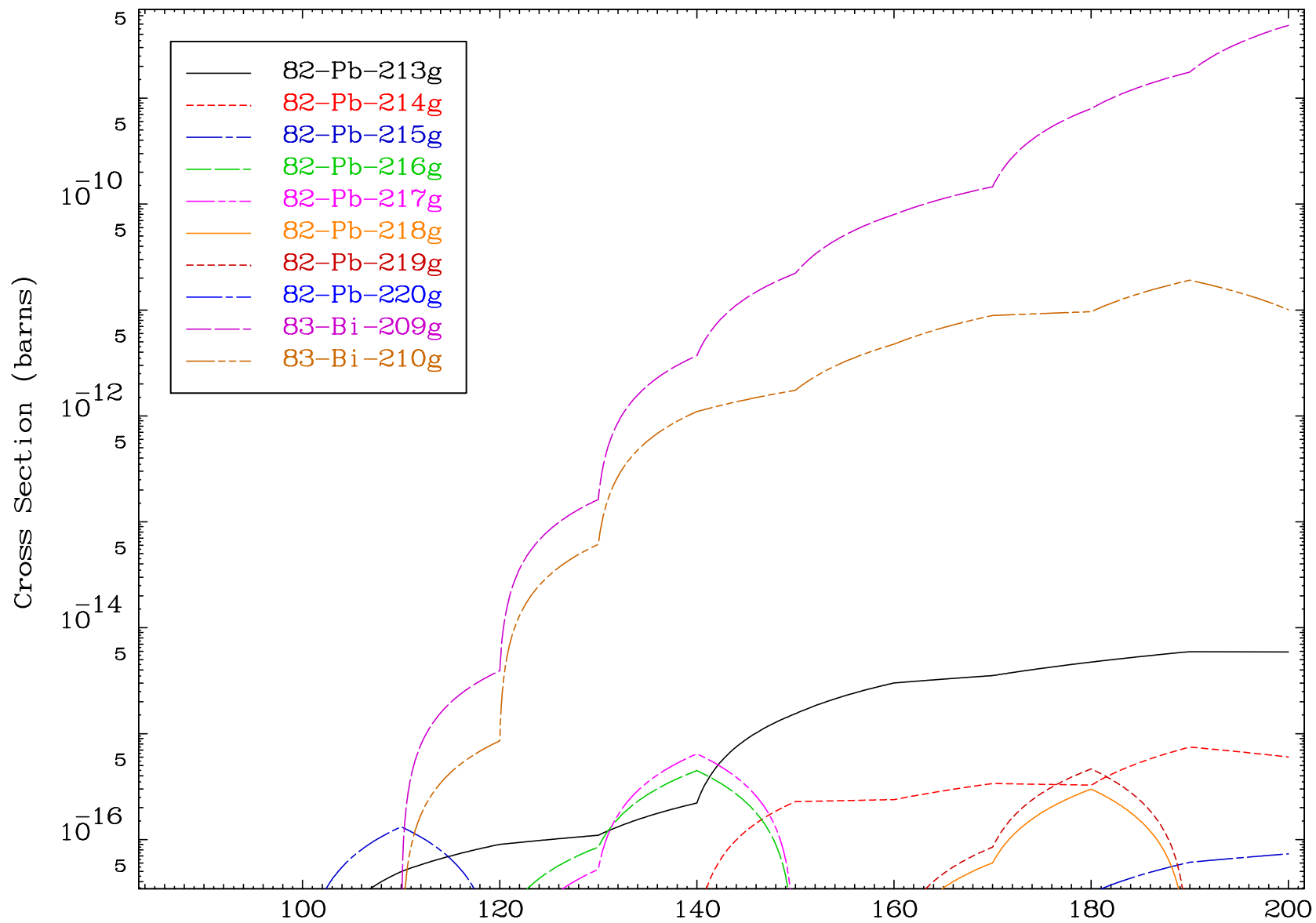
## Radionuclide Production Cross Section



## Radionuclide Production Cross Section

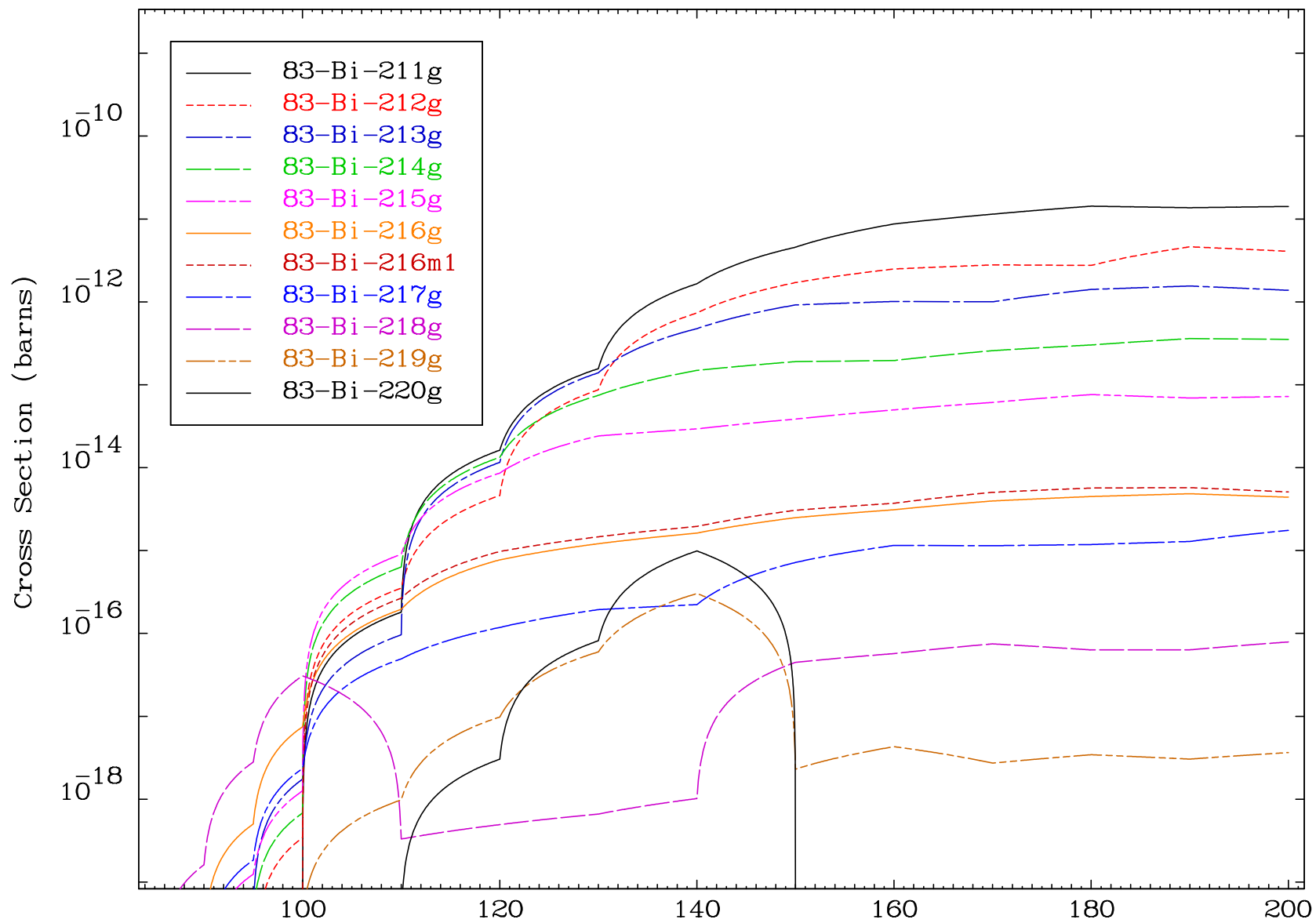


## Radionuclide Production Cross Section





## Radionuclide Production Cross Section

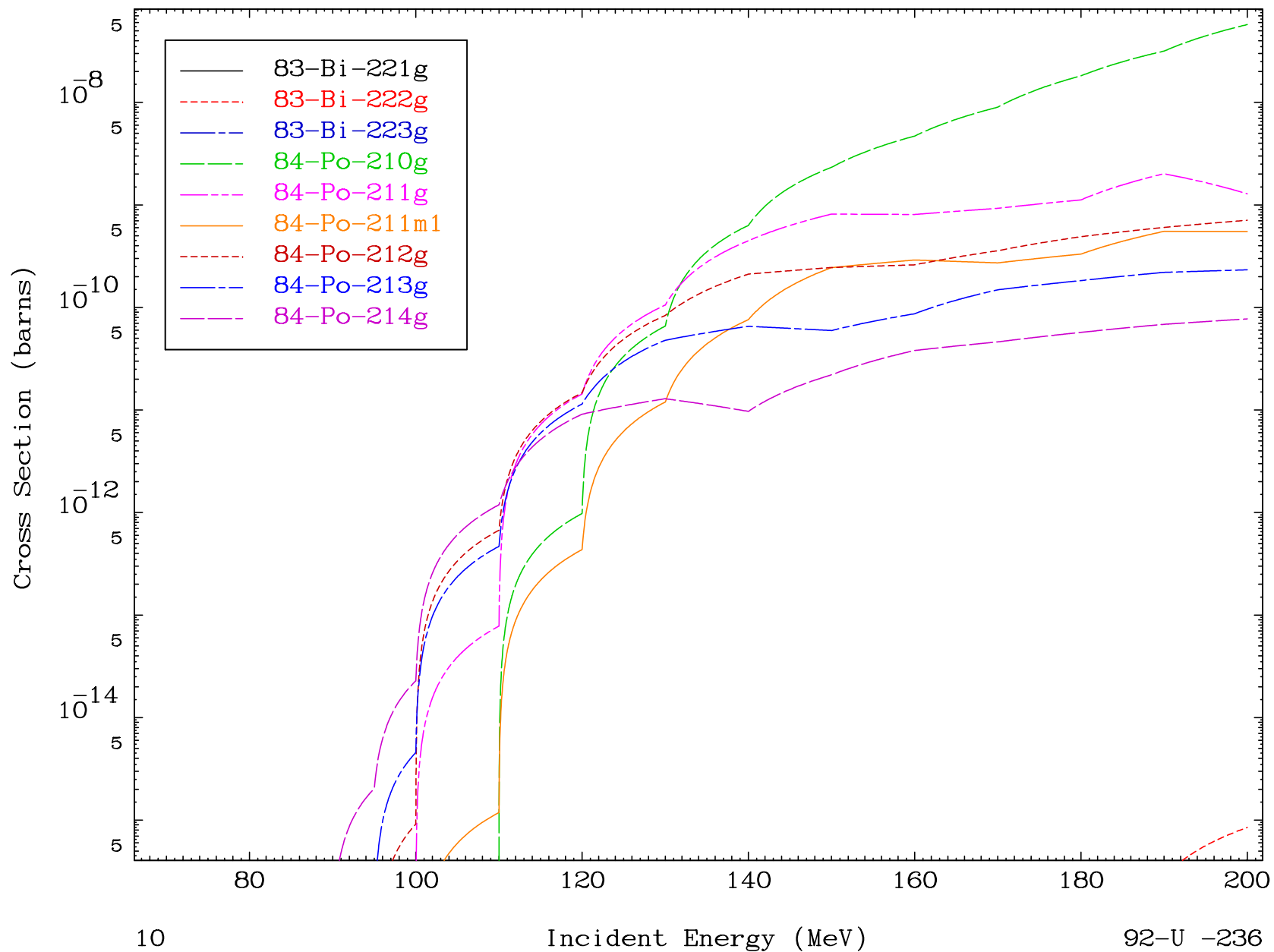


MAT 9231

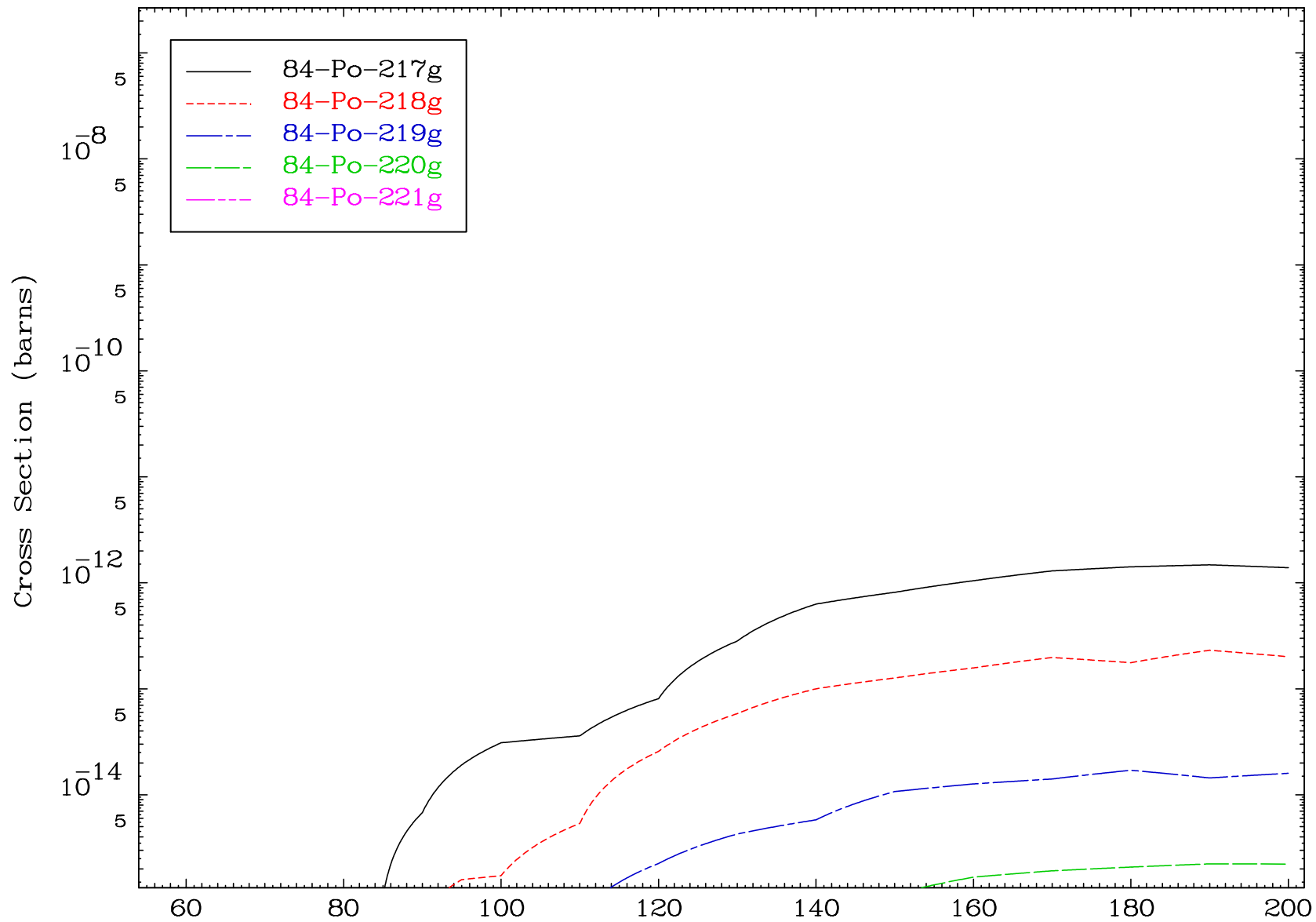
 $(\gamma, \text{remainder})$ 

92-U -236

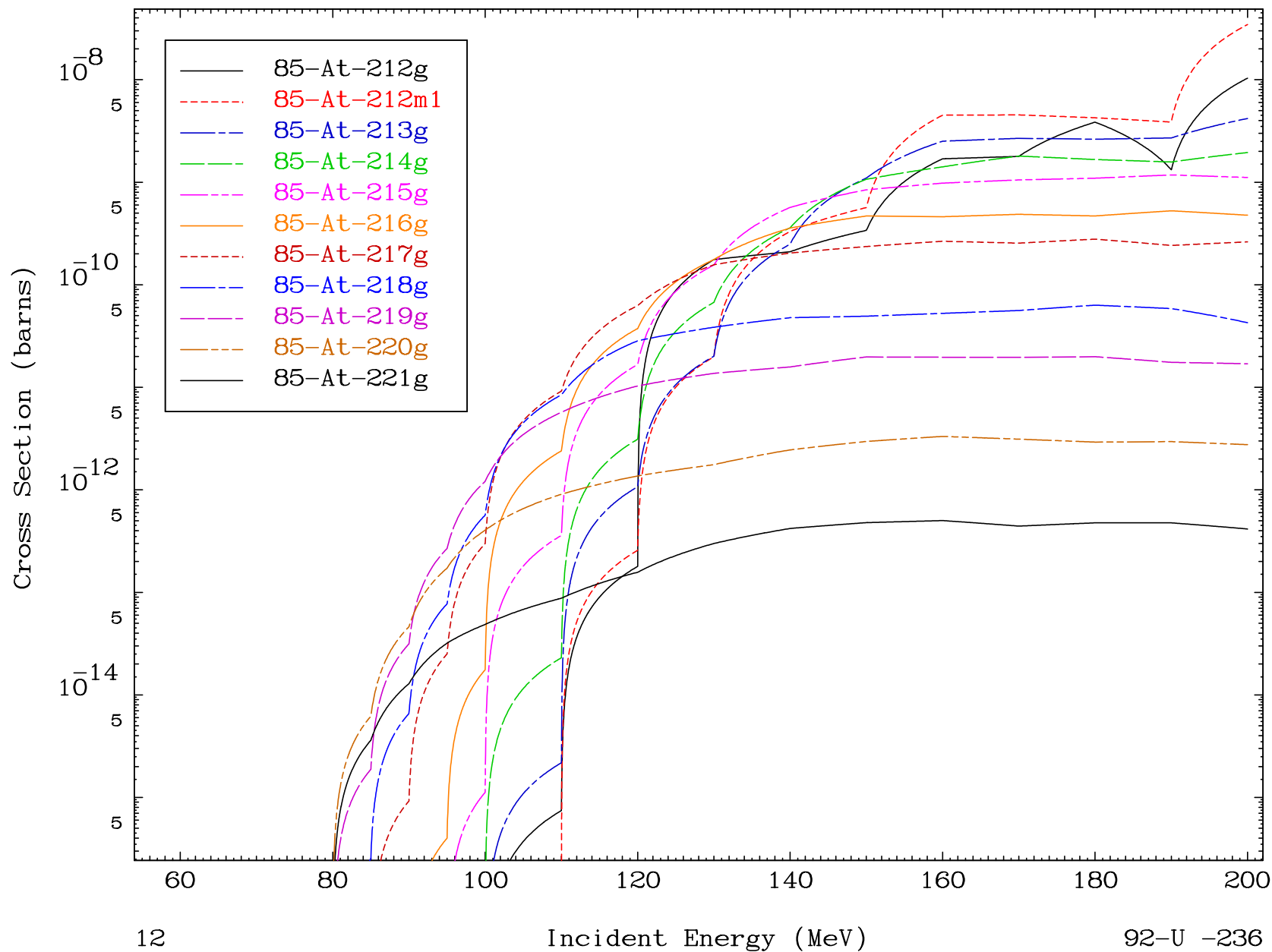
## Radionuclide Production Cross Section



## Radionuclide Production Cross Section



## Radionuclide Production Cross Section

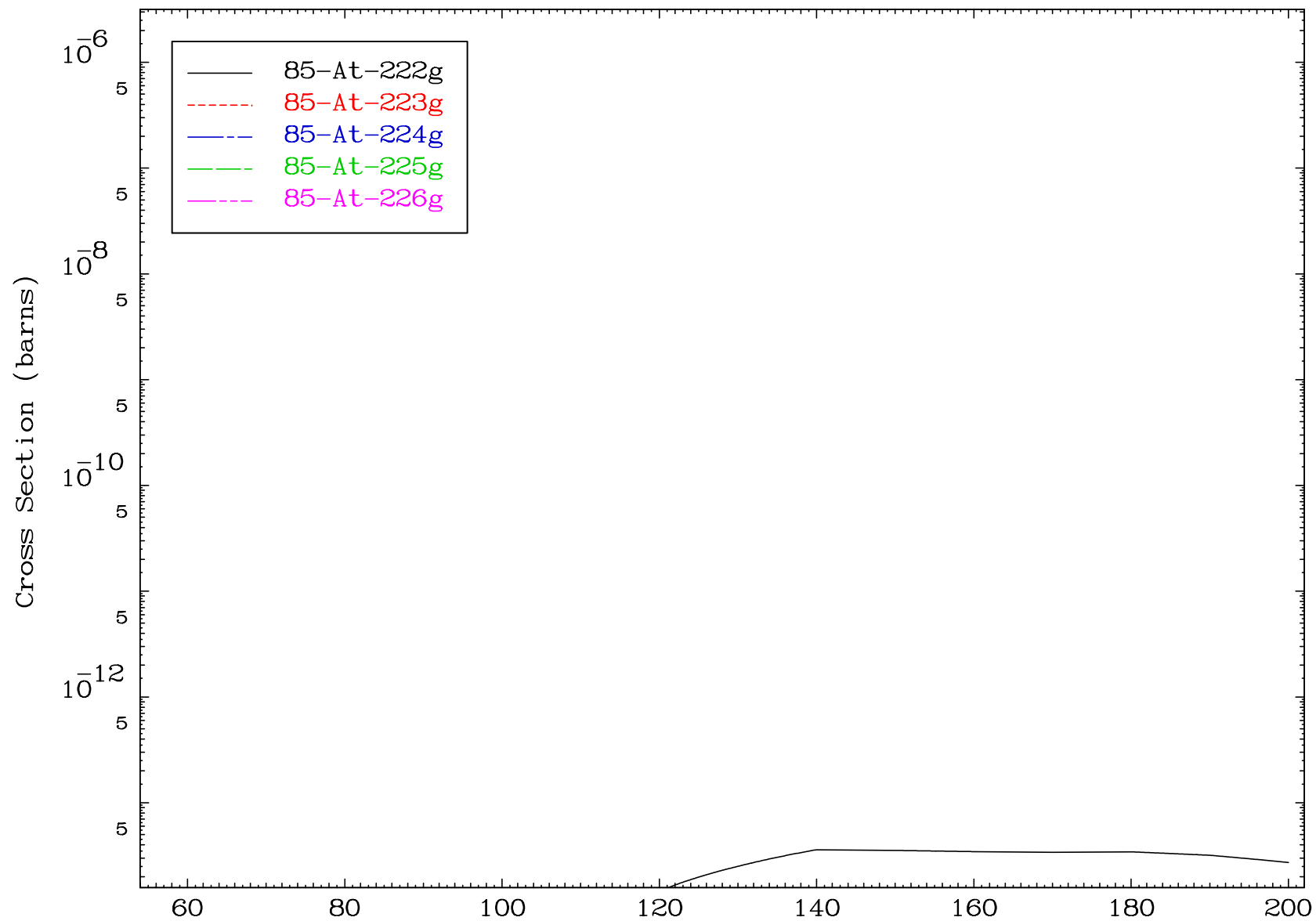


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section

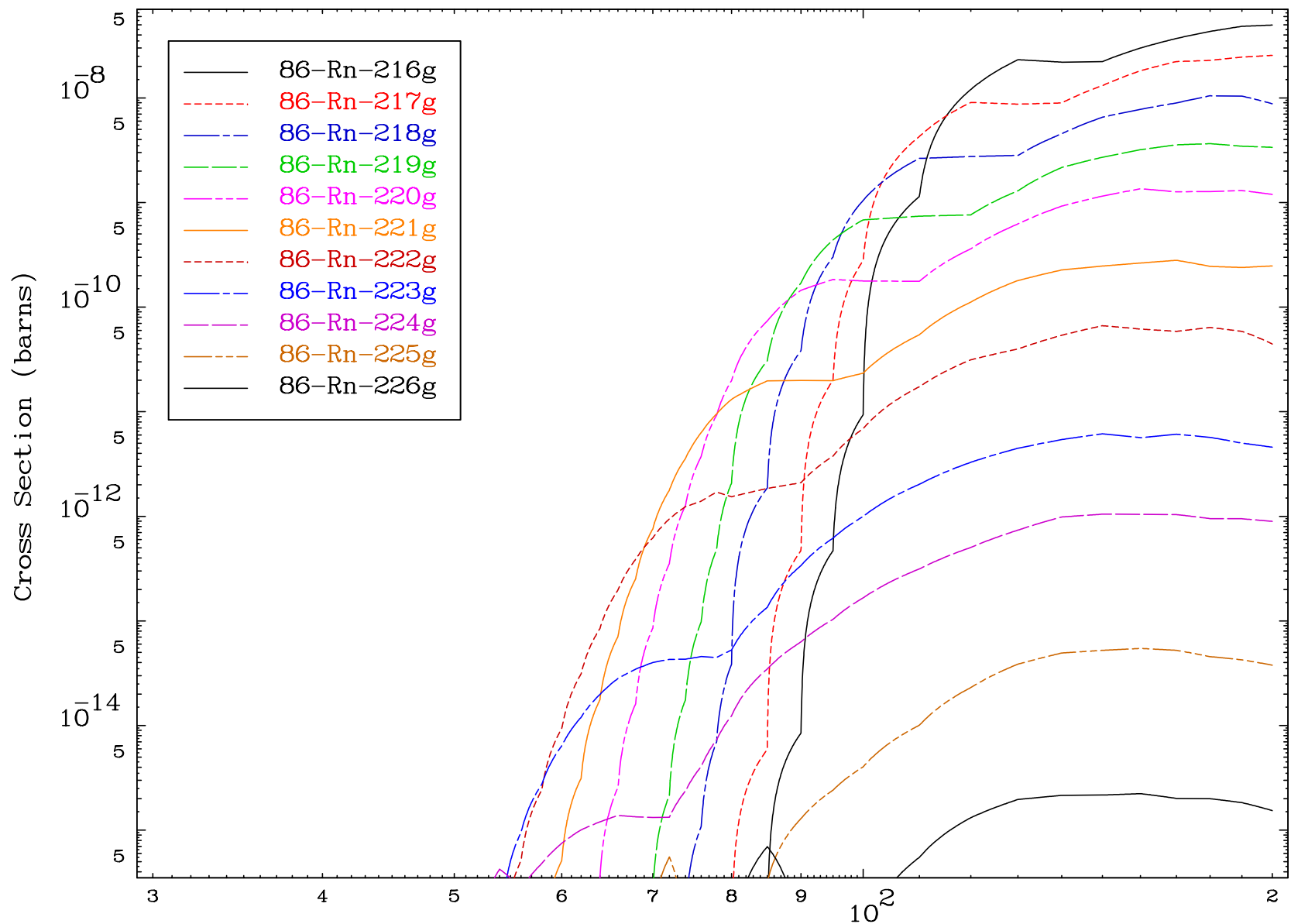


13

Incident Energy (MeV)

92-U -236

## Radionuclide Production Cross Section

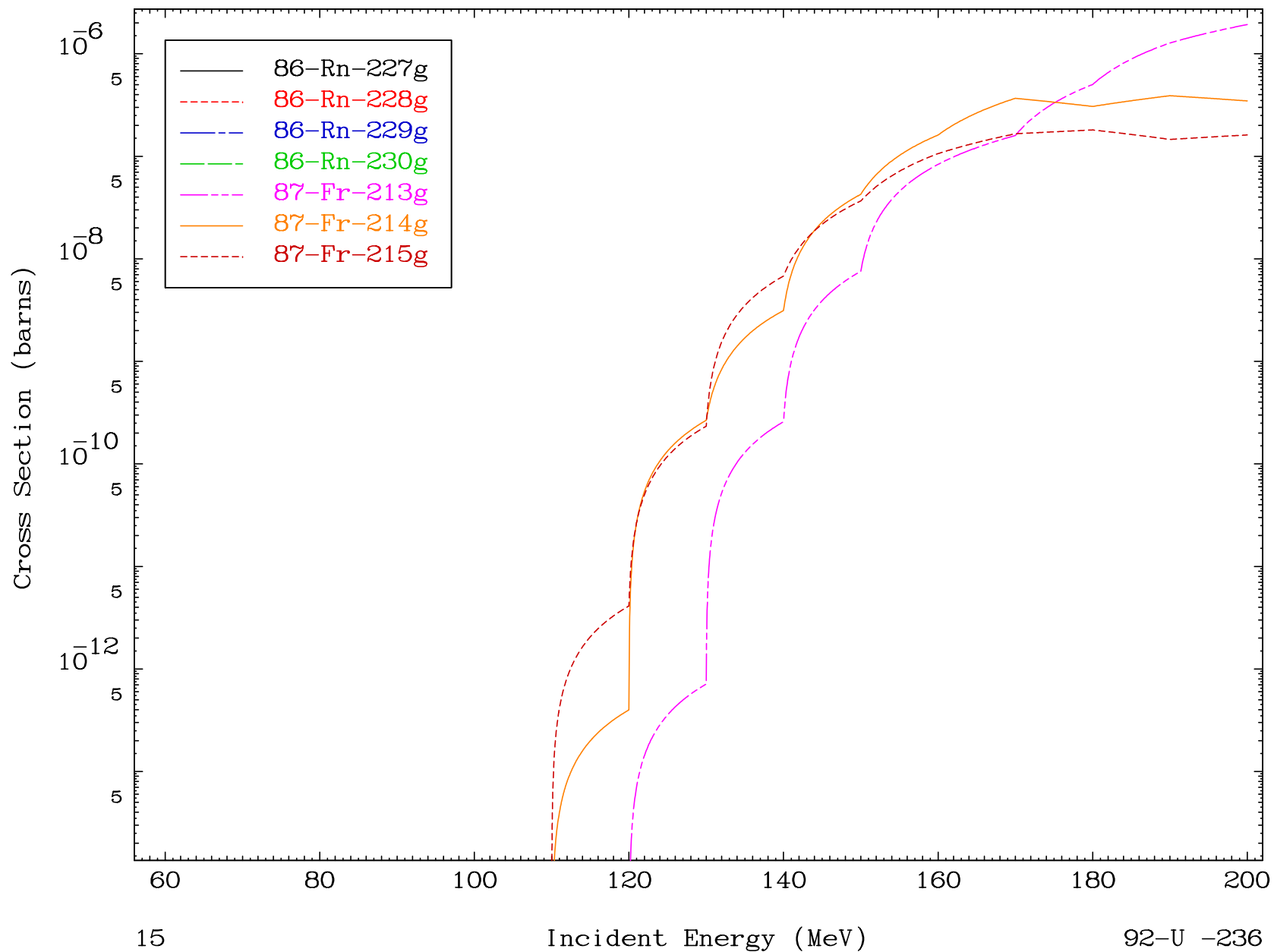


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section

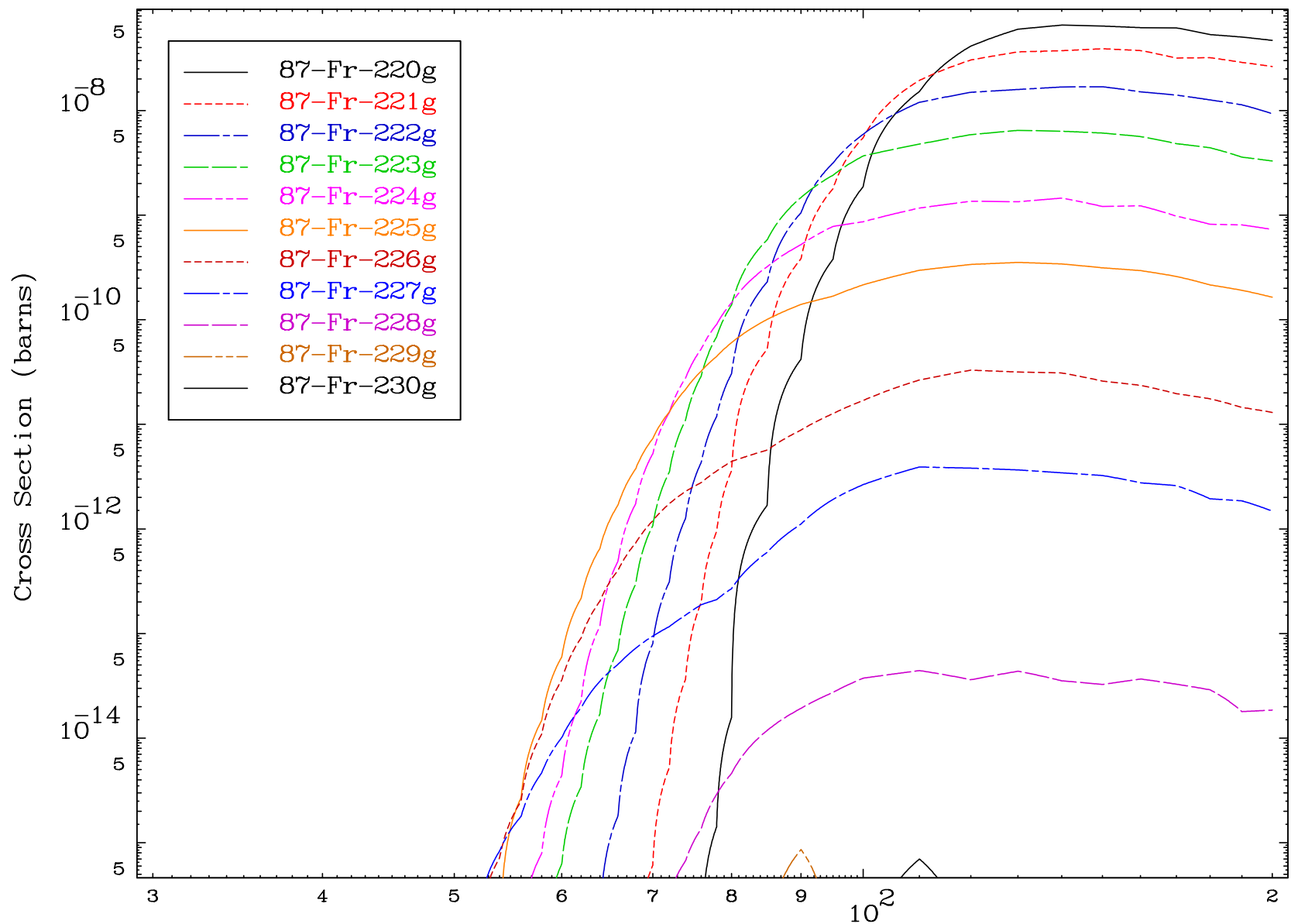


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section



16

Incident Energy (MeV)

92-U -236

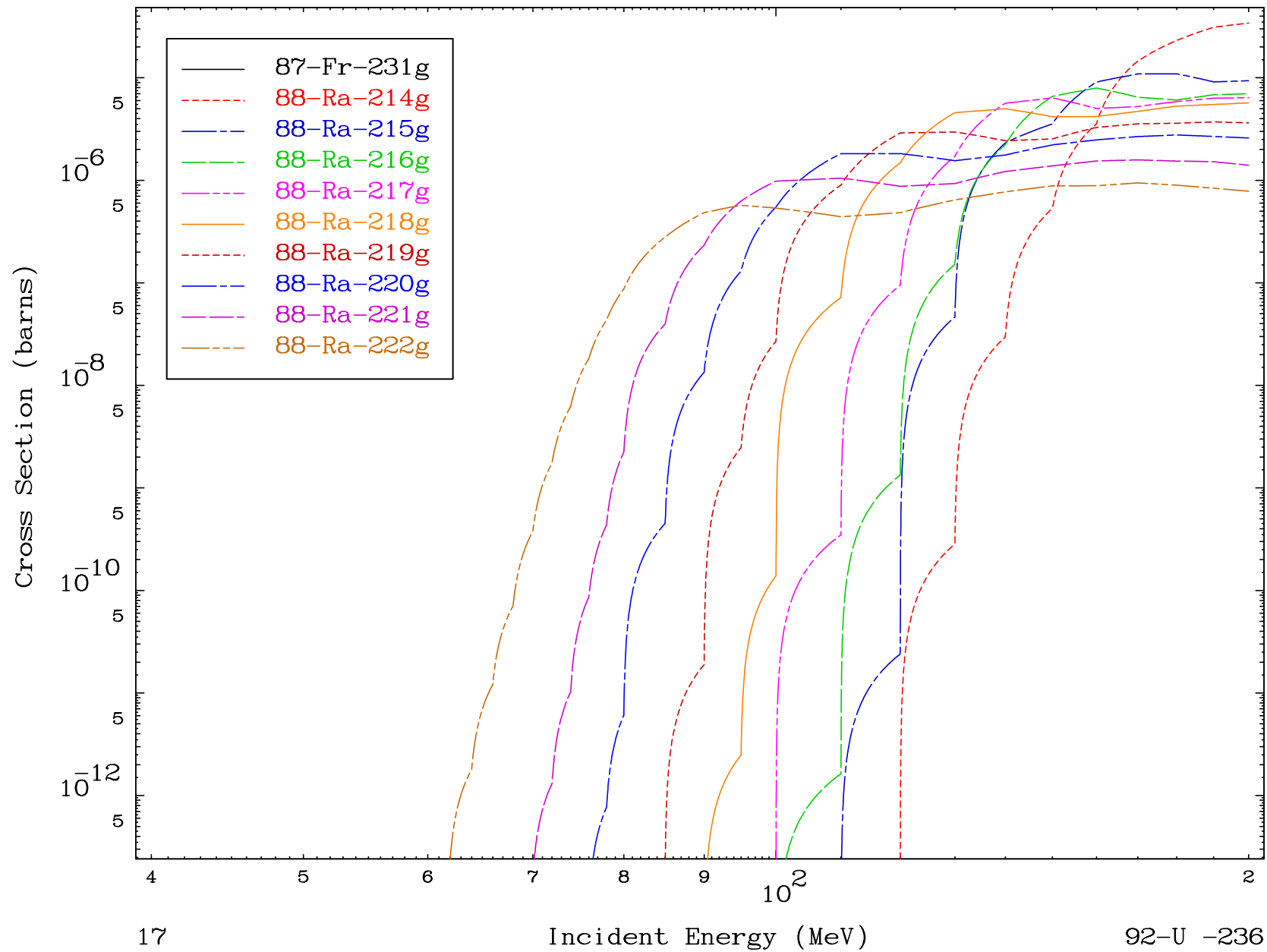


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section

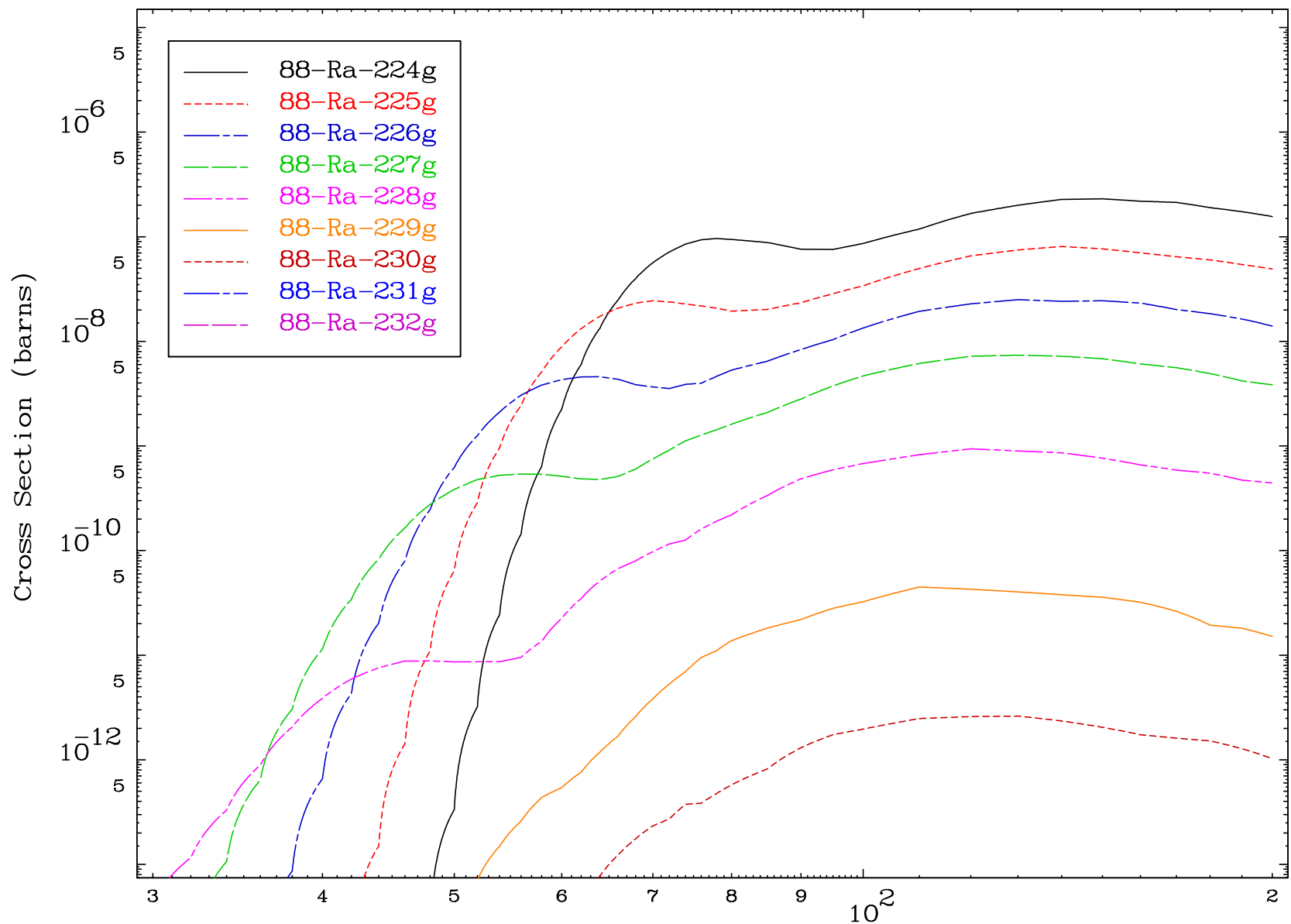


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section

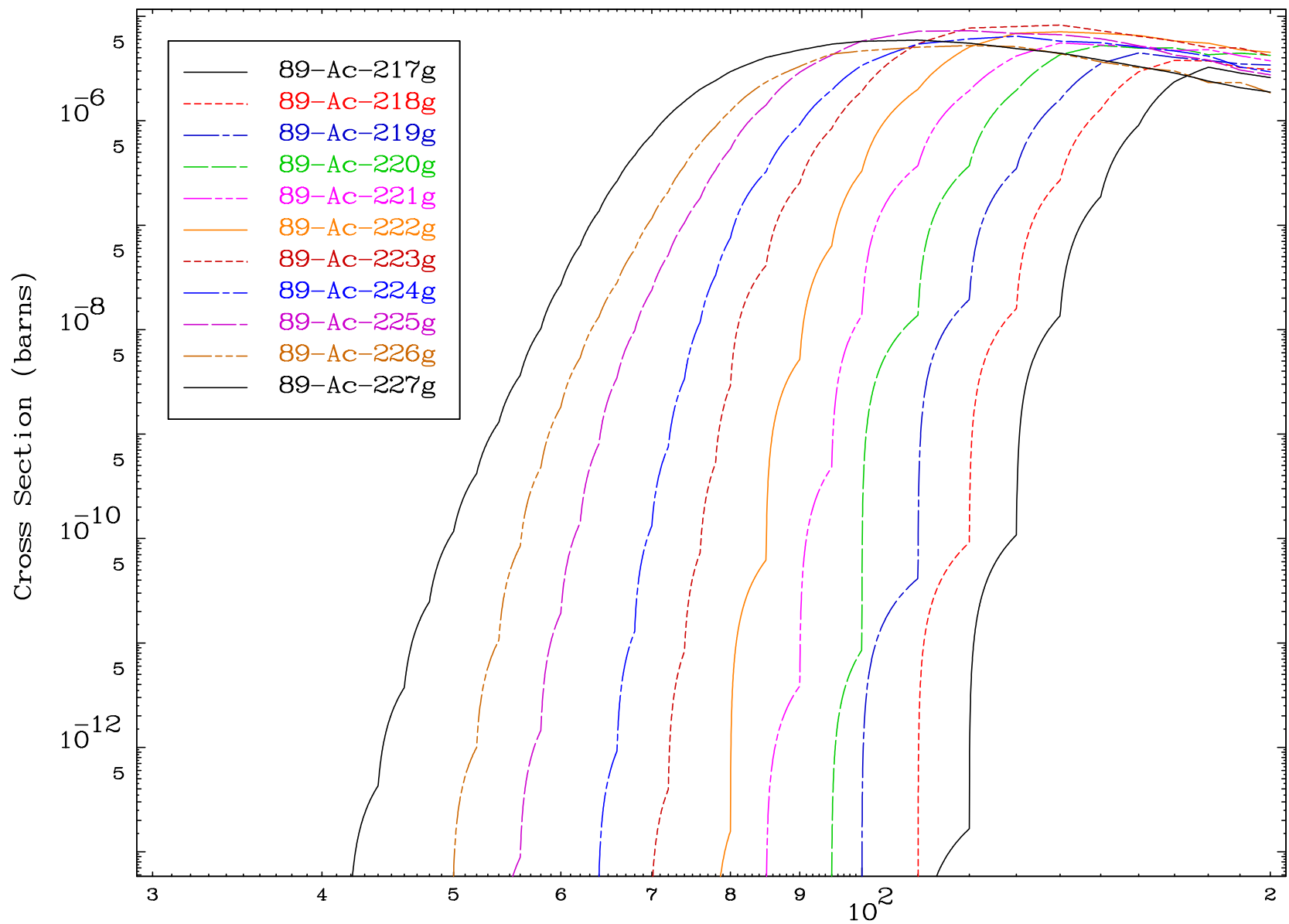


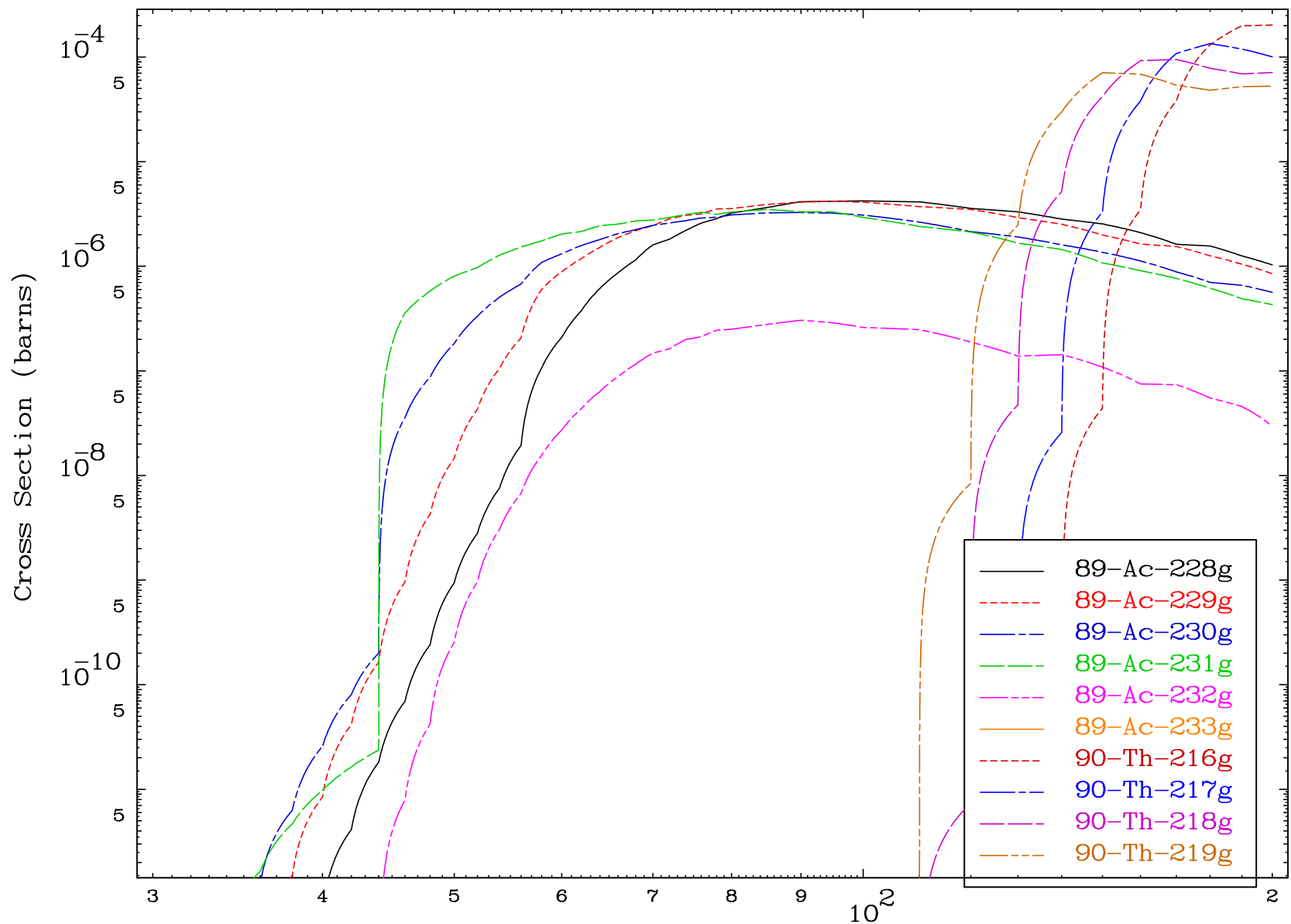
18

Incident Energy (MeV)

92-U -236

## Radionuclide Production Cross Section



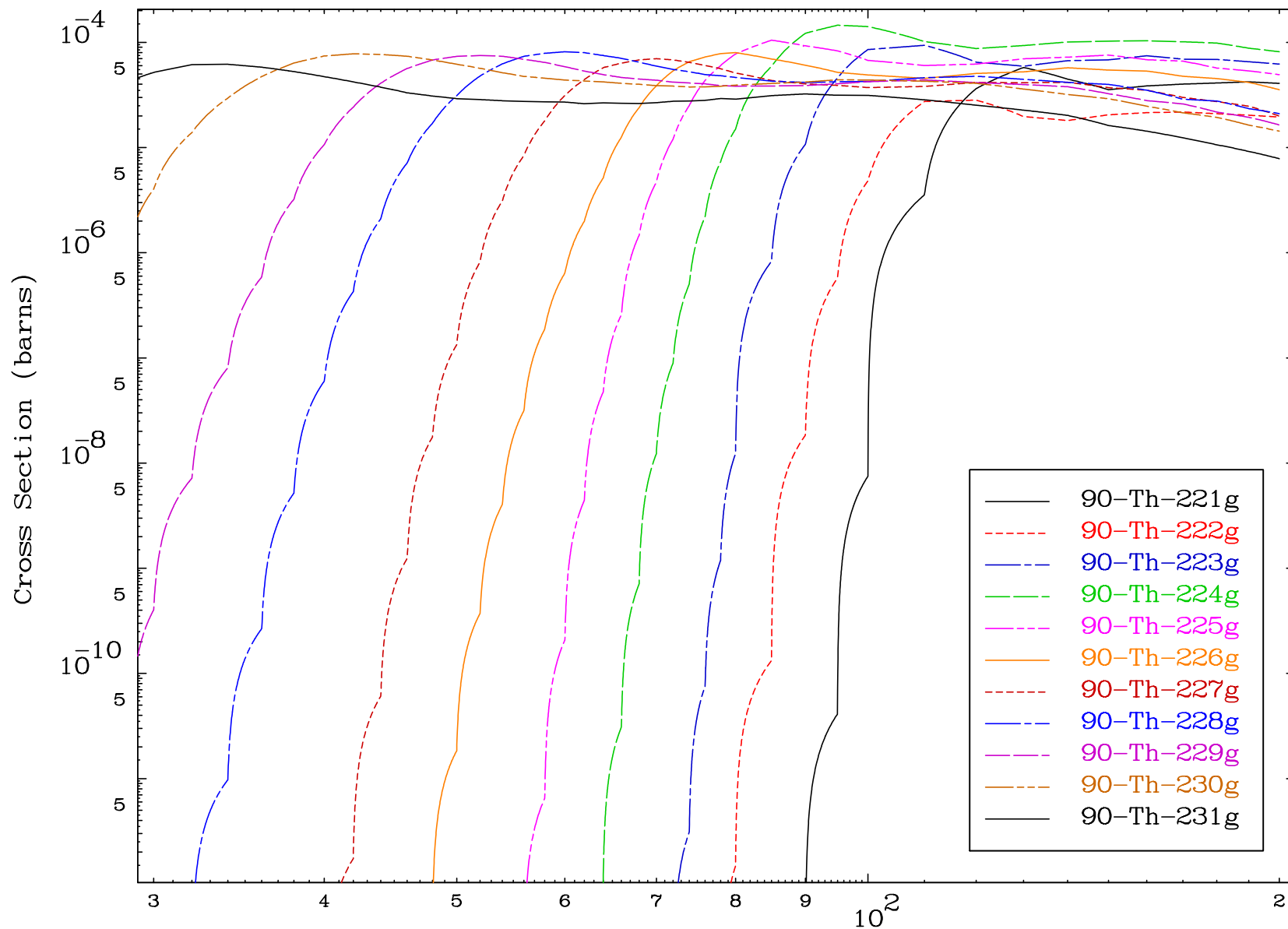


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section



21

Incident Energy (MeV)

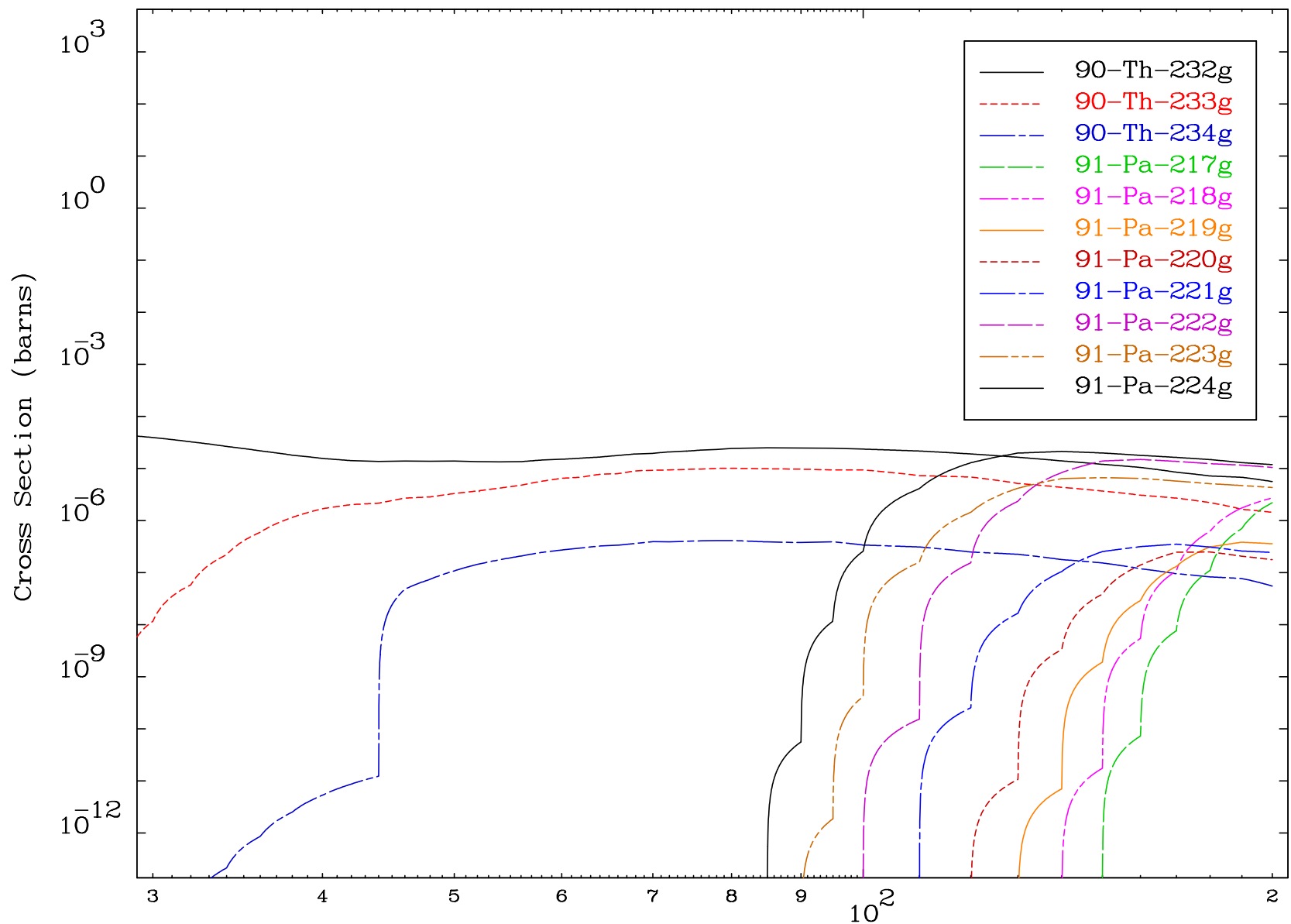
92-U -236

MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section



22

Incident Energy (MeV)

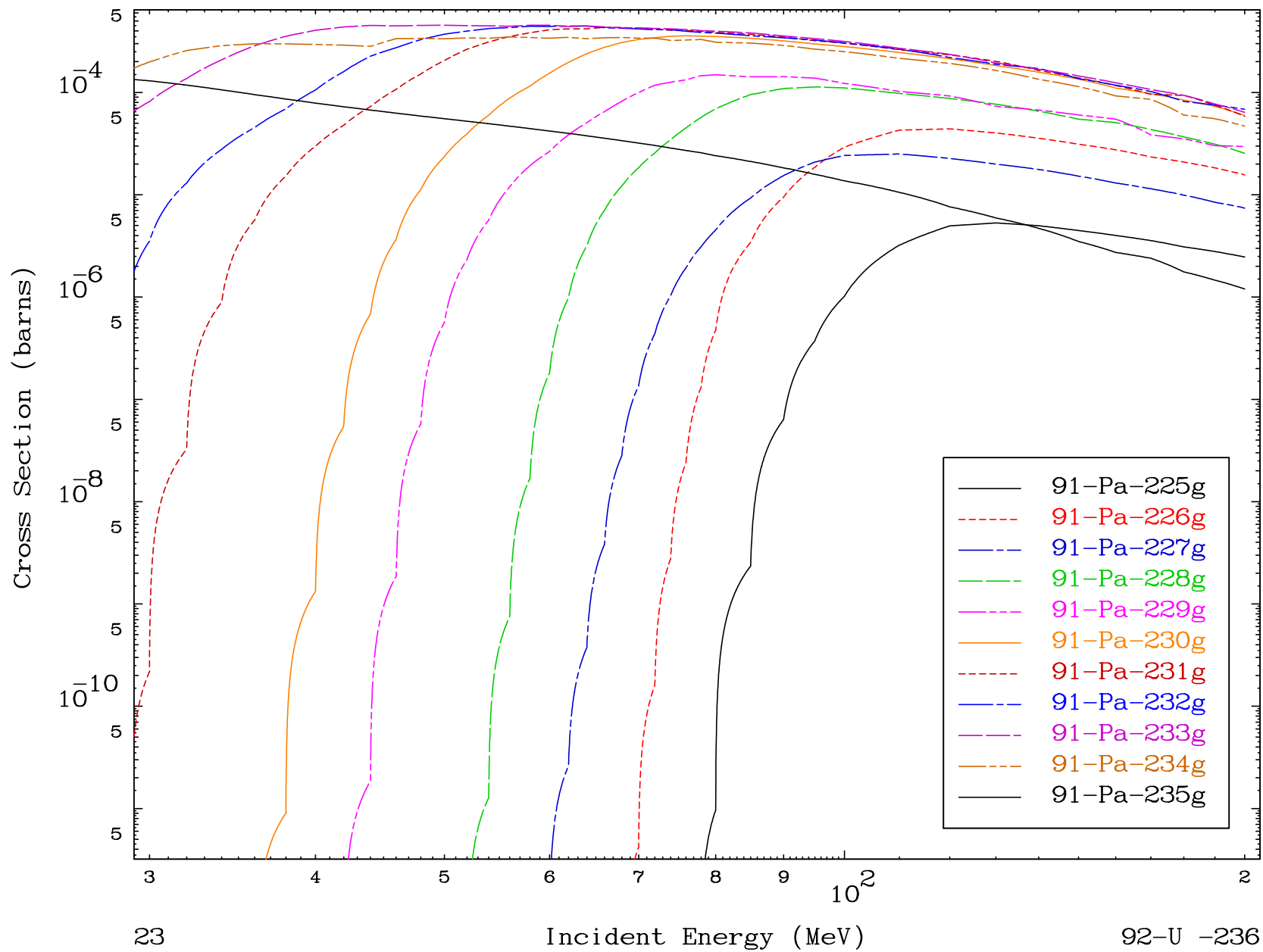
92-U -236

MAT 9231

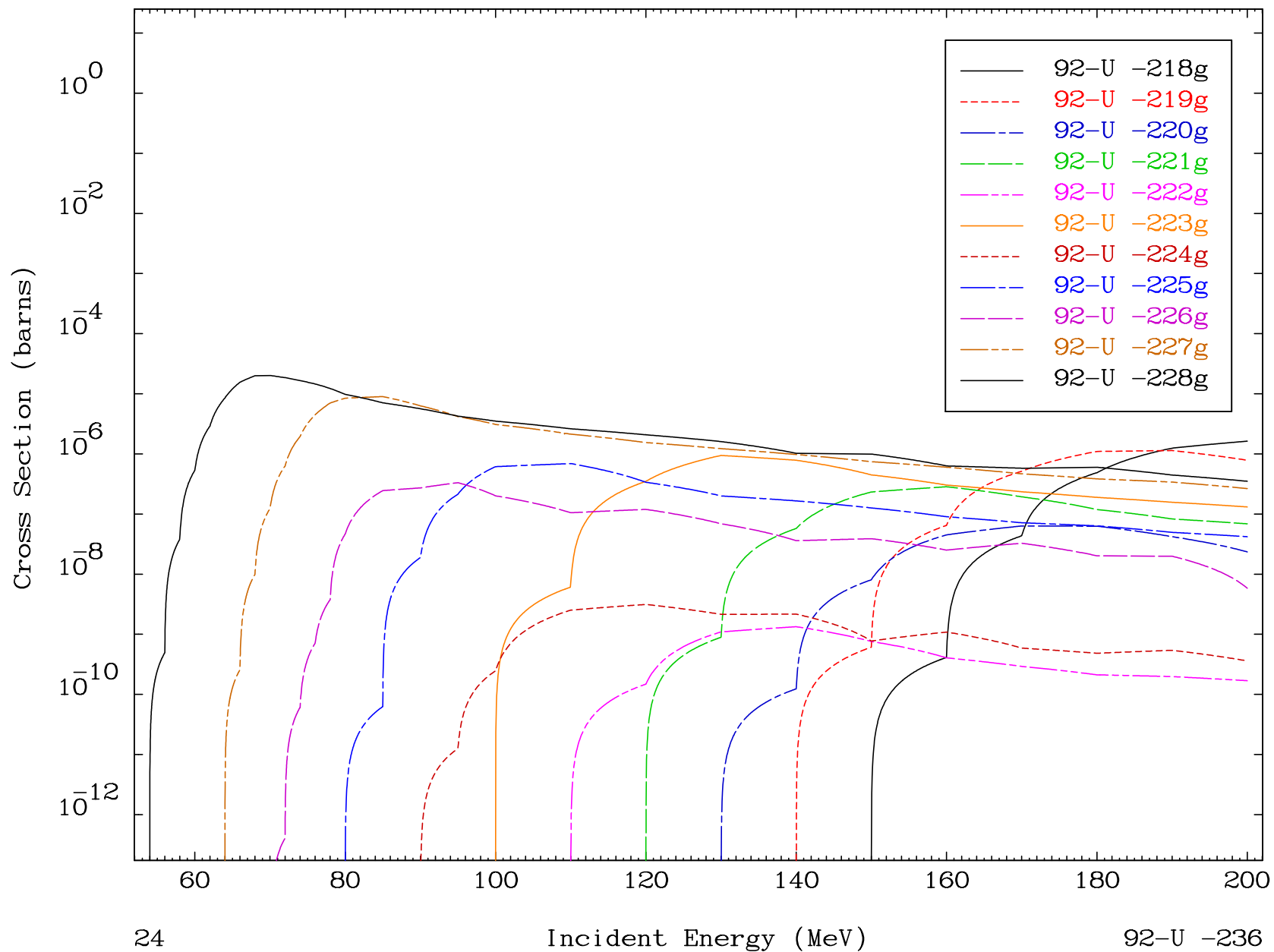
 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section



## Radionuclide Production Cross Section



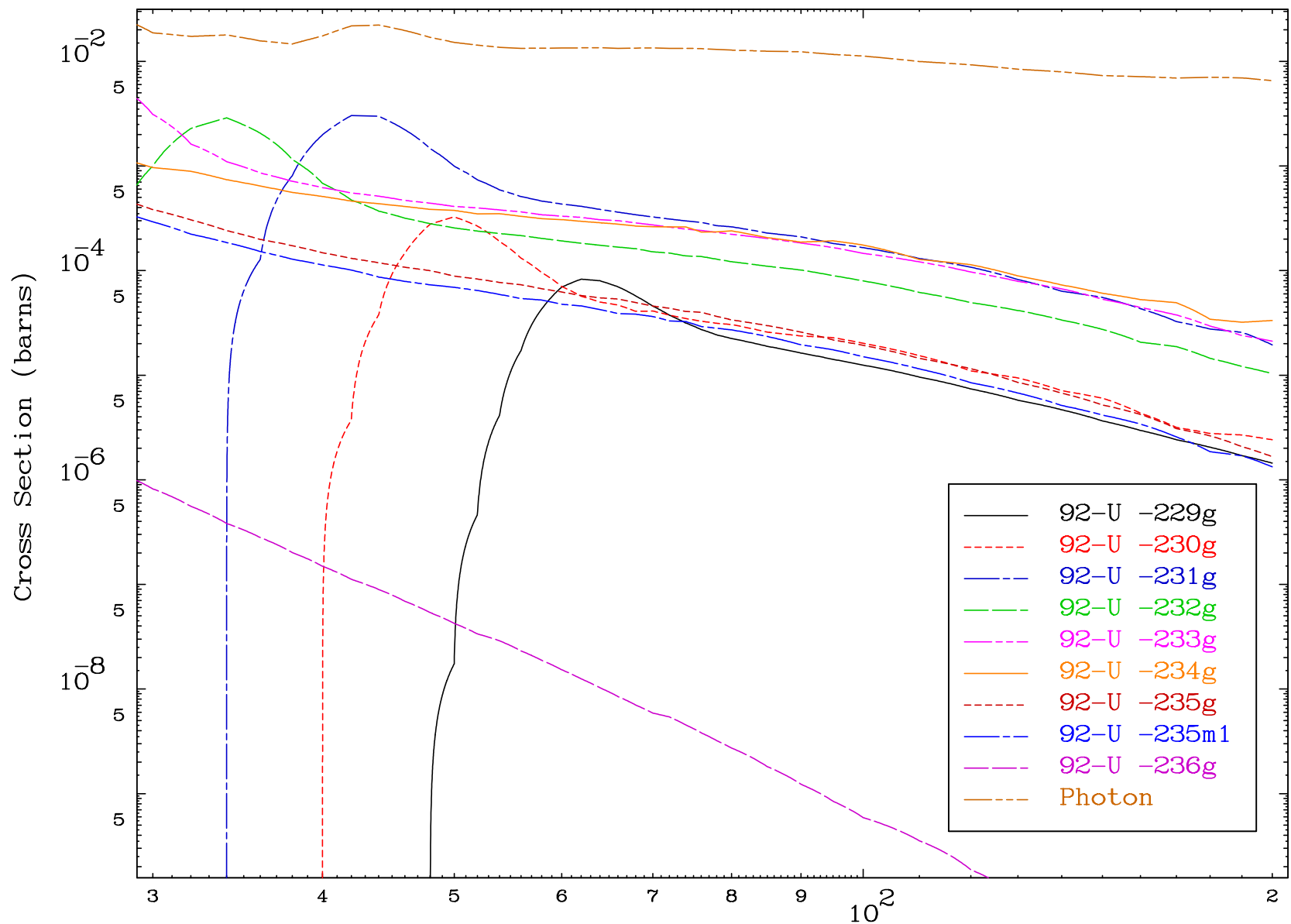


MAT 9231

 $(\gamma, \text{remainder})$ 

92-U -236

## Radionuclide Production Cross Section



25

Incident Energy (MeV)

92-U -236