

**ENGR 55500/G5300 REACTOR THERMAL-HYDRAULICS**

**Assignment #1** (due February 26, 2024)

- To build a containment wall for a nuclear reactor, concrete has been poured to form a 1.2m thick slab. The hydration of the concrete results in the equivalent of a constant heat source of  $q_o''' = 100 \text{ W/m}^3$ . If both surfaces of the concrete slab are kept at  $16^\circ\text{C}$ , determine the maximum temperature,  $T_{\max}$ , that would be reached, assuming a steady state condition. The thermal conductivity of the wet concrete may be taken as  $0.84 \text{ W/mK}$ .
- A nuclear fuel element of thickness,  $2L$ , is covered with a steel cladding of thickness  $b$ . Heat generated within the nuclear fuel at a rate  $q_o''' (\text{W/m}^3)$  is removed by a coolant at  $T_\infty$ , which flows past the surface at  $x = L+b$  and is characterized by a heat transfer coefficient  $h$ . The other surface at  $x = -L-b$  is well insulated, and the fuel and steel have thermal conductivities of  $k_f$  and  $k_s$ , respectively.
  - Obtain an expression for the temperature distribution  $T(x)$  in the nuclear fuel. Express your answer in terms of  $q_o'''$ ,  $k_f$ ,  $L$ ,  $b$ ,  $k_s$ ,  $h$  and  $T_\infty$ .
  - Sketch the temperature distribution  $T(x)$  for the entire system from  $x = -L-b$  to  $x = L+b$ . At what  $x$  does the maximum temperature occur?
- Consider steady, one-dimensional heat conduction in a solid wall of thickness  $15 \text{ cm}$  without any internal heat generation. The thermal conductivity is not constant and varies with temperature as  $k = 2.0 + 0.005T (\text{W/mK})$ , where  $T$  is in degrees Kelvin. If one surface of this wall is maintained at  $150^\circ\text{C}$  and the other at  $50^\circ\text{C}$ , determine the rate of heat conduction per square meter ( $\text{W/m}^2$ ). Sketch the temperature distribution through the wall. Is it linear or non-linear?
- Consider a shielding wall of thickness  $L$  for a nuclear reactor. The wall receives gamma-rays such that heat is generated within the wall according to the relation,
 
$$q''' = q_o''' e^{-\mu x}$$
 where  $q_o'''$  is incident radiation flux (constant),  $\mu$  is a gamma attenuation coefficient, and  $x$  is the distance from the inner surface. Using this relation, derive expressions for the temperature distribution in the wall,
  - if both the inner and outer temperatures are maintained at  $T_i$  at  $x = 0$  and  $x = L$ ,
  - if the inner and outer temperatures are maintained at  $T_i$  at  $x = 0$  and at  $T_o$  at  $x = L$ , respectively.
  - For the temperature distribution obtained in (b), at what distance from the inner surface would the temperature be at a maximum?
- Consider a nuclear fuel shaped as a long slab of thickness,  $7.5 \text{ cm}$ , and with  $k = 12 \text{ W/m}^\circ\text{C}$ . The fuel generates heat internally at a rate of  $10^5 \text{ W/m}^3$ . One side of the wall at  $x = 0$  is insulated and the other side at  $x = 7.5 \text{ cm}$  is exposed to a convection environment with a heat transfer coefficient of  $h = 500 \text{ W/m}^2^\circ\text{C}$  and a fluid temperature of  $90^\circ\text{C}$ . Determine the temperature profile in the slab and calculate the maximum temperature in  $^\circ\text{C}$  assuming steady one-dimensional heat conduction.

- For the square solid without any heat generation shown on the right, numerically solve the steady state 2-D heat conduction equation for the temperatures  $T_1 - T_4$ . Thermal conductivity is  $k = 1.5 \text{ W/mK}$  and each square mesh is  $2 \text{ cm}$  wide. The four surfaces are kept at constant temperatures as shown.

