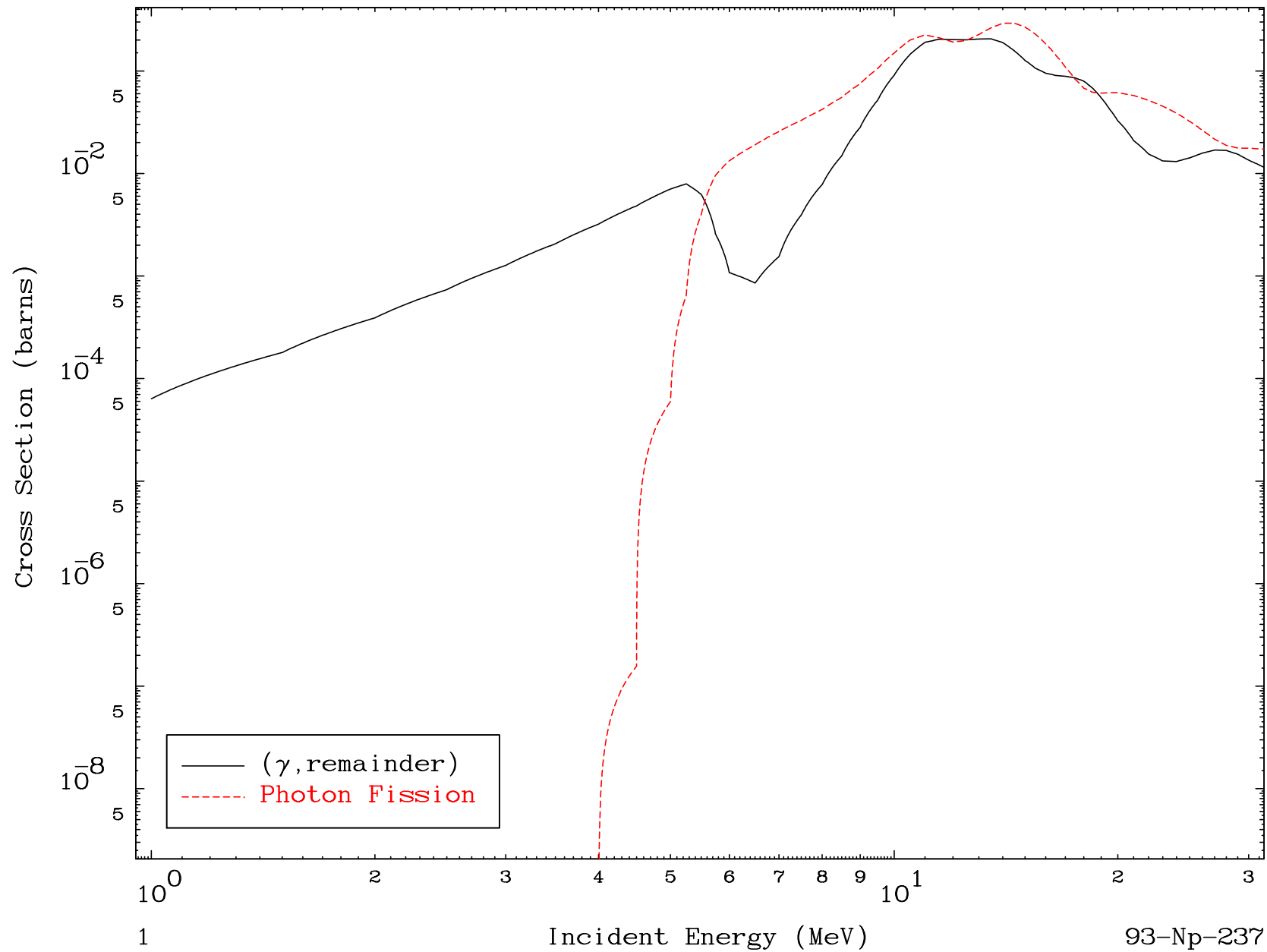


MAT 9346

Photon Major  
0 Kelvin Cross Sections

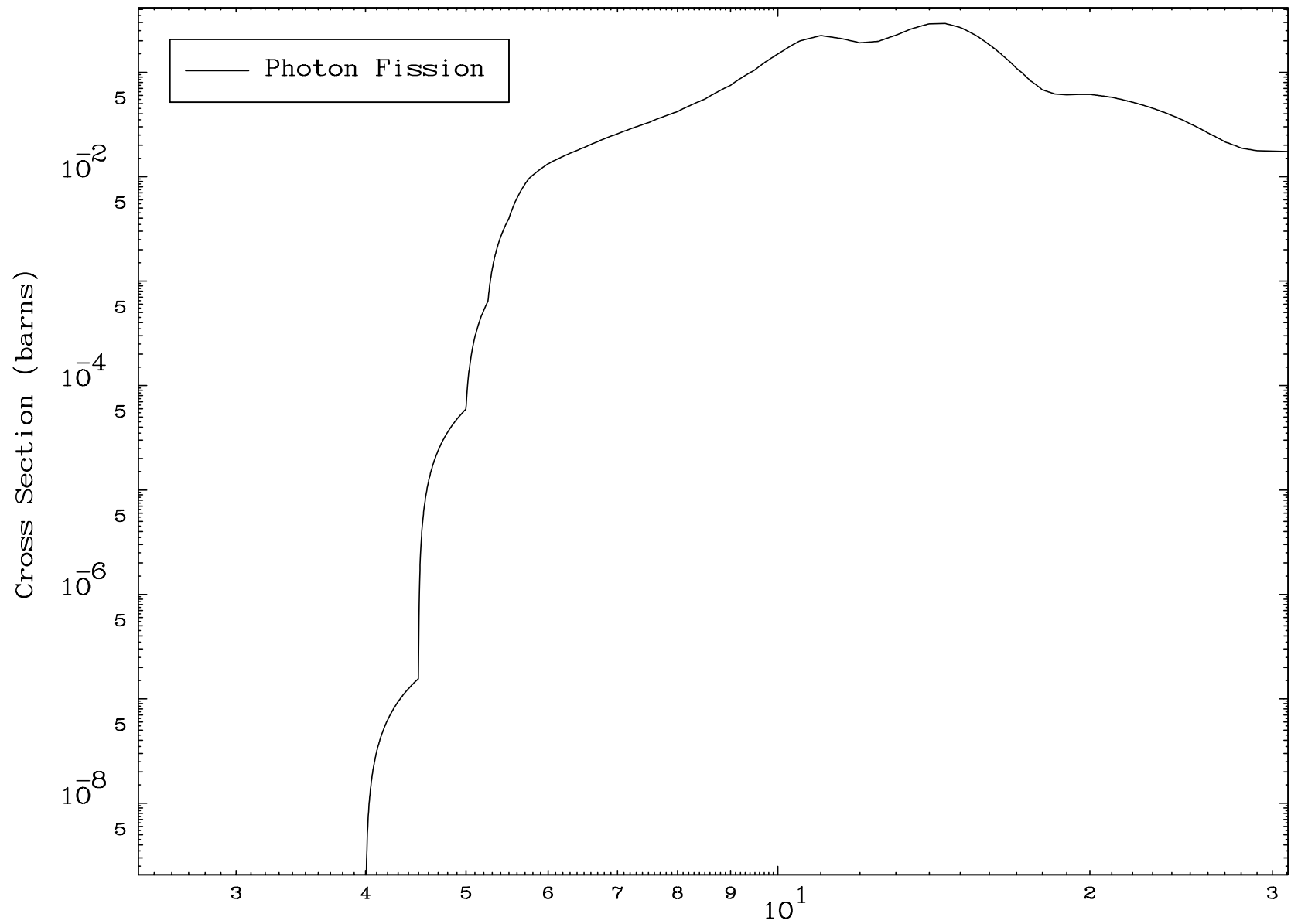
93-Np-237



MAT 9346

Photon Fission  
0 Kelvin Cross Sections

93-Np-237



2

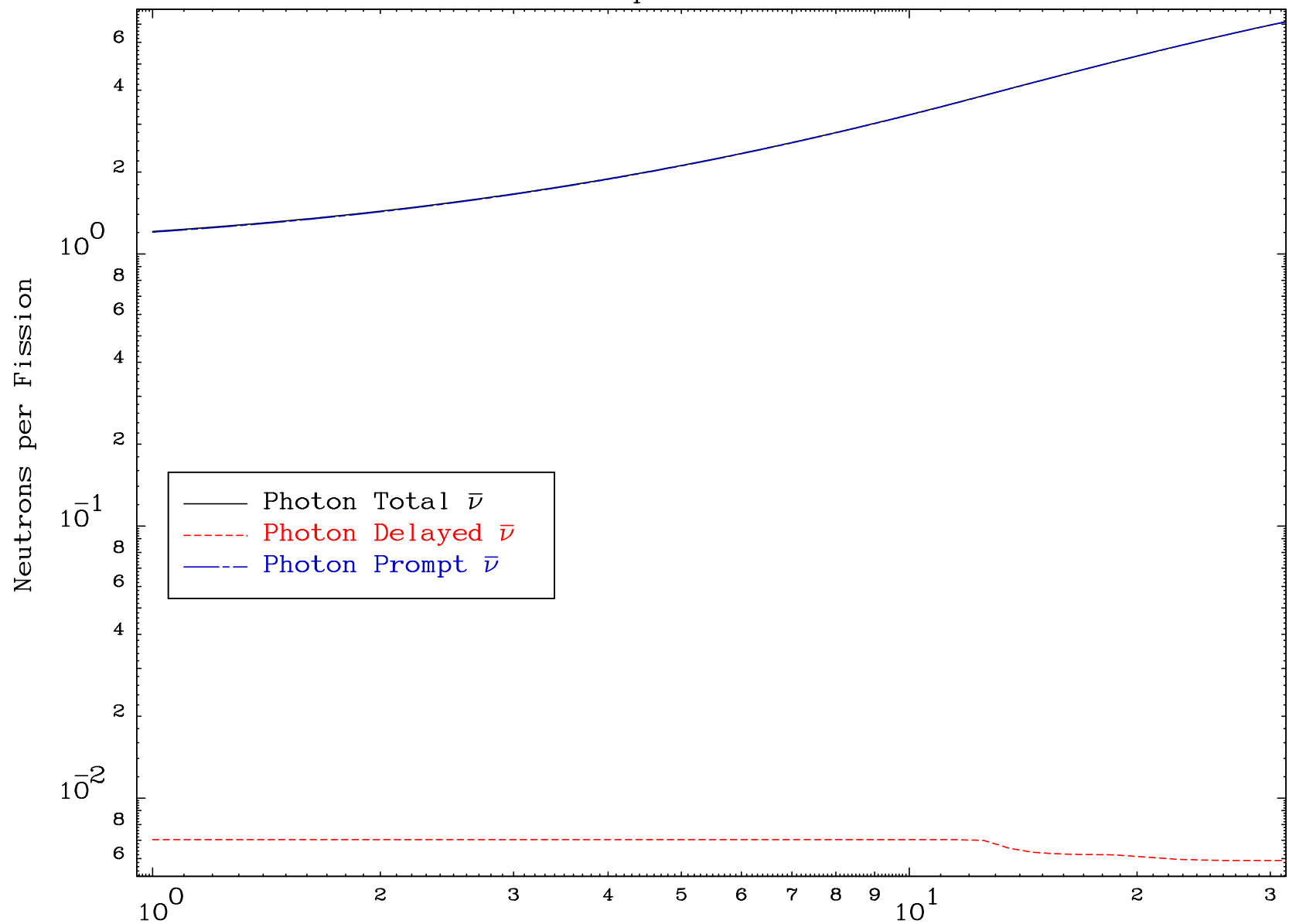
Incident Energy (MeV)

93-Np-237

MAT 9346

Photon Neutrons  
per Fission

93-Np-237



3

 $E \cdot \sin(\theta/2) \text{ (MeV)}$ 

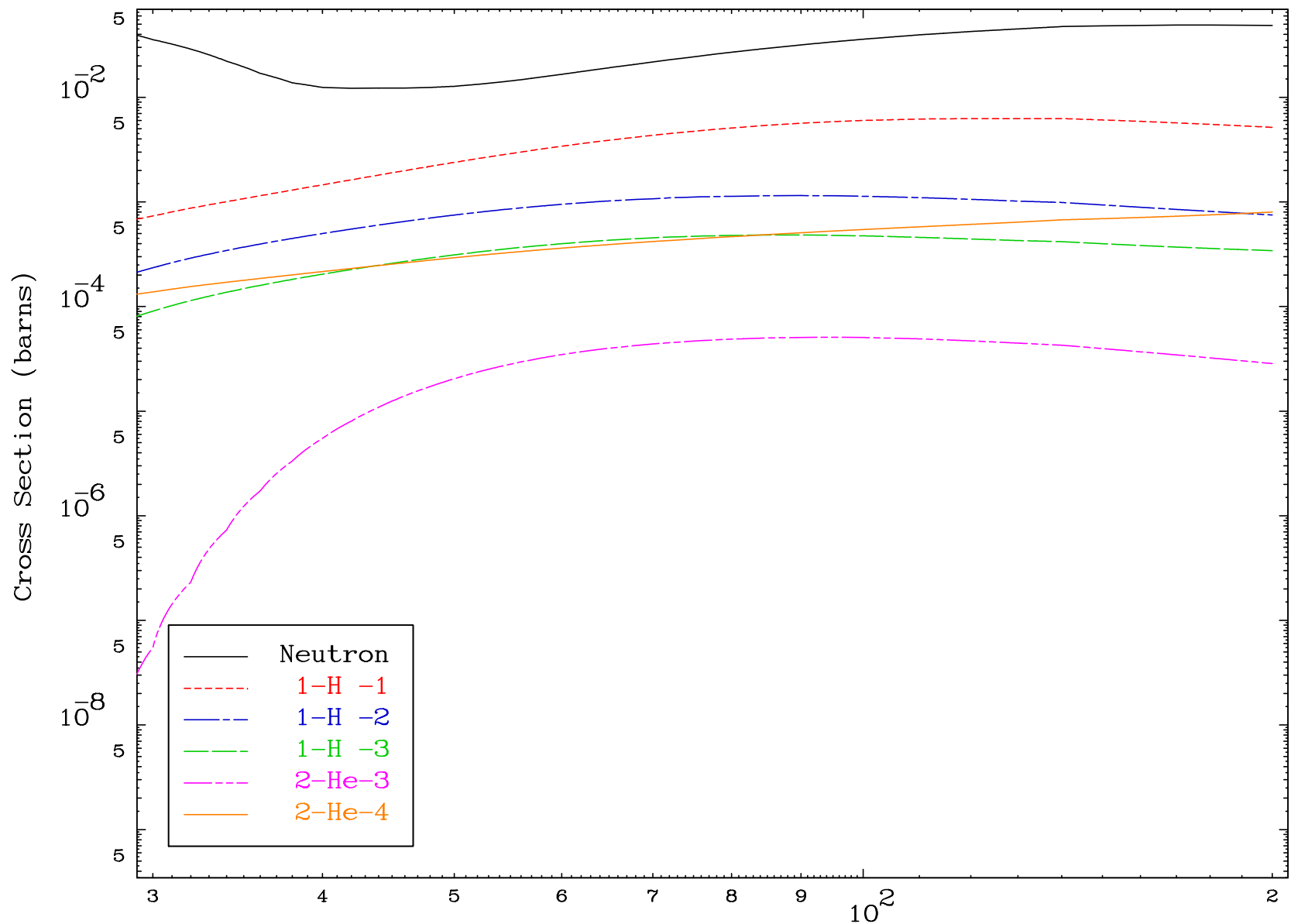
93-Np-237

MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

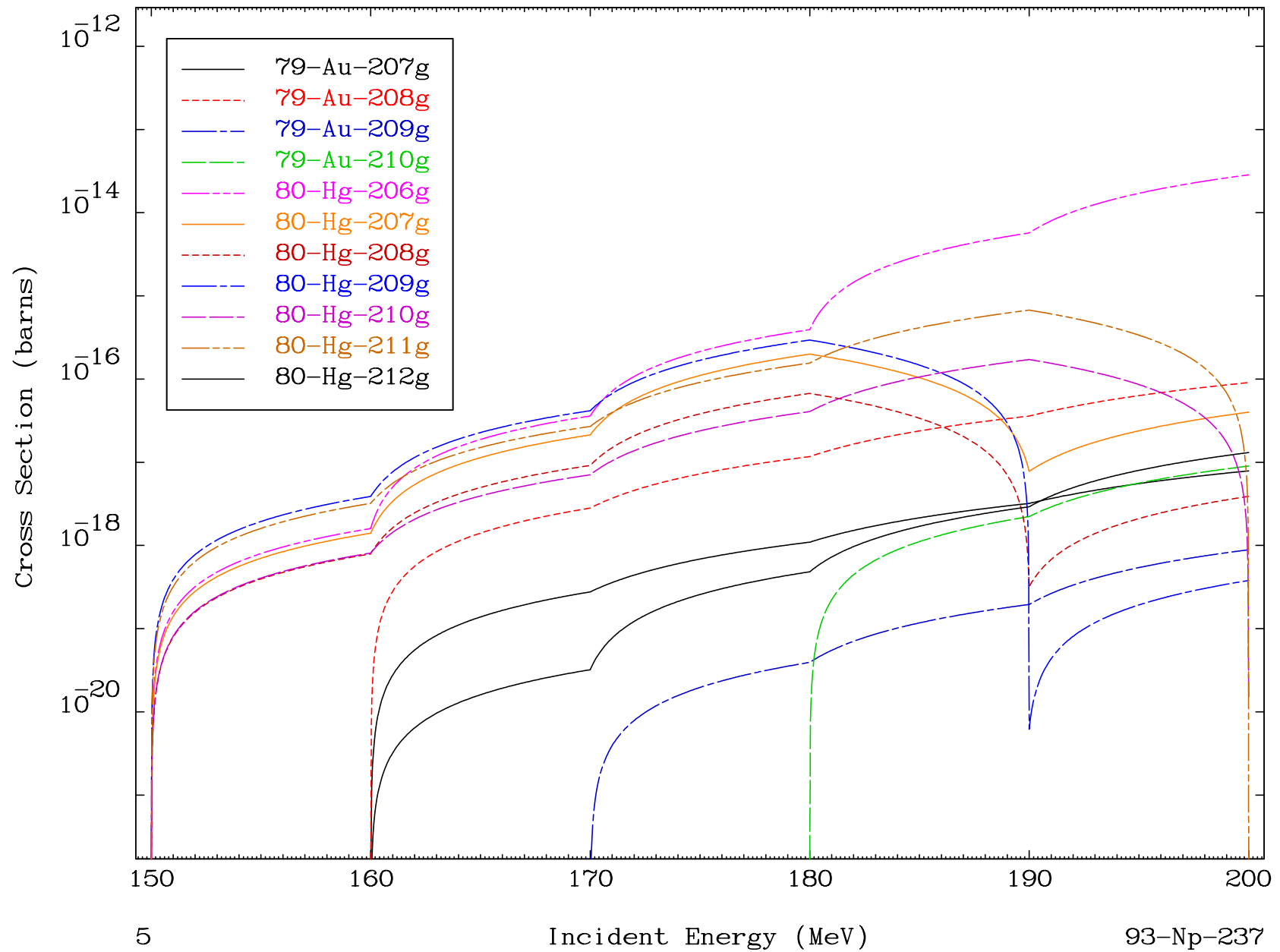
## Radionuclide Production Cross Section

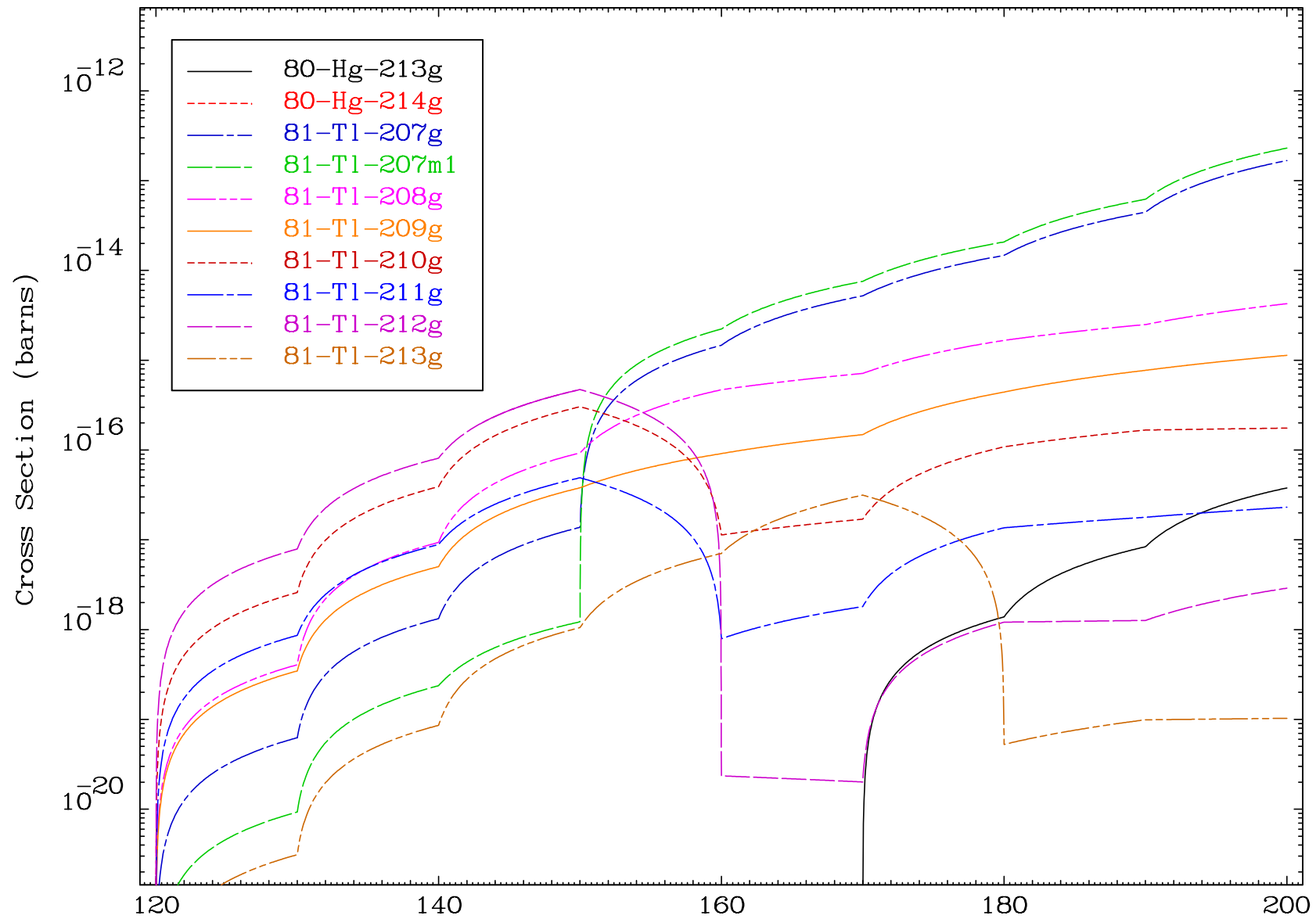


4

Incident Energy (MeV)

93-Np-237



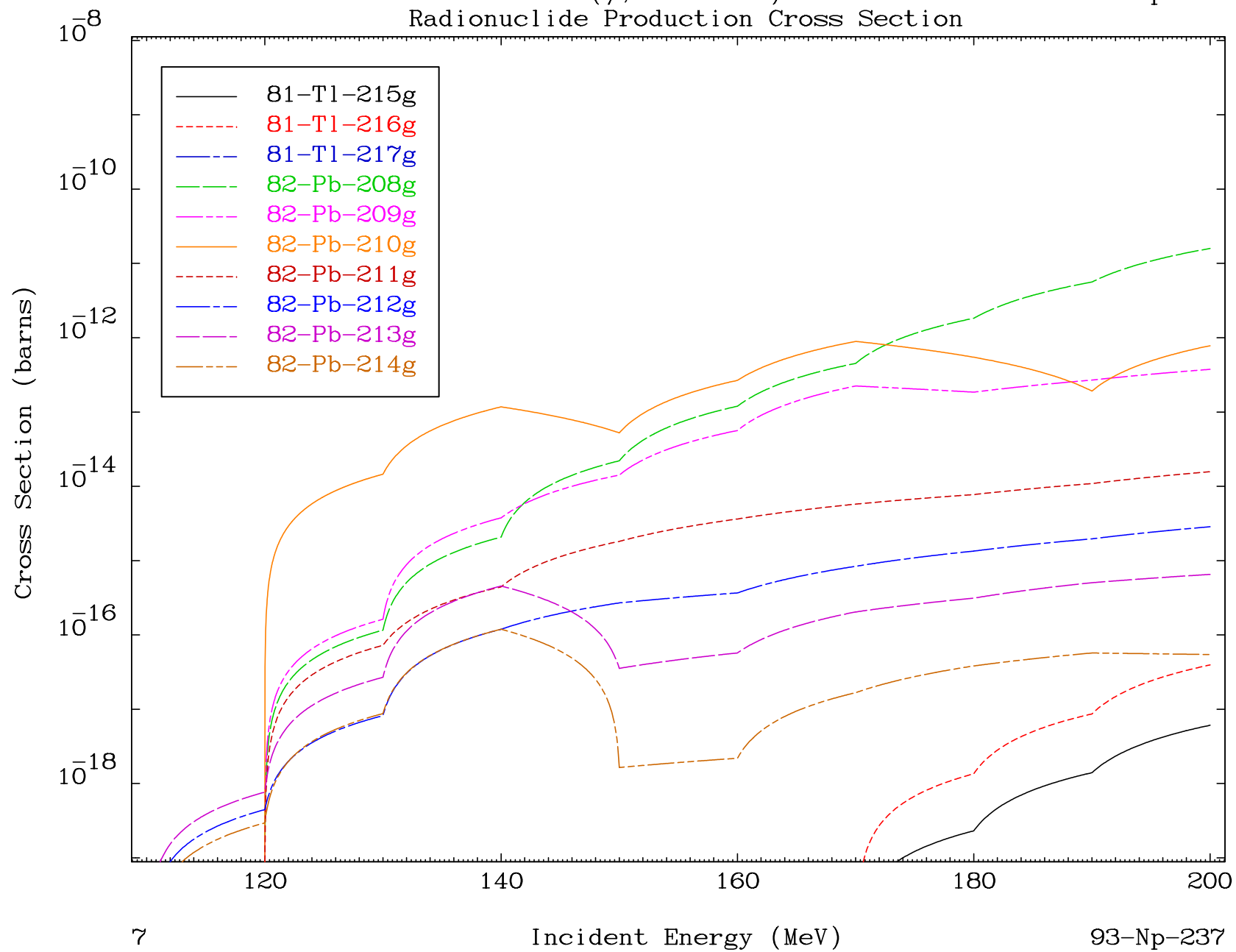


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section

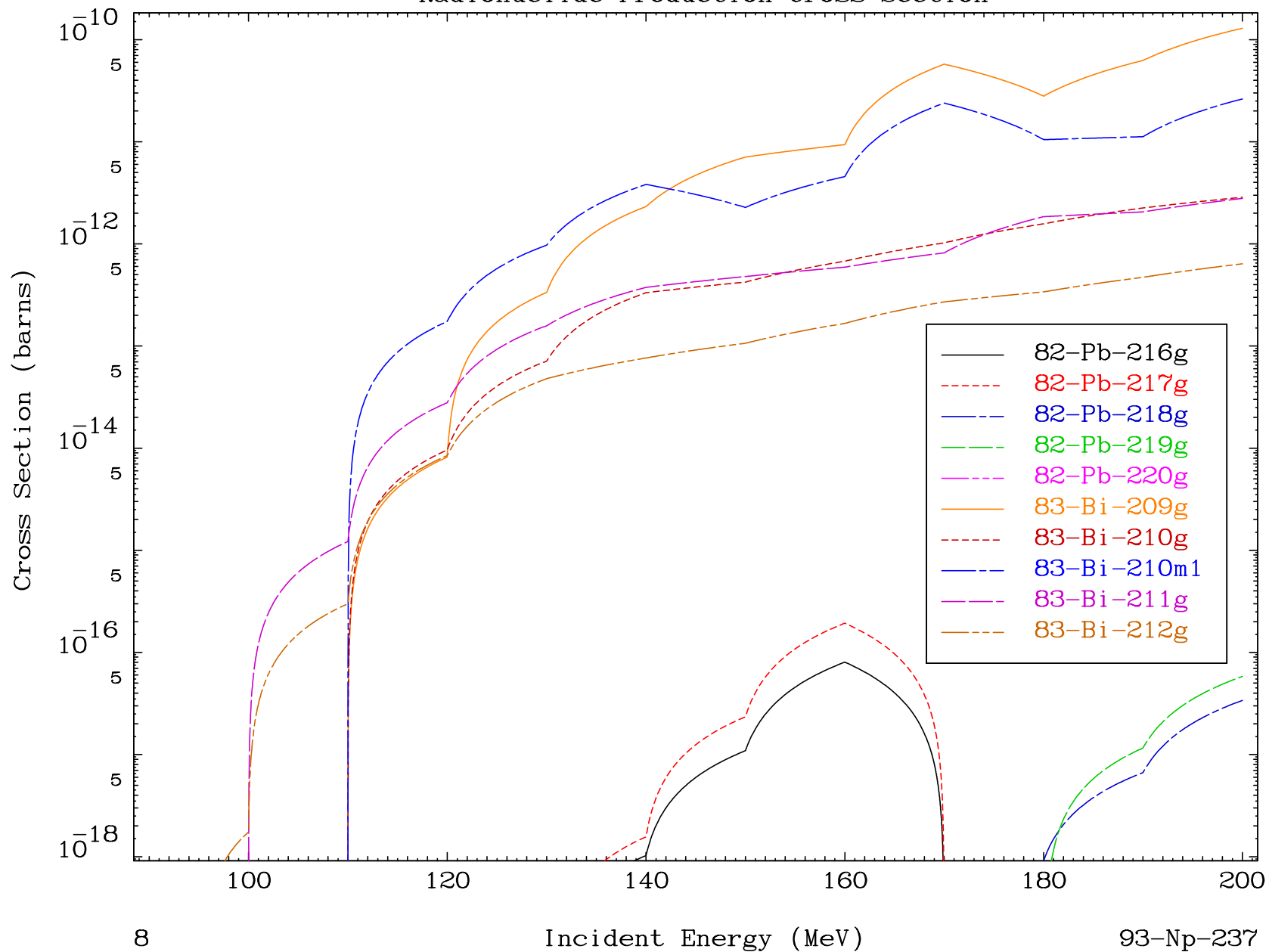


MAT 9346

 $(\gamma, \text{remainder})$ 

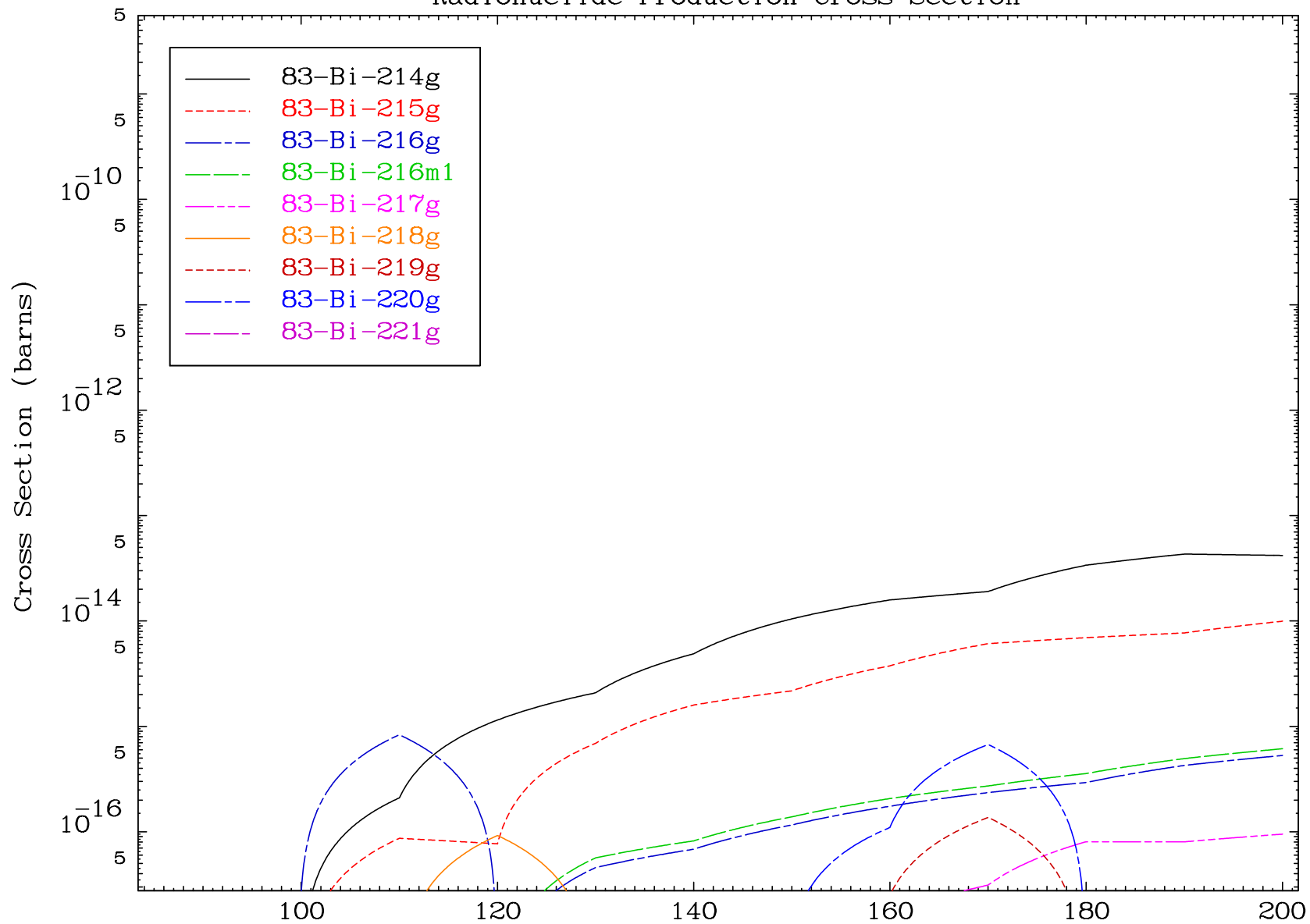
93-Np-237

## Radionuclide Production Cross Section





## Radionuclide Production Cross Section

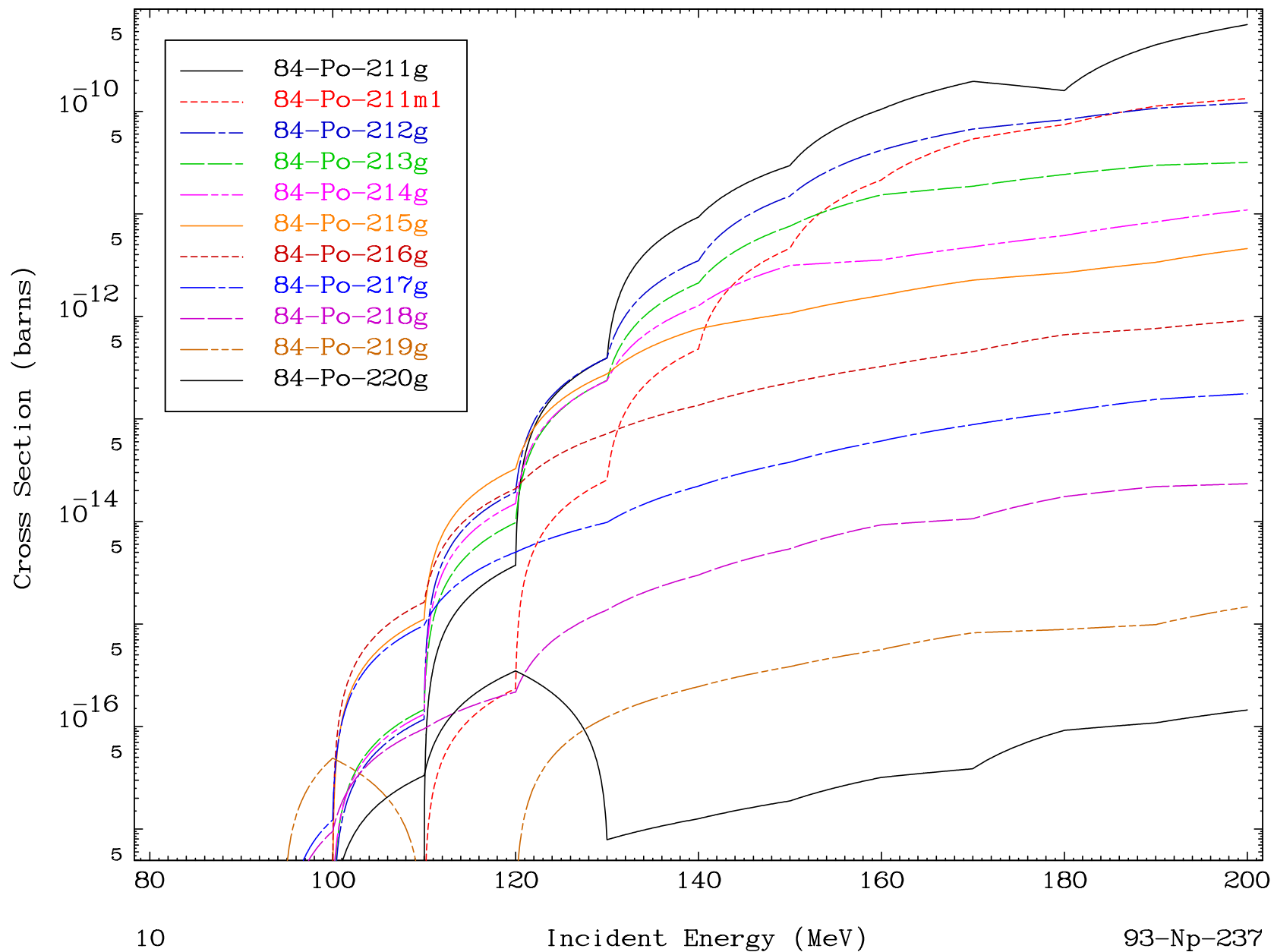


MAT 9346

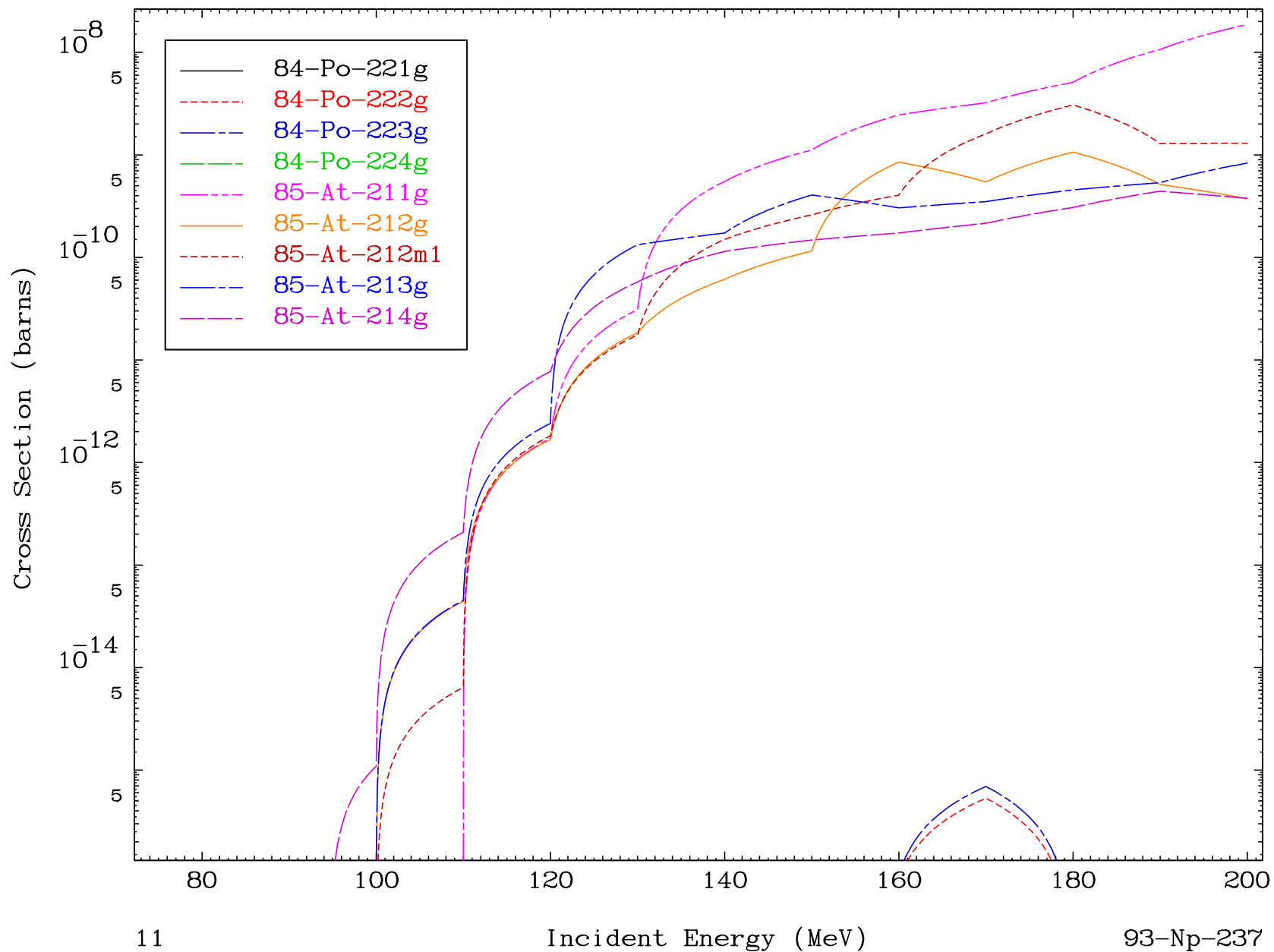
 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



## Radionuclide Production Cross Section

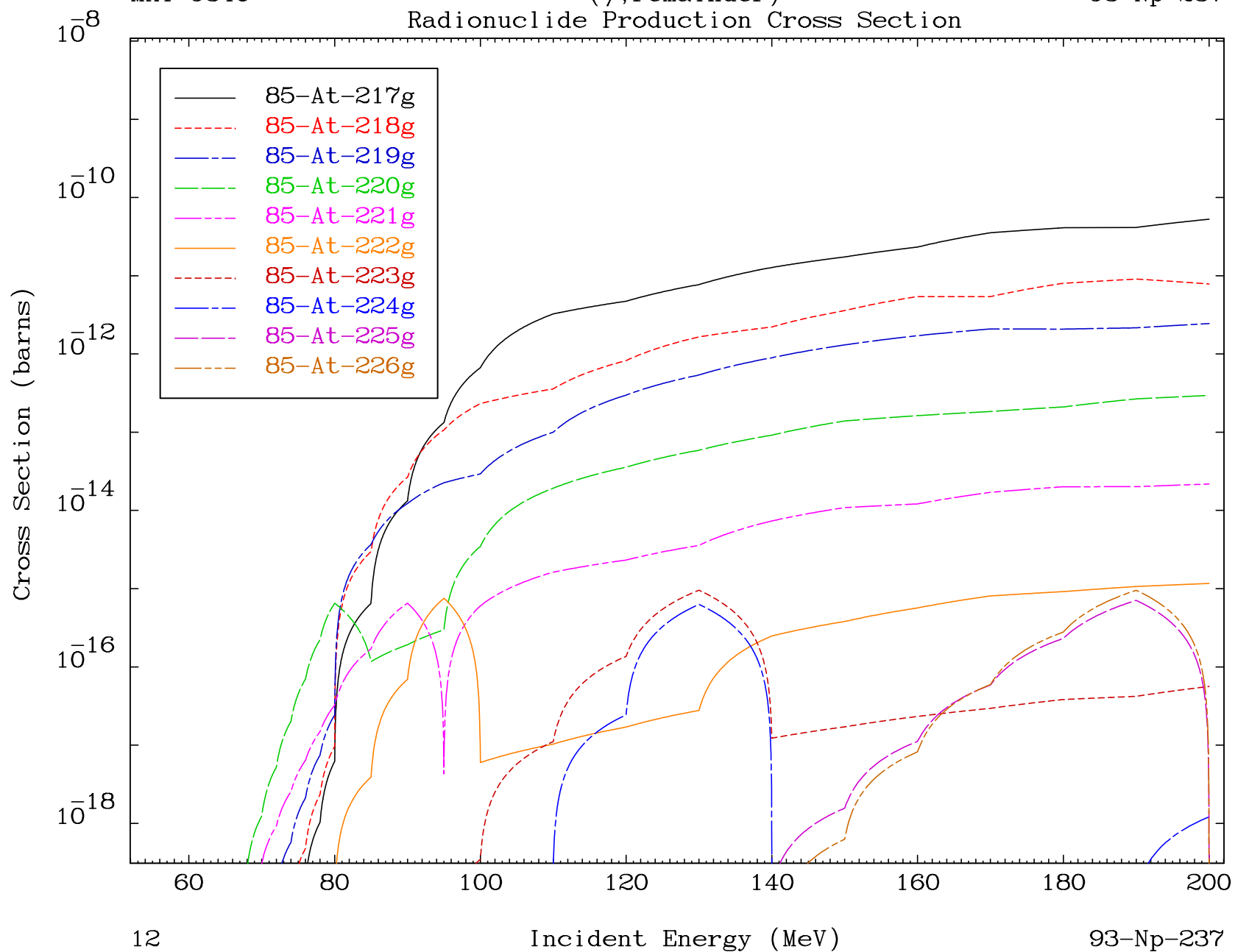


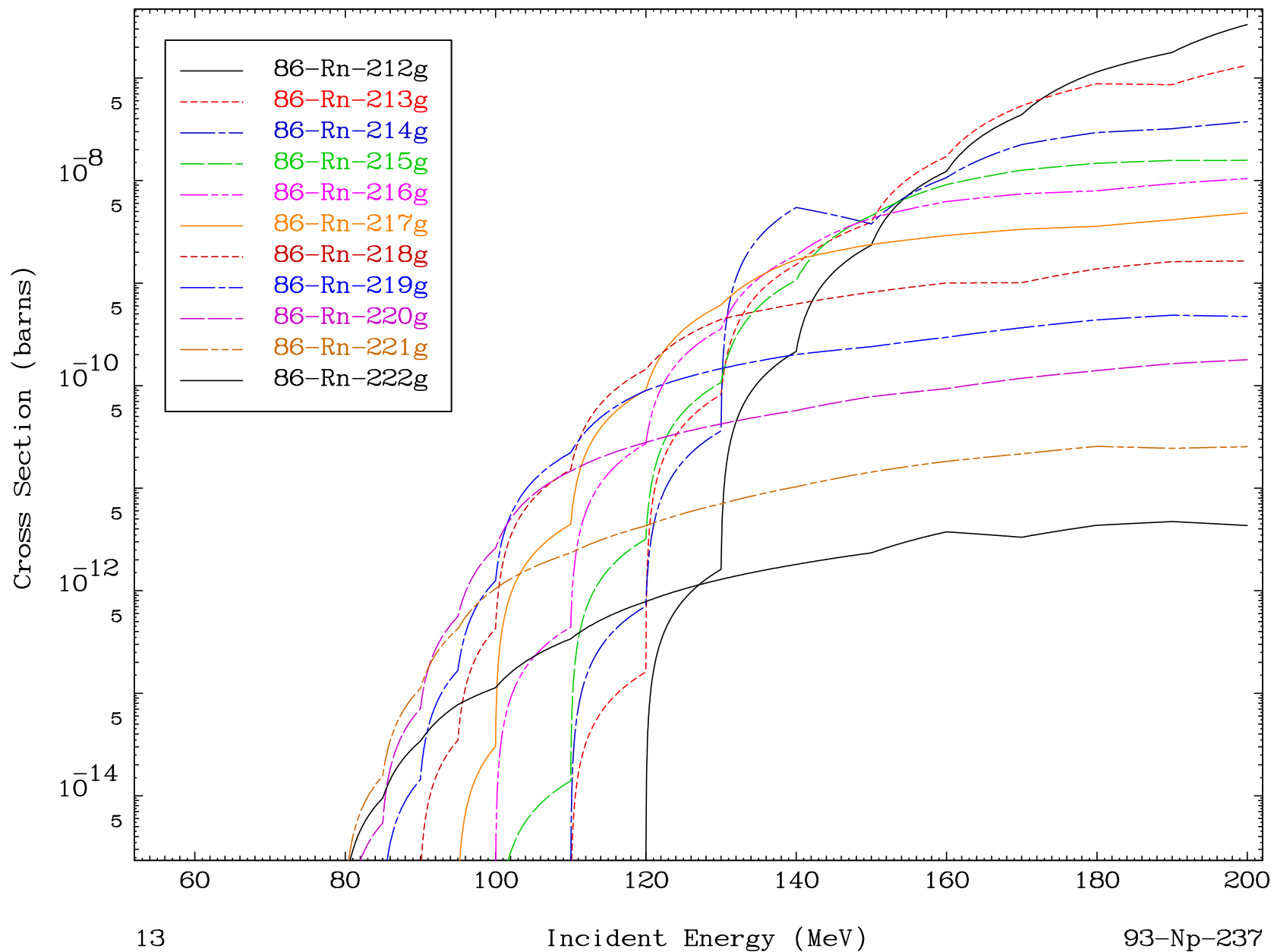
MAT 9346

 $(\gamma, \text{remainder})$ 

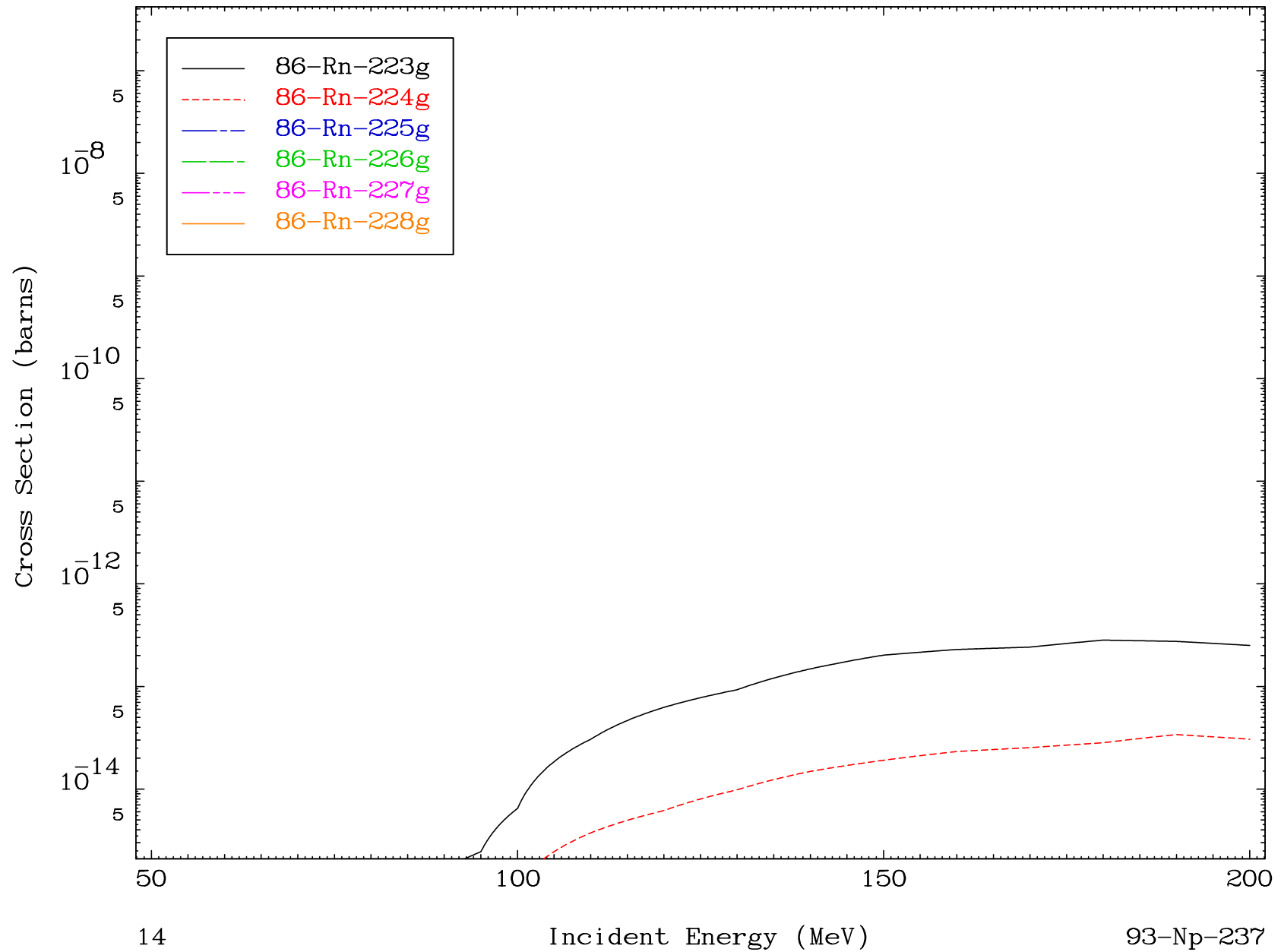
93-Np-237

## Radionuclide Production Cross Section





## Radionuclide Production Cross Section

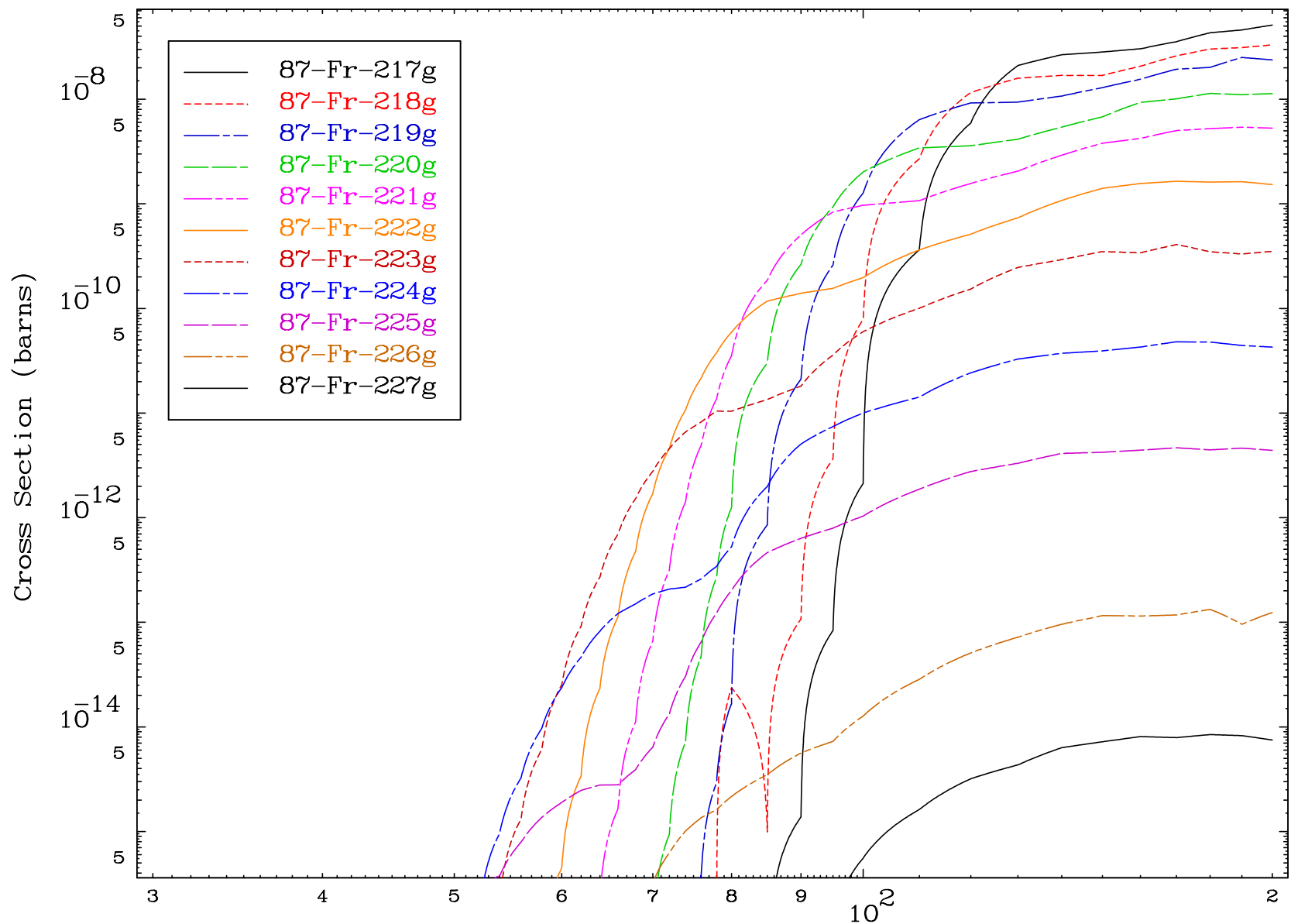


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



15

Incident Energy (MeV)

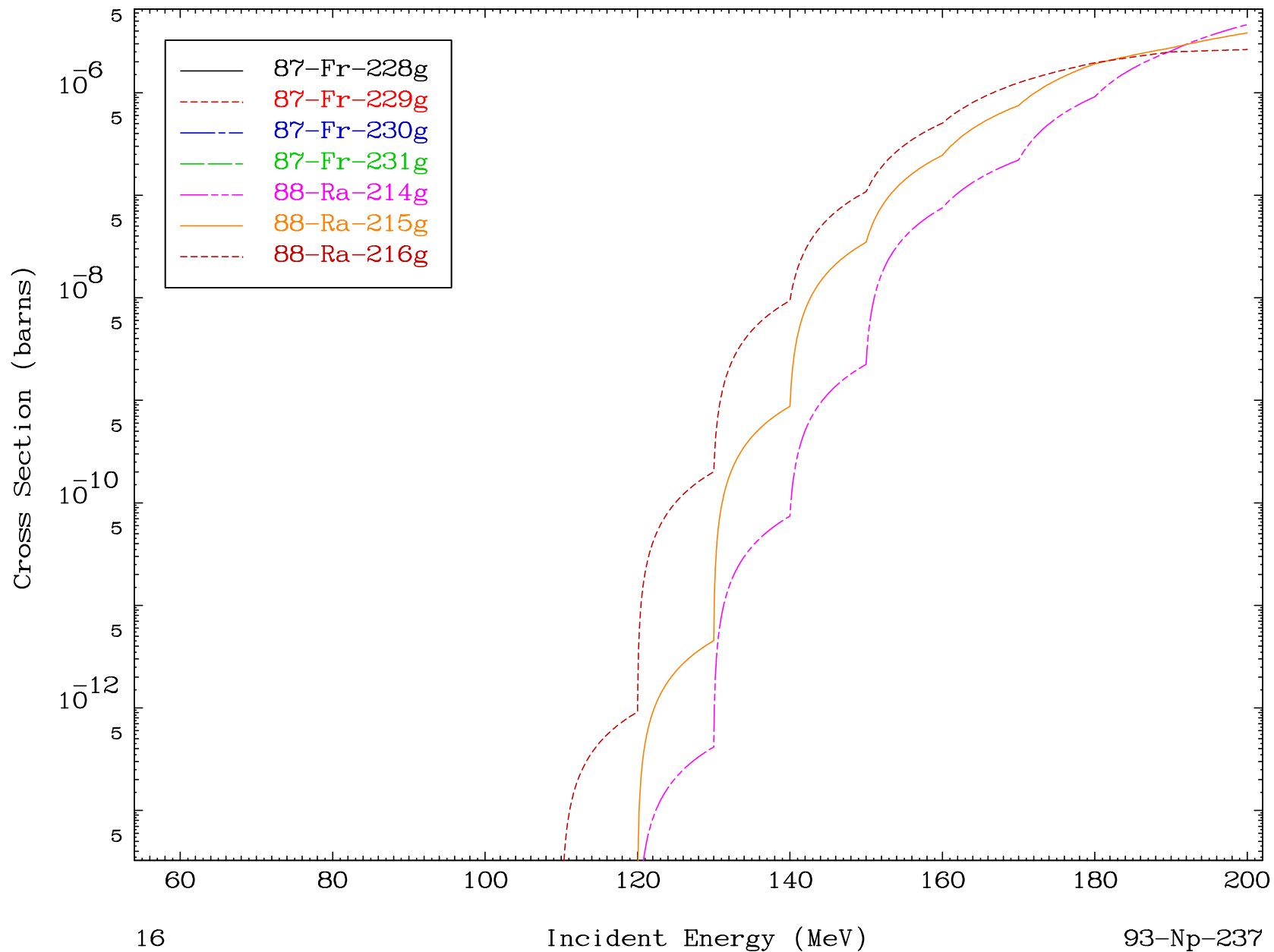
93-Np-237

MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



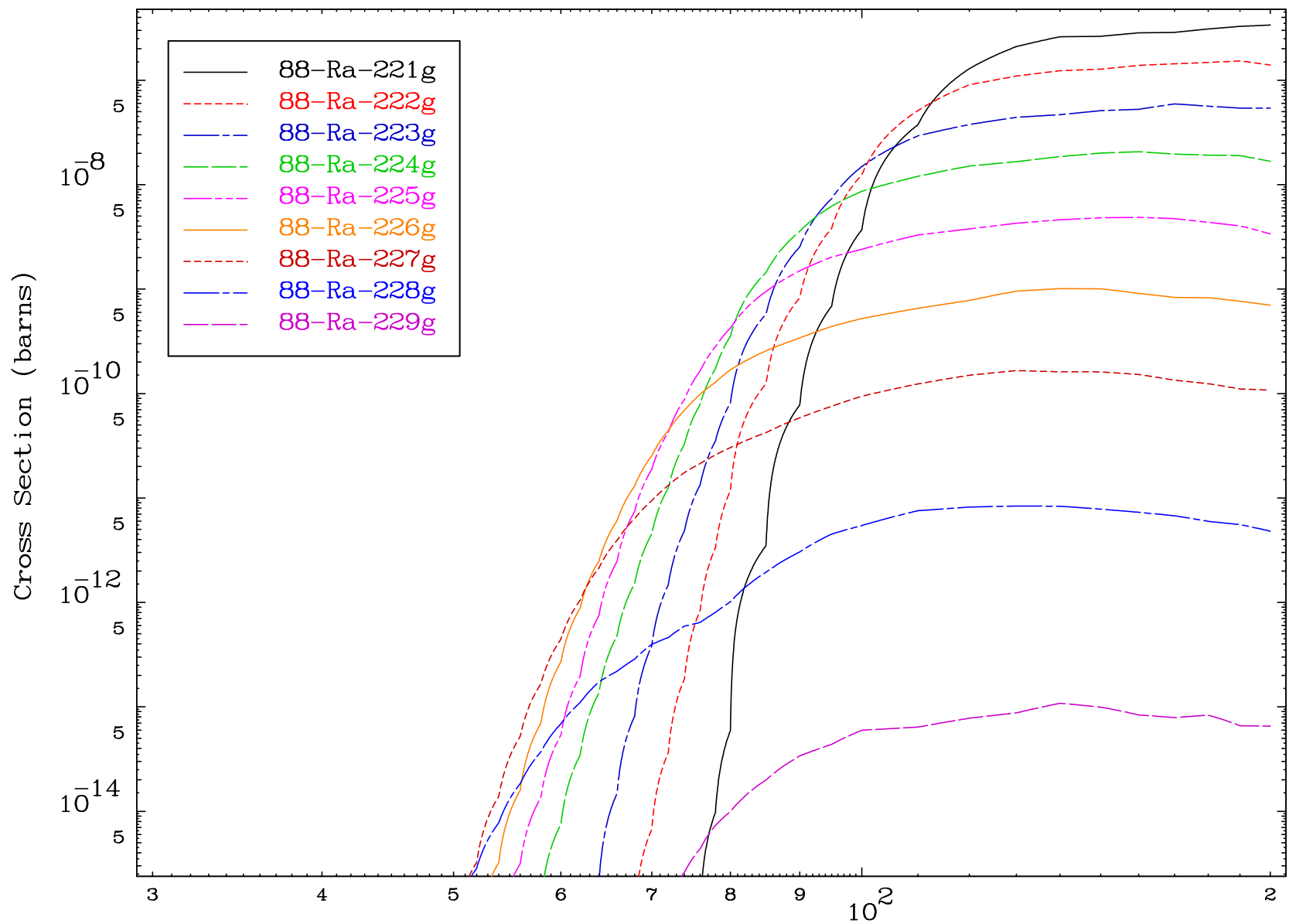


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



17

Incident Energy (MeV)

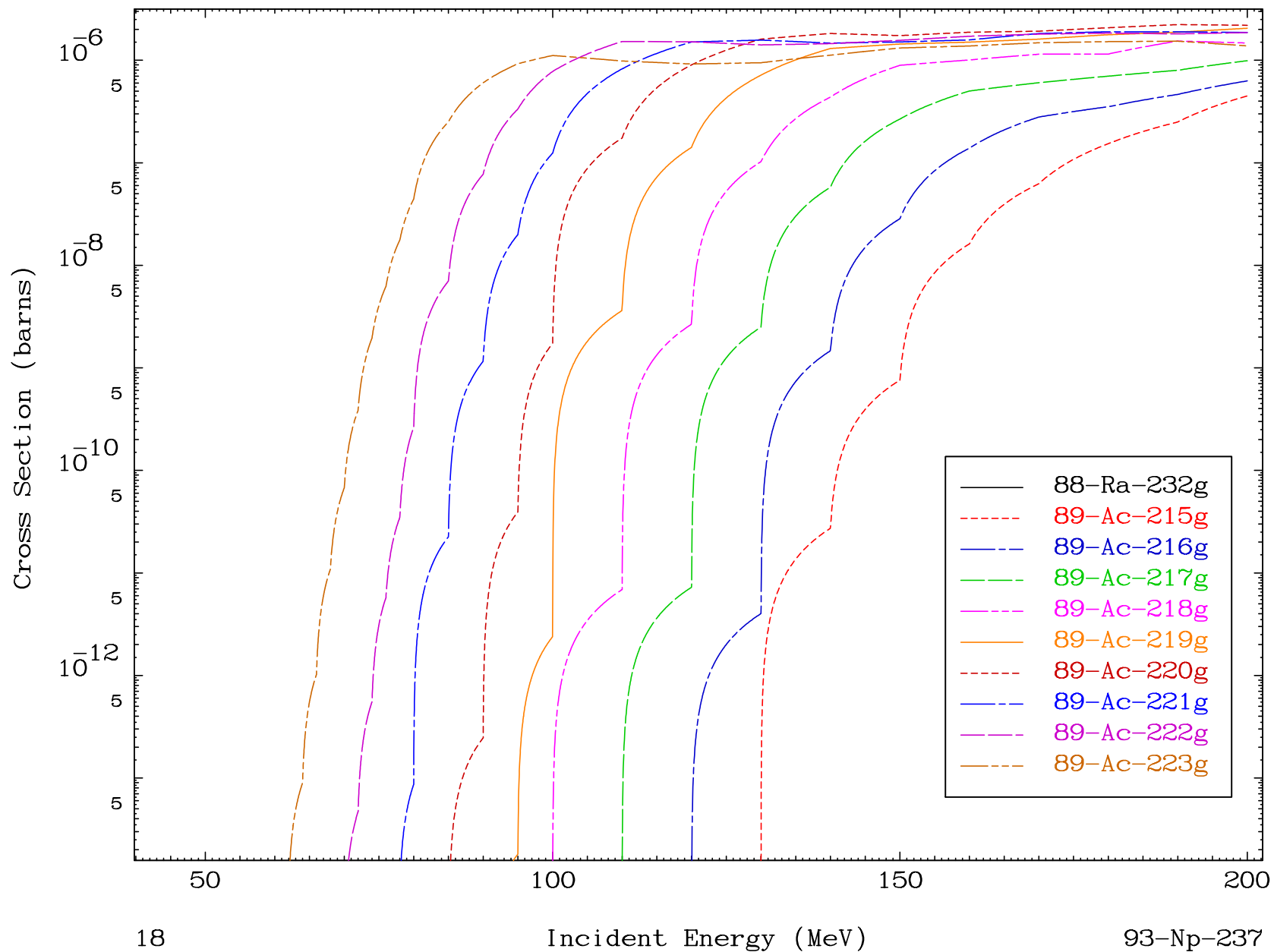
93-Np-237

MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

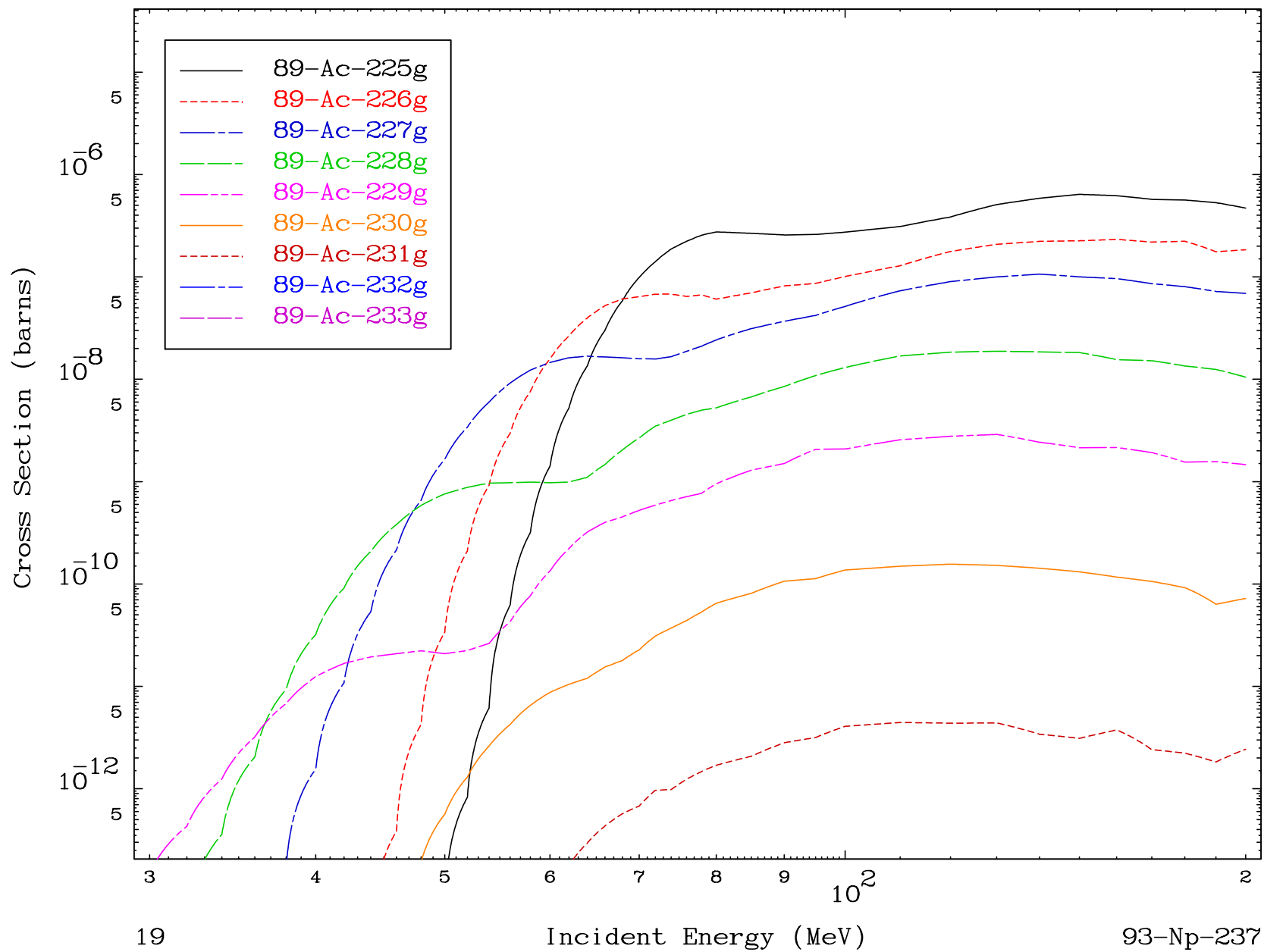
## Radionuclide Production Cross Section



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( $\gamma$ , remainder)  
Radionuclide Production Cross Section

93-Np-237

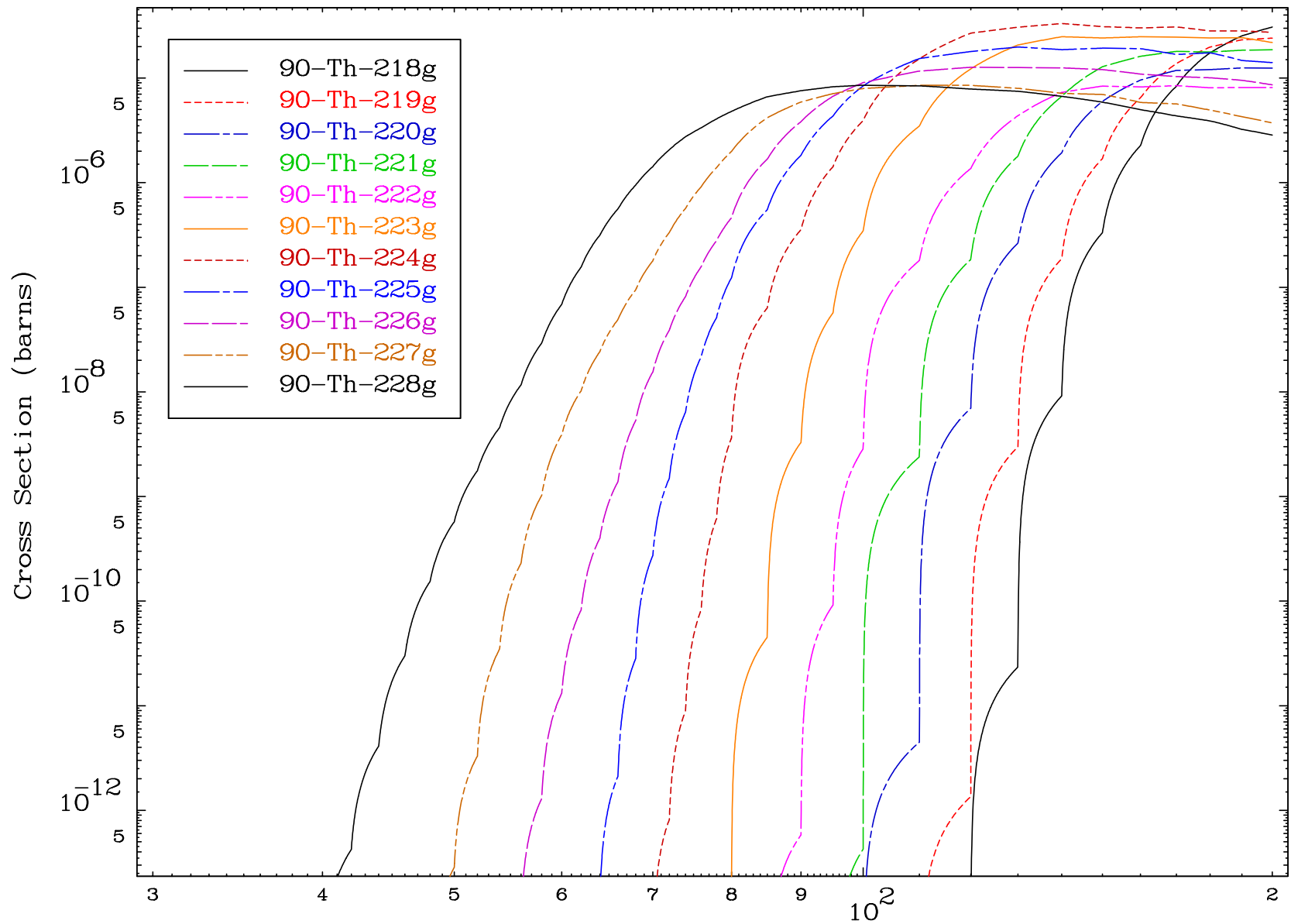


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section

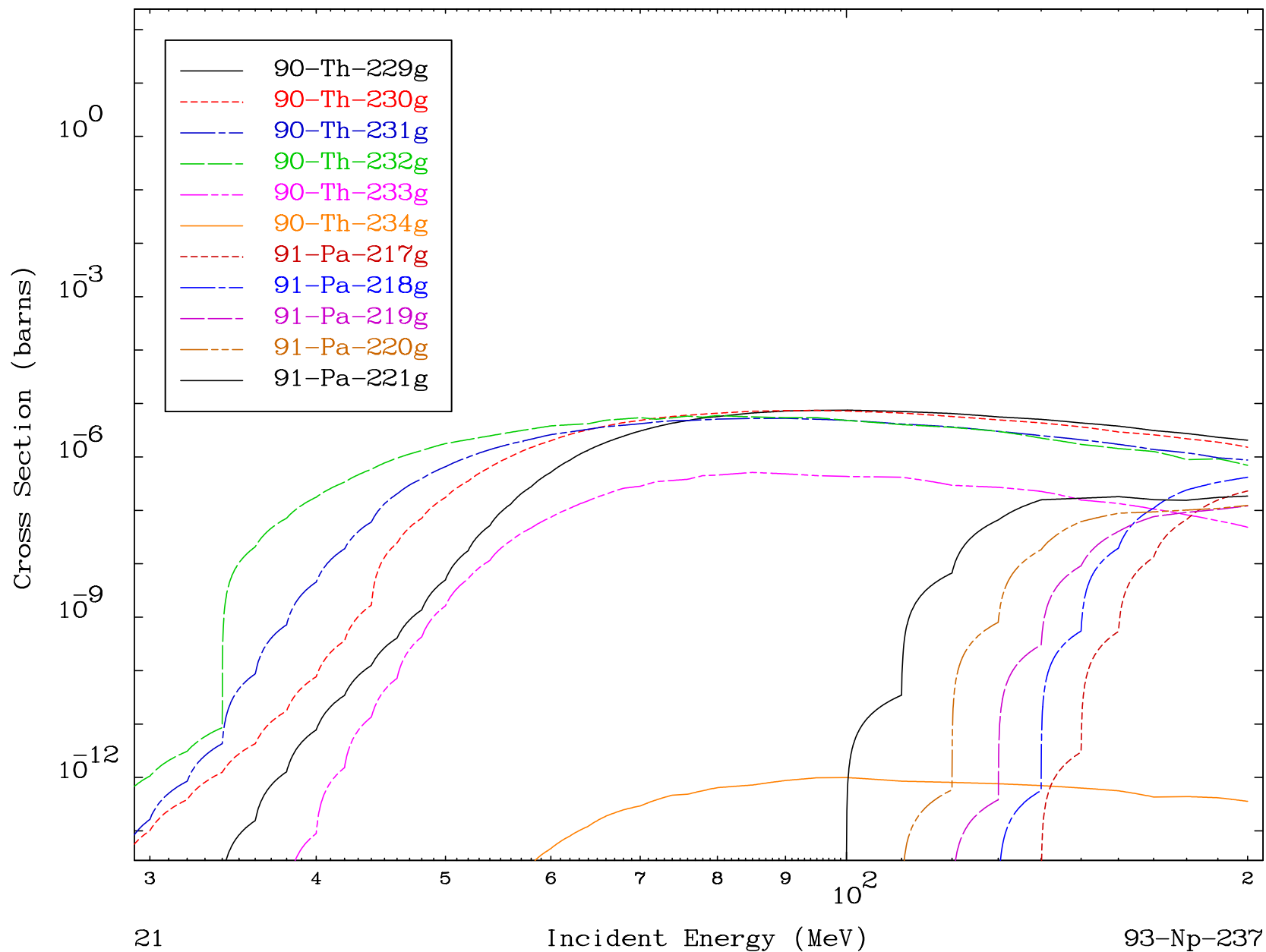


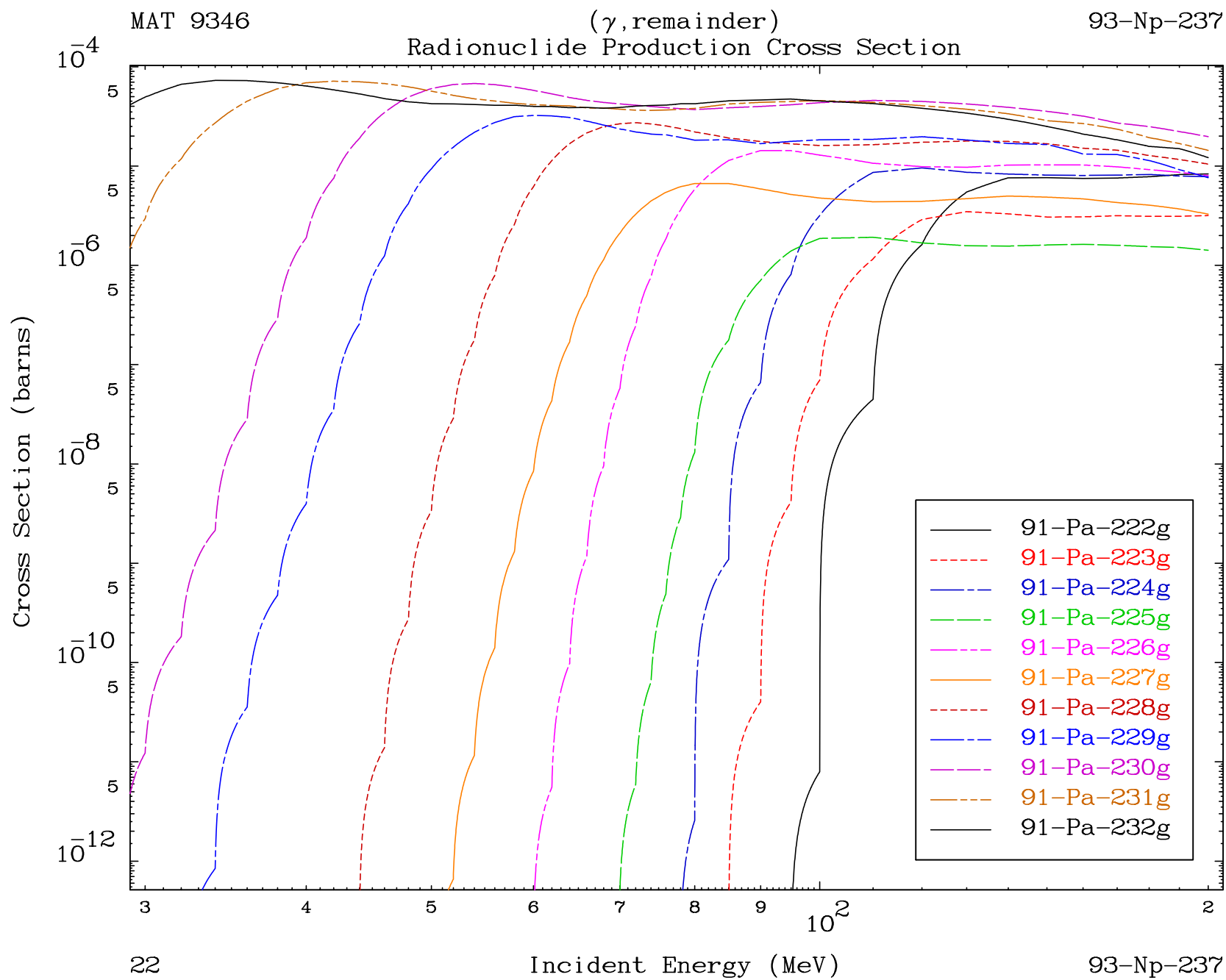
20

Incident Energy (MeV)

93-Np-237

## Radionuclide Production Cross Section



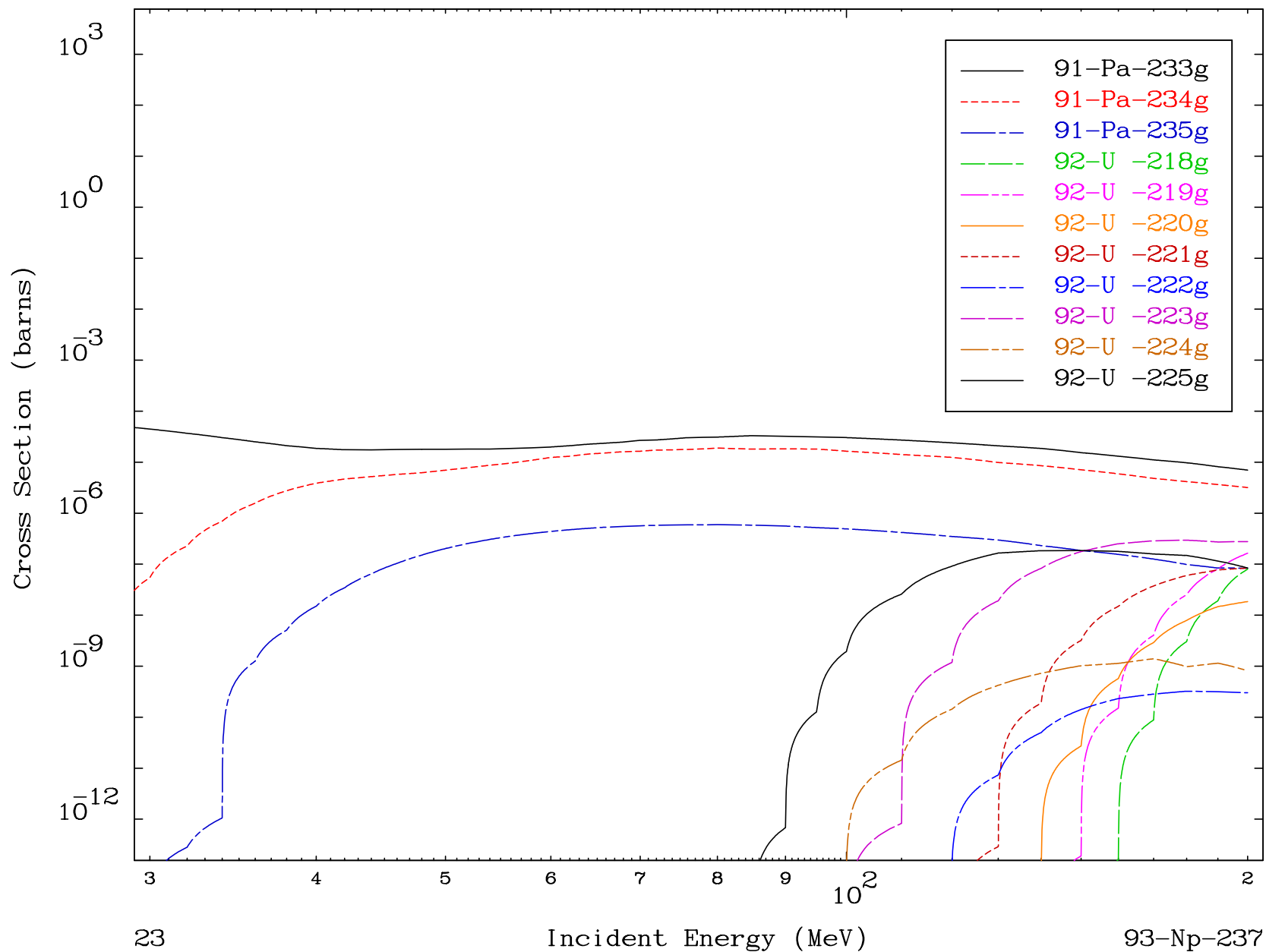


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section

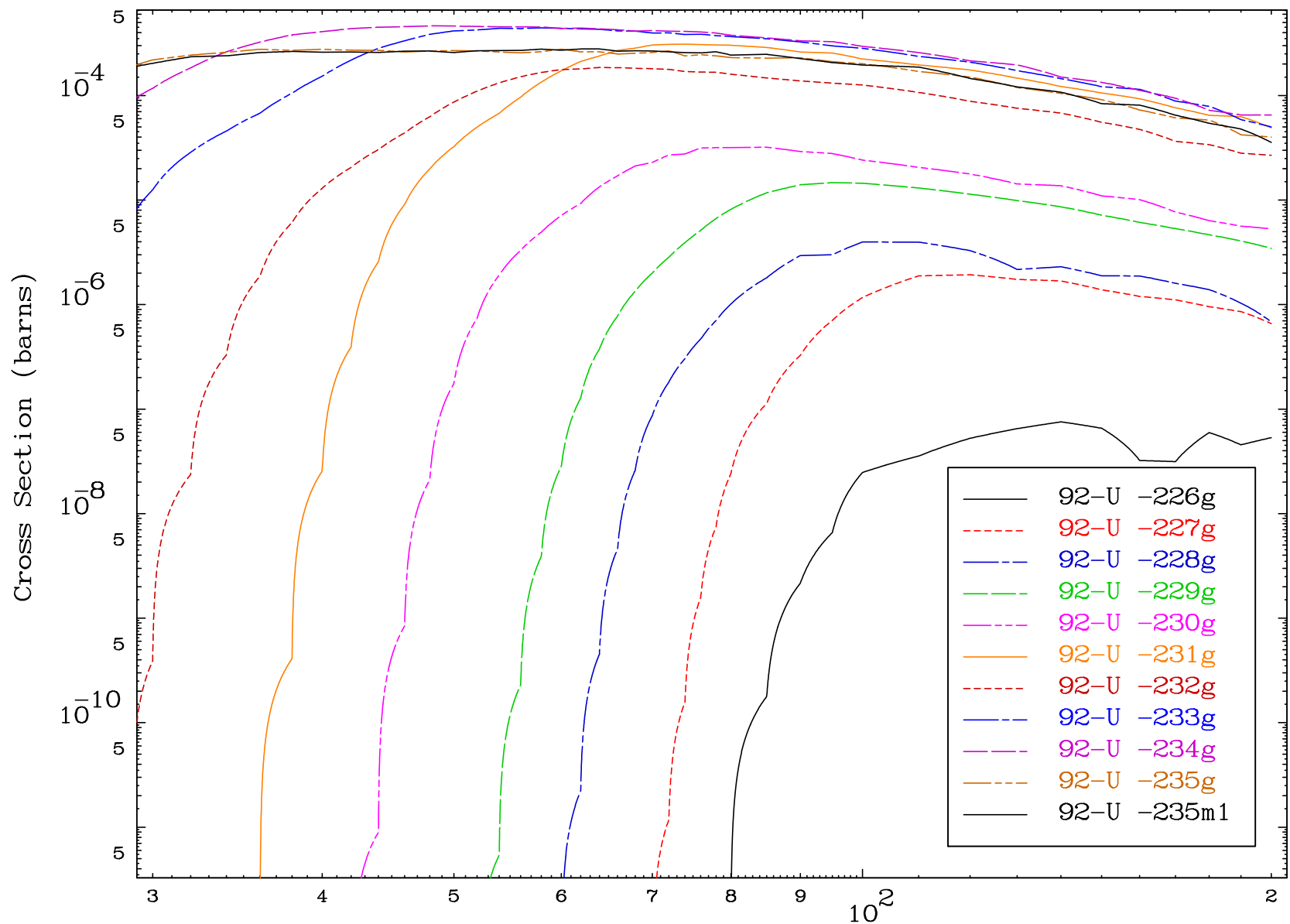


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



24

Incident Energy (MeV)

93-Np-237

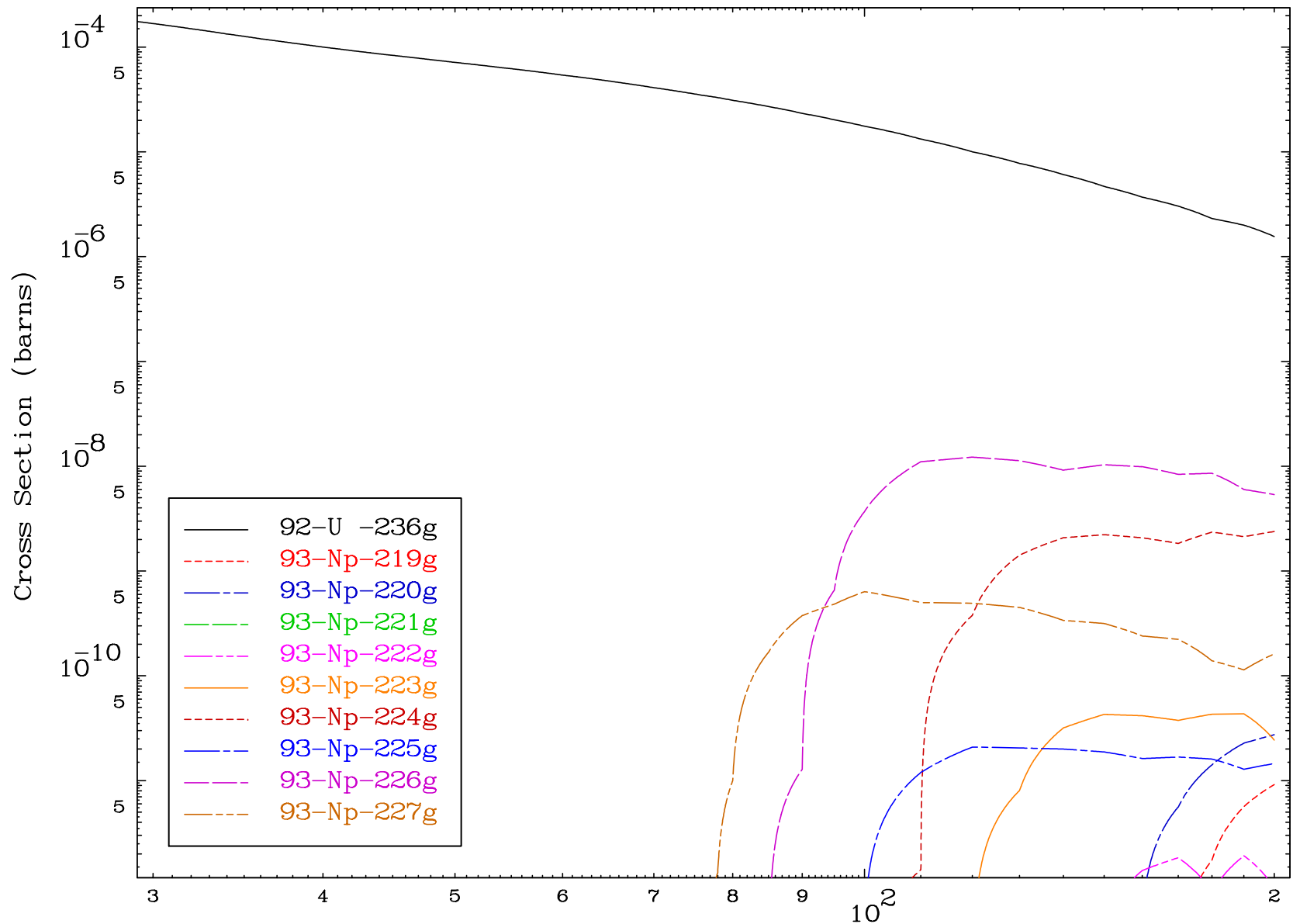


MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



25

Incident Energy (MeV)

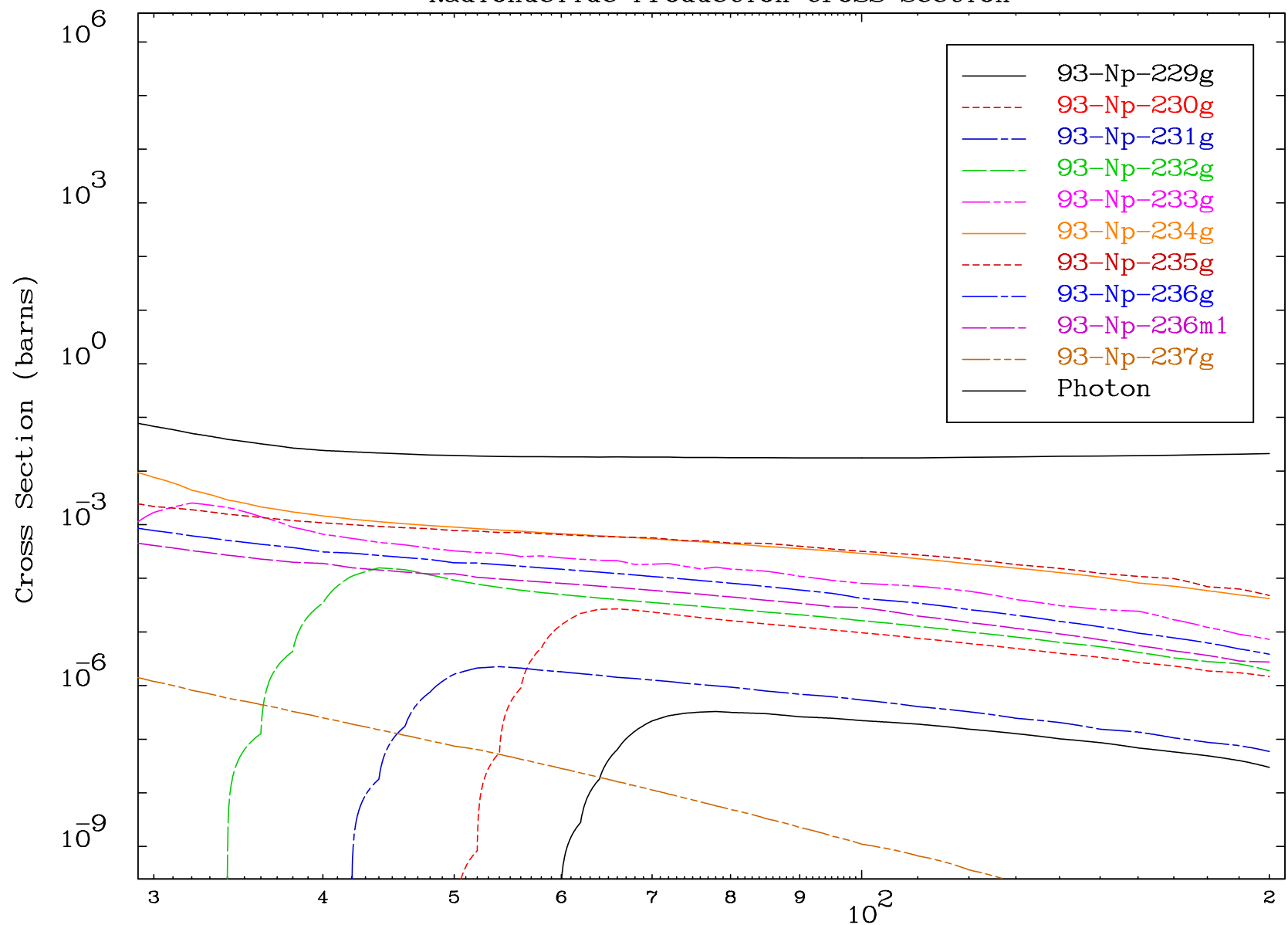
93-Np-237

MAT 9346

 $(\gamma, \text{remainder})$ 

93-Np-237

## Radionuclide Production Cross Section



26

Incident Energy (MeV)

93-Np-237