



# Comparison Between Continuous and Batchwise Online Reprocessing in Serpent2



Luke Seifert, Madicken Munk

University of Illinois at Urbana-Champaign, Department of Nuclear, Plasma, and Radiological Engineering, Urbana, IL 61801

## Introduction

### Molten Salt Reactor Online Reprocessing

- Depletion of Molten Salt Reactors requires accounting for reprocessing
- Batchwise modeling of Molten Salt Reactors is common [3, 2]
- Continuous modeling offers unique advantages over batchwise modeling

### Comparison of Methods

- An identical toy model is implemented for both methods
- Continuous model uses varying number of steps
- Multiple approaches are implemented for the continuous model



Figure 1: Geometry of toy model used in serpent2 for continuous and batchwise reprocessing models.

Table 1: Approach Acronyms and Descriptions

Approach	Description
SP pre	SaltProc steady batch pre-depletion
SP post	SaltProc steady batch post-depletion
CR	Cycle Rate continuous approach
SPCR	SaltProc Cycle Rate continuous approach
CTD	Cycle Time Decay continuous approach
CTRL	Control method (no reprocessing feeds or removal)

## Objectives

- Capture the precise differences in continuous and batchwise models
- Determine effective depletion step sizes for continuous reprocessing
- Investigate validity of using average feed rates during depletion

## Contact Information

- Email: seifert5@illinois.edu
- Phone: +1 865 279 0603

## Reprocessing Models

### Batchwise Reprocessing

- Iteratively perform depletion with external adjustments
- Steady Batch** uses small removals each depletion step.

$$X = \frac{1}{T_{cyc}} \quad (1)$$

- $T_{cyc}$  is the cycle time
- $X$  is the fractional removal rate
- Bulk Batch** uses full removal after a set number of depletion steps.
- SaltProc is used to run batchwise reprocessing for Serpent2
- The current version of SaltProc uses the Steady Batch method

Table 2: Batchwise Reprocessing Methods

Approach	Cycle Time	X [s <sup>-1</sup> ]	Step Removal
Bulk Batch [3d]	20 s	-	1
Bulk Batch [3d]	30 d	-	0*
Steady Batch [3d]	20 s	3.86E-6	1
Steady Batch [3d]	3 d	3.86E-6	1
Steady Batch [3d]	30 d	3.86E-7	0.1

\* Bulk removal occurs after 30 days, so the step fractional removal becomes 1 at 30 day step.

### Continuous Reprocessing

- Adds "decay-like" term to Bateman equation, less iterative [1]

$$\frac{dN_j}{dt_{base}} = \sum_{i \neq j} [(\gamma_{i \rightarrow j} \sigma_{f,i} \Phi + \lambda_{i \rightarrow j} + \sigma_{i \rightarrow j} \Phi) N_i] - (\lambda_j + \sigma_j \Phi) N_j \quad (2)$$

$$\frac{dN_j}{dt_{net}} = \frac{dN_j}{dt_{base}} - \lambda_{r,j} N_j + \sum_{mat} \lambda_{r,i \rightarrow j} N_i \quad (3)$$

The symbols given in the equations are defined as follows:

- $N_j$  is the atomic density of isotope  $j$ .
- $\gamma_{i \rightarrow j}$  is the fractional fission product yield of  $j$  in the fission of isotope  $i$ .
- $\sigma_{f,i}$  is the microscopic fission cross section of isotope  $i$ .
- $\Phi$  is the spectrum-averaged scalar flux in the fuel region.
- $\lambda_{i \rightarrow j}$  is the decay constant of decay  $i \rightarrow j$ .
- $\sigma_{i \rightarrow j}$  is the microscopic transmutation cross section of reaction  $i \rightarrow j$ .
- $N_i$  is the atomic density of isotope  $i$ .
- $\lambda_j$  is the decay constant of isotope  $j$ .
- $\lambda_{r,j}$  is the reprocessing constant for removal of isotope  $j$ .
- $\sigma_j$  is the microscopic total transmutation cross section of isotope  $j$ .
- $\lambda_{r,i \rightarrow j}$  is the reprocessing constant for feed of material  $i \rightarrow j$ .
- Cycle Time Decay** (CTD) model treats reprocessing as decay

$$\tau_{1/2} = \frac{1}{2} T_{cyc} \quad (4)$$

$$\lambda_r = \frac{\ln(2)}{\tau_{1/2}} \quad (5)$$

- Cycle Rate** (CR) treats as linear fractional removal, same as Steady Batch

$$\lambda_r = \ln \left( \frac{1}{1 - X} \right) \quad (6)$$

- SaltProc Cycle Rate** (SPCR) mimics batchwise reprocessing with continuous model

Table 3: Continuous Reprocessing Methods

Approach	Cycle Time	$\tau_{1/2}$	X [s <sup>-1</sup> ]	$\lambda_r$ [s <sup>-1</sup> ]
Cycle Time Decay	20 s	10 s	-	6.93E-2
Cycle Time Decay	3 d	1.5 d	-	5.35E-6
Cycle Rate	20 s	-	0.05	5.13E-2
Cycle Rate	3 d	-	3.86E-6	3.86E-6
SaltProc Cycle Rate	20 s	-	3.86E-6	3.86E-6
SaltProc Cycle Rate	3 d	-	3.86E-6	3.86E-6
SaltProc Cycle Rate	30 d	-	3.86E-7	3.86E-7

## Results

### Multiple Steps

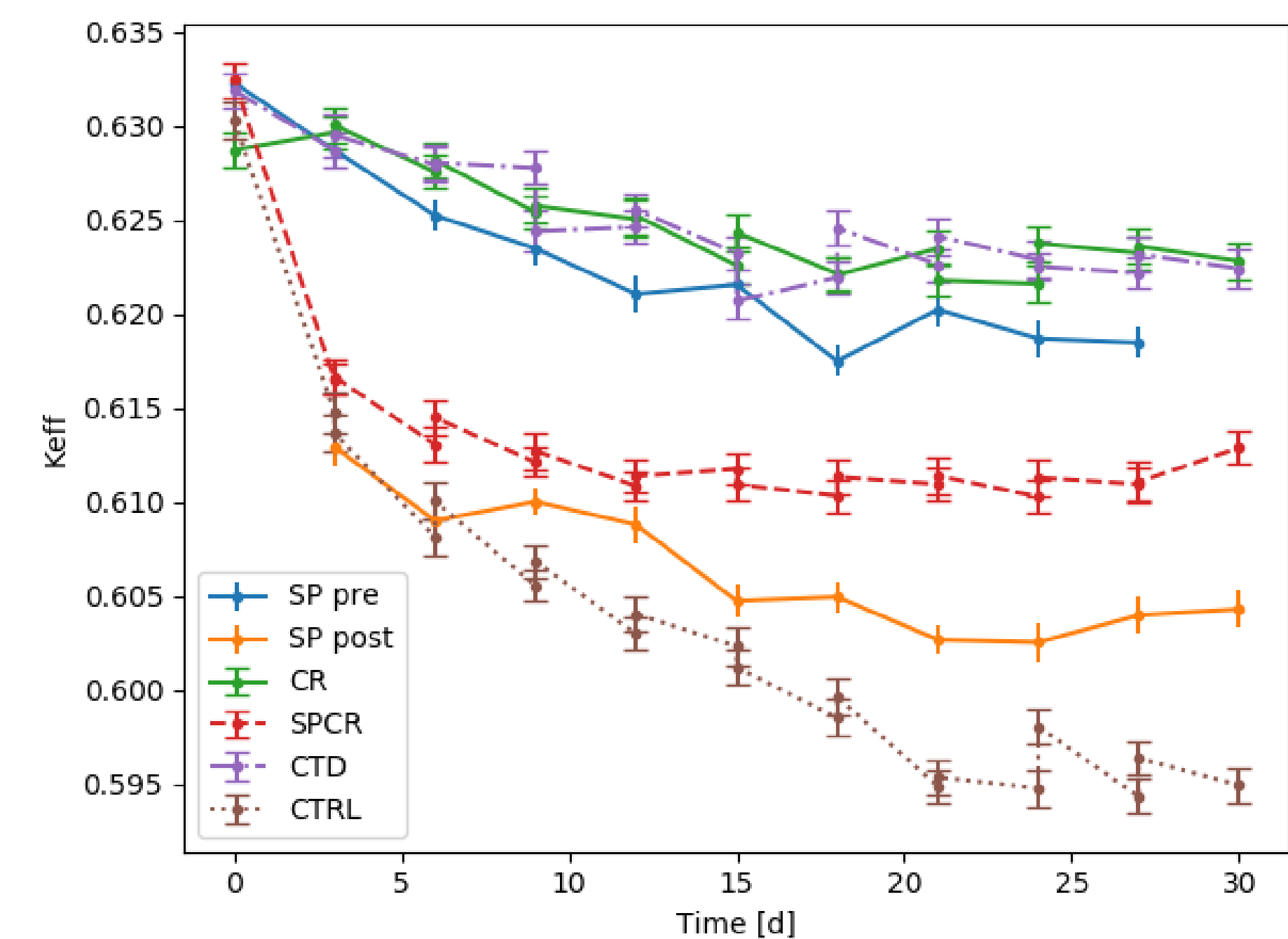


Figure 2: Continuous and batch models  $k_{eff}$  over time when using the matching depletion steps and feed rates.

### Single Step

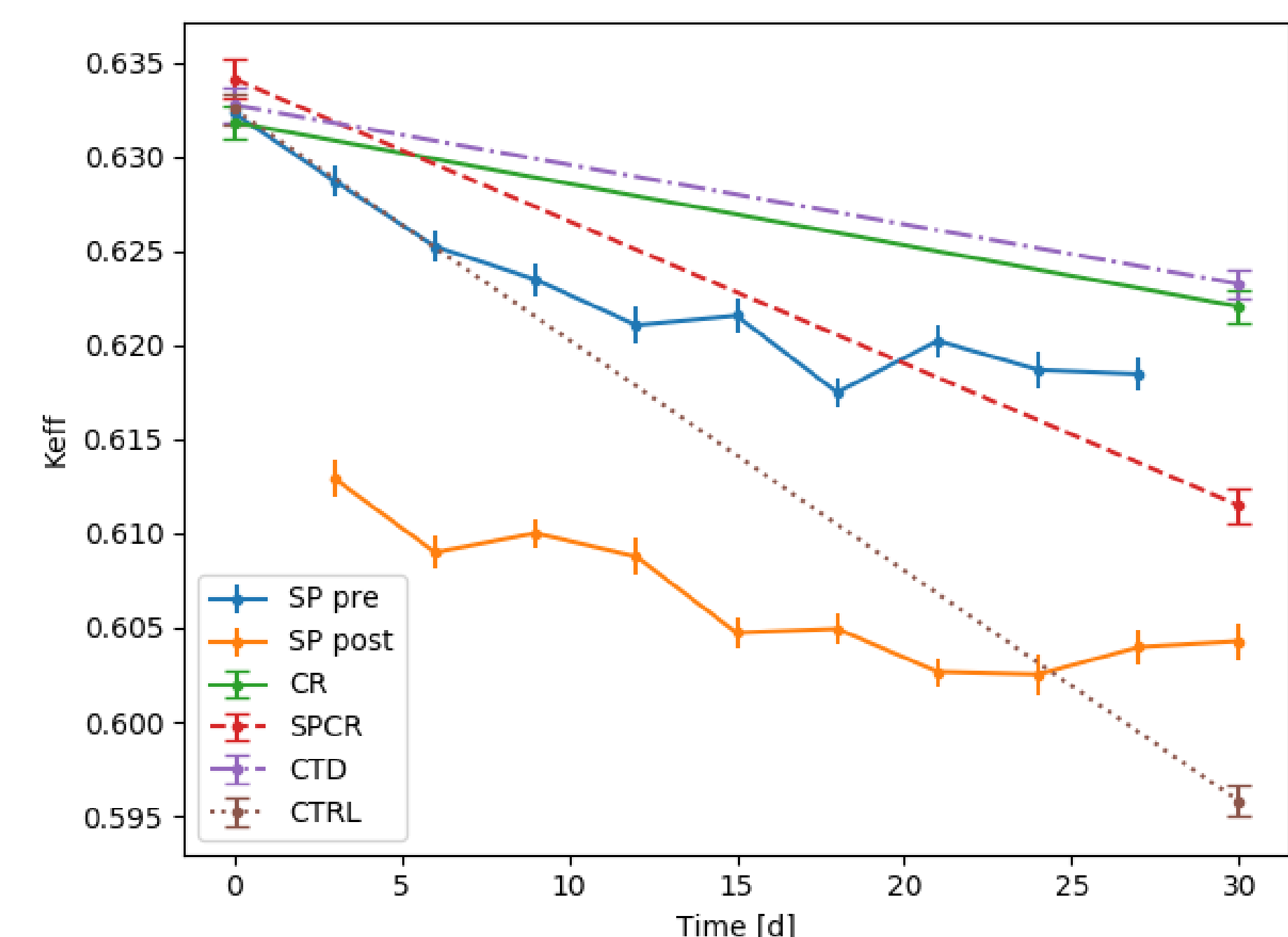


Figure 3: Continuous and batch models  $k_{eff}$  over time when continuous uses a single step and average feed rates.

## Overall Results

Table 4:  $k_{eff}$  at 30 Days for 3 and 30 Day Steps

Approach	3d Step $k_{eff}$	30d Step $k_{eff}$	Diff [pcm]
CR	0.622815	0.622043	77
SPCR	0.612871	0.611481	140
CTD	0.62241	0.623246	84
CTRL	0.594924	0.595784	86

- Continuous with same depletion steps results in higher  $k_{eff}$
- Step size order of magnitude increase and average feed rate usage resulted in 100 pcm difference in final  $k_{eff}$
- Step size increase reduces computational cost by a factor of 10
- Average feed rate use eliminates double running, making depletion twice as efficient

## Acknowledgements

This material is based upon work supported under an Integrated University Program Graduate Fellowship. The authors are grateful for this generous support.

The authors thank Professor Kathryn Huff for her contribution and support of this work in its early stages.

## References

- [1] M. Auferio, A. Cammi, C. Fiorina, J. LeppÄn, L. Luzzi, and M. Ricotti. An extended version of the SERPENT-2 code to investigate fuel burn-up and core material evolution of the Molten Salt Fast Reactor. *Journal of Nuclear Materials*, 441(1-3):473–486, Oct. 2013.
- [2] B. R. Betzler, J. J. Powers, and A. Worrall. Molten salt reactor neutronics and fuel cycle modeling and simulation with SCALE. *Annals of Nuclear Energy*, 101:489–503, Mar. 2017.
- [3] A. Rykhlevskii, J. W. Bae, and K. D. Huff. Modeling and simulation of online reprocessing in the thorium-fueled molten salt breeder reactor. *Annals of Nuclear Energy*, 128:366–379, June 2019.