



UNIVERSIDADE DE LISBOA INSTITUTO SUPERIOR TÉCNICO

Università degli Studi di Padova

Tokamak Magnetic Control Simulation: Applications for JT60-SA and ISTTOK Operation.

Doménica Corona Rivera

Supervisor: Prof. Horácio Fernandes Co-Supervisor: Prof. Nuno Cruz

External supervisor: Prof. Alfredo Pironti

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ABSTRACT			
Abstract en ingles			

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RESUMO

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	are connected [7]. 7

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LIST OF ABBREVIATIONS

@TODO: Review variable lists as writing the thesis

- AC Alternating Current
- ADC Analog to Digital Converter
- ATCA Advanced Telecommunications Computing Architecture
- CREATE Consorzio di Ricerca per l'Energia, l'Automazione e le Tecnologie dell'Elettromagnetismo
- DAC Digital to Analog Converter
- EO Electronic Offset
- IST Instituto Superior TÃl'cnico
- LQR Linear Quadratic Regulator
- MARTe Multi-threaded Application Real-Time executor
- MIMO Multiple-Input Multiple-Output
- PCS Plasma Control System
- PF Poloidal Field
- PID Proportional Integrative Derivative
- RFM Reflective Memory
- SCD Systéme de Contrôle Distribué
- XSC eXtreme Shape Controller
- WO Wiring Offset

LIST OF VARIABLES

@TODO: Review variable lists as writing the thesis

VARIABLES:

 \bullet B_p - Poloidal magnetic field

• I_p - Plasma current

• μ_0 - Vacuum permeability

INTRODUCTION

- 1.1 TOKAMAK PLASMA CONTROL
- 1.2 BEHIND THE PLASMA CURRENT
- 1.3 THESIS OUTLINE

PLASMA CONTROL SYSTEMS

2.1 OVERVIEW OF CONTROL SYSTEMS

The control of plasma position, shape and current among other parameters is one of the crucial engineering problems for present and future magnetic confinement devices. The Plasma Control Systems (PCS) lead with the overall control of fusion devices being responsible also for the plasma configuration and scenarios algorithms [1, Chapter 8]. Currently different PCS's are use in the tokamaks around the world. In this chapter the "DIII-D-like" PCS, the Systéme de Contrôle Distribué (SCD) and the Multi-threaded Application Real-Time executor (MARTe) will be approach, this last one being of special interest due to its extensive utilization in this work.

2.1.1 DIII-D Plasma Control System

The DIII-D-like PCS is use in various fusion research facilities such as EAST(China), K-STAR (South Korea) and MAST (UK). Early documentation regarding the PCS in DIII-D¹ reefers to digitalization of analog signals transmitted to a high speed processor executing a shape control algorithm and then writing the result to a digital to analog converter for driving the controlled systems. The real-time computer used allowed to performed operations with vectors and matrices required for the plasma shape control algorithm [2]. Figure 2.1 shows the block diagram of the DIII-D PCS 30 years ago.

In recent years the DIII-D PCS had extensive software and hardware upgrades. The PCS actual software consists of an infrastructure library core which provides all the routines that are necessary for implementing a basic and generic control system. The current PCS hardware configuration uses a collection of Intel Linux based multi-processor computers running in parallel to perform the real-time analysis and feedback control [3]. New digitizers have been added to the real-time network to increase the number of signals acquired an to control hardware on real-time, several real-time control algorithms were added and real-time data was added to external entities such as web server. [4]. In the current ver-

¹ DIII-D is a D-shape tokamak operated by General Atomics in San Diego, California.

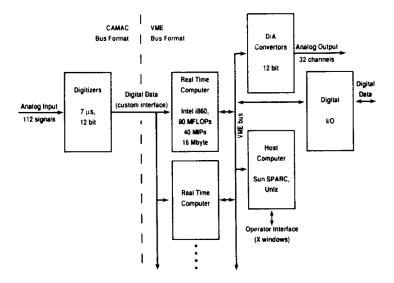


Figure 2.1.: DIII-D digital PCS in 1991 [2].

sion of the PCS, a Myricom 2 network has been replaced with a $40\,\mathrm{Gb/sec}$ InfiniBand 3 network based on the Mellanox Connect-X 3^4 hardware set. Figure 2.2 shows the currently overall networking diagram of DIII-D PCS .

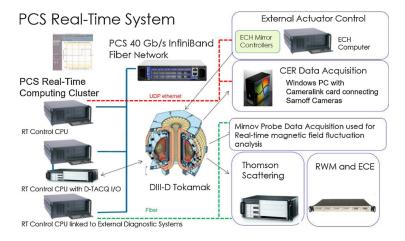


Figure 2.2.: Actual DIII-D PCS real-time systems [4].

² Myricom networks also called Myrnet are high speed networking systems used to interconnect machines to form computer clusters.

³ Is a network architecture from Mellanox designed to support I/O connectivity and reliability, availability, and serviceability Internet requirements [5].

⁴ The Connect-X from the Mellanox company are Ethernet network interface cards with PCI Express.

2.1.2 Systéme de Contrôle Distribué

The TCV^5 distributed control system uses a modular network of real time PC nodes liken by a real time network to provide feedback control over all of the actuator systems. Each node consists of a Linux PC either embedded on a Compact-PCI module or as a desktop computer with Intel CPU. A fiber optic ring network links the reflective memory (RFM) network cards in each node [6]. The design of the diagnostic signal processing and control algorithms is performed in Matlab-Simulink software. During the real-time execution C/C++ code is generated from the Simulink and compiled into a Linux shared library and distributed to target nodes providing the input/output interface to the control algorithm code [7]. Figure 2.3 depicts the TCV SCD layout with the connectivity to diagnostics and actuators.

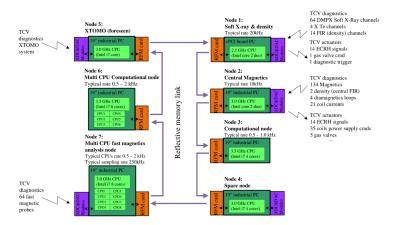


Figure 2.3.: TCV SCD. Real-time network nodes connection. The nodes configurations are shown together with the typical diagnostic and actuator systems to which they are connected [7].

2.2 MARTE FRAMEWORK

MARTe was developed in order to standardize general real-time control systems for the execution of control algorithms. MARTe framework is based on a multiplatform C^{++} library. [8]

⁵ The Tokamak á configuration variable (TCV) is a medium size tokamak localized in Laussane, Switzerland. It is characterized by a highly elongated, rectangular vacuum vessel.

PLASMA CONTROL SYSTEMS

- 2.2.1 *MARTe architecture*
- 2.2.2 Hardware containers
- 2.2.3 *MARTe* 2.0
- 2.3 EQUILIBRIUM AND CONTROL ALGORITHMS
- 2.3.1 PID control
- 2.3.2 Multiple-Input Multiple-Output control

JT60-SA CONTROL DESIGN

- 3.1 MACHINE DESCRIPTION
- 3.2 CREATE TOOLS
- 3.3 CONTROLLER DESIGNS
- 3.4 QST TOOLS IMPLEMENTATION
- 3.5 SIMULATION RESULTS

ISTTOK

- 4.1 MACHINE DESCRIPTION
- 4.2 DIAGNOSTICS AND ACTUATORS
- 4.3 ATCA-MIMO-ISOL BOARDS
- 4.3.1 Hardware layout
- 4.3.2 Real-time integration software
- 4.4 PLASMA CURRENT MAGNETIC FIELD

Retrieving the contribution of the plasma current in tokamaks ...

The methods of correction of the magnetic error fields due to inaccuracies of tokamak manufacturing and assembly are considered. The problems of the plasma position and shape reconstruction based on magnetic field measurements are discussed.

4.5 PLASMA CENTROID POSITION DETERMINATION

ISTTOK RESULTS

- 5.1 Implementation of the general application modules
- 5.1.1 PID control implementation
- 5.1.2 Multiple-Input Multiple-Output control implementation
- 5.2 RESULTS
- 5.2.1 PID control and LQR control results

CONCLUSIONS

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DEMONSTRATIONS