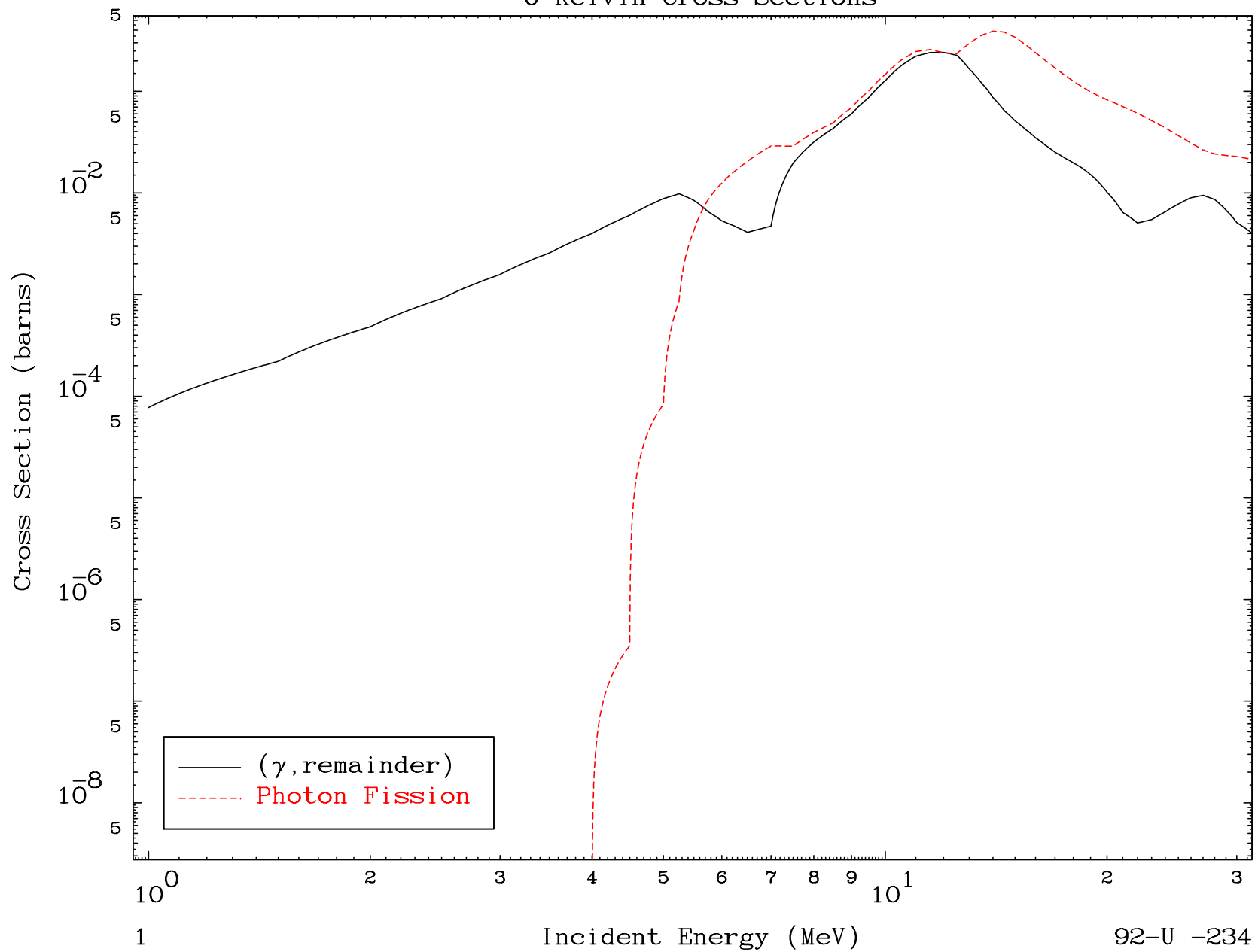


MAT 9225

Photon Major
0 Kelvin Cross Sections

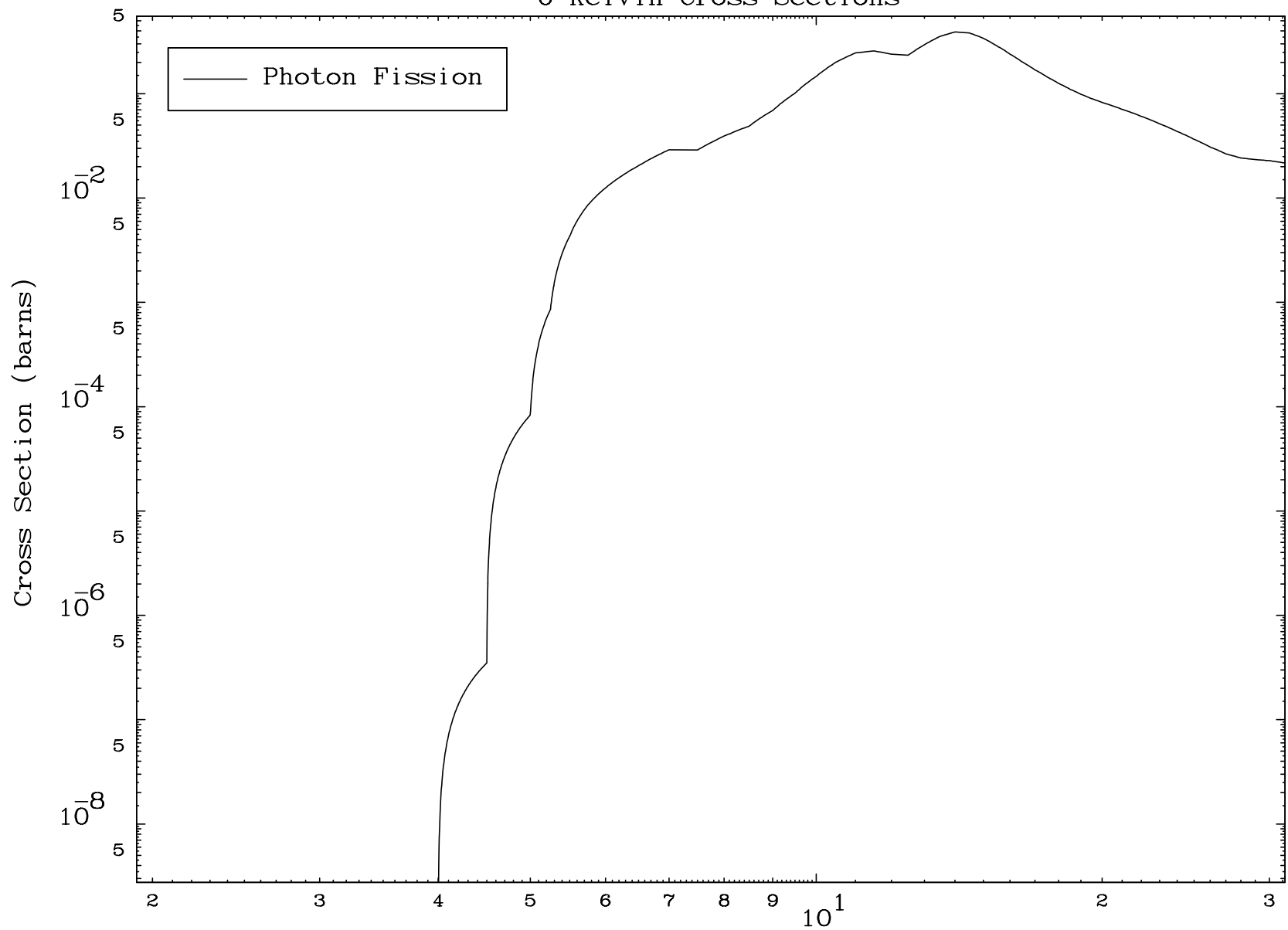
92-U -234



MAT 9225

Photon Fission
0 Kelvin Cross Sections

92-U -234



2

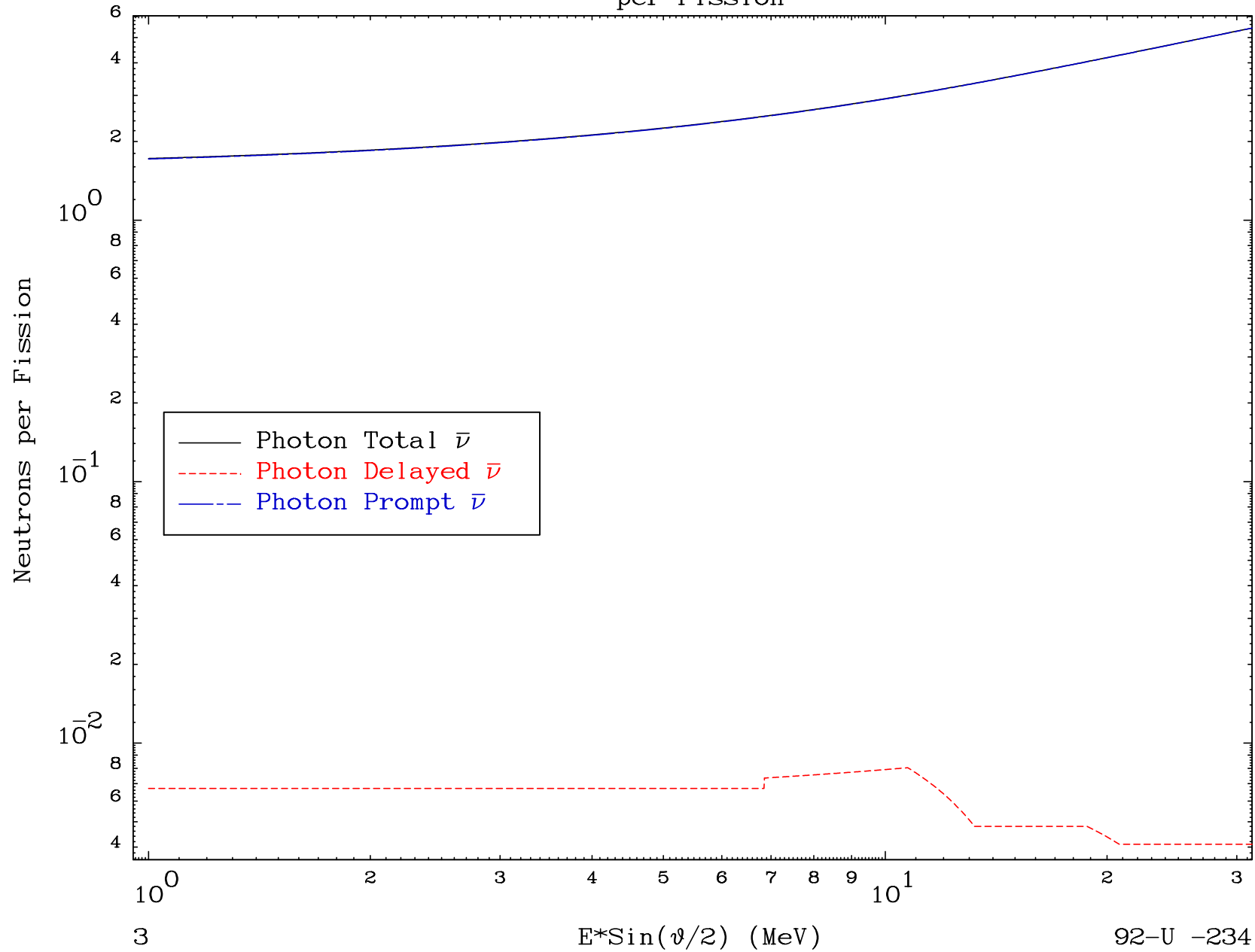
Incident Energy (MeV)

92-U -234

MAT 9225

Photon Neutrons
per Fission

92-U -234

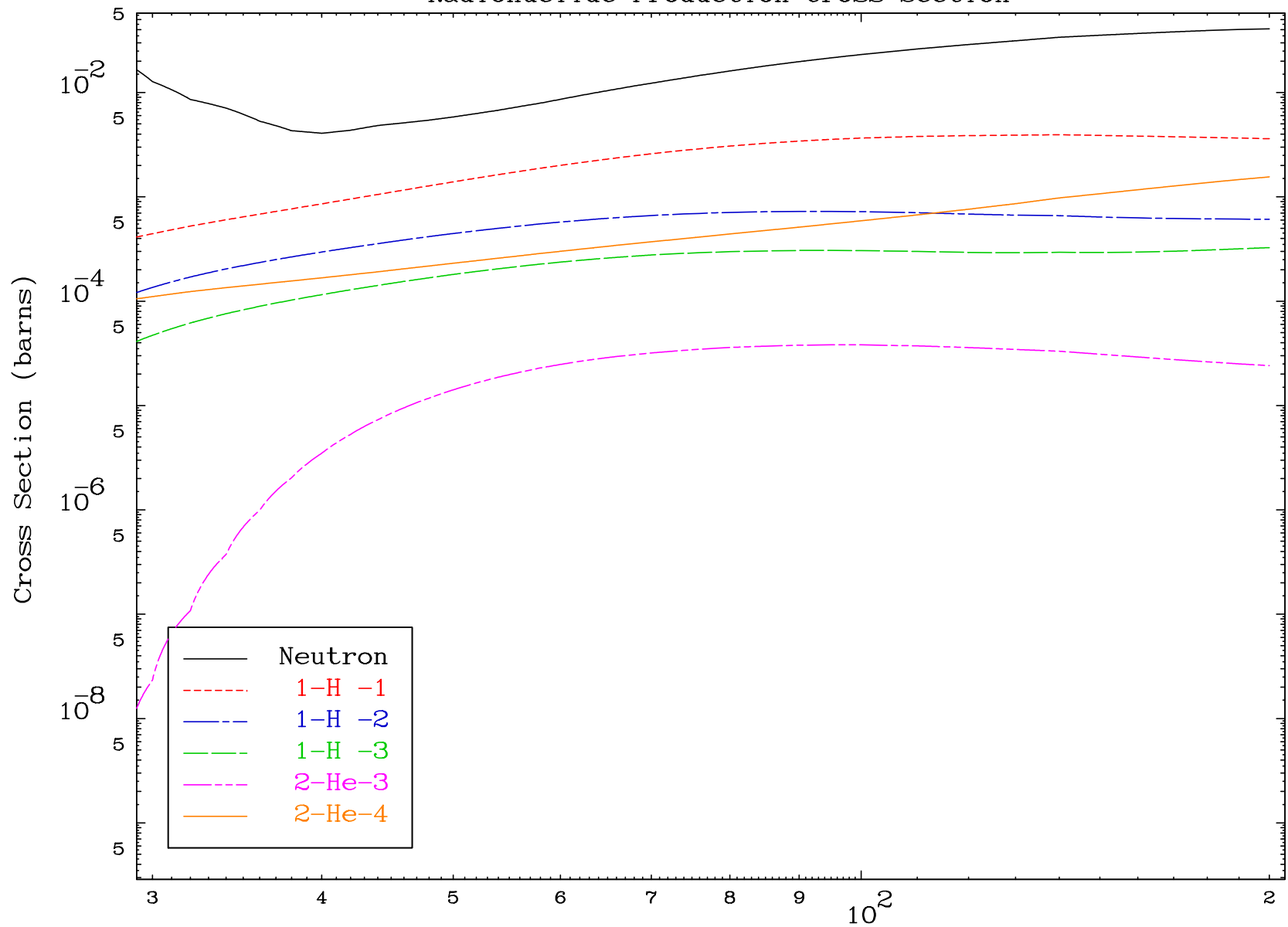


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

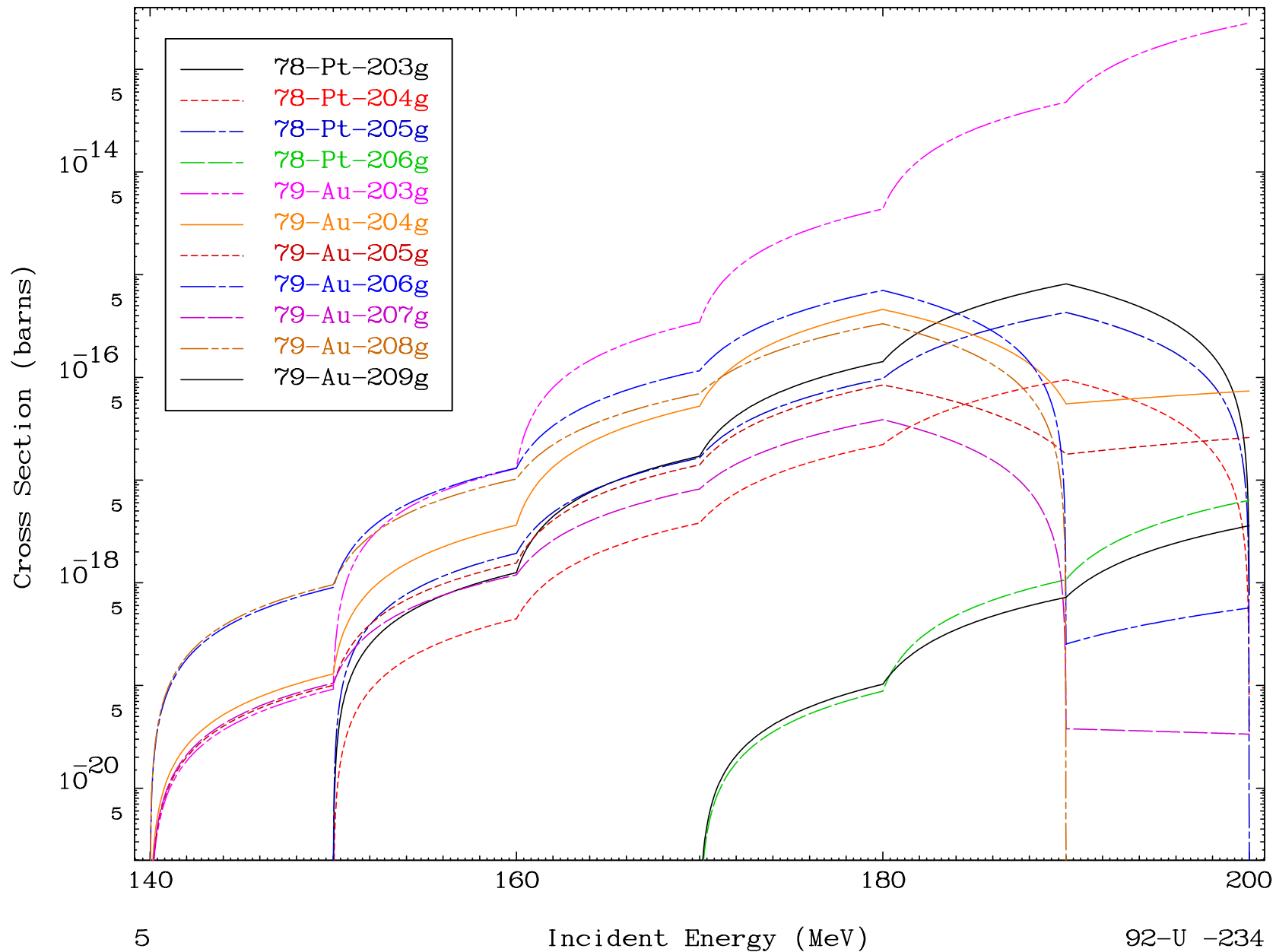


4

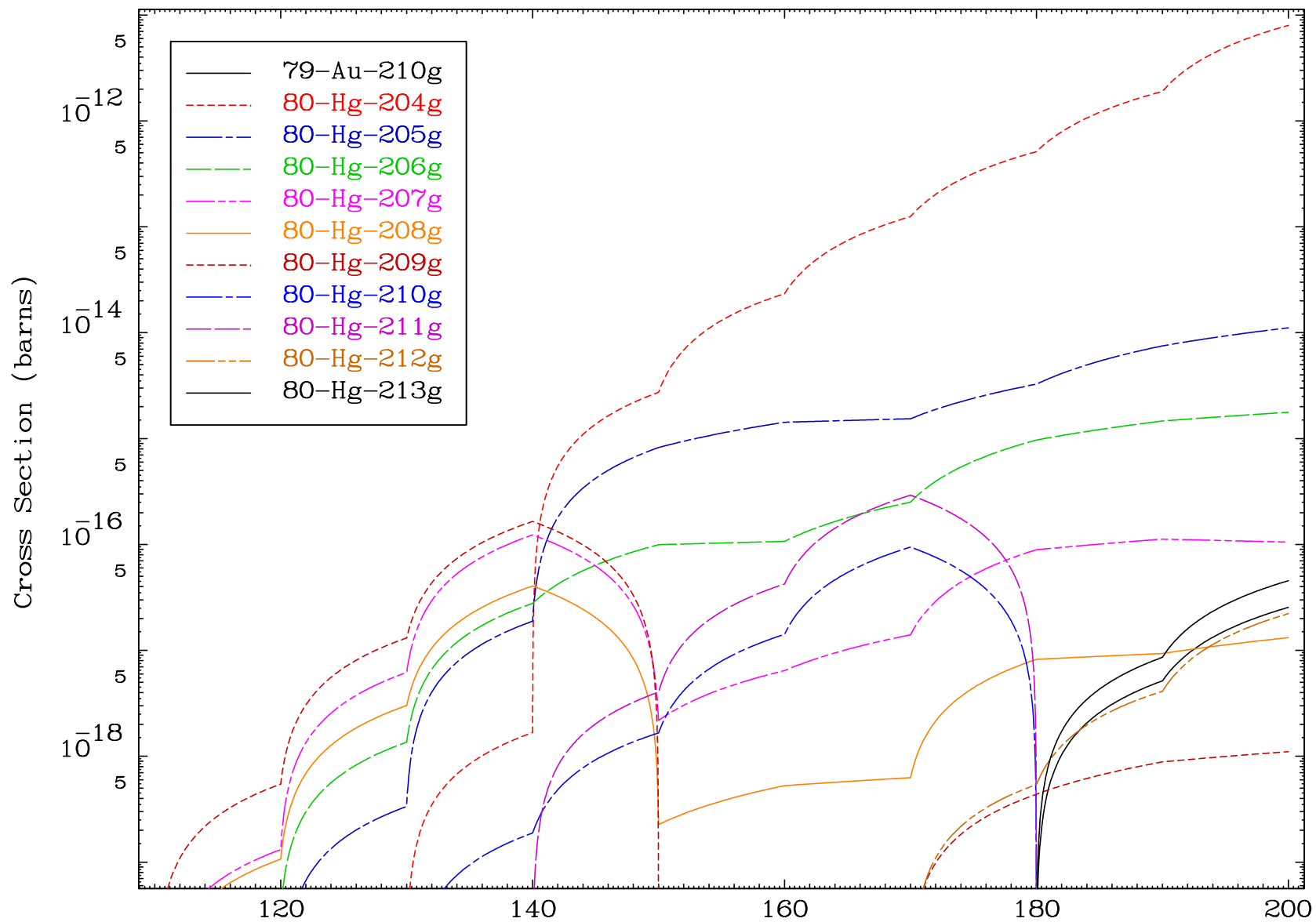
Incident Energy (MeV)

92-U -234

Radionuclide Production Cross Section



Radionuclide Production Cross Section

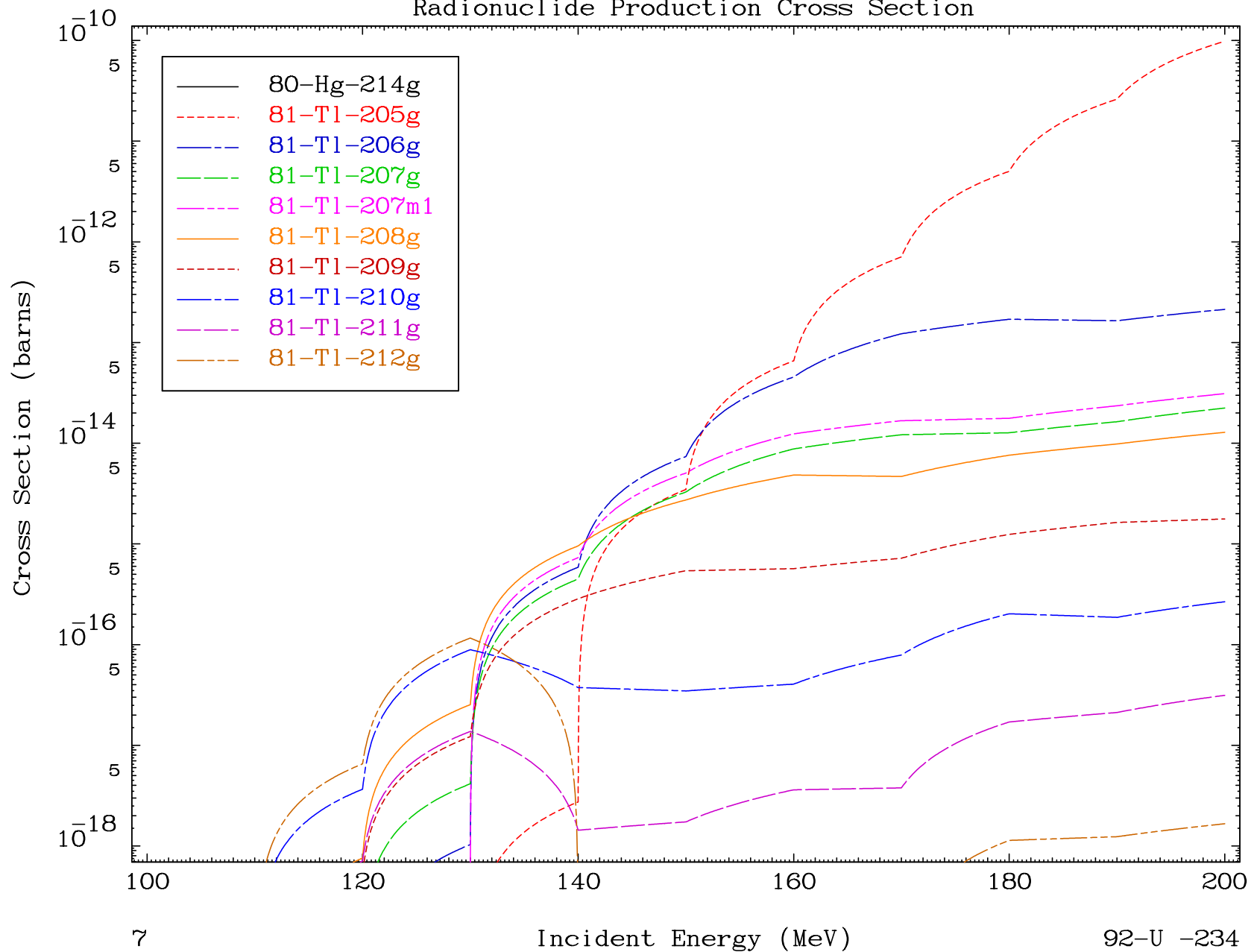


MAT 9225

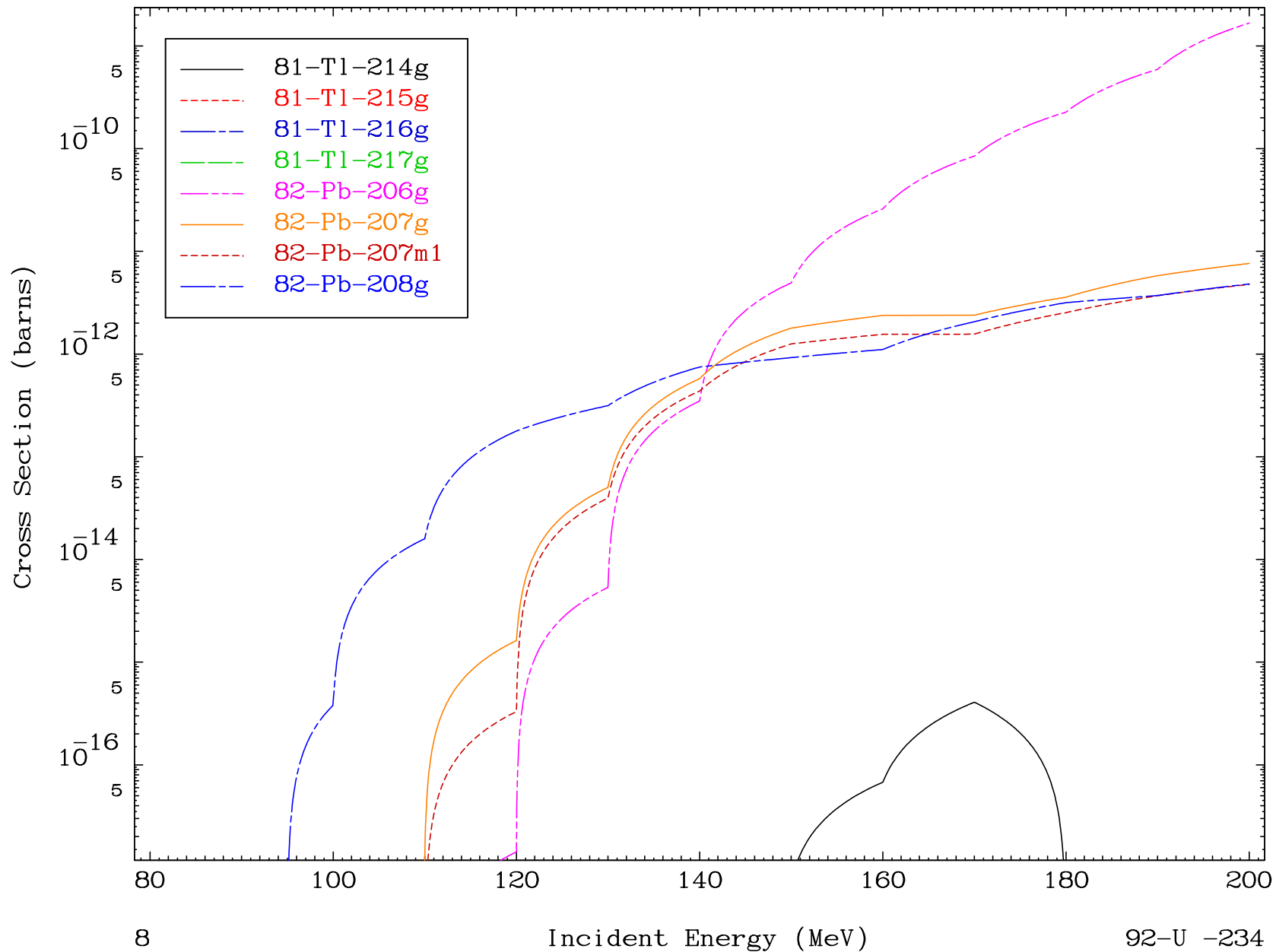
 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section



Radionuclide Production Cross Section

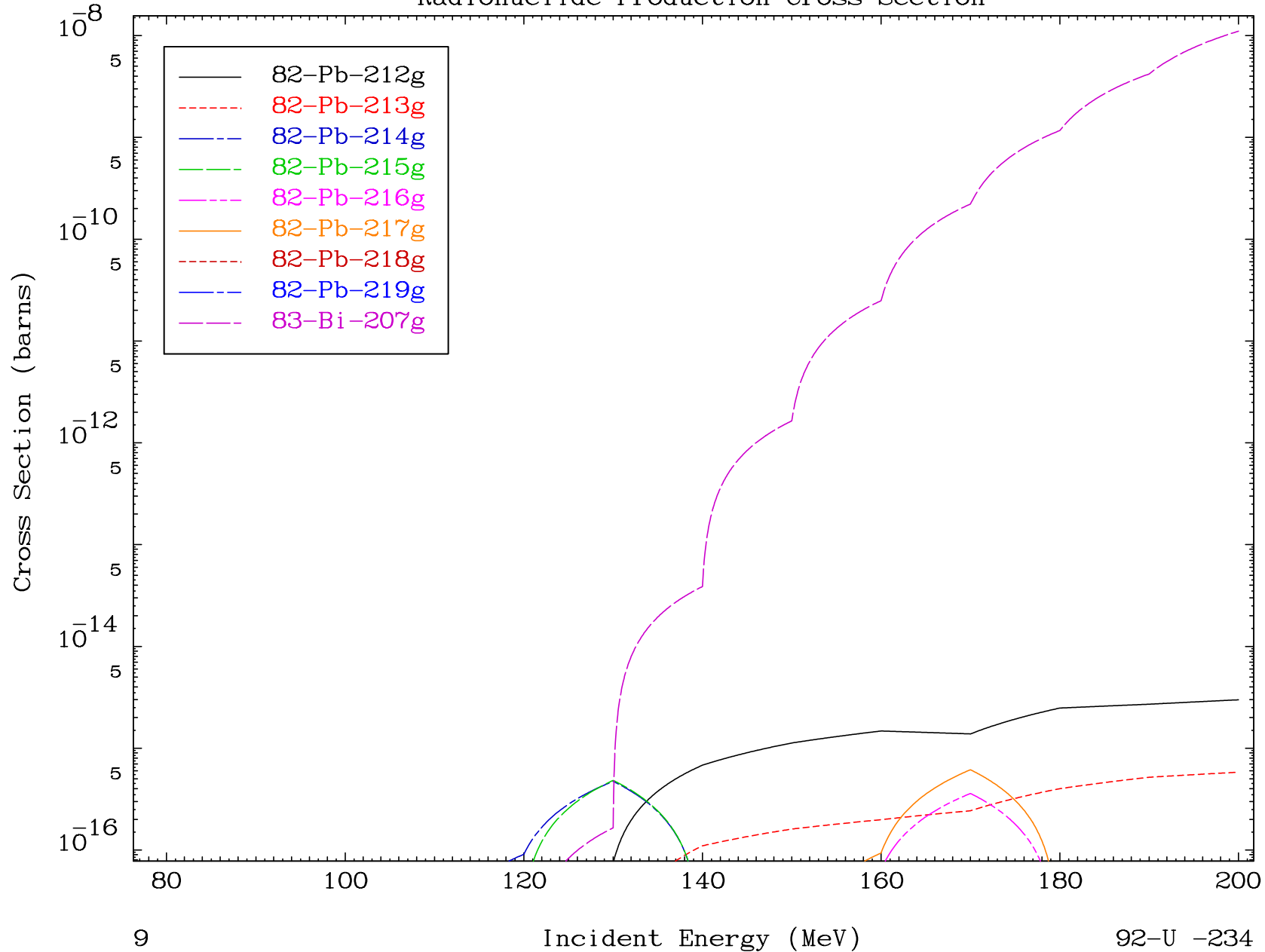


MAT 9225

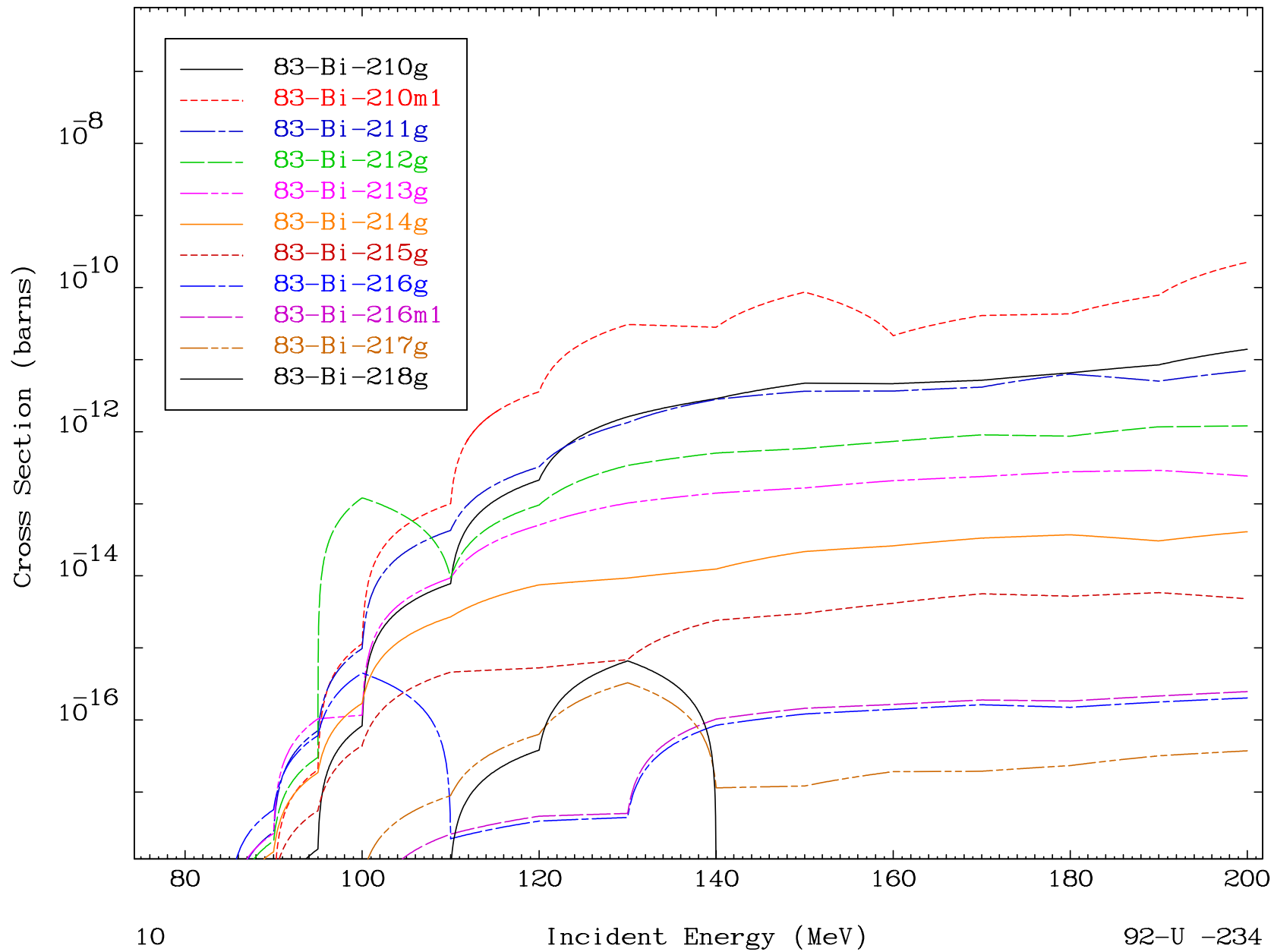
 $(\gamma, \text{remainder})$

92-U -234

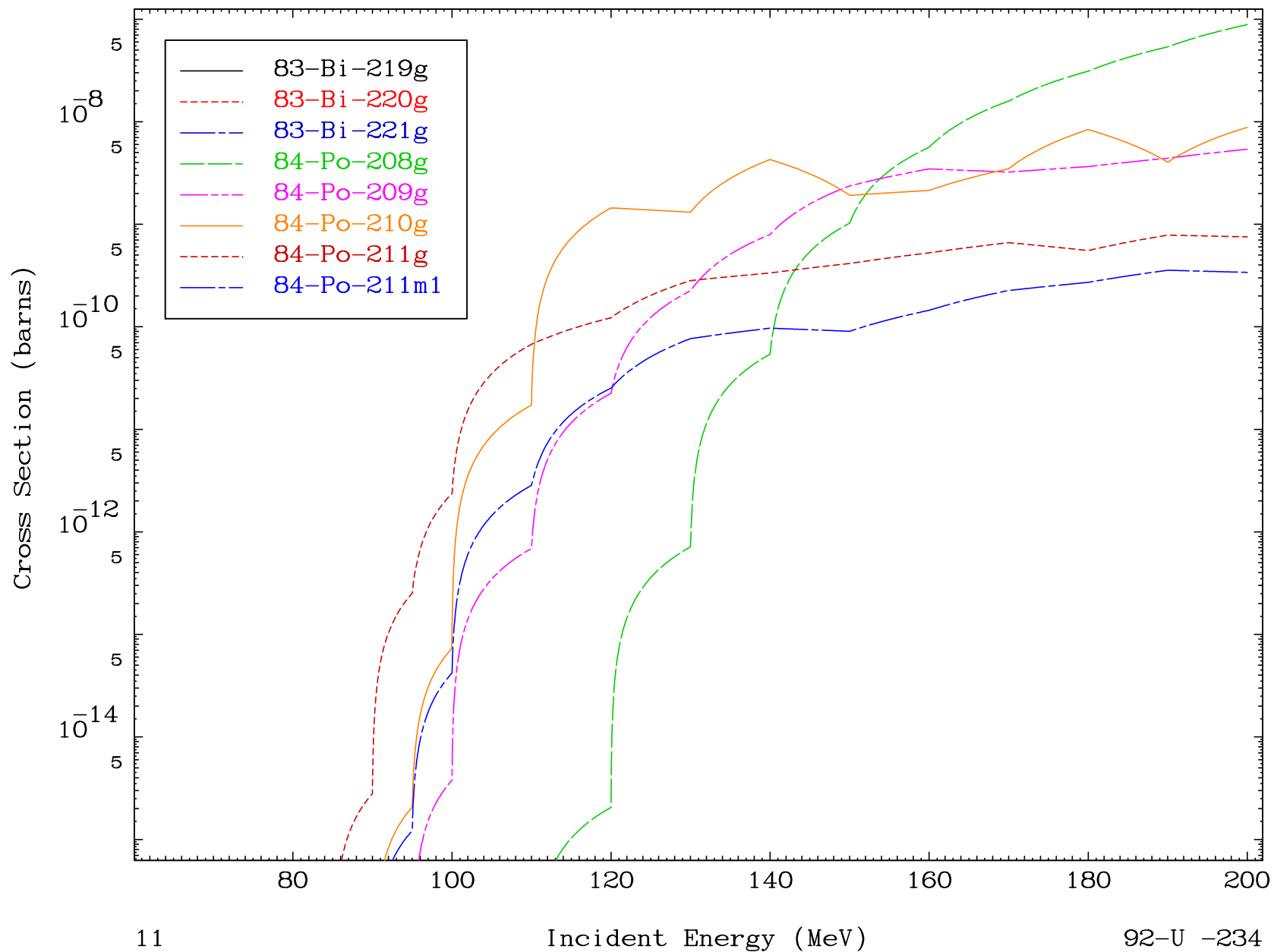
Radionuclide Production Cross Section



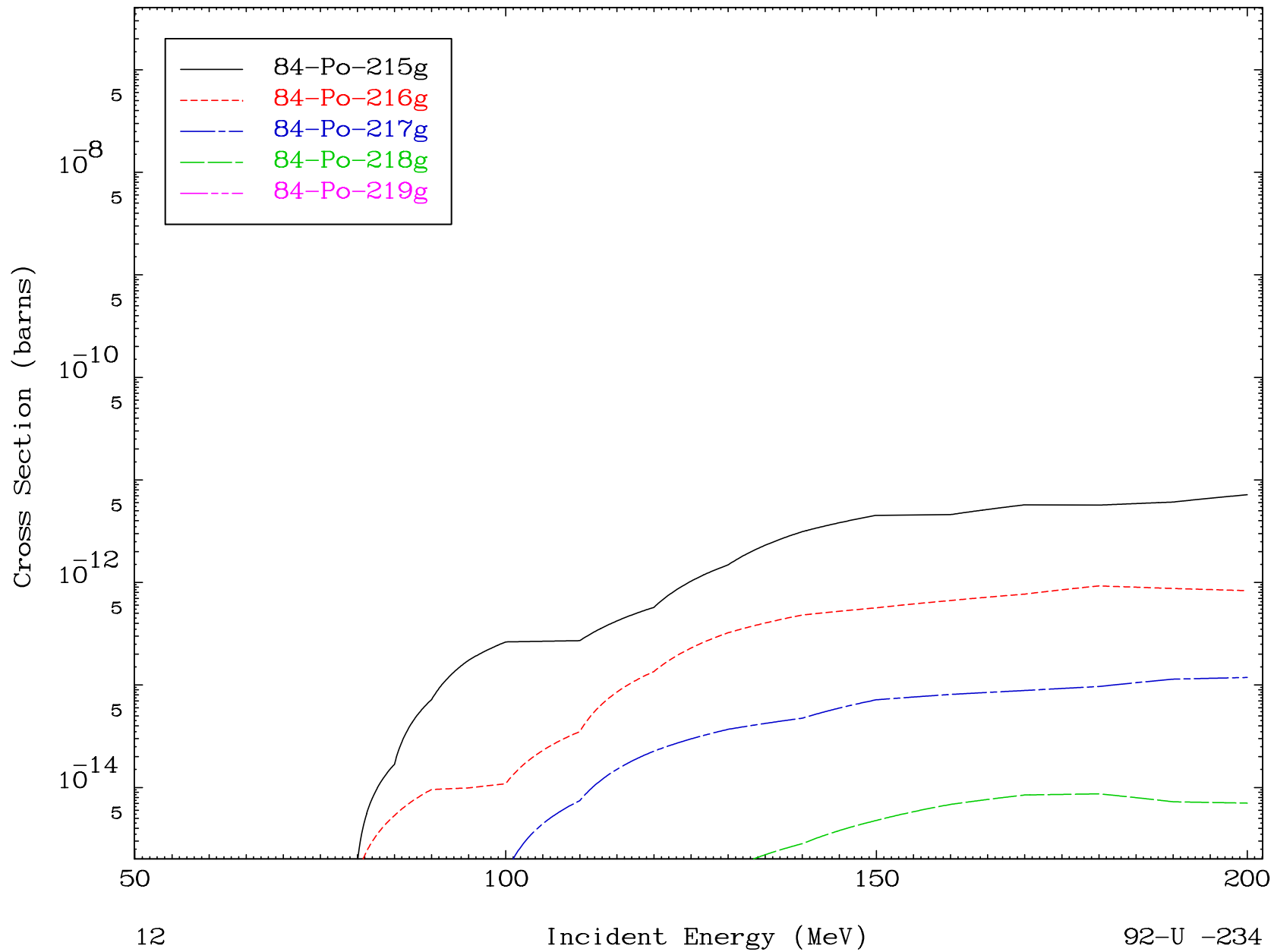
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

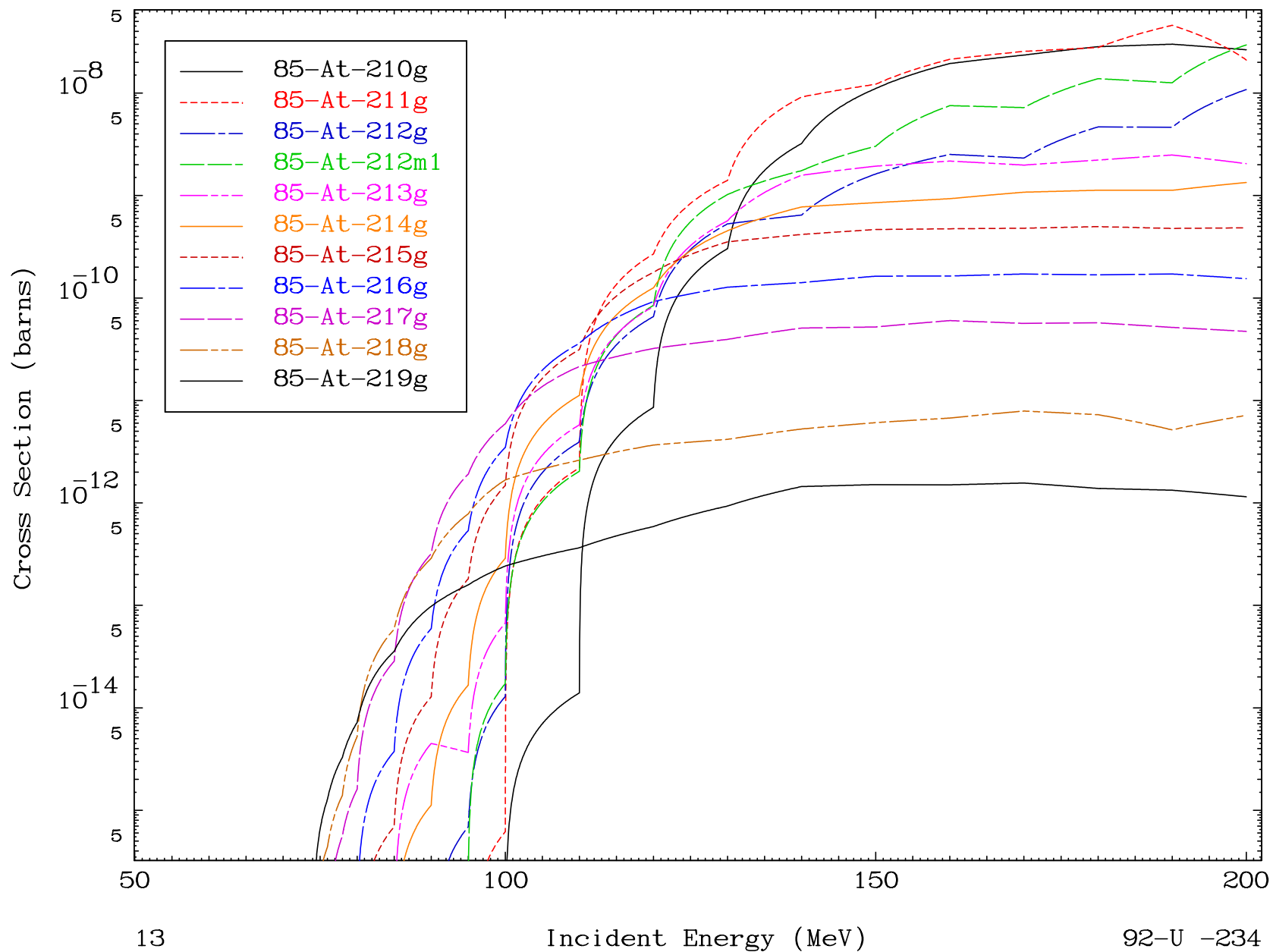


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

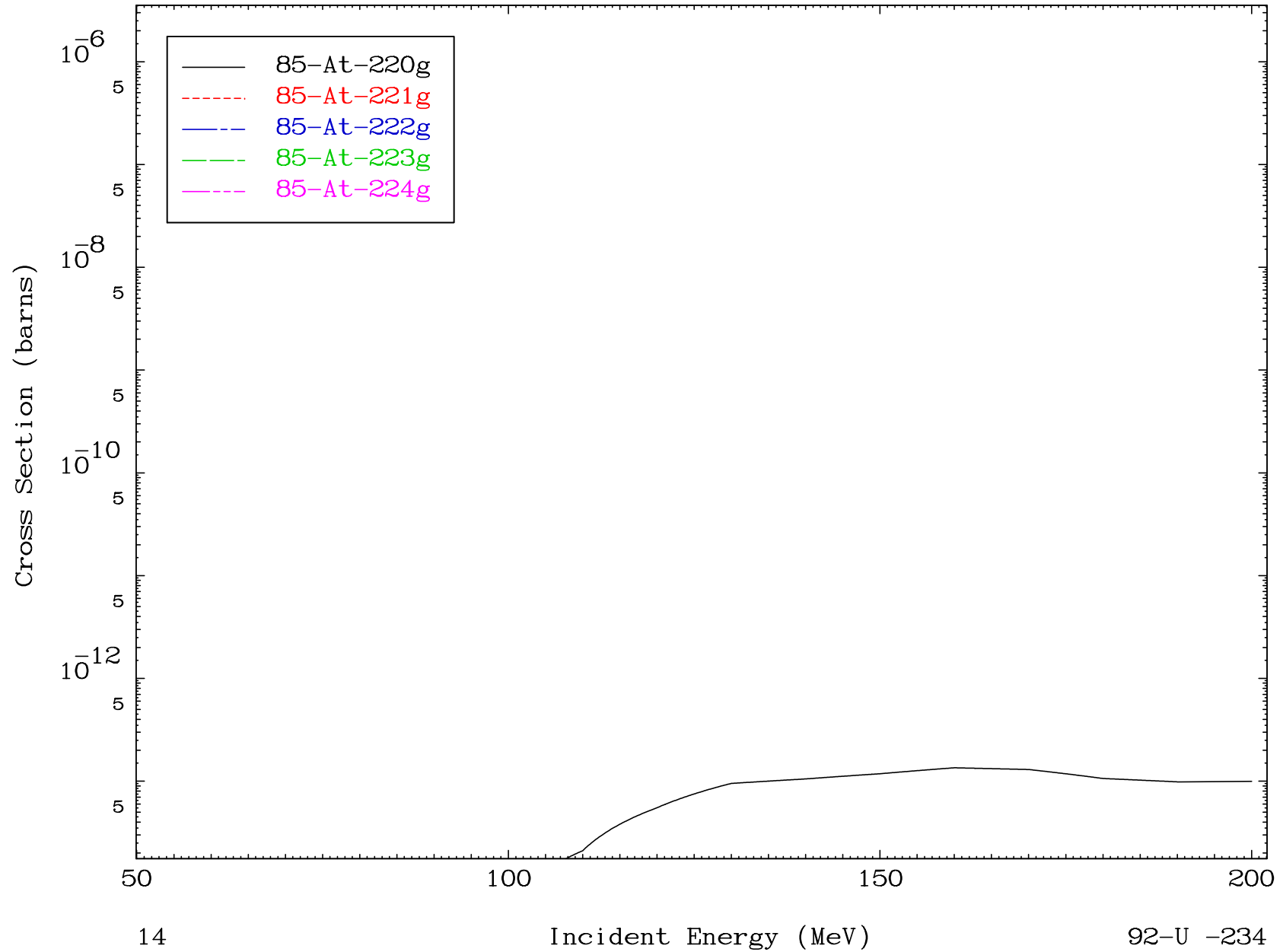


MAT 9225

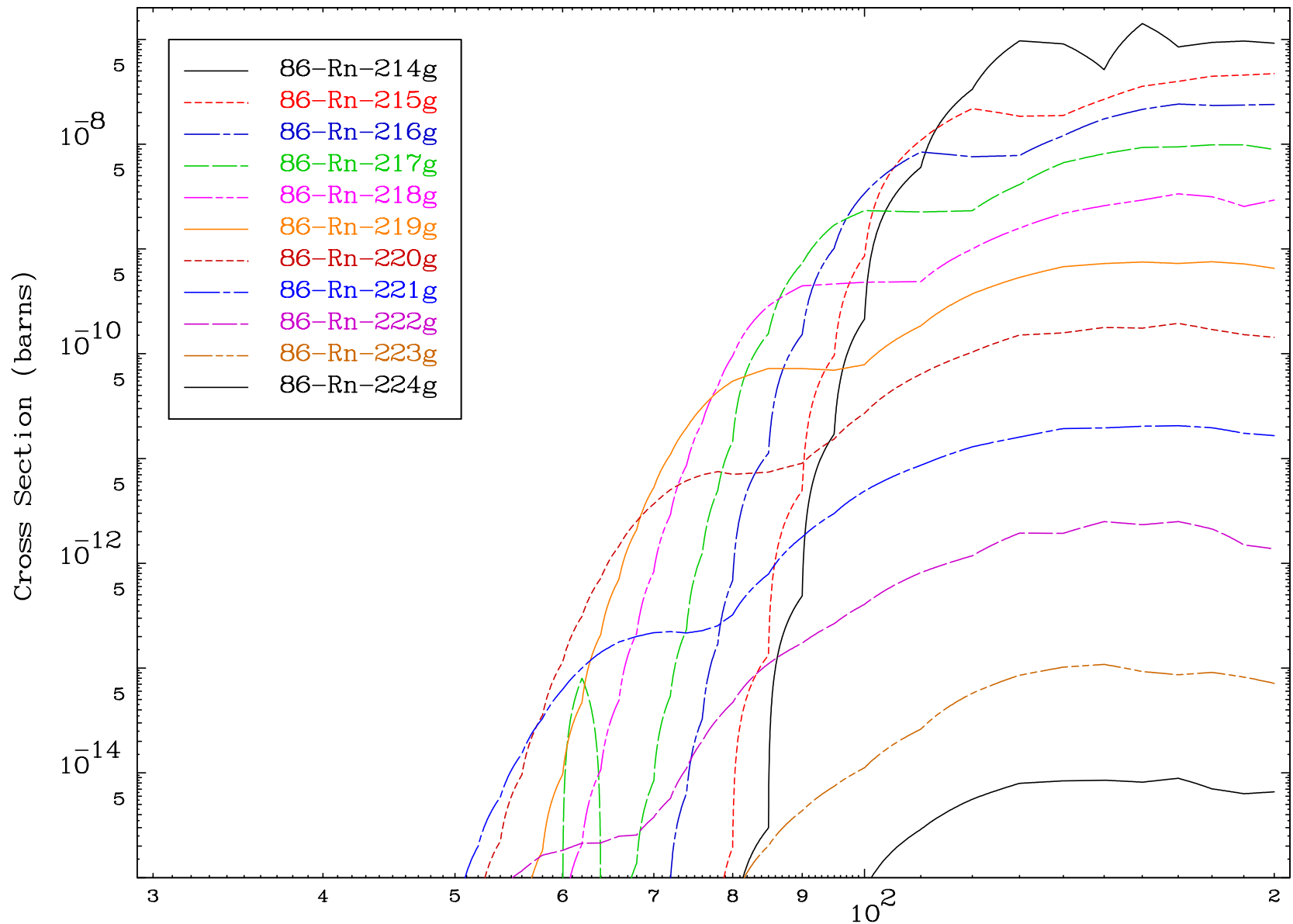
 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section



Radionuclide Production Cross Section

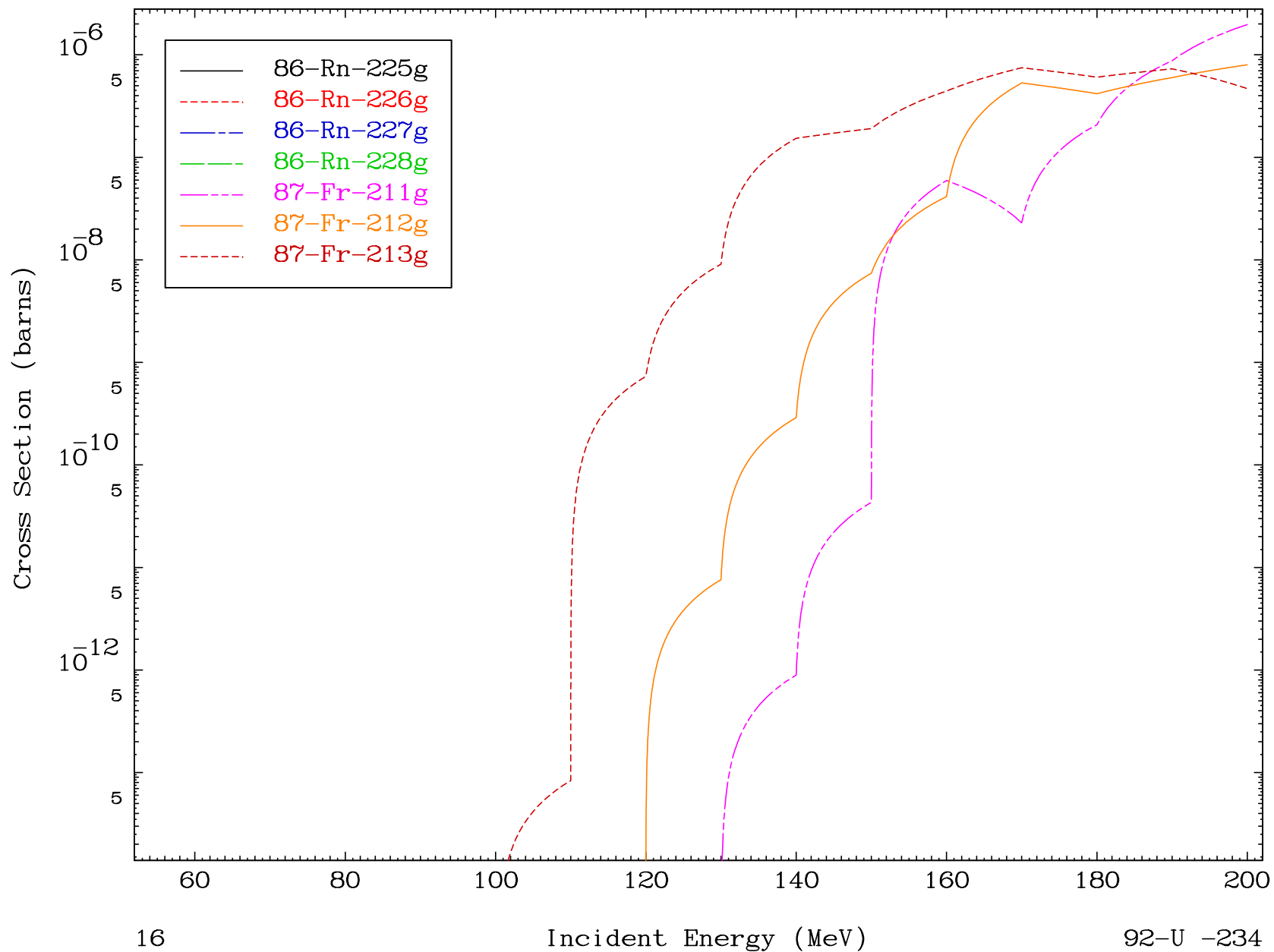


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

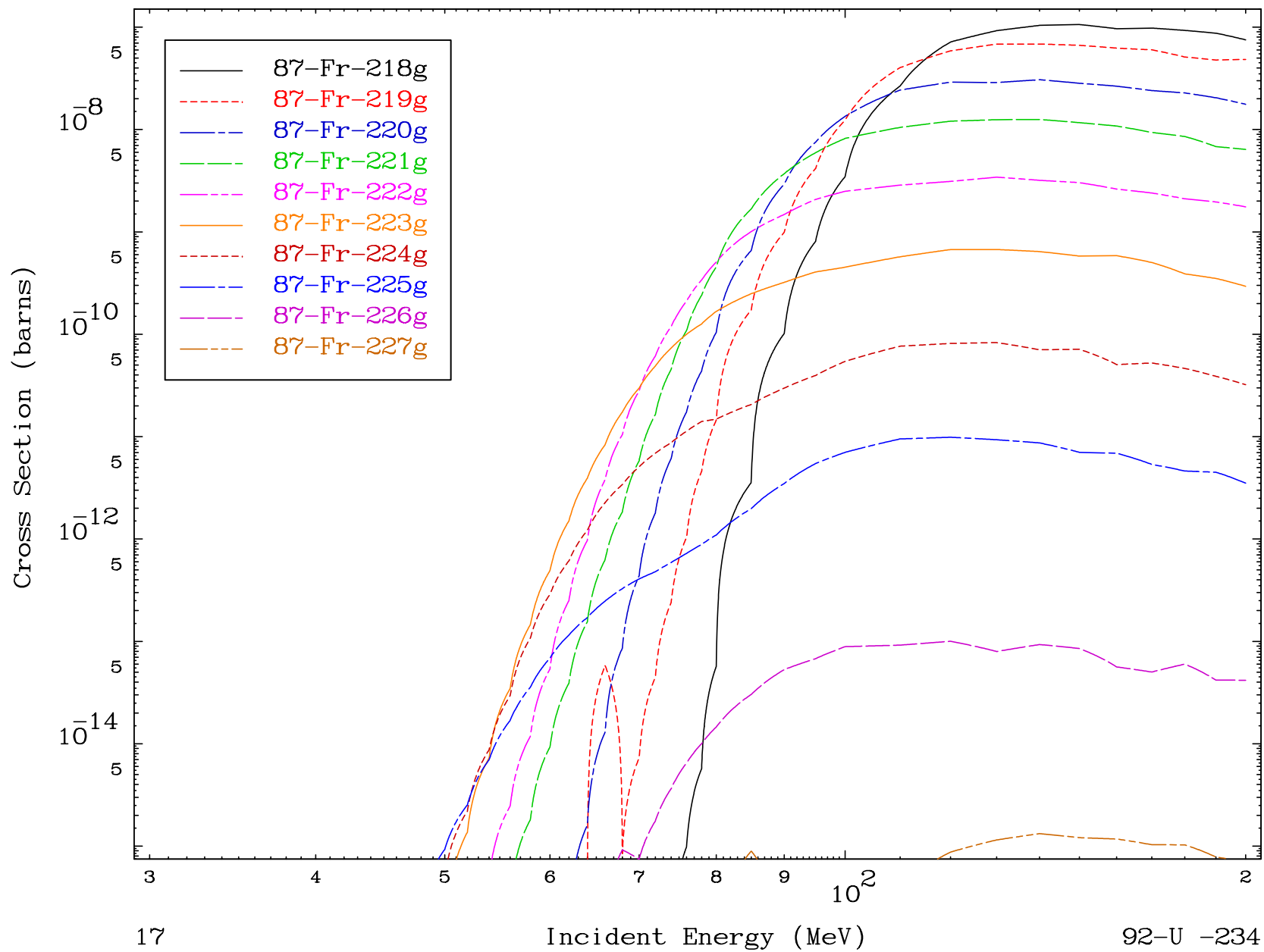


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

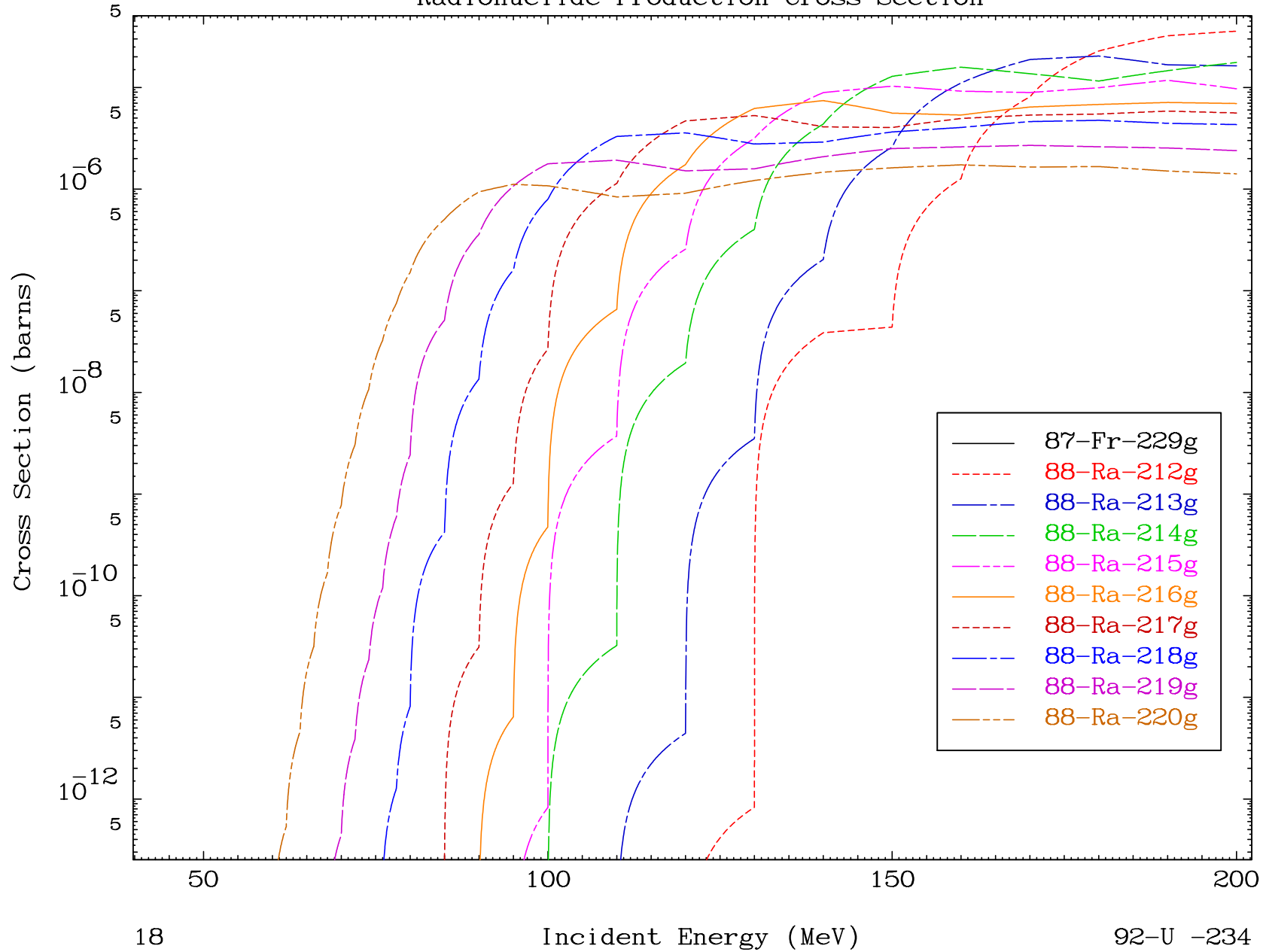


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

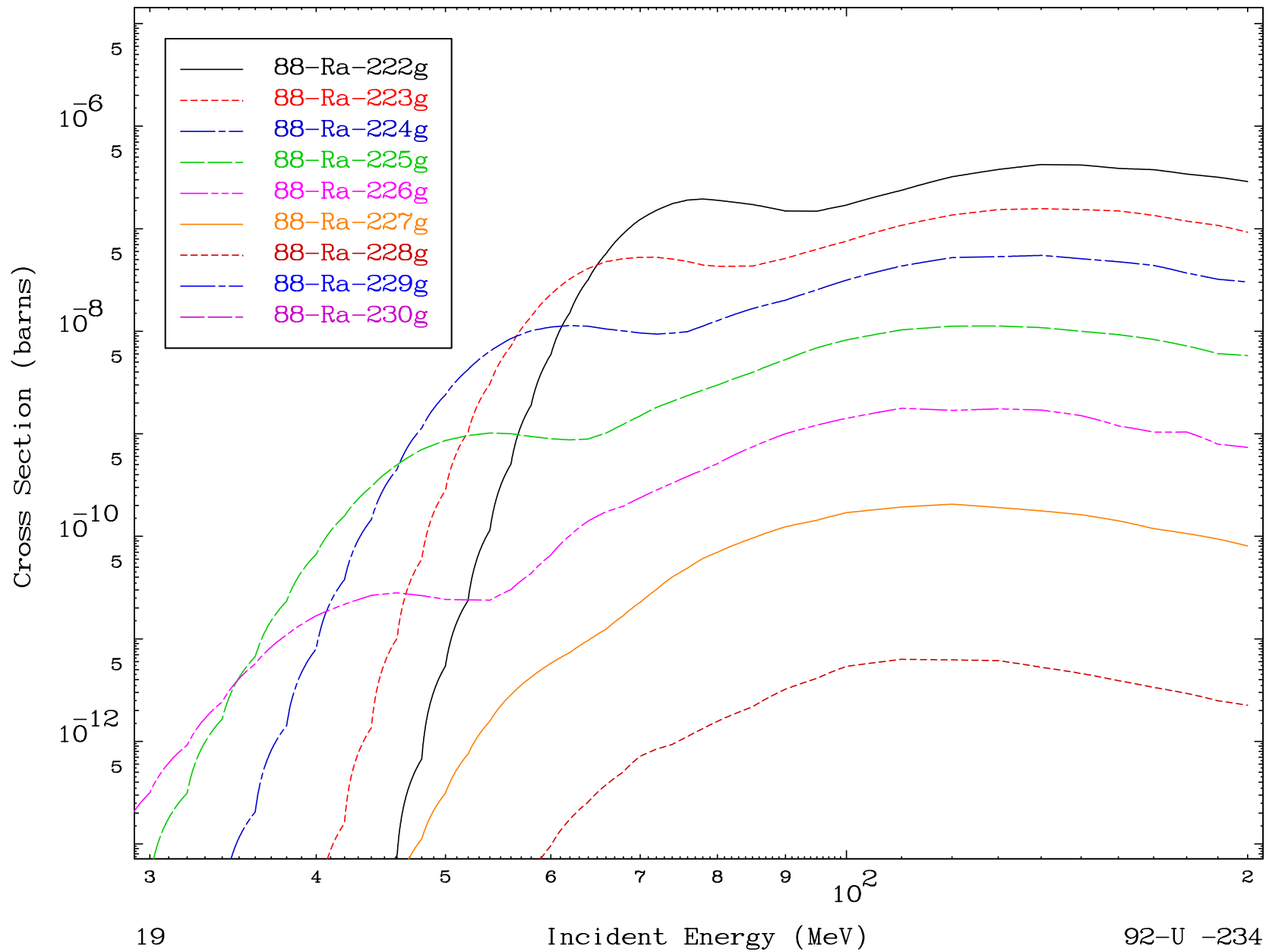


MAT 9225

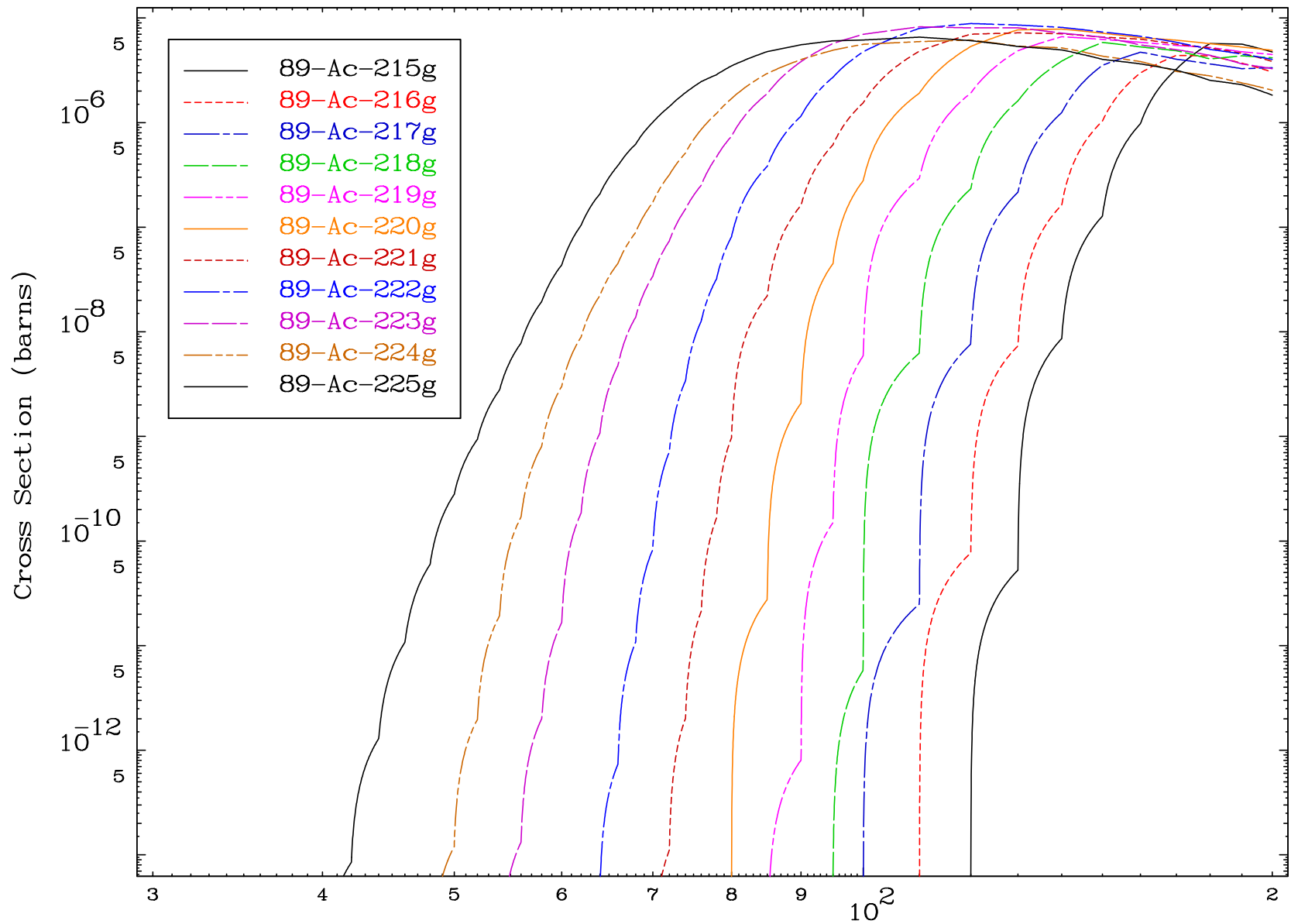
 $(\gamma, \text{remainder})$

92-U -234

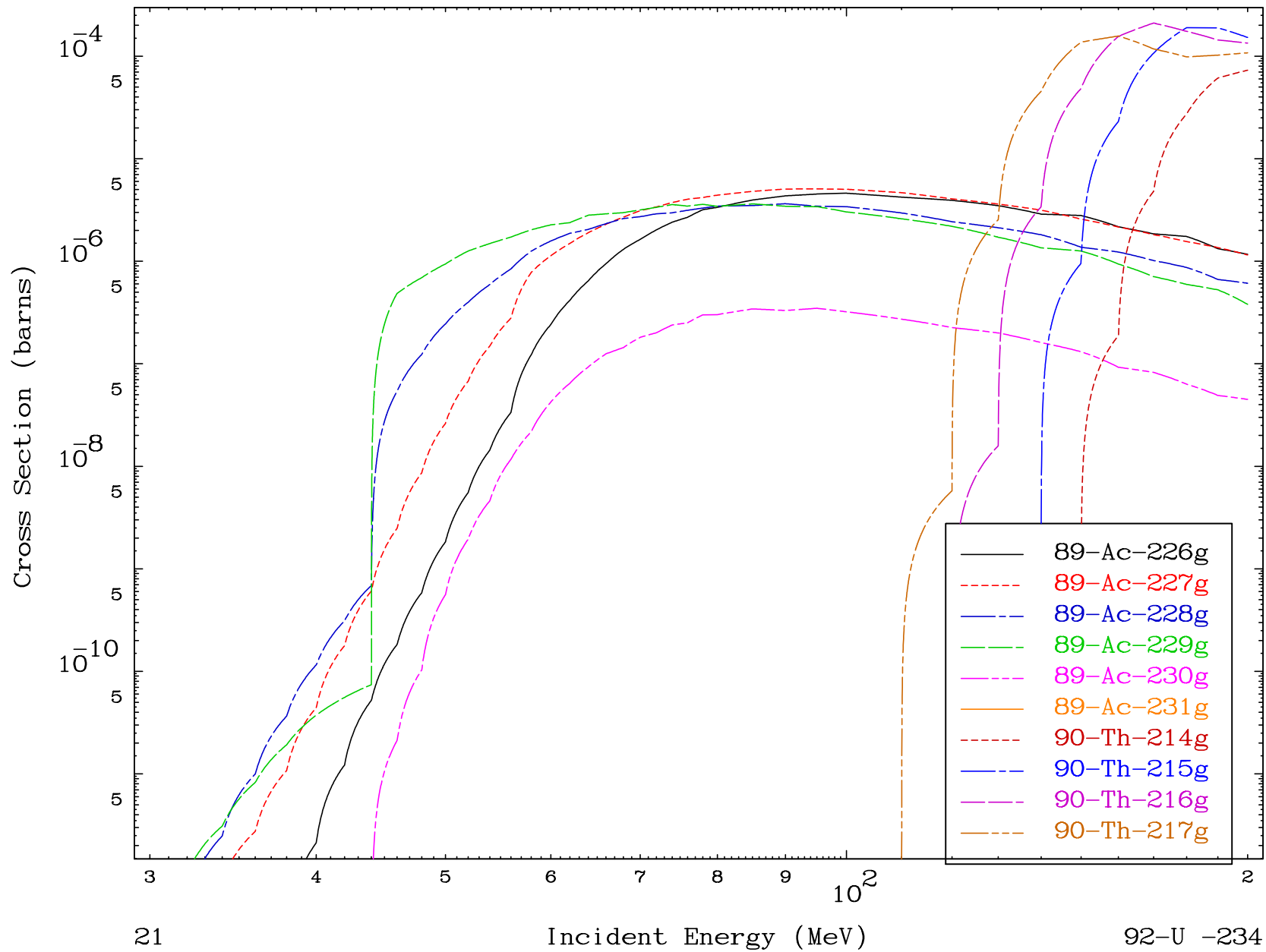
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

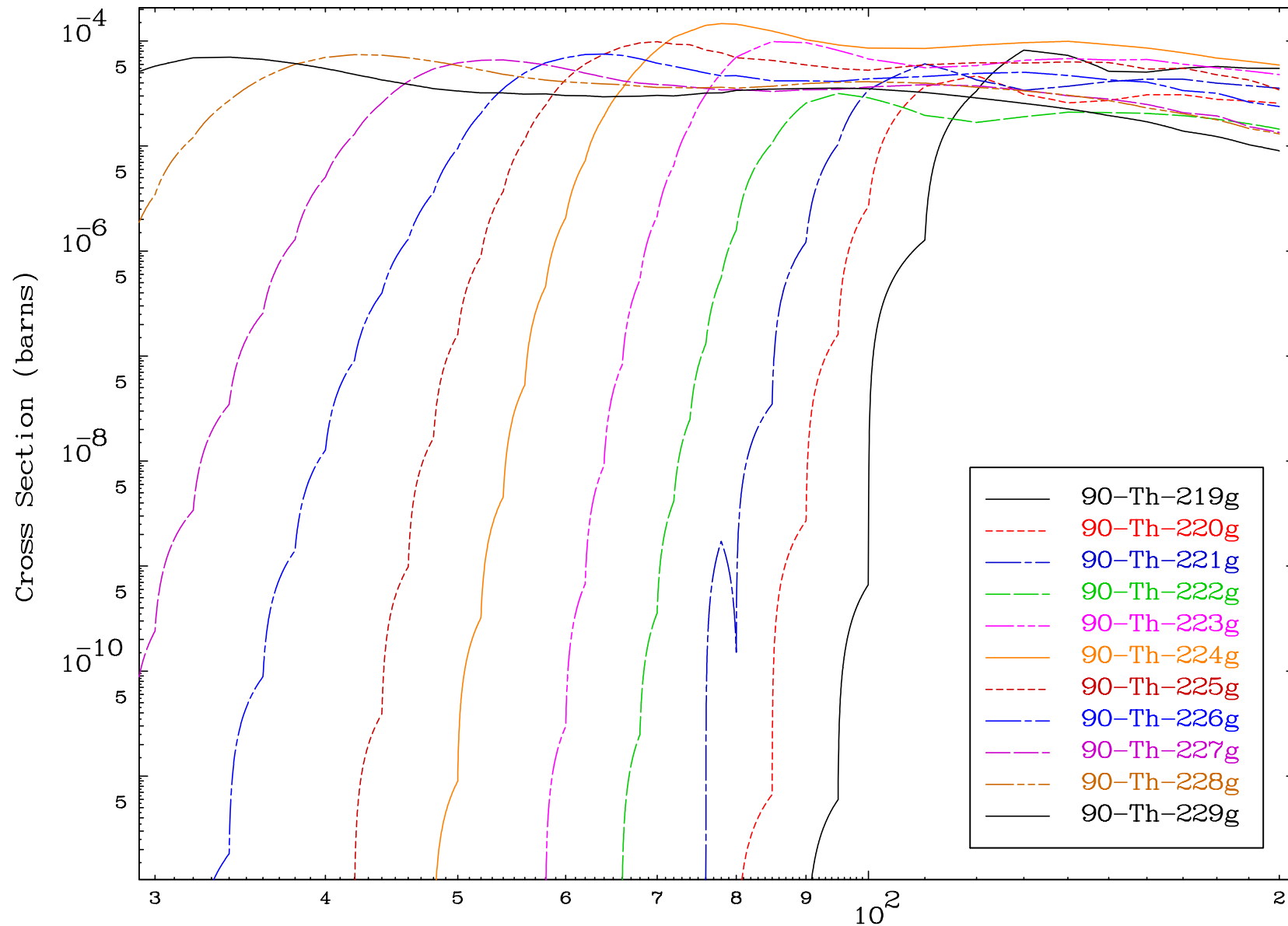


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section



22

Incident Energy (MeV)

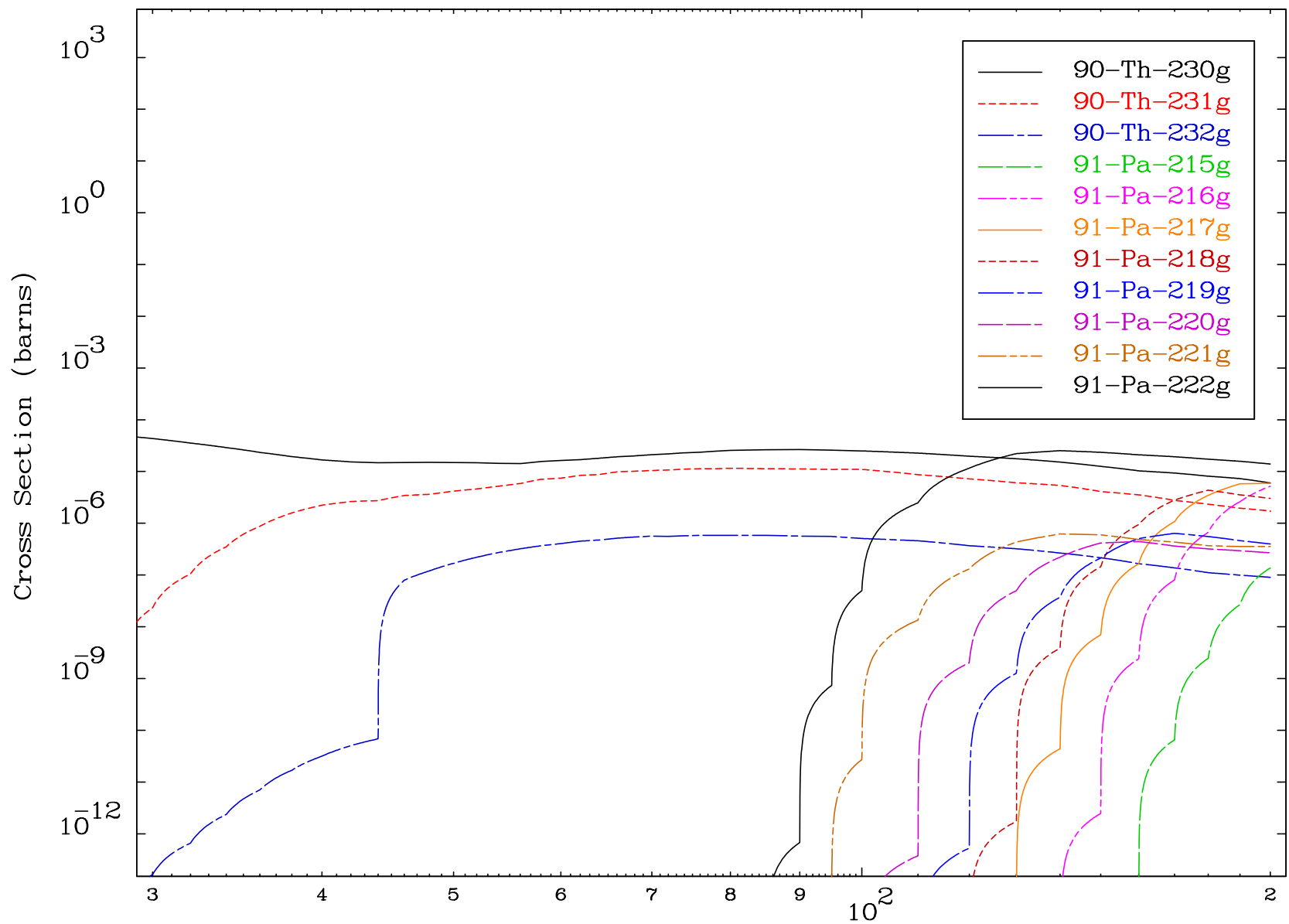
92-U -234

MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section



23

Incident Energy (MeV)

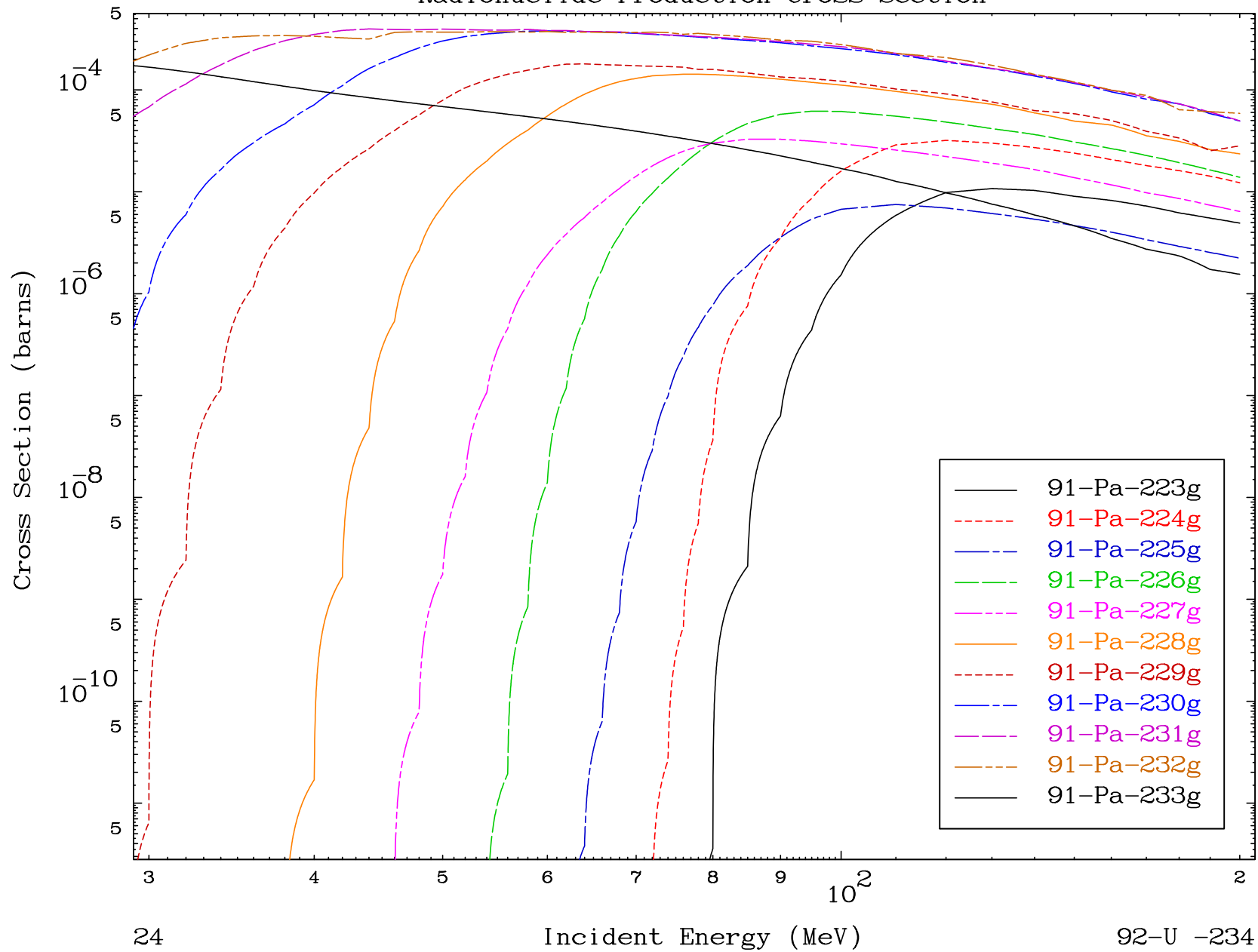
92-U -234

MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

Radionuclide Production Cross Section

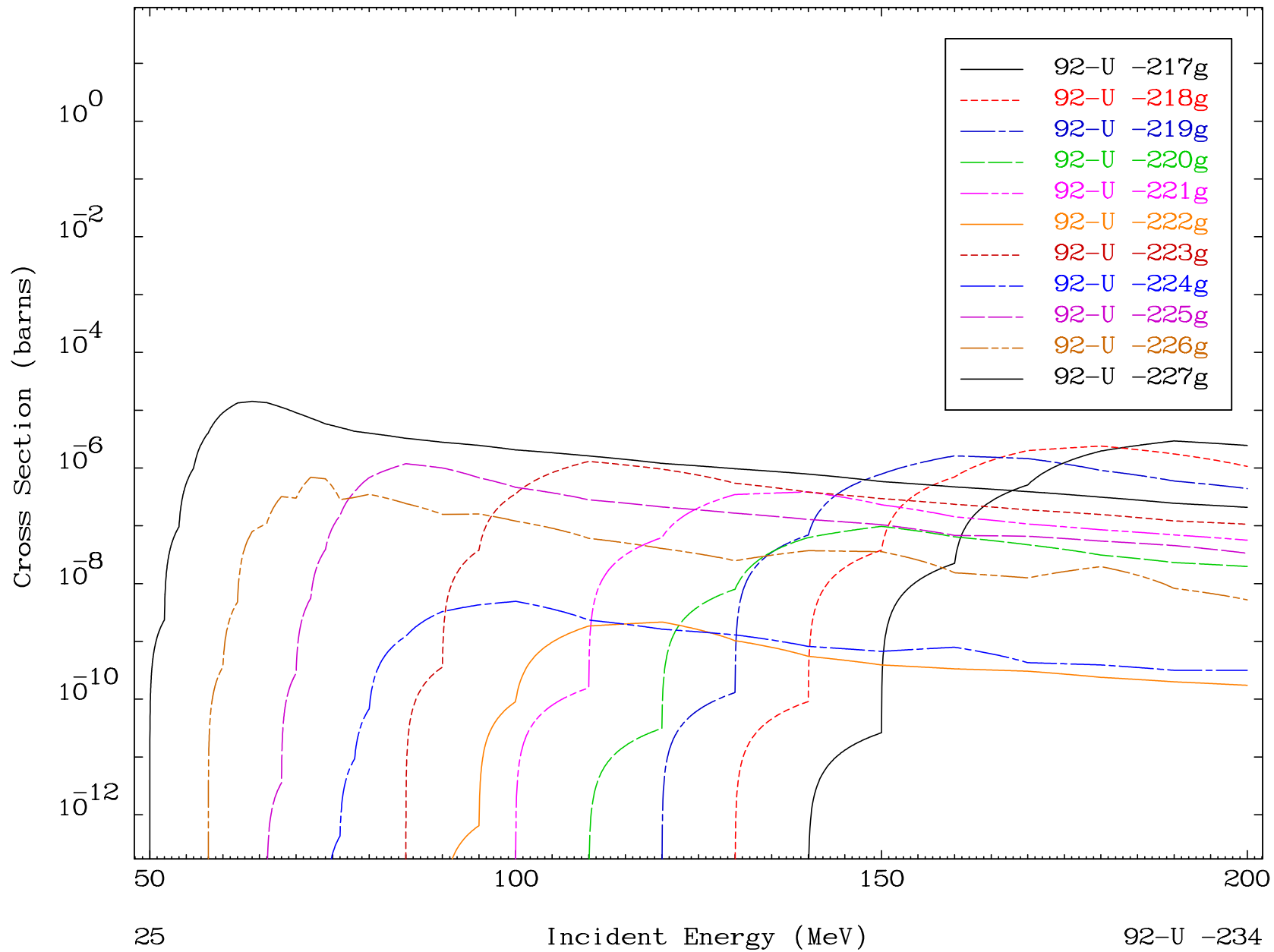


MAT 9225

 $(\gamma, \text{remainder})$

92-U -234

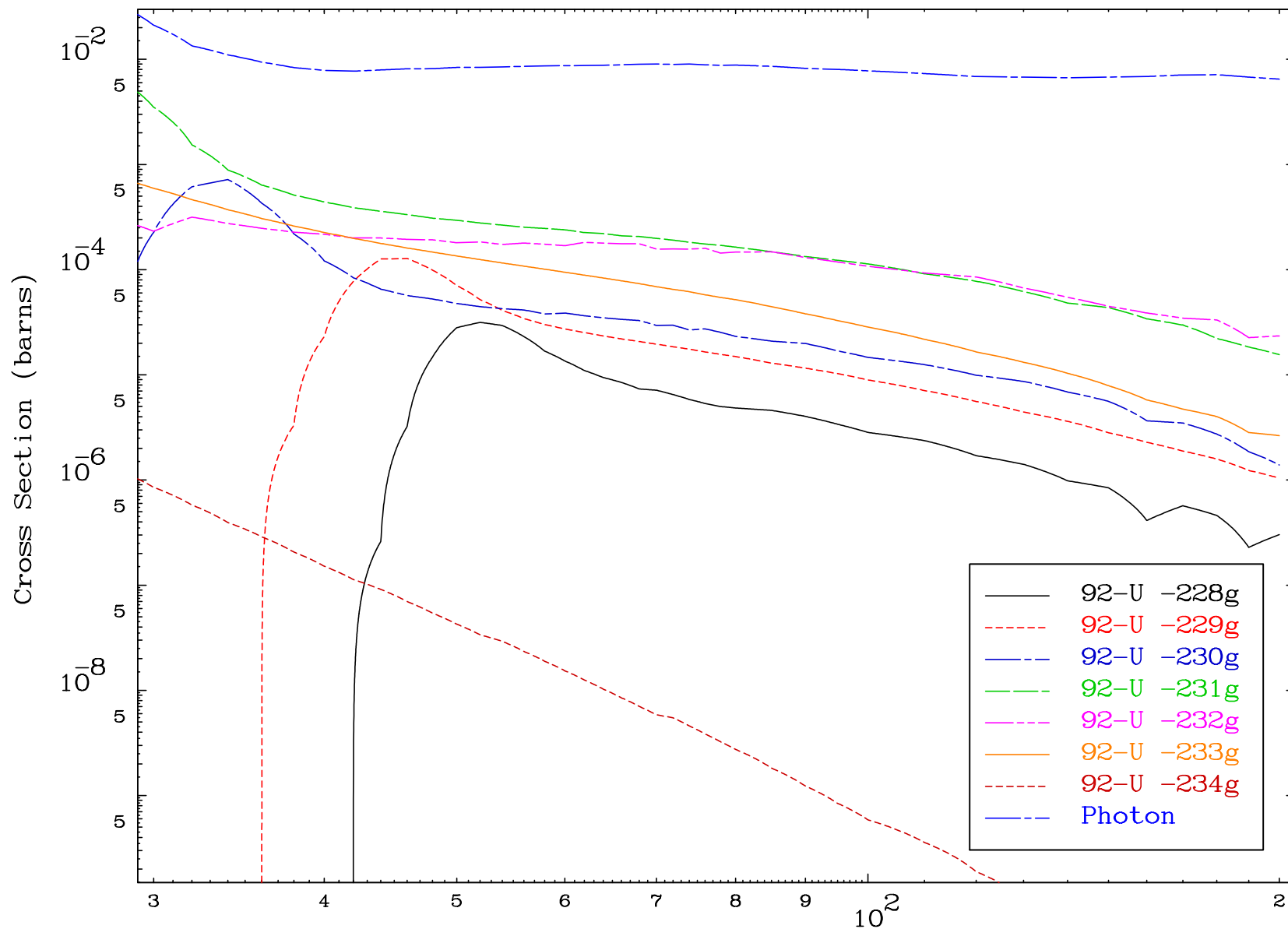
Radionuclide Production Cross Section



MAT 9225

(γ , remainder)
Radionuclide Production Cross Section

92-U -234



26

Incident Energy (MeV)

92-U -234