

3

Plasma radiation feedback control

This should go to the introduction

On the way to a magnetically confined plasma fusion reactor it is of upmost importance to develop a strategy to remove as much energy from the scrape-off layer in order to minimize the thermal power load onto vessel components[67–71]. A decreased plasma flux across the separatrix is also favourable due to the reduction of erosion rates, tritium retention and neutron permeation for potentially all-metal plasma facing elements of such a reactor. Strong plasma power losses in the scrape-off layer lead to changes in the density and temperature profiles therein. The resulting transport along the open field lines at this point, i.e. the plasma exhaust, decreases significantly and therefore the load on limiting (divertor) elements is reduced[69]. A theoretical scenario to visualize could be the one proposed by Tenney et. al.[72]: the neutral pressure close to the divertor is assumed to be relatively high, while impurity radiation, i.e. from extrinsically seeded argon in this case, causes the plasma temperature to drop tremendously outside the separatrix along the divertor channel. The latter is defined as the collection of open magnetic field lines that cross the limiting surface. This is accompanied by a high particle recycling regime, including incident working gas ion backscattering, trapping, diffusion, re-emission and desorption at or close to the plasma boundary[73, 74]. In front of the divertor the recombination rate is increasing to the point where no plasma flux is expected to reach the wall due to the very low temperature, high neutral and low plasma pressure. This state is called *detachment* or *plasma detachment* and can also occur partially, where only the ion flux close to the separatrix is

Remove half tree
and high should be
for pumping

Vague

not relevant

note in W7X

would reduce recombination

a figure to visualize?

decreasing in contrast to the whole scrape-off layer[75]. One might also say that the limiting surface shifts inward and becomes a *pseudo wall* with steep increase in neutral pressure profile. For a significant plasma power exhaust in the SOL and subsequent profile changes, edge radiation from low- to medium-Z impurities, Z being the atomic number, might be necessary. This is being investigated ~~upon~~ in many magnetically confined plasma fusion experiments[69, 70]. Additionally, the feasibility of a divertor concept is characterised by its stable performance and resilience against thermal or radiative collapse in scenarios with plasma detachment. Among important features are a high efficiency of particle exhaust under conditions of reduced recycling flux. Furthermore, at high radiation power losses, impurities should be transported into and retained inside the scrape-off layer to prevent accumulation ~~in~~ of the core plasma. Finally, the most critical attribute and limit of such a divertor is its capability to ~~dissipate~~ power from the plasma and to ~~reduce the~~ dissipate this heat load ~~from the targets~~.

For the stellarator Wendelstein 7-AS, which, in comparison to W7-X, featured a similar magnetic configuration space at a smaller volume, major radius and rotational transform and therefore poloidal mode number of edge islands, EMC3-Eirene simulations indicate that the majority of intrinsic impurity radiation from carbon is located at the *X-points*. Those are intersections of inner and outer separatrices, in-between magnetic islands[76]. Around X-points the radial width of magnetic field lines is largest, which greatly reduces the power transport across the field lines and therefore enables condensation of radiation[68, 77]. Simulations of impurity radiation and transport of the scrape-off layer for Wendelstein 7-X indicate that the radiation is also located around X-points. Additionally, the separatrix impurity density and neutral particle pressure at the divertor depend on the island configuration, i.e. geometry and size. Calculations show an optimum of impurity radiation for the efficiency of recycling neutrals. Around 60% to 70% (from input) of impurity radiation power loss yield adequate particle and power exhaust. The divertor neutral pressure is expected to decrease, while the impurity density and radiation front shifts inward towards the separatrix for power loss fractions of 80% and above - as calculated before in f_{rad} by equation (2.22). It is assumed that detached, higher density scrape-off layers are operational at lower impurity concentrations and higher neutral pressures. For some configurations of SOL islands, the neutral particle recycling flux does not refuel the core plasma, which possibly offers an independent

~~Simple~~
Simple
~~Relevant~~
Relevant

Set the scene
for why these
things are important.

2)

→ You need a consistent story line. Don't jump:

- 1) Transport \rightarrow Transport (not really art...) \rightarrow Strong localization of heat flux
- 2) Target heat flux needs to be mitigated
- 3) Dissipation by radiation (low T for SOL, need / mitigate foreseen)
- 4) Control required for operation + reactor
- 5) Needs to be consistent with core performance (dilution) and particle exhaust (pumping). This state only and give references --- not your dogfores

Which?

control of both densities [67].

In order to achieve this desired level of plasma control in magnetically confined fusion devices, the external seeding of impurities is inevitable to increase the power exhaust in the scrape-off layer in a controlled manner [67, 69–71]. However, an unprompted, continuous feeding of the plasma with an extrinsic impurity will eventually corrupt the scrape-off layer and lead to a radiative collapse. An active control system to describe and inject low- to medium-Z impurities into the SOL of W7-X to increase the radiation power loss and exhaust, potentially achieving controlled plasma detachment, is needed. Similar systems that control the plasma density via high-pressure gas boxes have been used and verified in past experimental campaigns at the stellarator. They are installed inside the vessel about 20 cm from the last closed flux surface, where each of them can be operated independently with a standalone or predetermined mixture of working gases like H, He, Ar or Ne. Incidentally, plasma density control is one of the most important but equally evident requirements to the experimental exploration of a plasma fusion device [43, 78, 79].

The two-camera resistive bolometer system at W7-X provided a real time evaluation of the global radiative power loss for plasma feedback purposes during the last experimental campaign. Its application has been designed, tested, implemented and verified before and during the procedure of OP1.2b. A set of lines of sight has been used to estimate the radiation level for plasma control, with fast auxiliary gas fueling as an actuator. Actuator of the impurity seeding was the thermal helium beam diagnostic [81].

This chapter will introduce and characterise the real time bolometer feedback system. This also includes a brief introduction of the thermal helium beam and other connected diagnostics that have been designed to achieve plasma control and possibly stable detachment through seeding of impurities. After introducing the hardware and software setup of the system, laboratory tests for the performance of the feedback will be presented. Finally, the results of experiments incorporating plasma radiation feedback are shown, including a comparison to other feedback candidates and a discussion of potential advantages and disadvantages.

Systèmes

3.1. Configuration *this goes into the introduction!*

During the last experimental campaign, the stellarator Wendelstein 7-X was operated in island divertor configuration for the first time. The corresponding toroidal length of open field lines, which are pathways of heat flux to the wall and eventually end on plasma facing components, increased about ten-fold in comparison to the limiter scrape-off layer, which was explored during the very first W7-X startup campaign. Results of the first divertor experiment campaign, previous to the one mentioned above, presented target heat fluxes that promise stable operation at high density[82]. This operation phase also featured the first, as well as second and third, boronization or boronized wall in this machine. Improvement of wall conditions through boronization is considered a standard technique to reduce the extent of intrinsic impurities, most prominently carbon and oxygen, from plasma facing components into the plasma[83–88]. A thin film of diboron (B_2) compounds is deposited onto plasma facing components, which are inert when exposed to molecular oxygen and do not show any oxidation in plasma environments. They are amorphous, transparent and kinetically hard. Boronized walls show overall lower chemical reaction rates, while retaining oxygen and metal to a much better degree[84]. Each of the three performed boronisations led to a significant reduction of intrinsic impurity radiation from carbon and oxygen[89].

Hereafter presented experiments took place after the aforementioned boronisations and were performed in magnetic stand~~and~~ ^{up} high mirror configurations with a rotational transform $\iota = 5/5$ [91, 90]. Five individual island bands wrap around and enclose the magnetically confined plasma region in toroidal direction. The magnetic island chains circle helically between upper and lower divertor elements around the torus, which cut through the enclosing field lines and corresponding separatrices in poloidal direction[92]. Figure 3.1 shows the assembly of the ten divertor elements. On the left, each colored band represents an island chain that revolves around the confined plasma region and cycles between upper and lower divertors in consecutive half-modules[93]. The right shows the five half modules, which each contain an upper and lower divertor, HM10/1 through HM50/1 respectively. Red and blue bands around the torus indicate traced field lines that originate and terminate in target elements, where the thermal helium beam box is located and gas is injected into the scrape-off layer. Both traces

3.1. Configuration

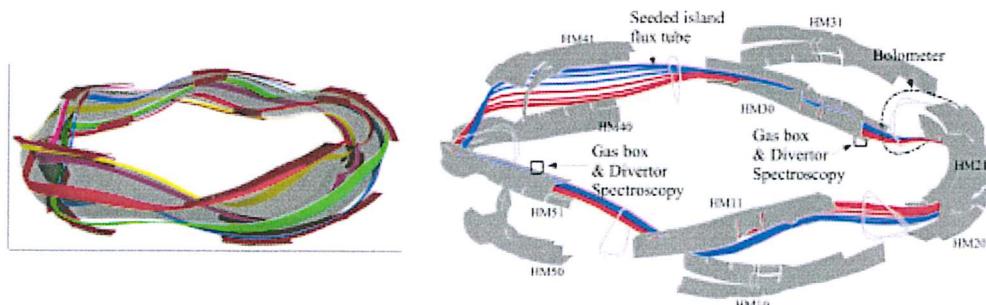


Figure 3.1.: (left): Standard magnetic configuration last closed fluxsurface (transparent, gray) inside ten divertor modules assembly (dark red). Five fluxtubes that helically revolve around the core plasma and follow the island chains in front of the divertors included (red, yellow, purple, green, blue)[93]. (right): Ten divertor elements (half-module HM 1-5, up (1) and down (0)) and helium beam gas boxes with positions of bolometry system (108° toroidally) and field lines that connect at the seeding points (red, blue)[91]

have a target-to-target connection length in the order of $\mathcal{O}(100\text{--}1000\text{ m})$ and follow field lines from one (~~upstream~~) divertor, in clockwise (red) and counter-clockwise (blue) direction, until they reach another (~~downstream~~) ~~to a downstream~~ target[91].

Thermal helium beam diagnostic

The line ratio spectroscopy of helium is used to measure electron density and temperature in the scrape-off layer and plasma edge. Due to the intrinsic nature of the fully optimized stellarator W7-X, ~~very~~ low temperatures and densities in divertor regions, especially under conditions of plasma detachment, are expected. Therefore, the spectroscopy is extended by ~~seding~~ ^{??} seeding of extrinsic impurities through thermal gas injection valves. These low-Z impurities include ~~helium, neon, argon, methane (CH_4) and mixtures thereof.~~ ^{using molecular He-seeding} Due to their lower excitation energies, the yield of line radiation at low temperatures is much better[94]. Electron temperature T_e and density n_e can be derived from the ratio of line radiation intensities of atomic helium, which are based off ~~of~~ calculations for emission rate coefficients from a collisional-radiative model.[81, 95]. The thermal helium beam diagnostic consists of

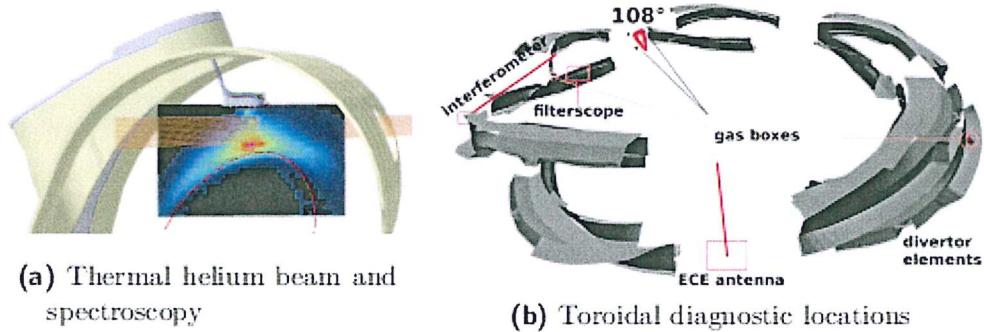


Figure 3.2: (a): Gas injection box at the upper plasma edge with all 5 capillary nozzles as used for divertor operation. Included are spectroscopy lines of sight (orange) perpendicular to the beam and the last closed flux surface (red). Superimposed is the result of one EMC3-EIRENE for He line emission[94]. (b): Relative toroidal position of all involved diagnostics. One filterscope system is located in the same divertor as one of the gas valve boxes, while interferometer and electron cyclotron measurements are done in different locations.

two *in vacuo* boxes at different toroidal locations in one upper lower divertor element. Each is equipped with five, millisecond fast piezo valves that allow for independent gas injection of He, Ar, Ne, N₂, H₂ or CH₄. The nozzles on the gas lines are mounted behind the target elements and directly seed into the same island chain of the $\iota = 5/5$ magnetic standard configuration. Line pressures range from a few mbar to a maximum of 60 bar, where the latter is used for seeding experiments, at around 1.5 km/s exit velocity and an opening angle of 40°.[81, 94]

However, the measurement of electron density and temperature in the divertor region by line ratio spectroscopy is of no concern for the plasma feedback control. The fast acting gas valves of the thermal helium beam diagnostic will be used as an actuator for the injection of extrinsic impurities. Other diagnostics provide experimental information that will be used for the feedback system, which ultimately controls the gas boxes. One of those diagnostics is the dispersion interferometry that provides line integrated core plasma densities in real-time.

| Then why discuss it?!

3.1. Configuration

Dispersion interferometry

Interferometry is a fundamental measurement technique to assess the line integrated plasma density. Due to its simple methodology, it is a prime candidate for reliable real-time feedback[96, 97]. An easily tunable laser is split via a frequency doubling crystal. Both beams pass the plasma co-linearly. Because of their differing group velocities, i.e. dispersion of the plasma with its density dependent refractive index, both beams experience different phase shifts based on the plasma density along their path. After a second pass through the crystal, their interference image can be used to calculate the line integrated density. The disadvantages of this generally very robust measurement method are phase shifts that are introduced via perturbations perpendicular to the beam direction and potentially unfavourable SNR due to both reference and measurement beam passing the plasma[98]. For the single channel integral electron density dispersion interferometer (IEDDI), a real-time evaluation system based on FPGAs has been implemented. This assembly is capable of providing density measurements with a 23 µs latency, which is dominated by the laser diodes modulation. This has a maximum datarate of 50 kHz. However, the density feedback frequency is mandated by the internal W7-X system at 1 kHz. The latter is used for feedback by the central stellarator control system and plasma fuelling[99]. The density data is forwarded with an error of $2 \times 10^{18}/\text{m}^2$ and signal drift of $6.7 \times 10^{15}/(\text{m}^3 \text{s})$ from the last performed laser and phase calibration[80].

Density feedback... Why relevant for Proj 2?

Besides the dispersion interferometry, there are two other possible candidates for thermal helium beam feedback. One of them is the spectroscopic imaging system, which measures impurity heat and particle fluxes from the plasma onto the divertor target. It can provide spatial-temporally resolved impurity line radiation, e.g. of carbon species like C³⁺, for feedback purposes.

only FS

+ spectroscopic
line measurement
IR: heat flux
FS: impurity line
classification

Filterscope and IR-visible camera system

are cameras used at all?

Two divertor targets have been under special investigation by the spectroscopic imaging system at W7-X in past experimental campaigns[100]. In one toroidal position, a combination of high resolution visible-infrared cameras and filterscope systems were able to provide the totally calibrated, spatially

and temporally resolved line radiation intensity of H_{α} , H_{β} , He-I, He-II, C-II and C-III from and in front of the target. The filterscopes at W7-X are based on the design from the *Oak Ridge National Laboratory*¹. Lenses located outside the vacuum vessel focus light from the plasma and divertor onto a linear array of optical fibers. At focus lengths of 25 mm and 50 mm, the spatial resolution at the target becomes 3.6 cm and 7.1 cm, respectively. Those fibers are connected outside the machine to individual optical bandpass filters, which are attached to photomultiplier tubes (PMT), whose full-width at half maximum is 1–3 nm for the above emission lines. Data acquisition with a temporal resolution of 100 kHz is taken care of by a LabVIEW® control program. The filterscopes provided measurements of H_{α} , H_{β} , He-I, He-II, C-II lines and visible bremsstrahlung[100]. For feedback purposes, the filterscope system has to provide the accurate photon flux as the brightness of the scrape-off layer changes, which challenges the PMT voltage control. In certain scenarios, relative errors as high as 20% after gain normalization can occur. For fast plasma transitions with large increments in emission brightness, occasional clipping of the filterscope signal is unavoidable[101, 102].

The infrared-visible imaging system is based on the design from the *Los Alamos National Laboratory*¹. It is located outside the vacuum vessel as well, while looking at the divertor plasma from 1.5 m away through a 21.5 cm diameter, 6 mm thick sapphire vacuum window. The infrared camera is acquiring between wavelengths of 3–5 μm at a 125 Hz acquisition rate of 1344×784 pixel resolution images. The calibration of the camera was performed with a 50 mm or 200 mm lens for temperatures of up to 1200 °C. In contrast, the visible system is acquiring 1024 pixel square images at 100 Hz with only a 50 mm lens. Both cameras achieve a 1 mm spatial divertor resolution[103].

I think we only fueled with H, didn't we?

After characterising the dispersion interferometry and filterscopes as candidates to control the impurity seeding, one additional constraint to the performance of the thermal helium beam valve box was made. The central line integrated electron temperature needs to be above 2 keV in order to

¹ORNL: Oak Ridge National Laboratory, Oak Ridge, Tennessee, United States of America; 37831-8072

¹LANL: Los Alamos National Laboratory, Los Alamos, New Mexico, United States of America

the predictive value $P_{\text{pred}}^{(1)} \approx P_{\text{rad}}$ the full set of information. An equivalent requirement is the adequate description of the local emissivity distribution by the set of lines of sight S , though this abstract concept is difficult to quantitatively assess from the standpoint of the bolometer diagnostic alone. However, an attempt for a rigorous and thorough investigation of the accuracy and reliability of $P_{\text{rad},S}$ is made in the later section 5.1. The subset S of the total selection of lines of sight, which are available through the HBC and VBC arrays, needs to be much smaller than the main cameras in order to be a candidate for radiation feedback. A decreased amount of channels for calculation means less computational load and therefore promises feasibility for real-time application. A key distinction between P_{rad} and $P_{\text{pred}}^{(1)}$ is that the prediction has to be limited between 0–10 MW. less worky

$$P_{\text{pred}}^{(n)} \in (0, 10) \text{ MW} , \\ U_{AO,n} [\text{V}] = P_{\text{pred}}^{(n)} [\text{MW}] , \quad n = \{1, 2\}$$

On one hand, this is due to the analog output limitation of the hardware to a range of 0–10 V. On the other, radiation power loss estimations > 10 MW for a total possible heating power of 10 MW do not make sense for the aspired feedback system and its requirements. In equation (3.1), the analog output voltages that are used to provide feedback to the gas valve control are noted as $U_{AO,n}$, with $n = \{1, 2\}$.

The performance of the real-time radiation system, especially with respect to latency, is crucial to the achievement of stable plasma detachment through impurity seeding in the scrape-off layer. A calculation like in $P_{\text{pred}}^{(1)}$ takes a certain, non-negligible amount of computation cycles and time, which inevitably increases the latency of the feedback response, relative to the acquisition of the underlying samples. One way around this issue is to make further simplifications to the approach of equation (2.25) or equation (3.1). In steady state scenarios, with slow perturbations and small gradients in the radiation power distribution, let us assume:

$$P_M = F_M \left(\frac{d(\Delta \tilde{U}_M)}{dt} + f_M \Delta \tilde{U}_M \right) \approx F_M f_M \Delta \tilde{U}_M , \\ P_{\text{pred}}^{(2)} := \frac{V_{P,\text{tor}}}{V_M} \cdot \frac{P_M V_M}{K_M} = a_M \Delta \tilde{U}_M , \quad M \in (S_{\text{HBC}}, S_{\text{VBC}}) \quad (3.2)$$

This is separate
and should be
included in
a discussion
of the PID
controllers

3.2. Real-time radiation feedback system

*→ elaborate of reference
subsequent*

avoid poor ECRH coupling and therefore radiation collapses. The electron cyclotron diagnostic (ECE) is measuring line integrated T_e along a line of sight through the plasma core and along the separatrix, closer to the edge[104–106].

Finally, the real-time radiation feedback system, which provides information about the total radiative power loss of the plasma to the thermal helium beam, will be introduced.

3.2. Real-time radiation feedback system

The goal of the real-time feedback system is to provide an estimate of P_{rad} , as derived in equation (2.25), for each sample during data acquisition, with as small as possible latency to the underlying measurement. This means a calibrated, error and offset corrected, filtered absorber voltage $\Delta \tilde{U}_M$ needs to be calculated within one signal integration cycle. Assuming current results from the *in-situ* calibration routine are available, one is able to find the power onto the absorber for every point in time P_M , as described by equation (2.19). In order to calculate a total radiation power loss in real-time from P_M , information about the line of sight geometry have to be provided via K_M and V_M . For an extrapolation of P_{rad} similar to equation (2.25), the calculation of geometric coefficients is too computationally expensive and can not be performed during the data acquisition. Therefore, K_M and V_M have to be provided beforehand, limiting their specificity towards the experiment scenario, e.g. in regards to the magnetic configuration. Finally, due to computational limits, P_{rad} of the full HBC or VBC arrays can not be calculated within one sample period. Simplifications to equation (2.25) have to be made to achieve a real-time proxy of the radiation power loss. Assume there exists a collection of lines of sight S that measure the radiation power loss from the plasma, so that

$$V_S = \sum_M^S V_M ,$$

$$P_{\text{pred}}^{(1)} := P_{\text{rad},S} = \frac{V_{\text{P,tor}}}{V_S} \sum_M^S \frac{P_M V_M}{K_M} , \quad S \subset (S_{\text{HBC}}, S_{\text{VBC}}) \quad (3.1)$$

3.2. Real-time radiation feedback system

Equation 3.2 yields a different approach to the radiation power loss proxy. Assumptions for $P_{\text{pred}}^{(2)}$ require $d\Delta\tilde{U}_M/dt \ll f_M \Delta\tilde{U}_M$ and therefore temporal changes in the radiation distribution to be negligible. This saves additional computational steps by eliminating the derivative. Furthermore, removing the sum in equation (3.1) assumes the radiation to be located mainly in or around one radial position, i.e. one point in the chord brightness profile, which is symmetrically distributed in poloidal and toroidal direction. The channel constant a_M can be calculated in advance to the measurement. For example: with $R_M = 1 \text{ k}\Omega$, $\tau_M = 110 \text{ ms}$ and $\kappa_M = 0.8 \text{ mW/k}\Omega$, which gives $F_M = 0.343$ and $f_M = 0.993$, $K_M = 1.1 \times 10^{-10} \text{ m}^3$ and a total radiating plasma volume $V_{P,\text{tor}} = 45 \text{ m}^3$, the channel constant to upscale the absorber voltage $\Delta\tilde{U}_M$ to feedback proxy becomes $a_M = 141.18 \text{ MW/mV}$. At an absorber signal of $\Delta\tilde{U}_M = 0.1 \text{ mV}$, equation (3.1) will estimate and extrapolate a total plasma radiation of $P_{\text{pred}}^{(2)} = 1.41 \text{ MW}$.

Equation 3.1 will be the reference for a radiation power loss proxy that can be implemented for the scrap~~off~~ layer seeding system. Nevertheless, both of the previously introduced concepts have been used during the last experimental campaign to provide radiation power loss predictions. Details of how the implemented diagnostic hardware and a tailored algorithm calculated the radiation power proxy will be described below.

To start, the setup that was used to provide radiation loss information to the thermal helium beam box will be introduced. The center component of the bolometer feedback system is the PCI Express add-in card and multipurpose input/output device NI® 6321. For the purposes of this experiment, only the analog output capabilities of this device are of interest. The cards specifications have been rated for constant environmental conditions, i.e. temperature and humidity, which are provided by the air-conditioned cabinet in the torus hall, where the bolometer system PC~~of~~this device has been installed on is located. The NI® 6321 incorporates two analog output channels that are both able to provide a DC coupled voltage signal in the range of $\pm 10 \text{ V}$, with 16 bit or 0.3 mV resolution. If both channels are being used, the maximum update rate becomes 840 kS/s , which is equal to a $1.2 \text{ }\mu\text{s}$ sample time with an accuracy of 10 ns . The output has a buffer array of 8191 samples, where the first sample in is replaced by sample number 8191+1 and so on (FIFO¹, see

irrelevant for us
with St+L.Gres

¹FIFO: First In – First Out

property	σ_G	s_G	s_r
value	8×10^{-5}	$1.13 \times 10^{-5} / ^\circ\text{C}$	$5 \times 10^{-6} / ^\circ\text{C}$
property	σ_{off}	s_{off}	ΔINL
value	5.3×10^{-5}	$4.8 \times 10^{-6} / ^\circ\text{C}$	1.28×10^{-4}

Table 3.1.: Output and temperature characteristics of NI® 6321.

figure 3.3). The absolute error ΔU_{AO} for output voltage U_{AO} of the device is given by:

$$\begin{aligned}\Delta U_{\text{AO}} &= U_{\text{AO}} \Delta G + R \Delta U_{\text{off}} \\ \Delta G &= \sigma_G + s_G \Delta T_{\text{in}} + s_r \Delta T_{\text{ex}} \\ \Delta U_{\text{off}} &= \sigma_{\text{off}} + s_{\text{off}} \Delta T_{\text{in}} + \Delta \text{INL},\end{aligned}\quad (3.3)$$

where ΔT_{in} and ΔT_{ex} are the temperature difference to the last internal and external calibrations, and R the set output range of the card. The noted properties and their value can be found in table 3.1. For a temperature change relative to the calibrations of 1°C and 5°C respectively, an output range of 20 V ($\pm 10\text{ V}$) and voltage of $U_{\text{AO}} = 1\text{ V}$ the analog error becomes $\Delta U_{\text{AO}} \approx 3\text{ mV}$. The characterisation of the hardware will later be put into context when addressing the performance and limits.

The NI® 6321 is only tasked with providing signals for the gas valve control. Therefore, the procedure of acquisition and the algorithm that are responsible for real-time feedback need to be presented next. In section 2.2.2, the bolometry system has been described. The following introduction builds on the specifications of the previously implemented system.

Measurement Algorithm

Drop?

Assume an incoming signal that functions as an event trigger, as is standard for the central W7-X control system, which marks the point in time $T_0 = T_1 - 60\text{ s}$, with T_1 being the moment the experiment starts - commonly the startup of ECRH microwaves[99]. A barebone, top level LabVIEW program, designated to receiving and interpreting any incoming trigger signals as fast as possible, is forwarding a start command to the core bolometer

3.2. Real-time radiation feedback system

control. The trigger routine is executed before an experiment session and is running with the lowest possible latency, which is $\ll 1$ ms. The initialization of the main bolometer LabVIEW routine is following the start signal from the trigger program. The user interface input parameters, which have been set before runtime, are stored in memory for later requests during execution. This includes links to local files containing the line of sight geometry information.

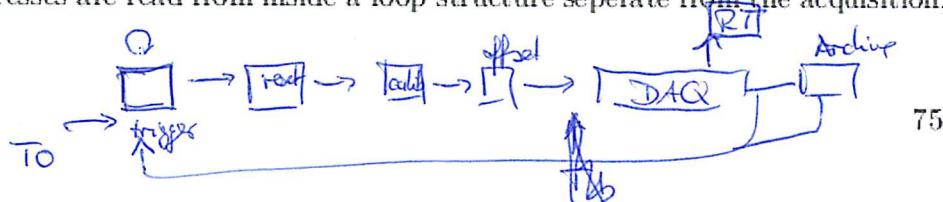
Thereby provided K_M and V_M are, for reasons stated above, derived from magnetic standard configuration calculations.

~~The reason for~~

A preventative reset and clearing of the bit registers and FPGA states of the NI® AD 7730 is carried out in the beginning of the execution. For each of the following internal routines, the bit programmable registers, which are responsible for the mode, voltage range and frequency of acquisition, are set to appropriate values. For the initial floating potential measurement, which has been mentioned in section 2.2.4, the device is put into full range mode, i.e. ± 80 mV and sample frequency of 0.4 ms. The subsequent measurement of 10 s yields the offset voltage and measurement noise per channel. Next, the registers are reset again and then put into calibration mode, which enables the NI® AD 7730 to measure the current through individual parts of the Wheatstone bridge. As lined out in section 2.2, a calibration routine incorporating two subsequent heating stages with 1.2 V and 2.5 V for 1.6 s each is then executed. Resulting currents are measured with a sampling time of 0.4 s. This, in total 4 s long procedure is repeated for both of the reference and measurement parts of the Wheatstone bridge. In succession to the four individual calibration routines, the respective properties κ , τ and R for reference and measurement absorbers are calculated using equation (2.7) and following (see section 2.2.4). Thereafter, filter and mode register are programmed to continuously acquire N data points at the given sample time of 0.8–6.4 ms for the measurement of experiment data. At this point, the program indicates a total runtime of 25–30 s, depending on small changes in the PCs execution prioritisation. The remaining part of the $T_0 + 60\text{s} \rightarrow T_1$ section is downtime.

$\hookrightarrow T_1 = T_0 + 60\text{s}$ above?

Simultaneously, the NI® 6321 is programmed for two-channel analog output. A global variable for each channel, i.e. memory address for writing and reading is provided beforehand. The most current radiation values are calculated using $P_{\text{pred}}^{(n)}$ from equation (3.1) and equation (3.2) individually, inside the acquisition routine, which will be presented further down below. The addresses are read from inside a loop structure separate from the acquisition,



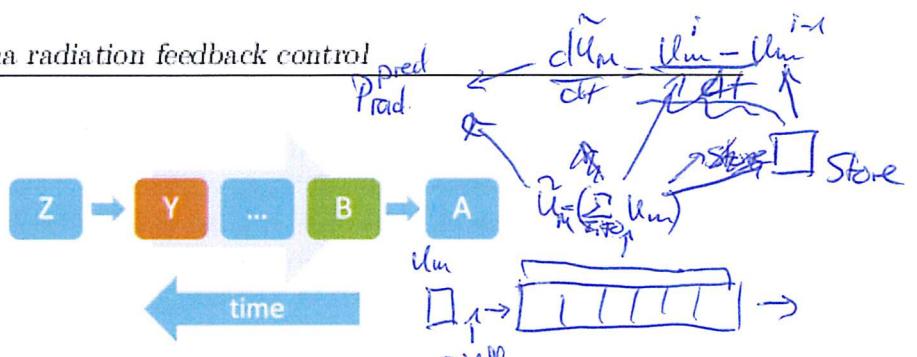


Figure 3.3.: Schematic of a FIFO array. Time progresses from right to left, i.e. sample A is acquired before B, ..., Y and Z. Currently stored samples are B, ..., Y. When sample Z is acquired, sample A is removed from the FIFO array, the oldest data point at this moment.

which runs as fast as allowed by the output performance of the card. A certain amount of time, e.g. 5 s before T_1 , the system starts acquiring samples. The main LabVIEW program initialises a sub-routine that only takes care of data acquisition. Required storage arrays and memory addresses are either referenced or created at first runtime. In scenarios where no feedback is required, only the raw data is returned from the sub-routine, with no additional tasks performed during sample cycles. Also, the initialisation of the NI® 6321 is omitted. For feedback purposes, multiple additional arrays have to be prepared. The raw samples ΔU_M of channels that are in the selection S are stored in an array with the number of rows $|S|$, the cardinality of the subset. In order to reduce the impact of signal noise on the prediction value, smoothing like in equation (2.18) has to be applied. In real-time application, performing a full width Savitzky-Golay filter the width of L samples for each channel is not possible. A computationally affordable approach to a boxcar filter is the FIFO array. A schematic can be seen in figure 3.3. Consecutive samples are stored in a container of length $L + 2$. Assume an iterator i that counts the progression of data points in time. For $i \leq L$ samples are placed in the container at index i . Once the iterator reaches $i > L$, the stored samples are replaced by new ones at their location $j \equiv i \pmod{L}$, hence *first in - first out*. For $i = M$, the mean over all elements is stored in column $L + 1$. In the next iteration, entry $L + 1$ is moved to column $L + 2$ and the new mean is calculated in $L + 1$. This is repeated in all following cycles. Hence, data of the real-time feedback algorithm is stored in a structure the size $|S| \times (L + 2)$.

After the required memory space and addresses have been reserved, the data

is the hardware detail necessary?
It divides the rest

If this is relevant discuss this in a block
and make it relevance clear there

3.2. Real-time radiation feedback system

acquisition is started. The enclosing loop over N repetitions is timed by the length of a sample, e.g. 1.6 ms. An acquired data point $\Delta U_M^{(i)}$ for sample number i is subtracted by the previously in-situ measured offset $V_{\text{off},M}$ and stored in the FIFO array at its corresponding location. After M iterations and onwards, for every sample i and channel M , the boxcar filtered $\tilde{\Delta U}_M^{(i)}$ is calculated. The temporal derivative is calculated from columns $L + 1$ and $L + 2$ using the discrete *forward Euler*¹ method:

$$\frac{d(\tilde{\Delta U}_M^{(i)})}{dt} \approx \frac{\tilde{\Delta U}_M^{(i)} - \tilde{\Delta U}_M^{(i-1)}}{\Delta t}.$$

*put extended Fifo figure here
S. above*

By referencing the current calibration results, one finds $P_M^{(i)}$ from the last and second to last column of the FIFO array using equation (2.19). With K_M and V_M provided through input files, the final step is done in equation (2.25), which yields $P_{\text{rad}} = P_{\text{pred}}^{(1)}$. Similarly, $P_{\text{pred}}^{(2)}$ is derived from a separate FIFO array which only stores samples of one channel. The scaling factor a_M is calculated beforehand.

For each iteration $i > L$, both $P_{\text{pred}}^{(n)}$ *are output as ZI signal and stored for archiving* are written to their respective memory addresses, which are read from in the top-level routine, where the analog output is carried out as fast as possible. Also, both values are written to individual arrays that are returned to the main program after the measurement loop has been completed. After N samples, P_M , ΔU_M and both $P_{\text{pred}}^{(n)}$ are forwarded to the primary LabVIEW application. The measurement procedure has been successfully completed at this point. Output loop and device status are reset and returned to default for the next experiment. At last, the data will be stored locally and uploaded to the central W7-X archive, including the calibration currents, results and feedback proxies. Measurement, as well as feedback routine and algorithm are shown in a schematic outline in appendix A by algorithm 1.

The performance of the *real-time radiation feedback system* has been tested and benchmarked before its application during the experimental campaign. An uncalibrated diode laser with 7 mW continuous power output was aimed at a single detector absorber, like it was described in section 2.2.2, outside the plasma vessel and inside the measurement cabinet of the bolometer diagnostic. A frequency generator with cycle frequencies of 1 Hz-1 kHz and output voltages < 10 V has been powering the laser diode. The detector

¹Leonhard Euler * Apr. 15. 1707 † Sept. 18. 1783

new
Subsection
Benchmark
Validation

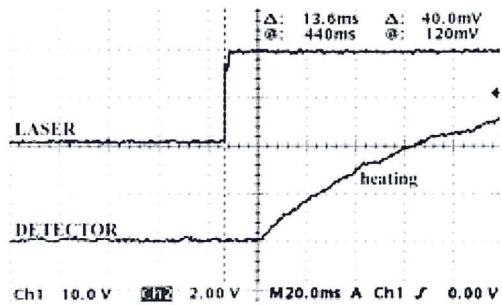


Figure 3.4.: Real-time feedback system benchmark result for 7 mW at 1 Hz laser diode power and frequency. The detector response was created using the algorithm for $P_{\text{pred}}^{(1)}$ with $M = 1$ the FIFO array length. A total, minimum latency of 13.6 ms was achieved during the test.

was connected to a vacant port in the acquisition electronics. In order to find the intrinsic, lowest possible latency of the system, the length of the FIFO buffer was set to the minimum $M = 1$. A measurement program was carried out while the laser was adjusted to its maximum 7 mW intensity at a square wave cycle frequency of 1 Hz. The output of the frequency generator was split between the diode and one input of an oscilloscope. The prediction $P_{\text{pred}}^{(1)}$ for the analog output was calculated using equation (3.1) and was also monitored by the oscilloscope with a separate input. The result of the benchmark can be seen in figure 3.4. A minimum intrinsic latency of the system of 13.6 ms was achieved. Prediction and analog output using $P_{\text{pred}}^{(2)}$ resulted in the same values. Further temporal delays are subject to the smoothing of the FIFO array over M samples of length Δt :

$$\Delta T \approx \frac{M \cdot \Delta t}{2} . \quad \text{regularly used } M \Rightarrow \Delta T ? !$$

In context of the other feedback candidates and actuator system, this latency is the largest among the individual diagnostics. This will become important when discussing initial results of the bolometer feedback and comparing its achievements to the results other control values.

78 In addition, there is a time delay of the reaction of P_{rad} to opening of the gas valve of $a - b$ ms. Due to plasma physics timescales

⇒ You need to discuss this here and conclude which is more important.

3.2. Real-time radiation feedback system

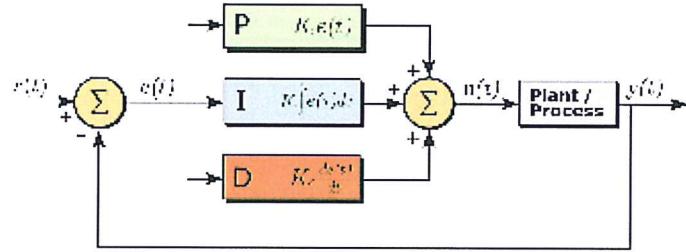


Figure 3.5.: A block diagram of a PID controller in a feedback loop. Desired process value or setpoint SP $r(t)$ and measured process value PV $y(t)$. The control function $u(t)$ is used for feedback[107].

PID controller

Both of the analog output channels of the NI[®] 6321 that provide $P_{\text{pred}}^{(n)}$ are connected via individual, BNC^I terminated, double shielded cables outside the vacuum vessel to an *Arduino*^{II} based *PID*^{III} that controls the impurity seeding valves. Such a controller is widely used in control systems that incorporate feedback mechanisms. The PID algorithm continuously modulates a control function, based on an error value $e(t)$ between a setpoint SP $\equiv r(t)$ and process variable PV $\equiv y(t)$. A correction to the feedback control is applied involving a proportional (P), integral (I) and derivative (D) term of $e(t)$. A block diagram of how such a PID works can be seen in figure 3.5. A setpoint $r(t)$ is defined as the desired state of the machine or system. The current state of the system is measured in $y(t)$. Error $e(t)$ is given by the difference of SP and PV. Individually optimized constants K_p , K_i and K_d scale the proportional, integral and differential part of the control function $u(t)$. The impact of the feedback mechanism, which is controlled by $u(t)$, is measured in $y(t)$ and is used for subsequent adjustments to the control. The control function is given by:

$$e(t) = r(t) - y(t)$$

$$u(t) = K_p e(t) + K_i \int_{t_0}^t e(t') dt' + K_d \frac{d e(t)}{dt}. \quad (3.4)$$

Figure 3.6 shows how the control function for the same setpoint and response

^IBNC: Bayonet Neill Concelman, Bayonet Nut Connector or British Naval Connector

^{II}open-source electronics platform, based on easy-to-use hardware and software for I/O

^{III}PID: proportional–integral–derivative controller

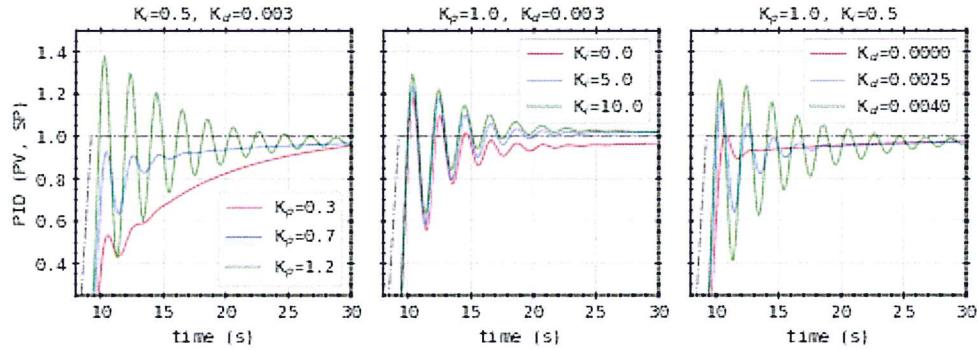


Figure 3.6.: Example of how the control function $u(t)$ changes for the same setpoint SP (grey dashed) and system but different proportional, integrals and differential parts.

changes with different K_p , K_i and K_d . The proportional component determines the approach of $u(t)$ to the setpoint. For larger K_p , the amplitude of the control increases and vice versa, i.e. smaller proportionality leads to a slower adjustment towards the SP. The integral component is responsible for the long-term development of the control. Scaling K_i up eventually can lead to a faster balancing with the setpoint, but too large values will force the control to underestimate the error more consistently. Smaller differential contributions with K_d dampen oscillations in the feedback control, but can also neglect and smooth over fast, transitional responses of the system.

For feedback control with electron density measurements by dispersion interferometry, the factors of proportional and integral part K_p and K_i have been individually optimized based on previous measurements and knowledge about delays in the feedback system itself, i.e. valve opening and closing times. A differential component $K_d = 0$ is excluded in those scenarios. During real-time radiation feedback-control, a different, also uniquely optimized set of parameters has been used. In this case, also $K_d \neq 0$ has been included. For both, there are two sets of PID factors that scale either more aggressively or conservatively.

3.2. Real-time radiation feedback system

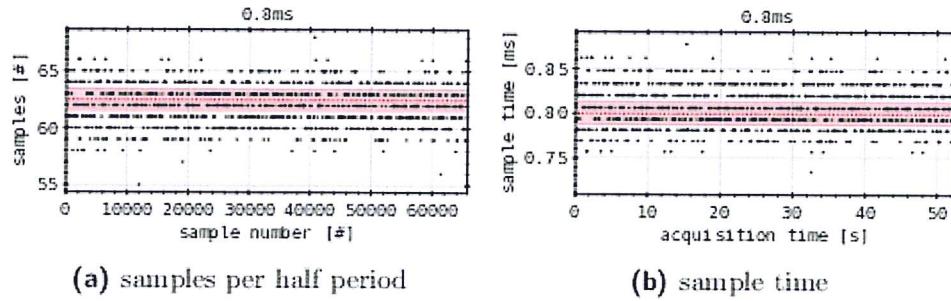


Figure 3.7.: Laser test results without real-time radiation feedback. The expected number of samples and average sample time are shown in red. (left): Number of samples per laser half period over sample number (right): average sample time over laser half period, as derived from the left over acquisition time.

3.2.1. Performance

The previous efforts around the real-time radiation system and underlying algorithm were dedicated to a reliable, low latency performance for detachment control purposes. In order to ensure a consistent sampling of the absorber signal and subsequent radiation power, while also calculating and providing feedback proxies $P_{\text{pred}}^{(n)}$, measurements of the acquisition cycle timings was performed. Therefore, the same setup as for the intrinsic latency test of the radiation feedback system was used. An uncalibrated diode laser with 7 mW continuous power output was aimed at a single detector absorber in a laboratory environment, outside the plasma vessel. A frequency generator with 1 Hz-1 kHz of < 10 V output voltages was powering the diode. The exposed absorber was connected to a vacant port in the diagnostic electronics. In order to measure the actual sample times for the individually set Δt , diagnostic programs were carried out with maximum acquisition length and a square wave signal laser facing the detector. The measurement procedure was performed while the laser was adjusted to its maximum 7 mW intensity with a frequency of 1 Hz. For a set sample time Δt and laser half period of 0.5 s, the number of samples that are expected to measure the laser light is given by:

$$N = \frac{0.5 \text{ s}}{\Delta t} \pm 1 .$$

Move to Chapter 2? Disrupts the Pt part..

Is also relevant for general diagnostic in terms of time shift.
Have you discussed this?

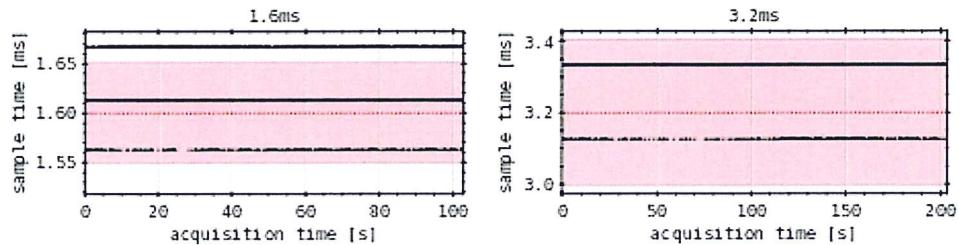


Figure 3.8.: Laser test results without real-time radiation feedback for 1.6 ms and 3.2 s. Both show the average sample times per laser half period over acquisition duration. The expected average sample times are shown in red.

An uncertainty of one sample is assumed, both for errors in distinguishing process of rising and falling slopes in the signal and the smallest possible resolution of the measurement. Rewriting N for Δt yields the *actual sample time* for the respective laser half period.

The results for no real-time radiation feedback, i.e. without the algorithm and calculations related to the proxies, for a set sample time of $\Delta t = 0.8$ ms can be seen in figure 3.7. This is the lowest possible sample time of bolometer data acquisition, irregardless of feedback and experiment configuration. The number of data points is limited by the memory of the system. Therefore, for longer periods of acquisition, larger Δt have to be selected in order to cover the experiment duration. For $\Delta t = 0.8$ ms, the actual sample time is found to be within 0.05 ms of the set sample time 0.8 ms. A large amount of data points indicate a divergence from the programmed acquisition time Δt , which is supported by the accompanying number of points per laser half period, which show the same behaviour. This is equal to a 6.25% error in sample timing, which is significantly larger than the, in section 2.2.5 already discussed minimum resolution of ADC programming and physical properties of the absorber design.

Results for $\Delta t = 1.6$ ms and 3.2 ms are shown in figure 3.8. Similar perturbations of the acquisition timing can be found for 1.6 ms, while the spectrum of measured sample times only consists three values. For $\Delta t = 3.2$ ms, the results are within the errorbar of the measurement and therefore in agreement with the predicted timing. Larger sample times show the same results and therefore support the findings of this benchmark. An erratic error in

How long?
Marketable

3.2. Real-time radiation feedback system

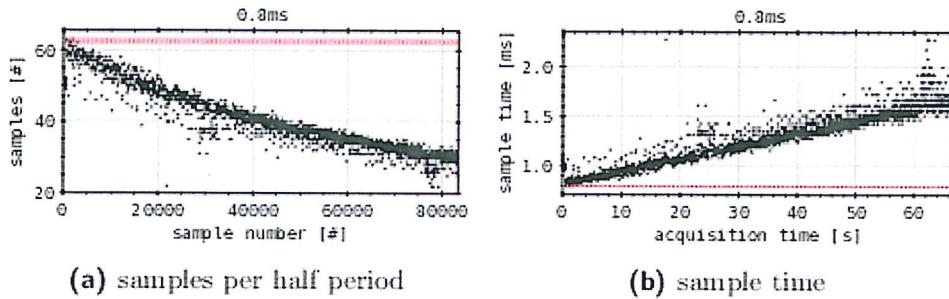


Figure 3.9.: Laser test results with real-time radiation feedback. The expected number of samples and average sample time are shown in red. (left): Number of samples per laser half period over sample number (right): Average sample time over laser half period, as derived from the left over acquisition time.

acquisition sample timing is found for $\Delta t < 1.6$ ms, which is presumably caused by the routine of signal integration in LabVIEW®. This specific part of the program is clocked at a fixed frequency for each set sample time Δt . Interferences can be caused by additional task, which are performed during each sample clock of the actual measurement.

In the next step, in order to compare the results without feedback, the same benchmarks are performed for the real-time radiation algorithm. The results for $\Delta t = 0.8\text{--}1.6$ ms are shown in figure 3.9. From the very first data point and laser period, an increasing sample time is found over the entire measurement duration. The first samples are already substantially longer than the set 0.8 ms, while their length only grows with 0.01 ms/s or 0.01 ps/S. This leads to an increase by factor two of the set sample time after 60s to ≈ 1.6 ms. For $\Delta t = 1.0\text{--}1.6$ ms, similar results can be seen in figure 3.10. However, an elongation of the sample cycle is found to occur after a certain number of samples have been processed, i.e. ≈ 28000 S for $\Delta t = 1.0$ ms. Like before, a linear increase of 0.01 ms/s in sample length is found from this point onwards. For an even longer sample time of 1.6 ms, only single data points deviate from Δt , while the majority of samples is still within the expected timing window of the program. However, in comparison to the previous results without real-time feedback, $\Delta t = 1.6$ ms exhibits greater uncertainties towards the end of the acquisition. Larger sample times show no perturbations due to the feedback algorithm and its interference with the

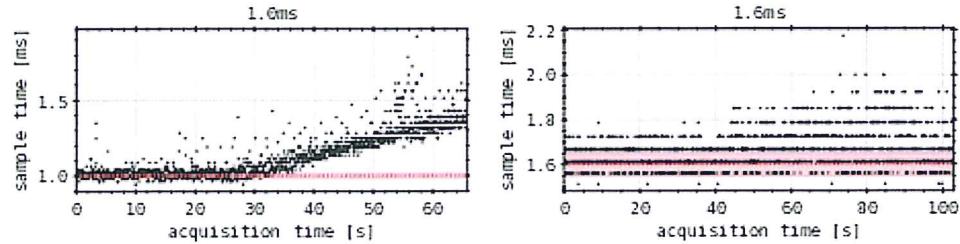


Figure 3.10.: Laser test results with real-time radiation feedback for 1.0 ms and 1.6 ms. Both show the average sample times per laser half period over acquisition duration. The expected average sample time are shown in red.

acquisition cycle.

In order to be able to rule out any interference of the PC system on the performance on the acquisition timing, the same benchmarks are repeated with a synthetic load of at least 80% of its capacity on the central processing unit of the system. If the stress on the computational unit does have any impact on the calculation speed, with which the algorithm is performed, and therefore timing of the diagnostic, a significant increase in deviation from the set sample time Δt is expected. The results for $\Delta t = 0.8$ ms are shown in figure 3.11. When comparing results to the previously presented measurements without synthetic PC load and real-time feedback, there is no noticeable difference in the number of samples per half period or actual sample time. This is the case for all larger samples times Δt . The computational stress of the computer is found to have no effect on the performance of the bolometer acquisition.

The presented results inevitably show that the real-time radiation feedback is not applicable for sample times $\Delta t < 1.6$ ms, or at higher acquisition frequencies for experiment times above 30 s. Furthermore, in its current configuration, the bolometer systems acquisition duration is limited by its memory capacity with regards to the selected sample time Δt . The perturbations in *actual* sample timing can be found for all individual sample time settings Δt . They show random behaviour and protrude in both directions around the set value with similar amplitudes. Both bolometer acquisition programs, with and without real-time feedback, present erratic divergence of the sample timing. However, the latter shows stretching of integration

3.3. Experimental achievements

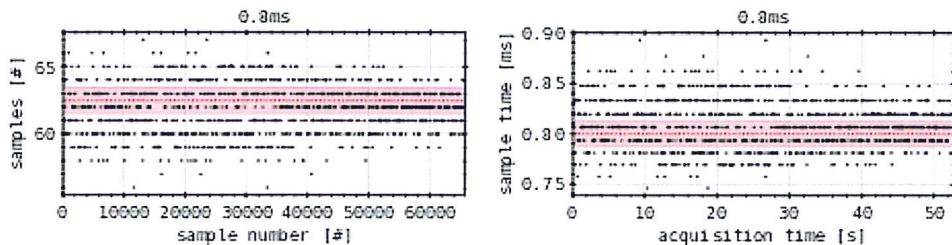


Figure 3.11.: Laser test results without real-time radiation feedback and additional, synthetic PC load. The expected number of samples and average sample time are shown in red. (left): Number of samples per laser half period over sample number (right): Average sample time over laser half period over acquisition time.

periods for lower set sample times. No coherent alteration of experimental data timelines due to random perturbations in sample timing is expected. Though, future bolometer measurements could greatly benefit from a similar setup to be permanently installed at the diagnostic.

The following experimental applications of real-time radiation feedback will incorporate the findings of this benchmark. If not stated otherwise, the set sample time of the bolometer acquisition is 1.6 ms.

→ mention effect on time shift
for regular DTQ.

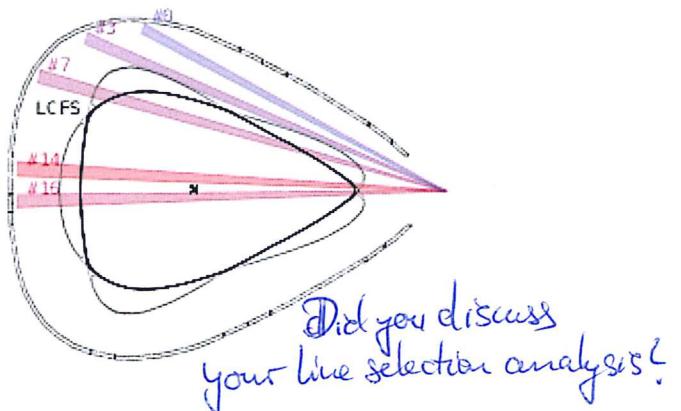
3.3. Experimental achievements

After characterising the *real-time radiation feedback* system in the previous section, one can now begin to present and discuss its experimental achievements from the last experimental campaign.

Initial results

The very first plasma discharge that incorporated radiation feedback control was experiment (number) XP20180920.29. After initial, functional tests of the feedback system, this was the first time the PID controlled gas valve actuator was applied based on radiation measurements. This plasma discharge took place after the first *boronization*, which significantly decreased the influx of intrinsic impurities oxygen and carbon from plasma facing components and vessel walls. The latter line radiation was decreased by a factor

Figure 3.12.: Line of sight selection for experiment number XP20180920.29, only containing of HBC channels from the top and center part of the triangular cross section, i.e. the bolometer measurement plane around $\varphi = 108^\circ$ toroidally.



of up to six[108]. The experiment was planned to include 10 s of 2 MW constant microwave heating. The ECRH was coupled to the second-harmonic extraordinary (X2-) mode of the plasma. For safety reasons, the heating and therefore discharge were terminated early after 1.3 s. The plasma vessel was pre-filled with hydrogen before heating startup by five short bursts of 5 ms each at ≈ 200 mbar L/s. The experiment was performed in standard magnetic field geometry with an $\iota = 5/5$ magnetic island chain configuration.

Plasma feedback valves have been seeding helium into the scrape-off layer from port AEH30 in half module no.3. Helium exited the valves at 50 mbar with a flow rate of 6×10^{18} atoms/s, injecting 1×10^{19} atoms in total. The setpoint SP of the PID was programmed for $f_{\text{rad}} = 90\%$, while $P_{\text{rad}} \approx P_{\text{pred}}^{(1)}$ from real-time feedback is assumed to be true. On one hand, this is done according to theoretical and experimental predictions for stable plasma detachment for W7-X, like it was described in the introduction of this chapter. On the other hand, in anticipation of errors in the radiation prediction, as well as retardations of the plasma response to gas injections, this is supposed to prevent oversaturation of the scrape-off layer and plasma with impurities, which can lead to a radiative collapse.

The temporal evolution of available plasma parameters can be seen in figure 3.13. In the top left, the input heating power and plasma stored energy, as well as its derivative can be found. Below, the total radiative power loss from the plasma, measured by both bolometer cameras can be seen. The real-time radiation feedback predictions, calculated by equation (3.1) and (3.2) are shown in the same plot. The prediction for P_{rad} in $P_{\text{pred}}^{(1)}$ was calculated

(is this relevant)

↳ You mean 9% instead of 100%

3.3. Experimental achievements

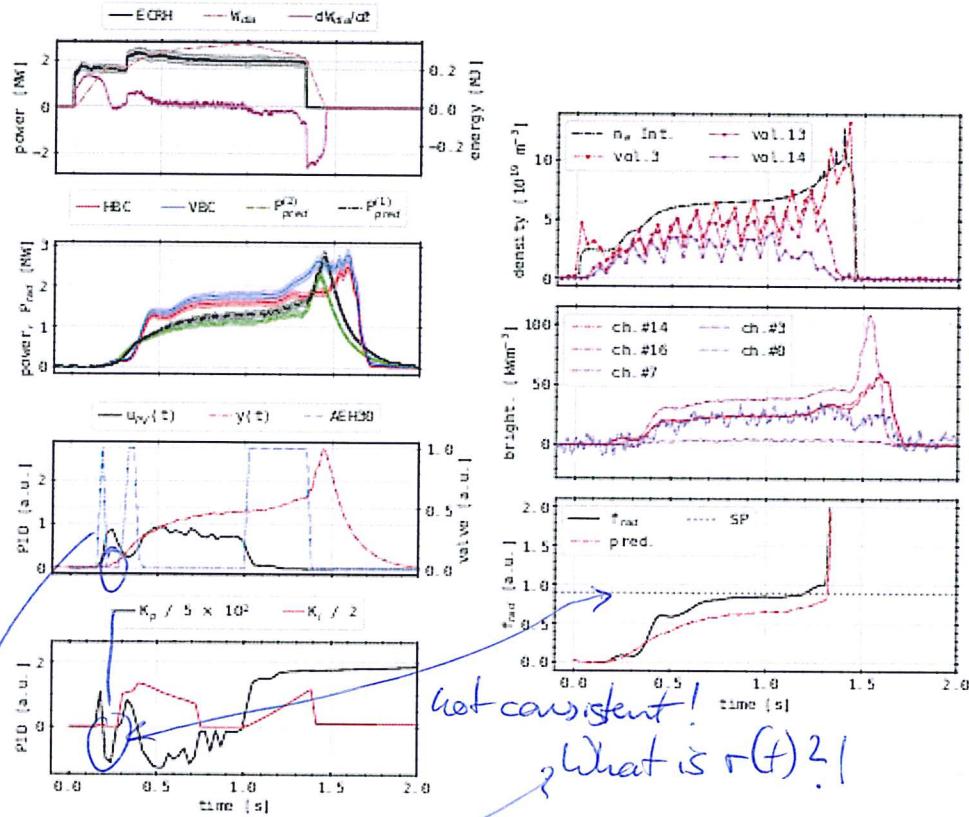


Figure 3.13.: XP20180929.29

Very first radiation feedback controlled plasma discharge. (top left): Microwave heating power, plasma stored energy and loss (center left top): Radiative power loss, both cameras and prediction values (center left bottom): PID control $u(t)$, reference $r(t)$ and valve actuation (center right top): Electron density of selected Thomson scattering volumes and line integrated ECE measurement (center right): Individual bolometer channels, used for $P_{\text{pred}}^{(n)}$ (bottom right): Radiation fraction(s).

using the detector subset $S = \{0, 3, 7, 14, 16\}$, as well as the approximation $P_{\text{pred}}^{(2)} = a_7 \Delta \tilde{U}_7$ with detector 7. This line of sight selection can be seen in figure 3.12. Below, the PID controls process value $u(t)$, the forward model $P_{\text{pred}}^{(1)}$ for comparison and the normalized feedback valve actuation

are presented. Only valves located in port AEH30 injected helium into the scrap~~off~~ layer. On the bottom left, the PID components K_p and K_i can be seen. No differential component was used in this case. The line integrated electron density from the dispersion interferometer and *Thomson scattering* for selected volumes {3, 13, 14} is shown in the top right. Below, the chord brightness evolution of the bolometer channel subset is presented. Finally, the radiation fraction f_{rad} , given by equation (2.22), next to the setpoint SP of the controller can be seen on the bottom right.

Both plasma radiation power loss predictions quantitatively roughly mimic the complete P_{rad} dataset. Like the full camera information set, the predictions feature a late rise from 0.2 s to 1.5 MW at 1.3 s. After, the proxies sharply rise and peak with 2.5 MW at 1.4 s, before rapidly decreasing to zero. Proxy $P_{\text{pred}}^{(1)}$ is slightly larger and reaches its maximum of 2.8 MW later than $P_{\text{pred}}^{(2)}$. The radiative power loss P_{rad} from both cameras has a much steeper increase at 0.35 s, with a small plateau around 1.4 MW. The vertical camera measures an increasing radiation power of 2.7 MW at 1.4 s, where oscillations lead to an abrupt drop off. Similarly, the horizontal camera measures oscillations for a shorter duration at a later point in time around 1.5 s. Generally, the VBC presents a larger P_{rad} of at least 10% than the HBC, while the latter shows radiation power values of up to 0.5 MW larger than the predictions. Although, in situations of peak radiation, the proxies $P_{\text{pred}}^{(n)}$ are able to represent the full dataset results within their respective errorbars.

The individual chord brightness of the channels used to calculate the predictions show the same behaviour. However, their temporal relation, i.e. the position of the peak before and after 1.5 s varies significantly. Channel no.7 is maximum at 1.55 s, while detectors viewing plasma volumes further inside with smaller $r_{\text{eff},M}$, no.14 and 16, reach their peak about 0.1 s later. Due to its orders of magnitude larger noise $\sigma_{\Delta U}$, channel no.0 has no meaningful contribution to the radiation prediction and must have experienced issues during data acquisition. Channel no.3 already shows very small values and therefore suggests no.0 not containing any more information on the emissivity profile, since the chord brightness indicates that the radiation is mainly located inside the last closed fluxsurface, which will be discussed further below with figure 3.14.

Similar behaviour can be seen in the Thomson scattering electron density

3.3. Experimental achievements

for three different volumes - the volumes are ordered so that vol.0 is located closest to the magnetic axis or center of the plasma and vol.16 the furthest away. Volume no.3 quickly rises after microwave heating startup and gradually increases towards its maximum of about $1.4 \times 10^{20}/\text{m}^3$ at 1.4s, before dropping to zero at 1.5s. The volumes further outward climb much slower and are as much as 50% smaller. They also experience an early drop off from 1.3 s and 1.0 s for vol.13 and 14 respectively. The line integrated measurement of the dispersion interferometry envelops the lines of the Thomson profile, leading up to the same maximum.

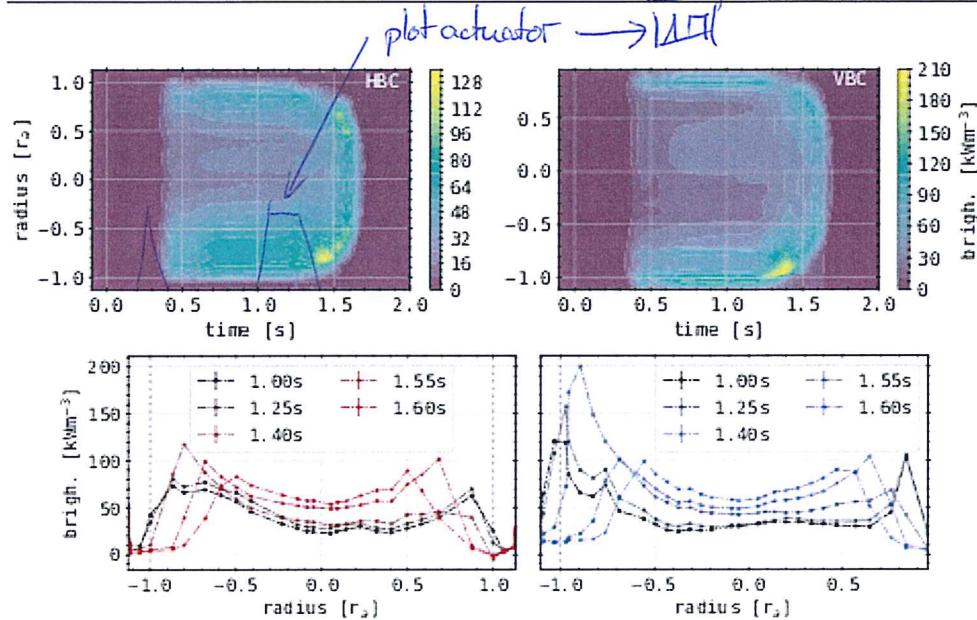
The resulting radiation fraction f_{rad} for a constant 2 MW heating quickly rises, after the slight delay in the radiation, to 80% at 0.7 s. It diverges beyond 1.3s, since P_{ECRH} becomes zero and the radiation power loss reaches its maximum. After the start of the ECRH and within the first 0.2s, the radiation power is close to zero, while the plasma stored energy is increasing until a plateau at 0.2 MJ. Its time derivative indicates that almost all of the heating power is deposited into the plasma. However, probes dedicated to measuring the shine-through of the microwaves through the plasma show that there is a non-negligible portion of the heating power beam which is not absorbed. After dW_{dia}/dt becomes zero, there is a small step in the ECRH power of 0.3 MW, which in turn again increases the stored energy, as well as P_{rad} towards its plateau. After ECRH shutoff, W_{dia} linearly decreases to zero, while the plasma radiation reaches its maximum.

The feedback can be seen on the bottom left of figure 3.13. According to the PV, the valve is opened and closed for injecting helium into the plasma, as is indicated by the actuation of the gas valves. The injection box is located in the diagnostic port AEH30, in the half module of the plasma vessel next to the bolometer measurement plane in counterclockwise direction. In terms of transport along magnetic field lines, the actuator is one full toroidal rotation away from the bolometer cameras, since the net parallel plasma transport increased to $\approx 200 \text{ A}$ in clockwise direction (if viewed from above). Due to the delayed response of P_{rad} and P_{pred} , the control function $u(t)$ triggers the gas injection early. Two smaller gas puffs of $< 0.1 \text{ s}$ are initiated within the first 0.5 s of the plasma and one longer, 0.35 s inlet after 1.0 s.

*Then it is in the
same location?
What is this?*

Figure 3.14 presents the (top) spatial-temporal resolved chord brightness profiles of the horizontal (left) and vertical (right) bolometer camera, as well as (bottom) selected snapshots of the above profiles for both. The

2D

**Figure 3.14.:** XP20180929.29

Chord brightness profiles (top) in two-dimensional contours and (bottom) at significant points in time for both cameras HBC and VBC.

radiation, measured by the HBC, is mainly located inside the last closed fluxsurface, i.e. $r_{\text{eff}} \in \{-1, 1\}$. After the delayed increase of radiation power, the chord brightness develops generally symmetric, with slightly larger and wider structures towards the negative end of the scale, or closer to the VBC in the bolometer plane. Very shallow peaks limit the radiation profile on both ends inside the confinement area, while a hollowness with a 20 – 30% reduction around $r_{\text{eff}} = 0$ can be seen. After the second gas injection, the brightness of the peaks increases gradually, until at 1.4s around $r_{\text{eff}} = 0.75r_a$ a pronounced maximum appears. This structure is followed by a quickly inwards, i.e. towards the magnetic axis shifting radiation front beyond 1.5s, which presents not emission after 1.7s. Similar results are found with the VBC chord brightness profile. Due to the asymmetric line of sight geometry of the vertical camera relative to the magnetic axis, the spatial range is slightly shifted towards the negative, which corresponds to the inside of the vessel or left of the triangular cross section. The VBC profile features a stronger hollowness of about 50%. Furthermore, the small asymmetry of

points out that SOL
 radiation is also seen
 on these LOS
 It's after the drop of
 signal noise
 Tell that
 Shows the in-
 ward movement
 property.

3.3. Experimental achievements

the HBC chord profile is much more pronounced in the vertical camera. The exemplary snapshots of the profile underline this circumstance. Shown there is also the inward shift of the radiation over time towards 1.7 s. Both profiles feature the asymmetric maximum at 1.4 s on the lower inside of the separatrix, close the aperture of the VBC.

maybe you want
to show inversion
once to visualize the
movement?

Due to the delayed response of plasma radiation and therefore difference to the setpoint, the controller engages and initiates two, separate gas pulses. One small and large dent in the radiation power can be found following the actuation of the gas valve and ionization of helium in the scrape-off layer. The radiation power increases accordingly after the first gas injection and step in ECRH around 1.0 s, which leads to $f_{\text{rad}} \approx 80\%$. However, due to the discrepancy between prediction and full dataset, the PID is assuming a much smaller radiation fraction, as shown by the red line in figure 3.13:(bottom right). The gas valves are opened again, since f_{rad} plateaued and has not yet reached the setpoint. The following 0.35 s long helium puff first slowly increases P_{rad} until the actual radiation fraction crosses 90%, where it again gains significantly beyond 95%. During that time, the electron plasma density collapses from the outside inwards, as at 1.3 s vol.14 is close to zero and vol.13 decreases about 50%. Meanwhile, vol.3, i.e. the core plasma, and central line integrated density reach their peak. At this point, the plasma became nearly transparent for the microwave beam and the discharge had to be terminated early.

Due to the more conservative PID coefficients and underrepresented true plasma radiation power by the prediction $P_{\text{pred}}^{(1)}$, the second long feedback gas puff essentially suffocates the plasma by increasing the impurity content beyond a sustainable level and thereby removes energy through radiative cooling from not only the scrape-off layer, but also the core plasma. Because of the shrinking density and temperature profiles, which changes the gradients, helium is transported inside the last closed fluxsurface and into the confinement regions core plasma. After 1.0 s, the plasma continuously loses $\approx 0.25 \text{ MJ}$ per second, or irradiates 0.25 MW due to seeding. At increasing density and declining plasma energy, the temperature is assumed to be strongly reduced. The previously discussed radiation assymmetries in chord brightness profiles of both cameras in figure 3.14 can be identified as the connected plasma volume of gas injection (see figure 3.1). Once these profile changes have reduced the absorption of the microwaves enough, the ECRH has to be terminated in order to avoid too much stray radiation on

measurements?
? What do you
mean?

You should plot profiles if you discuss them so long.
But is this relevant for your work?
in this detail

plasma facing components. The remaining 0.2 MJ of plasma stored energy are exhausted over the next 0.1 s at nearly 2 MW. This energy is partially irradiated, as P_{rad} increases by ≈ 0.75 MW. The remaining injected helium also dissipates energy by ionization and excitation in the core. The increasing core plasma density quickly drops, as the diamagnetic energy becomes zero and the remaining ionized gas neutralizes. The subsequent final radiation peak at 1.5 s is the result of the residual plasma and its recombination of hydrogen and impurities.

The radiation feedback XP20180920.29 has failed, since $P_{\text{rad}}^{(1)}$ did not provide the required accuracy for an approximation of P_{rad} and the system was not able to achieve a scenario of (stable) detachment. This led to an oversaturation of the scrape-off layer with helium, which caused plasma profiles to collapse and thus the discharge to be terminated early. Critical towards the efficiency of the feedback as well is the temporal delay, which is further exacerbated by the negligent, conservative scaling of the proportional K_p term in the process value $u_{\text{PV}}(t)$. From the chord brightness profile of the vertical and horizontal camera array, the radiation is found to be mainly to be located close the last closed fluxsurface, inside the confinement area. Therefore, additionally to excluding channel no.0, which was unexpectedly out of order in this experiment, the selection subset S has to be optimized towards this kind of plasma radiation profile. More lines of sight along this radial spectrum have to be included, while channels that do not see any or only small amounts of radiation have be removed from the feedback. However, in order to adequately represent any possible emissivity distribution for varying experiment scenarios, the subset S needs to contain detectors with lines of sight in all plasma regions.

more concrete title

Application progress

The second application of the radiation feedback occurred in experiment XP20180920.31. Regarding the boronization, the influx of impurities from the wall and general machine condition are assumed to be same as in the previous experiment. This discharge was also planned with 29.5 MJ of total input energy at 10 MW X2-mode microwave heating for 10 s in a similarly pre-fueled hydrogen plasma. Furthermore, the real-time feedback system used the same settings as experiment XP20180920.29, i.e. $S = \{0, 3, 7, 14, 16\}$ and $P_{\text{pred}}^{(2)} = a_7 \Delta \tilde{U}_7$, or as in figure 3.12. However, the injection valves for

Relevant?
This is a guess
Is it what you have
spectroscopic
measurements
to show

Inversion!

Is this relevant?
How?

3.3. Experimental achievements

Power balance
What is the target power

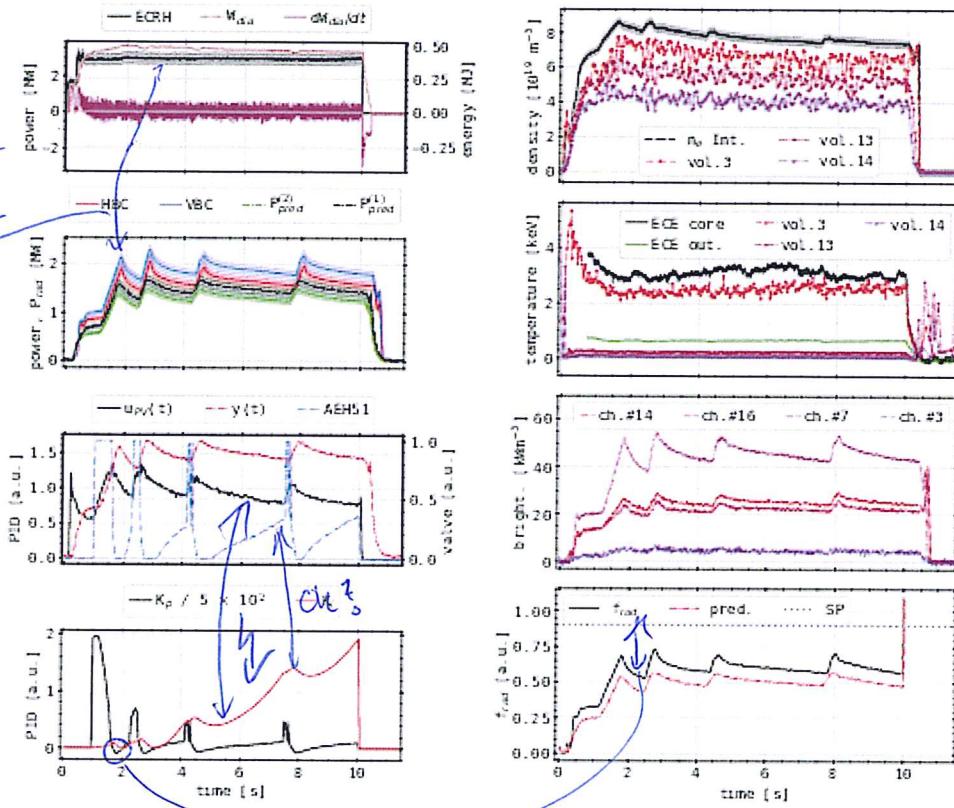


Figure 3.15.: XP20180929.31: (top left): ECRH power, plasma energy and loss (center left top): P_{rad} , prediction values (center left bottom): PID control $u(t)$, reference $y(t)$ and valve actuation (bottom left): Control component(s) (top right): Electron density of Thomson scattering and dispersion interferometer (center right top): Electron temperature of Thomson scattering and ECE (center right bottom): Bolometer channels for $P_{\text{pred}}^{(n)}$ (bottom right): Radiation fraction(s).

this experiment are located opposite of the bolometer plane, in half module no.5 and port AEH51, which is one half toroidal rotation away. The net toroidal current by parallel plasma transport increased over the course of the discharge to $\approx 2\text{kA}$ in counterclockwise direction.

The available plasma properties are shown in figure 3.15. The structure of the plots is generally the same as for the previous experiment. The dura-

tion of the discharge is now 10 s, given by the electron cyclotron heating of ≈ 2.9 MW. Additionally, the now accessible electron temperature measurements of both the Thomson scattering and electron cyclotron diagnostics are shown in the second plot on the right.

The heating power starts at 1.7 MW for the first 0.3 s, before increasing to ≈ 2.9 MW at 2.5 s, where it remains constant throughout the experiment. Accordingly, the diamagnetic plasma energy rises nearly linearly to 0.2 MJ. After a small plateau, W_{dia} increases more gradually together with the second heating stage towards 0.5 MJ. From there follows a slight decline of about 50 kJ until the termination of the microwave heating. Its time derivative, similarly to the heating power, rises to 1.75 MW and drops again as the stored energy plateaus. The same is repeated for the second stage. Afterwards, this peak decays with the same time constant as the positive slope of W_{dia} . After ECRH shutoff, the plasma stored energy quickly drops to zero within 0.3 s. This is also presented by dW_{dia}/dt , as the peak dissipation of ≈ -3 MW is followed by a sharp increase towards zero, with two small reversals in slope in between around -1.25 MW.

The global radiative power loss of both cameras show similar behaviour, though their discrepancy is generally $> 10\%$ for most parts of the experiment. Throughout the discharge, the vertical bolometer camera measures significantly more plasma radiation than its horizontal counterpart. The prediction values are similarly related, as $P_{\text{pred}}^{(1)}$ indicates about 10% more radiation power than $P_{\text{pred}}^{(2)}$, while the former is as much as 10% smaller than $P_{\text{rad}, \text{HBC}}$. At 0.25 s the radiation power rises quickly to 0.9 MW within the first 1 s. After a small increase to > 1 MW towards 1.3 s, $P_{\text{rad}, \text{VBC}}$ sharply jumps and peaks at ≈ 2.2 MW around 1.8 s. Afterwards, the radiation level drops with a gradually decreasing slope. This behaviour repeats three times, with varying levels of P_{rad} and time in-between the peaks. The second follows about 1 s after the first, with a marginal increase in power of 0.2 MW. The subsequent two occur around 1.5 s and 3.2 s later at about the same power loss level as the first. All of the peaks and valleys cycle between 1.7–2.4 MW. After ECRH termination at 10 s, the last stage has decreased to 1.8 MW. Beyond, the radiation power drops to 1.4 MW, where, after a small increment, P_{rad} goes to zero before 11 s. Differences between the full datasets and predictions are the less pronounced shape of peaks and slopes, smaller gradients in the rise and conclusion of the radiation and a substantial



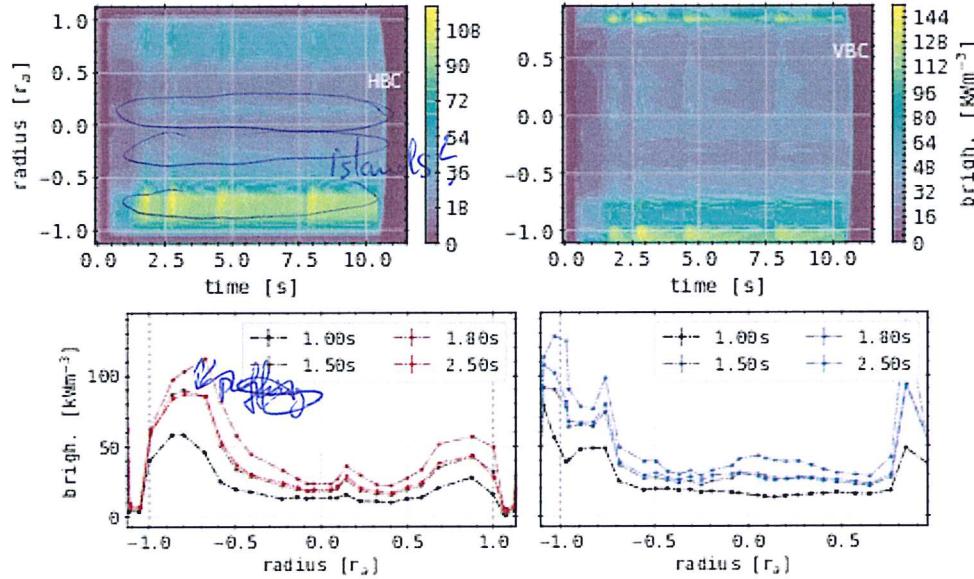


Figure 3.16.: XP20180929.31
Chord brightness profiles. (top) Two-dimensional contours (bottom)
snapshots for both cameras, HBC and VBC around feedback gas puff.

as well as the ECE measured temperature are significantly lower, well below 1 keV. After shutoff of heating power, the temperature drops quickly to zero, where large oscillations in Thomson scattering measurements can be found. For very low electron plasma densities, the measurement of temperatures by elastic Thomson scattering is not applicable. Hence, before and after the plasma, those values show very large errors.

The chord brightness of the subset that was used to calculate $P_{\text{pred}}^{(1)}$ presents the same behaviour as P_{rad} . As in the previous experiment, the channels further inward, no.14 and no.16 measure overall less radiation power and show smaller response to gas injections than detector no.3, which is viewing close to the separatrix on the inside of the magnetically confined region. Channel no.3 presents the lowest brightness measurement and a negligible reaction to helium seeding. As the plasma collapses after 10.5s, channel no.7 first drops before peaking and finally decreasing to zero, while no.16 and no.14 do not decrease.

The full chord brightness profile of the HBC and VBC, as well as snapshots

3.3. Experimental achievements

temporal deviation, which can be noticed around the maximums.

The control $u_{PV}(t)$ is composed of a proportional and integral part. During the rise of radiation in $y(t)$, the PV peaks before 0.5 s, accounting for the first heating stage and slower response of P_{rad} . After the gradient in $y(t)$ shallows and ECRH increases, the control triggers a helium gas injection and therefore an increase in radiation. Over the course of the activation, $u_{PV}(t)$ increases until the valve is closed, where the radiation still rises for ≈ 200 ms and the control conclusively drops. This behaviour is repeated for all subsequent gas puffs. Leading slopes in AEH51 signal are correlated to the increase in gradient in PV between injections.

The underlying system value $y(t)$, and therefore the prediction, underestimate the radiation level in P_{rad} significantly. Initiated actuations of the feedback valve in port AEH51 and subsequent injections of helium precede the peaks in radiation power. In the beginning, one gas puff leads to the first peak in P_{rad} . A second stage is enclosed before the next peak. For the following injections, incremental slopes lead up to the maximum amplitude activation, which increase in both width and height. A final, fifth stage is not completed due to the discharge being terminated beforehand. The behaviour of the valve can be derived from the two PID components the PID, which the control value is based on. The activation of the actuator is dominated by the proportional component of the PID. However, the potential fifth gas puff would have been initiated by the integral part K_i , as K_p has decreased substantially at this point. The leading slopes in actuator signal later in the experiment are supported by an integral component. Width and shape of the peaks in AEH51 signal are dominated by K_p .

The line integrated and innermost plasma electron densities exhibit the first step in heating power around $2 \times 10^{19} \text{ m}^{-3}$, while volumes further outward increase more gradually. From line integrated to core and outward volumes the density reaches about $8.5 - 4.5 \times 10^{19} \text{ m}^{-3}$ respectively, around 1.5 s after the first helium gas injection. Afterwards, additional gas puffs produce small peaks in the line integrated density. Generally, n_e decreases over the course of the discharge to $7.5 - 4 \times 10^{19} \text{ m}^{-3}$ from the core further outward. Inversely, the electron plasma temperature is highest after the first ECRH stage at 0.25 ms with 5.5 keV for the innermost Thomson scattering volume. The electron cyclotron emission measurements are not available in the first 0.75 s. Core ECE measurements indicate a higher temperature than volume no.3 of about 0.5 keV. The further outward volumes no.13 and no.14,

no. U_i is this part
+ K_p
The peaks
are a change from
consecutive trigger.
feedback coeff. and
hence too large K_p
 K_i is seen at
 ≈ 5 ($K_p = 0$)
 K_i is seemingly
way too low.

is this relevant?
How?

not at $\text{rad} \approx 0.5$! These are island hot spots ...

3.3. Experimental achievements

from before, during and after the first helium gas injection are shown in figure 3.16. From the perspective of the HBC, the radiation is mainly located inside the last closed fluxsurface, i.e. $r \in \{-1, 1\}$, with a strong poloidal asymmetry. The emission level on the bottom of the triangular cross section, inside the separatrix and close to the VBC aperture, is at least twofold of that on the top. During the first gas injection, the radiation level strongly increases towards approximately 1.9 s, where the maximum has significantly shifted inside. This is reverted after the injection is stopped. An emission front around $\approx 0.75r_a$ remains until the collapse of the plasma after 10.5 s, in which the following gas puffs each create small local maximums that are located slightly further inside. Radial gradients indicate that the entire radiation distribution is shifted inwards during that time. The same can be found on the top around $0.8r_a$, however at much lower power densities. The emissivity around the magnetic axis, i.e. $r = 0$ ~~is only 1/3 of the peak on the bottom side.~~ ^{True-intensity? This might not be at the axis. Be careful wording...} Towards the top, there is a third structure in the brightness profile at $0.2r_a$, which is consistently below the level of the upper peak. The vertical camera measures a radiation peak, similar to the negative end in the radial spectrum of the horizontal camera, outside and along the separatrix at $r = r_a$. The same behaviour in response of the helium gas puffs as on the left in the HBC can be examined here. The structures radial width however is much smaller, while being distinctly limited on the inside at $-0.95r_a$. Another structure can be found around $-0.75r_a$, with about 3/4 of the former intensity. On the opposite end of the radial profile, another emissivity peak can be seen, with comparable amplitudes after gas injections but much narrower radial proportions. A central structure can also be found $\approx -0.1r_a$, although with a much larger radial gradient as seen by the HBC. The poloidal asymmetry in the horizontal camera measurements is exemplified by the individual chord brightness snapshots of before, during and after the first gas injection. Both response and absolute level of the emissivity close to the separatrix are up to a factor of two larger on the left, or close to the VBC aperture on the bottom side of the triangular cross section. The power density is highest right before the end of the puff, from where it decreases approximately to the level in the beginning of the injection. With the increase in emissivity, the location of maximum intensity also moves inwards. The same can be found for the negative end of the radial spectrum in the vertical camera profiles. Moving inwards, the previously examined gap and second structure is found, with less pronounced maximums and no

~~check if you can identify with X-pt. or O-points?~~

~~TS doesn't see the increase. Is the interference passing an island (where puffed?) and might this be red- and confinement in an island O-point?~~

positional shift. On the other side, increments in radiation power density are significantly larger, while no movement is exhibited.

The *actual* and predicted radiation fraction, based on P_{rad} and $P_{\text{pred}}^{(1)}$ respectively, differ significantly. At most times, and especially during and after gas injections, the prediction underestimates the actual radiation fraction by at least 10%. Furthermore, f_{rad} is well below the setpoint of 90% reaching a maximum of 73% after the second gas puff. The conservatively set K_i was not able to affect and achieve a consistently higher radiation fraction by injecting more helium into the scrape-off layer. For a factor of ten larger K_i , the process value $u_{PV}(t)$ would be dominated by an integral component of error compensation later during the discharge and increase the radiative exhaust. At this point however, the gas injection is insufficient for radiatively cooling the SOL and increasing the radiative power loss to a high enough level for plasma detachment. Furthermore, this combination of K_i and K_p is unable to maintaining a constant, high radiation fraction $> f_{\text{rad}} 80\%$ in this scenario. The large gaps in gas feedback, which cause the radiation fraction to decrease significantly in-between valve activations, are basically defined by K_p in the beginning. This can potentially be bypassed and enhanced by a smaller, conservatively scaled differential component K_d . The significant drops in P_{rad} after each gas puff could be damped by a similarly proportioned K_d like the previously constructed $10K_i$, since the differential component scales $\propto dP_{\text{pred}}^{(1)}/dt$.

To summarize, this conservative set of proportional and integral PID components K_p and K_i , without a derivative part K_d , is not able to achieve stable detachment by helium impurity seeding. The gas injection is insufficient in radiatively cooling the scrape-off layer. A positional shift of peak radiation location can be observed in the chord brightness profile during impurity gas injections, although the outermost electron densities and temperatures do not indicate perturbations during those times. At the achieved radiation fraction of $\approx 75\%$, an improved particle and power exhaust is expected, which is supported by the increasing P_{rad} at relatively stable W_{div} under the assumption of a constant target heat load F_{div} . The desired, as well as theoretically predicted level of f_{rad} [67], at which a significant inwards shift of impurity radiation and plasma profiles is expected, was not achieved.

The possibility of feedback, based on a real-time radiation prediction was presented. Latency and accuracy of the proxies, defined in equation (3.1)

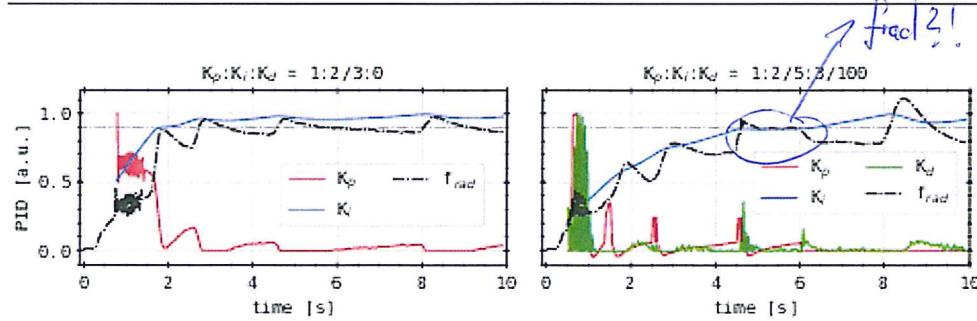


Figure 3.17.: Adjusted, simulated PID controller response based on f_{rad} in XP20180920.31. The impact of the feedback on the radiation fraction was modelled linearly by a cost function around a target \hat{f}_{rad} . The setpoint SP is set to 0.9 or 90%. (left): The algorithm was allowed to control K_p and K_i . The ratio is noted above, while their values are shown normalized. (right): PID control derived K_i and K_d . The proportional K_p was taken from the experiment data, as seen in figure 3.13.

therein are subject to the underlying $f_{\text{rad,XP}}$.

For a synthetic PID control, including a K_p as in the experiment of the modelled after radiation fraction, results also now feature a differential component K_d . The PV increases towards 90% much slower compared to the previous case. At 5s, $f_{\text{rad}} \approx 0.9$ is reached with a short plateau, after which K_i and K_d are not able to compensate the missing proportional scaling. Due to the consistently larger error, the integral component increases and adds to the existing peak in $f_{\text{rad,XP}}$ at 9s. The ratio between the individual contributions therefore also differs significantly, where $K_p : K_i : K_d = 1 : 2/5 : 3/100$. Larger values of K_d lead to strong oscillations, since no dampening of the control u_{PV} to $y(t)$ is applied. For an ansatz of

$$y(t) = (1 - \hat{f}_{\text{rad}}) e^{-(t-N\Delta t)} u_{\text{PV}}(t) + f_{\text{rad,XP}},$$

with exponential dampening over $N \in (5, 20)$ samples of length Δt , results do not improve. Including this model of response, the control in figure 3.18 does not manage to achieve a constant 90% radiation fraction within the 10s experiment duration. Though, the overshooting of the integral part is slightly reduced by a stronger K_d .

With this approach, it is possible to model and discuss the impact of latency and response of the plasma with regard to the feedback control $u_{\text{PV}}(t)$, as well as to study the influence of potential delays between the feedback

3.3. Experimental achievements

and equation (3.2) as $P_{\text{pred}}^{(n)}$ and calculated by the bolometer system using algorithm 1, are adequate for the application of impurity seeding feedback. A different combination of proportional and integral error compensation K_d and K_i , or even an entirely new set with a differential contribution K_d can provide the necessary feedback to achieve the desired $f_{\text{rad}} = 90\%$. To support this claim, a *synthetic* PID controller, which was previously applied to the system in figure 3.6, has also been used to find different contributions to the control function u_{PV} for achieving the desired 90% level of radiation fraction for the above experiment. In order to adequately address the plasma scenario, the process value is based on the provided $f_{\text{rad},\text{XP}}$ through the proxy $P_{\text{pred}}^{(1)}$. Two scenarios of PID control are presented: K_p and K_i can be adjusted, as well as K_i and K_d are optimized for a given K_p from the experiment. Assuming the system value to be

$$y(t) = (1 - \hat{f}_{\text{rad}}) u_{\text{PV}}(t) + f_{\text{rad},\text{XP}},$$

How?

where \hat{f}_{rad} is used to define the cost or efficiency of the control output towards plasma radiation adjustments. For example, to achieve the results shown in figure 3.17, $\hat{f}_{\text{rad}} = 0.5$ or 50%. This is in line with theoretical predictions of operational detachment scenarios, where higher density scrape-off layers are stable at lower impurity concentrations and higher neutral pressures. Furthermore, the control is given a 0.5s penalty, as is the case during the actual experiment in order to avoid early gas feedback and cooling of the plasma.

In the case of optimized integral and proportional components, K_p roughly mimics the behaviour that was experienced during the actual experiment. In the beginning 0.5s of feedback, K_p shows fast scale oscillations due to numerical artifacts. The response of K_i is two orders of magnitude larger when compared to the respective quantities in the experiment. The relative level is noted as $K_d : K_i = 1 : 2/3$. The integral compensation also starts early, together with the proportional contribution, in contrast to the experiment. The resulting *synthetic* radiation fraction model quickly, after the penalty time, rises to the setpoint of 90%. The enclosed small steps in K_p , in addition to oscillations in the underlying f_{rad} from the experiment, lead to under- and overshooting of the mimic radiation fraction around the setpoint. Beyond 5s, the integral K_i has significantly leveled out the process value, which causes K_p to decrease and become negligible. Remaining peaks

problem of
revered
plasma reaction
there is another
problem with the
setpoint, (think)
rescaling

I don't
understand this

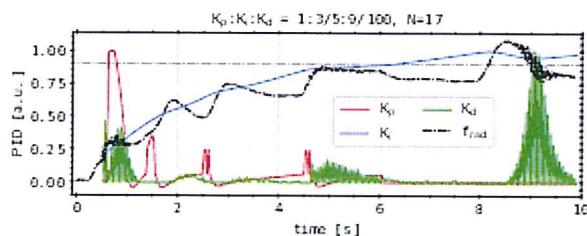


Figure 3.18.: Simulated PID response based on f_{rad} in XP20180920.31 including an ansatz with damping over $N\Delta t$ previous samples. Results are obtained for $\hat{f}_{\text{rad}} = 0.5$ and $N = 17$.

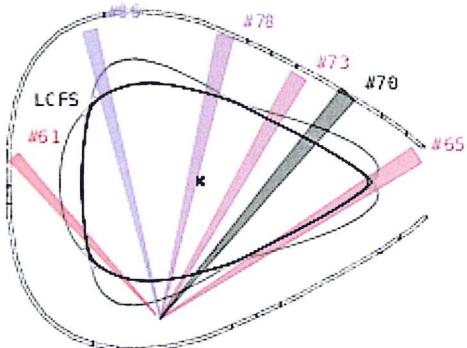
I don't understand what is done
and what we learn from this...
I have an idea, but
I am not sure. Sorry!
Rewrite this and
explain it to me
(we need to talk
here.)

3.3.1. Radiation feedback controlled discharge

The previously discussed real-time radiation feedback experiments were not able to achieve stable, controlled plasma detachment. However, the experimental results and control data indicate that optimizations of the feedback system and scenario may lead to controlled detachment. It was shown that, on the one hand, the large latency of the radiation feedback and plasma response require a more aggressive setting of the PID controller responsible for the gas injection, and on the other hand, the region of interest and channel selection of the bolometer for feedback purposes has to be optimized in order to adequately support the actuation of the valves. An experiment with a radiation fraction $f_{\text{rad}} \geq 90\%$ and a significant reduction of target heat loads, involving the injection of hydrogen into the scrape-off layer based on real-time approximations of the radiative power loss will be discussed hereafter.

The experimental data can be seen in figure 3.20. Discharge number XP20181010.32 consisted of 9.2 s of plasma with maximum 6.23 MW of electron cyclotron heating power and total input energy of 55 MJ. This plasma was heated by microwave radiation coupling to the second-harmonic ordinary mode. The plasma vessel was pre-filled by five, 5 ms gas puffs of hydrogen at ≈ 200 mbar L/s. The fast thermal gas injection system also seeded hydrogen at 750 mbar into the scrape-off layer. The responsible gas injection valves

Figure 3.19.: Line of sight selection for experiment number XP20181010.32, only containing VBC channels. The LOS cones cover all magnetic islands, as well as the core region and parts of the in-between X-points of separatrices.



are the same as in the previously presented experiment. (However, the net toroidal current increased to $\approx 0.9\text{ kA}$ over the course of the discharge in counterclockwise direction.) *Relevant?*

In order to provide a more representative radiation power loss proxy for a scenario involving SOL gas injections, the lines of sight selection was changed accordingly. The subset $S = \{65, 78, 61, 73, 86\}$ was adjusted to consist only of channels from the vertical bolometer camera. The line of sight selection can be seen in figure 3.19. Previously discussed results showed significant differences between the chord brightness measurements, as well as their toroidal, global power loss extrapolation between the two cameras. The vertical camera, thus far, has seen a greater response of the plasma radiation around the separatrix to the feedback gas injections and overall higher maximum emissivities due to its respective line of sight geometry. Because of the connection along the magnetic field lines between the seeding valves and lower scrape-off layer of the triangular cross section, the vertical camera is assumed to be more suitable to measuring the impact of gas feedback. So far, the chord brightness indicated the impact of radiation feedback on P_{rad} to be largest close to the VBC aperture and around the magnetic islands on the inboard and lower part of the triangular cross section, where the bolometer is located. The above line of sight selection of vertical camera detectors covers those particular areas, as well as the in-between located X-points, where at very high radiation fractions $f_{\text{rad}} \rightarrow 100\%$ condensation of radiation is expected.

Within the first 0.5 s, the ECRH increases to 2.3 MW and 4 MW for $\approx 0.2\text{ s}$ respectively. After, the input power is raised again to 6.1 MW between 1-

| *clarify*

3.3. Experimental achievements

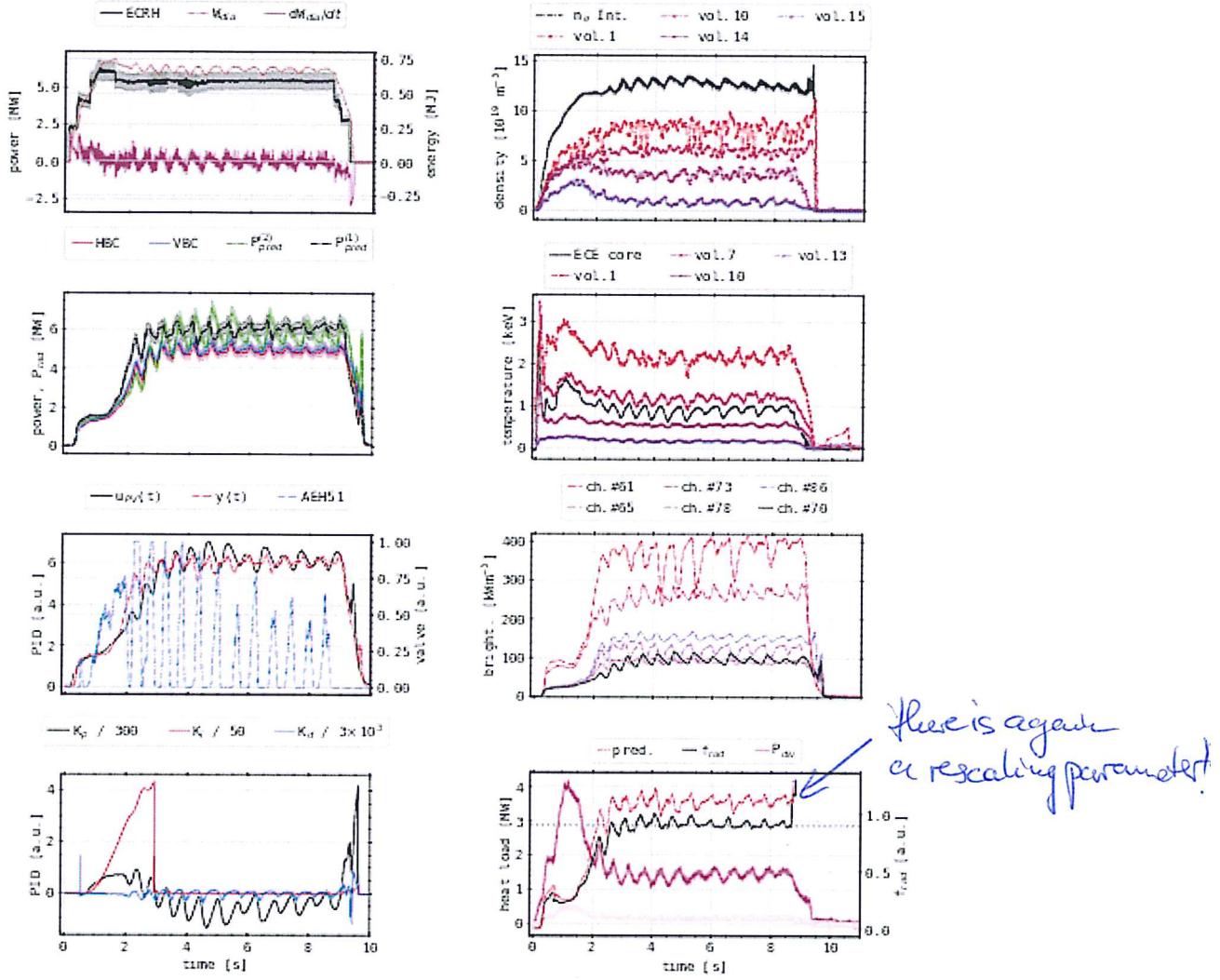


Figure 3.20.: XP20181010.32:

(top left): ECRH power, plasma energy and loss (center left top): P_{rad} , prediction values (center left bottom): PID control $u(t)$, process value $y(t)$ and valve actuation (bottom left): Control components (top right): Electron density (center right top): Electron temperature (center right bottom): Bolometer channels for $P_{\text{pred}}^{(n)}$ (bottom right): Radiation fractions and target heat loads (individual, integrated).

1.5 s. For the following 7 s, the microwave power is kept constant at 5.5 MW, with three smaller steps from 8.5 s to 9.2 s down to zero. The diamagnetic energy rises more gradually, with small changes in slope for each ECRH stage, to 0.75 MJ at 1.5 s. Afterwards, the level of W_{dia} decreases slightly over the course of the discharge to 0.65 s, on which oscillations of 50 kJ amplitude are superimposed. Those perturbations are in sync with the activation of the gas feedback. After the ECRH drops from 5.5 MW, the plasma energy declines within 0.5 s to zero. The resulting dW_{dia}/dt indicates that the initial heating stages are absorbed with up to 2 MW per increment by the plasma, while the power deposition decreases with increasing plasma energy towards the following microwave power step. Following the last ECRH stage before 1.5 s, the plasma energy exhaust oscillates accordingly between -1–1 MW, until the heating power steps down and W_{dia} drops to zero. Conclusively, dW_{dia}/dt represents the energy dissipated from the plasma with -3 MW from 9.2 s on until W_{dia} is zero.

Within the first two ECRH stages, the radiation power climbs to 1 MW, after which it shortly plateaus and slowly increases to 4.5 MW towards 2.2 s. On top of a slight increment of base level P_{rad} to 5 MW, superimposed oscillations as in W_{dia} with amplitudes of up to 0.5 MW can be seen. However, these perturbations are *retarded* to the activation of the gas valves, i.e. peaks in P_{rad} occur 0.2–0.3 s after the respective gas injection. They also decrease over the course of the discharge. After ECRH shutdown, the radiation power declines sharply, with a ~~conclusive~~ peak due to particle recombinations as the plasma collapses. The horizontal and vertical camera measure a P_{rad} that is within their respective error bars, while $P_{\text{rad}, \text{VBC}}$ is consistently larger. Both of the real-time proxies show larger radiation powers than the full camera extrapolations. Before the second oscillation in P_{rad} around 3 second, the single channel approximation $P_{\text{pred}}^{(2)}$, based on a vertical camera channel no. 70, indicates smaller values, however with greater perturbation amplitudes afterwards. The subset prediction $P_{\text{pred}}^{(1)}$ increases early and presents an up to 1 MW consistently larger radiation power loss. Oscillations therein have slightly smaller amplitudes compared to the other proxy, while both level around 6 MW.

The control function $u_{\text{PV}}(t)$ generally follows the behaviour of the process value $y(t)$. The gas injection valves are activated when, on the one hand, the overall radiation level as presented by the PV in the beginning is too low, or on the other hand, during falling edges in the oscillations thereof.

*Important:
Consistent with
above*

*too much
irrelevant
details*

3.3. Experimental achievements

directly caused by the subsequent gas puffs. At first, during the initial $y(t)$ plateau, the valves gradually increases gas injection to 75% of its maximum until 2 s. Within the next 2 s, the valve is opened four times fully for ≈ 0.3 s each. Afterwards, a similar frequency and duration is maintained at a significantly lower, over time decreasing intensity until 8.75 s. The individual PID components below indicate a dominant proportional and differential contribution K_p and K_d for most parts of the feedback. In the beginning however, for the increasing seeding before 2 s, an integral K_i equally contributes to the actuation of the gas valves. Hence, K_p and K_i trigger the system in the beginning, where the radiation level is still low and therefore f_{rad} far away from the setpoint of 90%. As soon as $y(t)$ indicates > 5 MW of radiation power, K_p and K_d particularly shape the control function and valve activation. The proportional component is compensating, for one thing, the individual oscillations due to the feedback, and then again a slight increase in process value, which in itself overshoots the setpoint. The significantly faster reaction of the differential K_d enhances the feedback response to the intrinsic oscillations. After 9 s, when the ECRH decreases and the discharge is eventually terminated, the PID components produce large values, which are not relevant to the feedback anymore.

The presented volumes of the electron density measurement by Thomson scattering have been changed with respect to the previous experiments, though their ordering remains the same. After a steep increase until 1.5 s into a plateau, the electron density of both line integrated interferometer and Thomson scattering measurements show oscillations similar to the ones in P_{rad} with varying intensity. The line integrated n_e goes up to $\approx 1.4 \times 10^{20} \text{ m}^{-3}$ during the on and off of the feedback. From the inside out, the Thomson scattering volumes show descending electron densities of $9 \times 10^{19} \text{ m}^{-3}$, $6 \times 10^{19} \text{ m}^{-3}$, $3.5 \times 10^{19} \text{ m}^{-3}$ and $1 \times 10^{19} \text{ m}^{-3}$ respectively. The further inward volumes no.1 and no.10 increase slightly during and beyond the initial gas injection up to 2 s, however no.14 and no.15 decrease after $P_{rad} \approx 3$ MW around 1.5 s. Line integrated interferometry measurements show sawtooth-like perturbations of the electron densities of $1 \times 10^{19} \text{ m}^{-3}$ in sync with the actuation of the feedback valves. Thomson scattering volumes no.1 and no.10 indicate no or negligible oscillations. Similar, but less pronounced peaks and drops compared to the interferometry are visible in volumes no.14 and no.15. All densities drop to zero after ECRH shutdown at 9.4 s.

Why is K_i shut off?
This is not its role and
prevents stable
operation...?!

Shorter

The electron temperatures are presented for different Thomson scattering, as well as a central electron cyclotron emission volume. A large maximum of 3.4 keV is found in the first 0.2 s after the start of ECRH. In descending order, this peak is shown by two innermost volumes no.7 and no.1, as well as the ECE core measurement and volume no.10. A further outward electron temperature measurement displays no such peak. By the initial ECRH stages, T_e increases again after a drop from the previous maximum to up to 3 keV in the innermost Thomson scattering volume. Number 7, ECE and no.10 respectively show lower electron temperature values down to 0.9 keV. During the first, longer gas injection, T_e collectively decreases for all measurement locations, with decreasing impact from the inside outwards. Volume no.10 and no.13 show only very little impact of the radiation feedback, though subsequent perturbations by the valve activations are still noticeable. The ECE and two inner Thomson scattering volumes indicate larger responses to the gas injection of up to 0.4 keV around their respective average of 0.8 keV, 2.2 keV and 1.2 keV. Further outwards, the electron temperature T_e in volumes no.10 and no.13 is around 0.6 keV and 0.2 keV, with variations of at most 0.1 keV. These oscillations are anticyclical to the gas injections and radiation power loss measurements, i.e. when P_{rad} increases due to a hydrogen puff in the scrape-off layer, the temperatures decrease and vice versa. After 8.5 s, where the ECRH starts to decrease, the electron temperature quickly drops to zero in all locations before 9.5 s.

The chord brightness of the subset $S = \{61, 65, 70, 73, 78, 86\}$ for radiation prediction $P_{pred}^{(1)}$ and channel no.70 for $P_{pred}^{(2)}$ largely resemble the already presented P_{rad} and aforementioned proxies. The line integrated radiation power density greatly differs among the selected channels and lines of sights. The color of the plots matches the, in figure 3.19 presented lines of sight. Channels no.70, no.86, no.73 and no.78 share a shallower slope than the overall P_{rad} . They also present similar or slightly larger average emissivities like in XP20180920.31 of $80\text{--}160\text{ kW/m}^3$. A noticeable temporal shift between the aforementioned channels can be seen, i.e. no.70 has peaks where no.78 and no.86 have local minima, while maxima are found later the higher the emissivity. The response to the gas injection is within 40 kW/m^3 for those channels during this experiment. The radiation intensities measured by channel no.65 and no.61 greatly differ from the previous ones. Both show a very steep rise during the initial heating stages, with a subsequent plat-

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3.3. Experimental achievements

and slight decrease until 1.5 s. After, channel no.65 goes up to average $\approx 290 \text{ kW/m}^3$, with oscillations of up to 60 kW/m^3 . Channel no.61 reaches a base line of 360 kW/m^3 , from which perturbations due to the gas injection vary between $40\text{--}100 \text{ kW/m}^3$ over the course of the discharge. Between 4–5.5 s and around 6.75 s, the peaks and drops in no.61 are extraordinarily larger than during other gas puffs. These oscillations appear retarded to the hydrogen injections by around 0.2–0.3 s. A small increase in base line of $\approx 40 \text{ kW/m}^3$ during the cycling of the feedback is also visible in channel no.65 and no.61. Their local maxima and minima occur later than the previous channels, which measure lower emissivities. However, after the ECRH step down at 8.5 s, the radiation intensity starts to decrease first in those channels, before no.73, no.78, no.86 and no.70 peak and sharply drop to zero around 9.5 s.

The resulting radiation fractions of both the presented proxy $P_{\text{pred}}^{(1)}$ and mean between $P_{\text{rad,HBC}}$ and $P_{\text{rad,VBC}}$ are shown next to the heat load of the individual divertors and the integrated P_{div} . Both proxy and P_{rad} show an overall similar behaviour of f_{rad} , though, as seen before in $P_{\text{pred}}^{(1)}$, the total radiation power loss is being overestimated by the prediction, which leads to a larger and even greater than unity ($P_{\text{rad}} > P_{\text{ECRH}}$) radiation fraction. During the initial stages in microwave heating, f_{rad} quickly increases to 30–40%, before dropping again after the final ECRH power is reached. By the first, longer gas injection, the radiation fraction goes up to 80%. Subsequent valve actuations increase $f_{\text{rad}} > 90\%$ towards 3.5 s. For the power loss derived from the mean P_{rad} of both camera arrays, the following gas puffs cycle the radiation fraction between 90%–100%. Towards the end of the experiment, i.e. 9 s, the perturbation amplitude decreases and the mean $f_{\text{rad}} \rightarrow 90\%$ due to compensations in control $u_{\text{PV}}(t)$ and reduced gas injection. Since $y(t) = P_{\text{pred}}^{(1)}$ and the prediction overestimates P_{rad} , the radiation fraction provided to the feedback control is significantly larger than 100% during the alternation of the valve control. In accordance to P_{rad} and the adjustments in gas injection, the perturbations in f_{rad} decrease over time, where in the beginning variations of up to 15% can be seen. From prediction $P_{\text{pred}}^{(1)}$, f_{rad} even reaches 120% and oscillates between 115–105%. The integrated target heat load increases quickly until the maximum microwave heating is achieved at $\approx 1 \text{ s}$ to 4.2 MW. Afterwards, by the initial long and subsequent shorter hydrogen injections, P_{div} decreases to 2 MW at first and then averages around 1.4 MW over the following gas puffs. Similar to P_{rad} , the response in target heat load

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~~Steffen Otter~~

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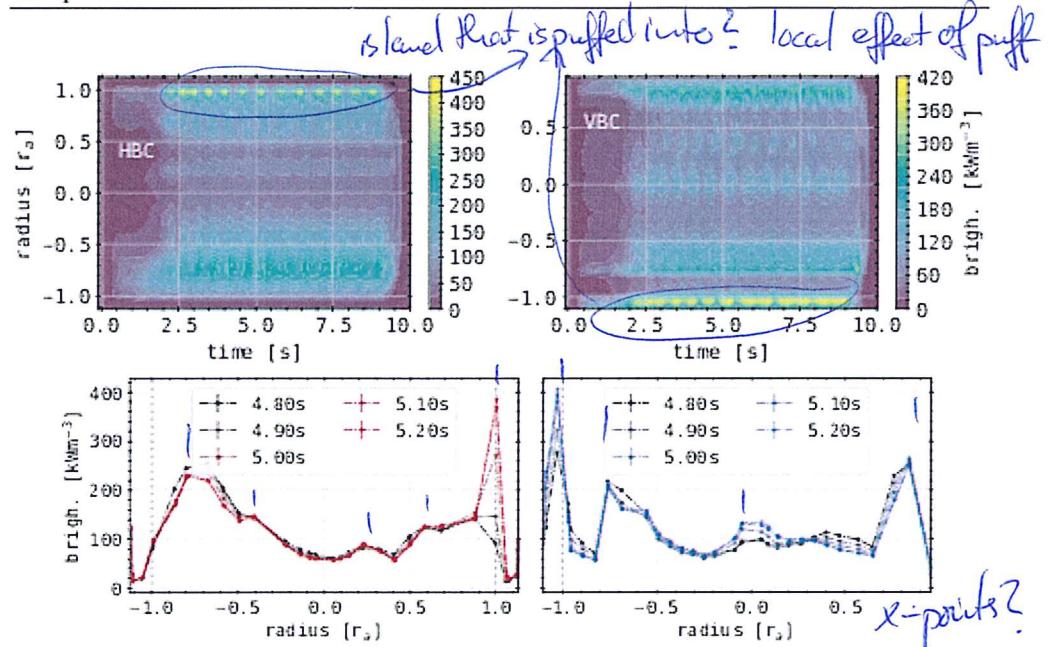


Figure 3.21.: XP20181010.32

Chord brightness profiles. (top) Two-dimensional contours (bottom) snapshots for both cameras, HBC and VBC around feedback gas puff.

to the later valve actuations decreases towards 8.5s. After the final gas puff, P_{div} decreases to 0.8 MW, from which it drops to zero at 9.3s. The heat load of the individual targets is largely symmetrically distributed. However, the divertors HM51 and HM10 indicate a non-negligible, larger thermal power of 50–100 kW, or 20–40% more than the other modules. In direction of the counterclockwise, net toroidal plasma current, the targets HM51 and HM10 in the respective half modules no.5 and no.1 are connected with each other by parallel transport along field lines. Furthermore, with respect to figure 3.1, this also connects to the responsible gas valves in module HM51, as well as the lower and inside magnetic islands. 22

In figure 3.21 the chord brightness profiles of both vertical and horizontal camera, as well as temporal snapshots around one hydrogen gas puff at $f_{\text{rad}} \approx 90\%$ can be seen. Similar to the previous experiment, the majority of emissivity is located at or around $r_{\text{eff}} = \pm r_a$. The poloidal asymmetry in chord brightness as measured by the HBC is reversed compared to before, with local maxima being strongly radially focused at the separatrix

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$r_{\text{eff}}/r_a = 1$. For each of the gas puffs and subsequent peaks in P_{rad} , a concurrent peak in the chord brightness profile with an overall radial inward shift of emissivity can be found. The temporal snapshots emphasize the (radial) asymmetry during gas injections, as well as show the evolution of the maximum emissivity at the separatrix $r_{\text{eff}} = r_a$. At $-r_a$, the emissivity increases from 80–380 kW/m³ for one hydrogen injection. The small inward shift is visible in the peak position on the negative end of the radial spectrum. The absolute maximum is located opposite of the peaks that have been measured during the previous experiments. The responsible lines of sight cover areas that are opposite to the aperture of the VBC and do not overlap with the magnetic island on the inboard side. The chord brightness of the vertical camera now shows a small poloidal asymmetry, which is supported by the individual profiles below. Local maxima can be found around the separatrix at $r_{\text{eff}} = -r_a$ after gas injections, with parallel inward shifts of the radiation on both sides of the profile. As in the top of the HBC chord brightness, the radial gradients are much stronger on the bottom of the vertical camera profile. In addition, there is a significant gap in the radial evolution of the emissivity on the inside of the separatrix of the inboard side. In comparison to the peak evolution for a single gas puff in the horizontal profile, the vertical camera measures only an increase of $\approx 33\%$ around $-r_a$.

The resulting power balance for this discharge can be seen in figure 3.22. Within the first 2 s $P_{\text{bal}} = 0$ is satisfied inside its errorbars, though the individual lines for P_{rad} indicate an underestimation of losses of ≈ 1 MW. After the first gas injection, the power balance starts to oscillate with the individual gas puffs, as seen in the radiation power loss P_{rad} , plasma energy W_{dia} and its derivative, as well as the divertor heat load P_{div} . On average, P_{bal} indicates a significant overestimation of the cumulative power losses of ≈ 1.5 MW. Perturbations due to the feedback injections cycle around this level by about 1 MW, with decreasing impact of each gas puff towards the end of the discharge. Due to the large errorbar of ≈ 0.8 MW, the power balance can be assumed to be ~~occasionally~~ off by ~~only~~ 0.3 MW following individual valve activations. After the ECRH starts to step down, P_{bal} declines in two drops due to the collapse of the plasma and dissipation of the stored energy to -2.5 MW and -4 MW.

The measurement data of this experiment presents evidence of *feedback controlled stable plasma detachment*. Hydrogen gas injection into the scrape-off layer, controlled by real-time radiation power loss measurements was able

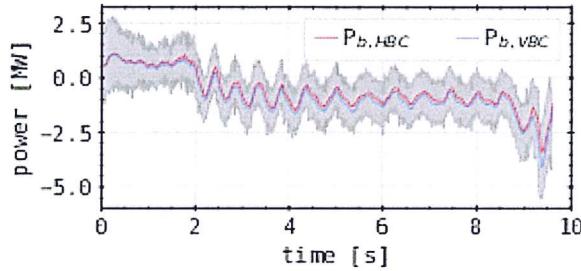


Figure 3.22.: XP20181010.32

Power balance for both P_{rad} of the individual camera arrays and a (grey) mean with errorbar, derived from the intrinsic errors of the contributing physical quantities.

to achieve and maintain a radiation fraction $f_{\text{rad}} > 90\%$ for extended durations. Its application raises the overall radiation level, as well as significantly increases the emissisivity along and close to the magnetic seperatrix. The achievement of $f_{\text{rad}} \geq 70\%$ is accompanied by an inward shift of radiation towards the seperatrix and center of the confined region. This is supported by the fact that the response to the individual gas puffs is, on the one hand, varying strongly in amplitude across the chord brightness profile, and on the other, temporally retarded with respect to different parts of the profile and their respective peaks and drops. The chord brightness profiles suggest radiation condensation, after increasing the radiation fraction $> 80\%$, in the inboard magnetic island and X-points towards the neighbouring islands on the top and bottom of the triangular cross section. Therefore, the lines of sights covering those areas measure factor two to three larger emissivities compared to detectors mainly viewing the magnetically confined region. The magnetic island located on the inboard side is connected to the responsible feedback valves in half module no.5 (AEH51) by the plasma trasport along field lines. The focussing of radiation power loss at X-points and close to the seperatrix, i.e. in magnetic islands at significantly higher $f_{\text{rad}} > 70\%$, was predicted by results from the similarly configured W7-AS experiment and simulations of impurity radiation using EMC3-EIRENE[67, 68, 77]. In addition, there is no degradation of the plasma stored energy, as W_{dia} ~~only~~ increases until $f_{\text{rad}} > 80\%$ and only small pertubations of the overall level due to individual gas injections can be found. The line integrated

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3.3. Experimental achievements

electron density presents similar behaviour, where n_e starts to plateau at $f_{rad} \approx 80\%$ and then slightly increases over the subsequent hydrogen injections. Individual Thomson scattering volumes close to the magnetic axis indicate smaller densities, though with comparable development as the line integrated density at lower values. No concurrent perturbations due to the gas puffs can be seen. The two outermost scattering volumes indicate a drop following their initial increase after the radiation fraction reaches 40%. They present, in contrast to the volumes further inwards, significant perturbations due to the gas injections, which are anticyclical to f_{rad} . The electron density close to the separatrix inside the confined area decreases $\approx 30\%$ when the radiation fraction goes from 90% to 100% and vice versa. Contrary, the electron temperature measured in the innermost volumes and from cyclotron emissions presents inverse behaviour to f_{rad} , while volumes further towards the separatrix display negligible interference by hydrogen injections. Also, the prior similarly drop from their respective maximum after $f_{rad} > 40\%$. Towards high radiation fractions $f_{rad} > 70\%$, a significant decrease of the plasma density close to the separatrix is expected, which leads to a steepening of the profile. For $f_{rad} \approx 90 - 100\%$ the perturbations in volumes closer to the separatrix, opposite to changes in radiation fraction, indicate the inward shift of the plasma and therefore reduction of perpendicular transport to the scrape-off layer. However, in this case the accompanying temperature distribution flattens and only changes significantly closer to the magnetic axis. Though, assuming constant temperature profiles close to the separatrix, the net perpendicular plasma flux across field lines is reduced due to smaller gradients and the decreased density. Lastly, the integrated divertor heat load supports the previous arguments. After the radiation fraction increases beyond 40%, P_{div} decreases with increasing f_{rad} until it is more than halved at $f_{rad} \approx 80\%$. With each subsequent gas injection and peak to 100% radiation fraction, the target heat load decreases further, before receding again with the concurrent drop in f_{rad} . The significantly larger heat loads of divertor modules HM51 and HM10 suggest a substantial contribution of the plasma radiation to the incident power of plasma target components in scenarios with very high f_{rad} and controlled radiative cooling in neighbouring plasma volumes. In this particular case, the parallel transport along field lines directly links the aforementioned modules, feedback gas valves and, indirectly in the second to next half module, inboard magnetic island in the bolometer measurement plane. The chord brightness reflects this as

2 How
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where? Why?
Profiles?
Where do you get
with this?

well, where the emissivity is twofold of that on the opposite side of the profile and at 400 kW/m^3 extraordinarily high when compared to the previous experiments.

In conclusion, this radiation feedback controlled experiment has achieved *stable plasma detachment* by gas injections into the scrape-off layer. The target heat load was significantly reduced by a factor of over two, while a radiation fraction and therefore radiative power loss $> 90\%$ of the input heating power was maintained by the PID controlled feedback system. During the gas injection, neither degradation of the plasma stored energy nor the core electron density, i.e. close to the magnetic axis, was found. However, the evolution of the electron temperature profile in its entirety indicates a stronger coupling to the radiation fraction than the electron density. A small but noticeable inward shift of the radiation distribution profile is measured by both bolometer cameras. The feedback control is able to consistently increase the radiation fraction $> 80\%$ by controlled injection of hydrogen, in this case the working gas in contrast to low- to medium-Z impurities, like helium in the previous experiments. In succession to reaching $f_{\text{rad}} \approx 90\%$, the feedback input underestimates the impact of the individual gas injections, which leads to strong oscillations in P_{rad} of up to 10% of the heating power for full activation of the valves. Towards the end of the experiment however, the PIDs differential component can steadily reduce the overshooting of the control and therefore reduce the oscillations in P_{rad} and conclusively f_{rad} . Another significant factor to the cycling of PID control and valve activation is the latency of the feedback prediction and response of the plasma to the injections. Although the radiative power loss proxy $P_{\text{pred}}^{(1)}$ is consistently larger than P_{rad} from either cameras measurement, the adjusted channel selection S in figure 3.19, consisting of detectors from the vertical camera arrays viewing the magnetic islands and X-points, has improved the quality of the feedback.

3.3.2. Comparison to different feedback control

The real-time radiation feedback system was able to perform plasma control based on other properties than the radiative power loss. In section 3.1, the dispersion interferometer, measuring the line integrated electron density, and optical fiber filterscopes, with lines of sight parallel to the divertors were introduced as additional candidates for plasma detachment control with the

3.3. Experimental achievements

thermal helium beam gas valves as an actuator. In order to fully discuss the quality of the radiation feedback and categorize its results, a comparison to the achievements of the different control proxies is imperative. ^{ers} ~~the following~~ ^{After} ~~done in~~ ^{the following}.

Electron density from dispersion interferometer

Similar to XP20181010.32, which featured controlled detachment by radiation feedback, this example of electron density feedback in experiment number XP20181016.16 consisted of hydrogen gas injections in half module HM51 by the same valves. Measurement results of this pre-filled hydrogen plasma can be seen in figure 3.23. A constant 5 MW of O2-mode microwave heating was applied for 21 s, after two initial, smaller steps at 3–4.5 MW within the first 1 s, before decreasing to 4.5 MW for the remaining 9 s. The plasma energy increases with small changes in slope, according to the steps in heating power, to 0.65 MJ. After the feedback gas injection starts at 2 s, there is a small drop in W_{dia} , which is followed by a decline towards 0.6 MJ until the step down in ECRH. This is also reflected in the plasma energy. Besides the initial heating of the plasma and its collapse after 32 s, the plasma power loss dW_{dia}/dt presents high frequency, random perturbations between -0.1–0.1 MJ, which decrease slightly towards the end of the discharge.

Radiation power loss P_{rad} from both bolometer cameras are within their respective error bars. Following the heating stages in the beginning, P_{rad} plateaus around 2 MW before increasing to 4.15 MW after the hydrogen gas feedback starts at 2 s. The radiation power increases ≈ 0.1 MW before the step down in ECRH. Afterwards, P_{rad} also steps down with the heating power to 4.05 MW, from which it drops to zero as the plasma collapses after 31.5 second.

The process value and feedback variable of this detachment experiment was the line integrated electron density, which was provided by the dispersion interferometer. A slowly declining n_e ramp around $1.1 \times 10^{20} \text{ m}^{-3}$ for this microwave heating and feedback configuration was developed from experience~~s~~ of previous experiments. The density setpoint for this feedback was designed to achieve a constant, high plasma radiation fraction $f_{\text{rad}} > 90\%$ [104]. Consequently, the proportional PID component K_p prohibits gas injections before 2 s, as n_e sharply rises towards its setpoint value. The following gas valves activation is exclusively determined by the integral component K_i . When the downward ramp in density begins, K_i also decreases similarly to

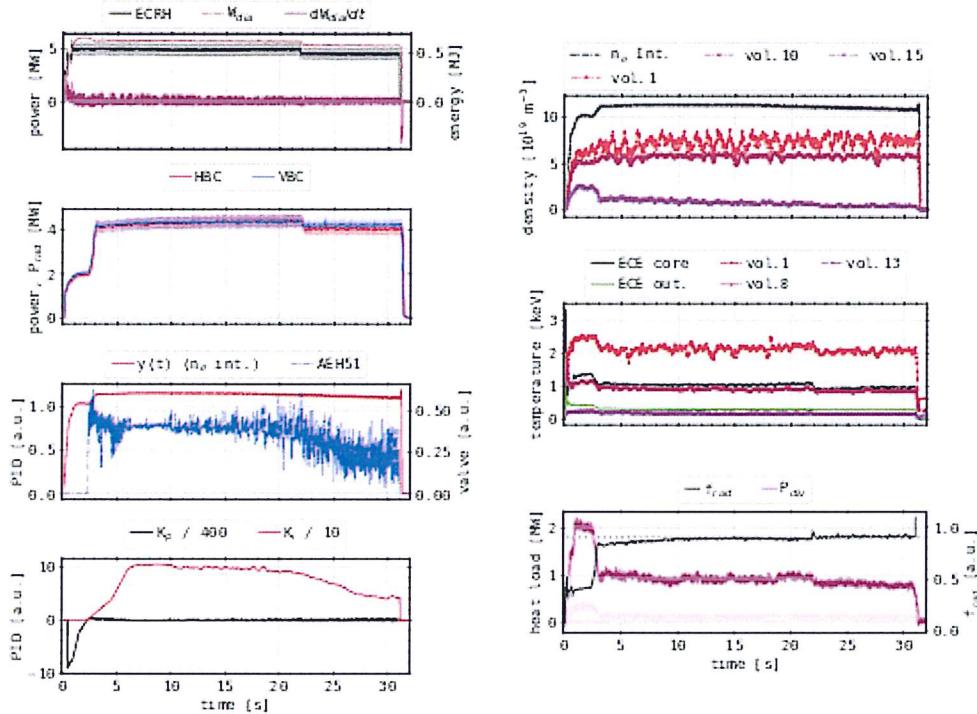


Figure 3.23.: XP20181016.16:

(top left): ECRH, plasma energy and loss (center left top): P_{rad} (center left bottom): Process value $y(t)$ and valve actuation (bottom left): Control components (top right): Electron density (center right): Electron temperature (bottom right): Radiation fractions and target heat load.

the valve actuation by a factor of two. The subsequent hydrogen gas injection starts at 2 s, which increases n_e from $1-1.15 \times 10^{20} \text{ m}^{-3}$. After 17 s, the electron density declines to $1.08 \times 10^{20} \text{ m}^{-3}$ toward the end of the discharge. Thomson scattering volumes further inward exhibit similar behaviour, however at lower values and with smaller responses to the hydrogen injection. Volume no.15 shows a decline after 2 s and a stronger ramp down than the line integrated density towards 31 s, while n_e of inside volumes increases minutely. The electron temperature initially increases quickly to its maximum 2.6 keV in the innermost Thomson scattering volumes. Afterwards, T_e

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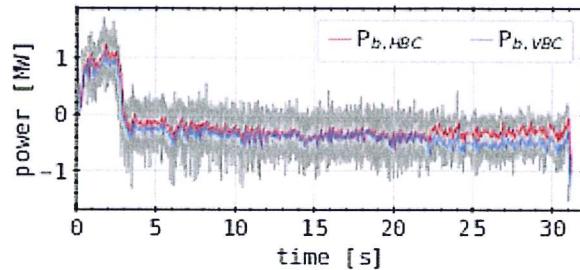


Figure 3.24.: XP20181016.16

Power balance for both P_{rad} (red, blue) of individual camera arrays and a (grey) mean with errorbar.

declines with the gas feedback and then again with the downward step in heating power. This behaviour is found for all temperature measurements, at lower values further outward and closer to the separatrix. Overall, the average electron temperatures are comparable to measurement results in XP20181010.32.

The resulting radiation fraction and target power adjust accordingly to the increase in radiation fraction. As the gas injections increase n_e to its ramp-like setpoint, the divertor heat load P_{div} drops by 50%, while f_{rad} rises to 80% with a small incline to 90% towards 10 s. Integrated heat load and radiation fraction remain relatively constant throughout the rest of the discharge. Similar to XP20181010.32, two divertor modules in HM51 and HM11 close to the gas valves and along the direction of parallel plasma transport show significantly larger heat loads in comparison to the others. ~~Q_{HM51+11} are not connected~~ ~~field lines~~ ~~parallel plasma transport~~ Also, similar to the radiation feedback detachment, P_{div} already starts to decrease substantially as $f_{\text{rad}} \approx 50\%$ and above, as well as T_e and n_e of volumes further outward begin to decline.

The global power balance for this experiment is shown in figure 3.24. There is a surplus of 1 MW of heating power in the initial 2.5 s, which is not accounted for in loss terms. After the feedback starts injecting hydrogen, P_{rad} increases by over 2 MW and P_{div} drops by 1 MW. The resulting power balance adjusts accordingly and indicates ≈ -0.2 MW, while $P_{\text{bal}} = 0$ is satisfied within the errorbar. From 20 s and following the step down in heating power, P_{bal} begins to increasingly deviate from zero over time. The individual lines for either cameras P_{rad} start to diverge more, with P_{bal} from radiative power

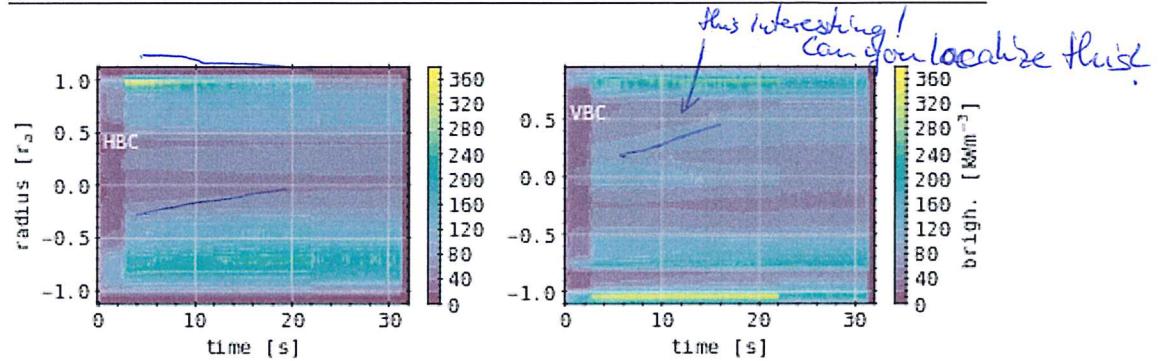


Figure 3.25.: XP20181016.16: Chord brightness profile of HBC and VBC.

loss measurements of the horizontal camera begin closer to zero than from the vertical.

The chord brightness profile for both bolometer cameras can be seen in figure 3.25. Very similar results to XP20181010.32 are presented. The HBC measures a strong poloidal asymmetry, with the maximum emissivity on the top of the triangular cross section throughout the entire discharge. A similar structure on the inside, closer to the magnetic axis, is found for $f_{\text{rad}} > 50\%$. The peaks on both ends of the radial spectrum are located in the same position as before, i.e. $r_{\text{eff}} = r_a, 0.8r_a$. Poloidal asymmetries for $f_{\text{rad}} > 50\%$ in the HBC chord profile decrease towards the step down in heating power at 22 s. The maximum in brightness shifts to the other side at $r_{\text{eff}} \approx 0.7r_a$, while the peak on the negative end of the radial spectrum has almost entirely flattened. Vertical camera measurements also present local maxima at $r_{\text{eff}} = -r_a, 0.9r_a$ for the entire duration of the discharge, with a substantial gap on the inside of the separatrix and negative end of the radial spectrum in the chord brightness. There is no reversal of the local maxima after 22 s in the VBC profile. However, the peak around $-r_a$ has decreased significantly to a similar level of the local maximum at $0.8r_a$. A small inward shift towards the magnetic axis of the chord brightness can be seen. For both cameras profiles, in agreement with the temporal evolution of P_{rad} and in contrast to XP20181010.32, there are no oscillations in the profiles due to cycling of gas injections. Furthermore, the total radiative power loss is 0.5 MW lower than the average during the radiation feedback detachment. The maximum chord brightness however is about 20% and 15% lower for the horizontal and vertical camera, respectively. With the

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step down in heating power at 21.8 s, the chord emissivity also decreases slightly.

In conclusion, the electron density feedback results in a significant reduction of divertor heat load by radiatively cooling the scrape-off layer and increasing the radiation fraction, like it was discussed for the case of radiation feedback. The gas injection system aims to achieve a high performance plasma in terms of core electron density, which is defined by the previously explored n_e setpoint at $1.1 \times 10^{20}/\text{m}^3$. Additionally, a slow ramp down in electron density is programmed in order to compensate for a gradual increase in P_{rad} over time due to a slowly increasing amount of intrinsic impurities. Accordingly, the radiation fraction is well adjusted above 80%, which leads to the already alluded ~~to~~ shaping of the electron density and temperature profiles. The more direct coupling, with less latency when compared to the radiation feedback, of hydrogen gas injections to the feedback proxy n_e yields much smoother and ~~stable~~ reliable results, without oscillations, in target heat load reduction. *Check the controls settings with Neerg. I think we need to discuss this*

Proof?
Zeff, RGT, Hexos

C-III radiation from filterscope

The third candidate for plasma feedback control was the C³⁺ (carbon) radiation intensity in front of the target, which is measured by filterscopes. The results of XP20180920.32 can be seen in figure 3.26. Like before, hydrogen is seeded into the scrape-off layer of a pre-filled hydrogen plasma by thermal gas injection valves in AEH51. In contrast to the previous experiments, this plasma was heated by microwave radiation coupling to the second-harmonic extraordinary (X2-) mode. After an initial step at 1.5 MW, a constant 3 MW of ECRH was applied to the plasma for 9.5 s. The plasma energy increases to 0.5 MJ, from which it declines towards the termination of the discharge. When the gas feedback sets in at 2 s, there appears a small increase in W_{dia} . Its time derivative reflects this behaviour, as two initial increments in heating power and the plasma collapse at 10 s can be distinguished in dW_{dia}/dt . Both bolometer cameras measure similarly behaving radiation powers, while the VBC indicates $\approx 10\%$ larger values than the HBC. After the start of the ECRH, P_{rad} plateaus at 1 MW, with a minor incline slope until 2.5 s. Following the beginning of gas injections, the radiation power loss gradually increases, until slowly equilibrating around 8 s at ≈ 2 MW. About 0.5 s after the ECRH is shut off, P_{rad} drops to zero as the plasma collapses.

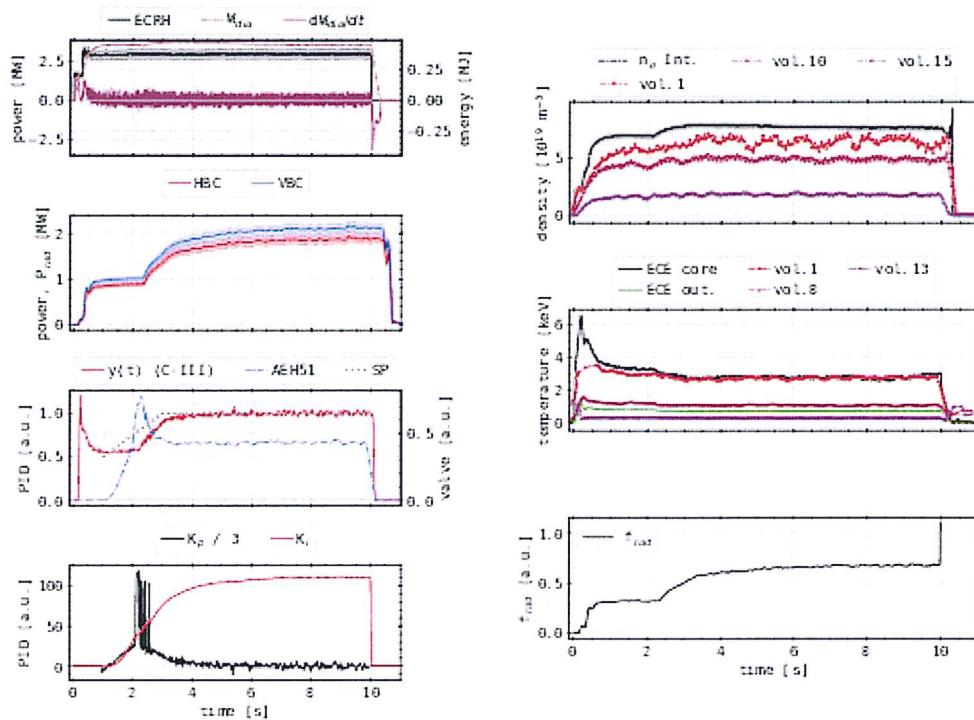


Figure 3.26.: XP20180920.32:

(top left): ECRH, plasma energy and loss (center left top): P_{rad}
 (center left bottom): Process value $y(t)$ and valve actuation (bottom left): Control components (top right): Electron density (center right): Electron temperature (bottom right): Radiation fractions and target heat load.

The PID process value for this feedback control is the ionisation ~~radiation~~ ^{line} of triple ionized carbon C^{3+} , as measured by the filterscopes from their lines of sight parallel to the divertor module HM51. The goal of this kind of control is to ~~reduce~~ ^{target in} ~~localize~~ ^{at a specific location above the target by controlling the} ~~reduction of the~~ ^{heat load of} ~~influence of~~ ^{heat load of} 3LCs

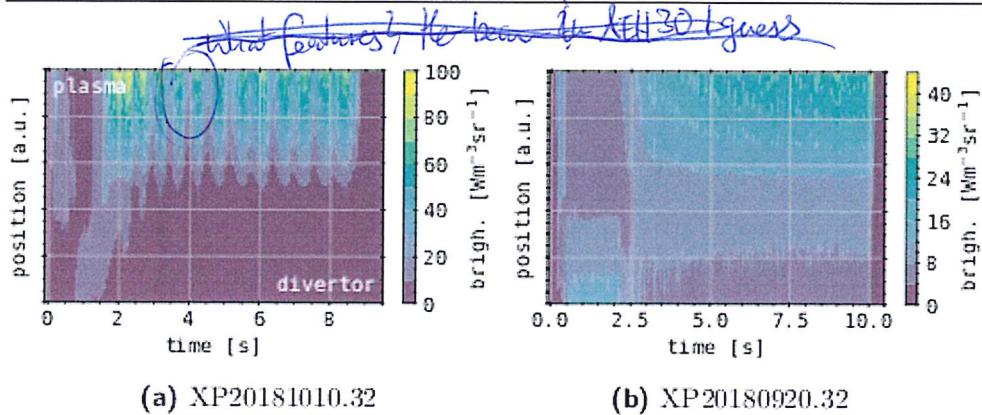


Figure 3.27.: Spatial-temporal evolution of C³⁺ radiation in front of the divertor in HM51. Plasma and target facing side (direction) are indicated. The measurements are performed by the same filterscopes, i.e. same lines of sight.

transport. The specific line of sight is measured from location AEI51, which is, in this specific case, the furthest away from the target. Therefore, the setpoint aims to increase the radiation in this location. The control function activates the valves after 2s in order to radiatively cool the scrape-off layer and increase $y(t)$. After the process value reaches the setpoint, the valve actuation equilibrates in order to maintain the emissivity. Beside the initial opening of the valves, the control function is dominated by the integral PID component K_i .

Electron temperature and density are largely constant throughout the discharge, with minor perturbations due to the hydrogen gas injections. Closer to the magnetic axis, the plasma reaches a maximum density of $8 \times 10^{19} \text{ m}^{-3}$ at $\approx 3 \text{ keV}$. Further outward and closer to the separatrix, the electron density and temperature decrease to $2 \times 10^{19} \text{ m}^{-3}$ at $< 200 \text{ eV}$.

The resulting radiation fraction is significantly lower than during the previously presented detachment experiments. After the start of the gas injections, f_{rad} reaches a maximum 70% at the end of the discharge. In order to further discuss the results of this feedback experiment, the full set of filterscope measurements have to be compared to XP20180920.32. Their respective profiles can be seen in figure 3.27. Experiment XP20181010.32 shows emission closer to the separatrix at the start and during the early development of the plasma radiation, i.e. at low radiation powers. Afterwards, during the first

hydrogen injections, the emissivity is higher closer to the target. After f_{rad} increases $> 80\%$, the C^{3+} radiation detaches from in front of the divertor and moves towards the plasma, where the maximum intensity for this set of lines of sight is found. The previously discussed oscillations in radiation intensity due to the cycling of the gas injections are also present here. The individual peaks appear in sync with the plasma radiation and with retardation to the respective gas puffs. By the results of the filterscopes, there is no emission of C^{3+} ~~emissions~~ close to the divertor for $f_{\text{rad}} > 70\%$. A similar evolution of ~~ionisation~~ radiation from C^{3+} is found for XP20180920.32. At the start of the plasma, the emissivity is largest on the plasma side. This is inverted before the hydrogen injection starts. The emissivity profile changes again with the activation of the gas valves. The local maximum C^{3+} radiation is found on the plasma side. However, the emissivity closer to the divertor is not zero, indicating residual carbon ionisations in this location.

From the results of the filterscope measurements, a significant shift in temperature and density profiles is expected. With respect to the Thomson scattering and ECE electron temperature and density measurements, both feedback experiments show similar results closer to the separatrix. A reduction of impurity radiation close to the target and shift towards the core plasma is found. Presumably, the perpendicular particle and energy transport across the separatrix and scrape-off layer, as well as heat load of the divertor are reduced. In contrast to before, this method does not achieve higher radiation fractions $f_{\text{rad}} > 70\%$. Previous feedback experiments have shown similar results of temperature and density profile changes, as well as reduction of target heat loads for smaller radiation fractions $f_{\text{rad}} < 70\%$. This is also reflected here, as the emissions from C^{3+} are reduced by a factor of two to three for $f_{\text{rad}} \approx 50\%$. Since the impurity radiation is strongly coupled to the underlying n_e and T_e plasma profiles, and the filterscope measurement provides much lower latencies and higher acquisition rates than the radiation proxy, much smoother and more reliable results without oscillations are achieved by C^{3+} emission feedback. The overall plasma performance during the detachment process, in terms of electron density and temperature profile, radiation power and divertor heat load, does not compare to the other experiments due to their inherently different microwave heating configuration[109].

No guess work

still think it's a PID issue

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3.4. Conclusions

In this chapter, the radiation feedback system was introduced. This system has been designed to achieve stable, controlled plasma detachment in cooperation with fast acting, thermal gas valves. Actuation of these valves, which either injected low-Z impurities, like He, Ne or Ar, or working gas ^{fuel} into the scrape-off layer, was controlled by an integrated PID. The goal of the radiation feedback control is to achieve and maintain $f_{\text{rad}} > 80\%$, ^{a preset fraction} (here 80%). ^{wrongly worded} which is expected to significantly reduce the perpendicular particle and heat transport across the separatrix and SOL onto the divertor.

In order to find low latency, high accuracy real-time representations of P_{rad} , two different proxies for predicting the total radiative power loss of the plasma have been established. Due to the intrinsic nature of the comparatively slower radiation measurement of the bolometer and the limited computational performance of the electronics, several assumptions about the radiation distribution have to be made. A subset S of the full set of lines of sight from both cameras is used to calculate $P_{\text{pred}}^{(1)}$ in a similar approach to equation (2.25) in equation (3.1). This simplification is based on the assumption that the radiative power loss can be adequately extrapolated from measurements taken in a smaller area of the chord brightness profile in the bolometer plane. Additionally, information about the line of sight geometry and magnetic configuration, necessary to calculate P_{rad} after the experiment, have to be provided beforehand, therefore limiting the flexibility of the prediction. The second radiation proxy $P_{\text{pred}}^{(2)}$ is based on the measurements of a single absorber \tilde{U}_M , further reducing the contributing area from the triangular cross section to the feedback. This approach also eliminates the time derivative in the bolometer equation (2.19), which requires relatively slowly transitioning or constant radiation distributions. In order to minimize the impact of noise to the feedback, an algorithm within the measurement procedure has been developed, featuring a FIFO array for smoothing and computational load reduction. A minimum total latency of the system of 13.6 ms for sample times of 1.6 ms was found. However, the M sample FIFO array further exacerbates this delay by smoothing over the absorber response with $\approx M \Delta t / 2$ width. ^{usually used $M \rightarrow 4$ delay ≈ 1 ms} Intial feedback experiments with helium lead to a radiative collapse of the plasma by oversaturation with impurities. The latency of the real-time system and the delayed response of the plasma radiation to gas injections can

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≈ 1 ms
+ delay of plasma
response of 200-
400ms

not be compensated by the conservatively programmed PID control. For a more aggressive set of integral and proportional control components K_i and K_p , the system is able to consistently achieve radiation fractions $f_{rad} > 50\%$. Synthetic benchmarks for such a scenario of a virtual PID controller indicate that a set of parameters exists that is capable of increasing and maintaining $f_{rad} \approx 80\%$ with minor oscillations.

Experiment XP20181010.32 shows results of radiation feedback that present stable plasma detachment. For a radiation fraction of 50%, a substantial reduction of target heat load and transport is found. The PID controlled gas injections increase the radiative power loss to a level of $f_{rad} > 85\%$, with remaining oscillations from cycling in valve actuation due to the large latency. Divertors close to the gas valves and along the parallel transport show a significantly larger heat load. The total integrated P_{div} is reduced by at least a factor of two for $f_{rad} \geq 90\%$. From filterscope measurements of C^{3+} ionization radiation ($\text{plasma detachment can be seen for much lower radiations fractions of } \approx 50\%$) *more careful working → heat loads!*

In comparison to other feedback candidates, the latencies of plasma response and measurement complicate the application of real-time radiation predictions. Electron density and impurity radiation feedback perform more *stable* and *smooth* detachment experiments. However, P_{pred} does not require preceding explorations of plasma parameters, like electron density feedback, and the plasma radiation intrinsically provides a scale for detachment quality through f_{rad} . In the current state, real-time radiation control is not feasible for sample times $\Delta t < 1.6$ ms due to the perturbations of the bolometer measurement timing by the feedback algorithm. In the future, a similar setup to the benchmark for actual samples times can be used with a blind absorber, outside the vessel, in order to disentangle the plasma radiation latency and errors in the measurement process.

The real-time radiation system *was* proven successful in achieving stable plasma detachment. In upcoming experimental campaigns, the application of such feedback is supported by the results presented in this chapter. Assumptions about the radiation distribution and its evolution remain valid for the purpose of providing predictions P_{rad} for this purpose.

problem with
PID

Rephrase
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