

PAPER

Feedforward and feedback control of locked mode phase and rotation in DIII-D with application to modulated ECCD experiments

To cite this article: W. Choi *et al* 2018 *Nucl. Fusion* **58** 036022

Recent citations

- [Implementing a finite-state off-normal and fault response system for disruption avoidance in tokamaks](#)
N.W. Eidietis *et al*
- [Relationship between locked modes and thermal quenches in DIII-D](#)
R. Sweeney *et al*

View the [article online](#) for updates and enhancements.

Feedforward and feedback control of locked mode phase and rotation in DIII-D with application to modulated ECCD experiments

W. Choi^{1,d}, R.J. La Haye², M.J. Lanctot^{2,a}, K.E.J. Olofsson^{1,b}, E.J. Strait²,
R. Sweeney^{1,c}, F.A. Volpe¹ and The DIII-D Team²

¹ Columbia University, New York, NY 10027, United States of America

² General Atomics, San Diego, CA 92121, United States of America

E-mail: wc2468@columbia.edu

Received 26 September 2017, revised 30 December 2017

Accepted for publication 11 January 2018

Published 5 February 2018



Abstract

The toroidal phase and rotation of otherwise locked magnetic islands of toroidal mode number $n = 1$ are controlled in the DIII-D tokamak by means of applied magnetic perturbations of $n = 1$. Pre-emptive perturbations were applied in feedforward to ‘catch’ the mode as it slowed down and entrain it to the rotating field before complete locking, thus avoiding the associated major confinement degradation. Additionally, for the first time, the phase of the perturbation was optimized in real-time, in feedback with magnetic measurements, in order for the mode’s phase to closely match a prescribed phase, as a function of time. Experimental results confirm the capability to hold the mode in a given fixed-phase or to rotate it at up to 20 Hz with good uniformity. The control-coil currents utilized in the experiments agree with the requirements estimated by an electromechanical model. Moreover, controlled rotation at 20 Hz was combined with electron cyclotron current drive (ECCD) modulated at the same frequency. This is simpler than regulating the ECCD modulation in feedback with spontaneous mode rotation, and enables repetitive, reproducible ECCD deposition at or near the island O-point, X-point and locations in between, for careful studies of how this affects the island stability. Current drive was found to be radially misaligned relative to the island, and resulting growth and shrinkage of islands matched expectations of the modified Rutherford equation for some discharges presented here. Finally, simulations predict the as designed ITER 3D coils can entrain a small island at sub-10 Hz frequencies.

Keywords: magnetic island, modulated ECCD, feedback control

(Some figures may appear in colour only in the online journal)

1. Introduction

Current perturbations, in the form of a deficit in pressure-driven bootstrap current, can cause magnetic reconnection and formation of Neoclassical Tearing Modes (NTMs) in the plasma [1]. Islands—identified by poloidal and toroidal mode numbers m

and n respectively, form at ‘rational’ surfaces, where the safety factor equals $q = m/n$ —can cause a non-axisymmetric local flattening in the pressure profile, which increases the deficit in bootstrap current and drives the island to grow larger.

While initially rotating at the plasma’s natural rotation frequency, these non-axisymmetric current filaments can induce eddy currents in the vacuum vessel wall, which in turn drag on the NTM, causing it to slow and eventually lock to residual error fields [2]. These locked modes (LMs) can quickly grow in amplitude, deteriorating plasma confinement by deforming pressure profile as in [3] and, in the worst case scenario, cause a major disruption [4].

^a Present address: U.S. Department of Energy, SW Washington, DC 20585, United States of America

^b Present address: General Atomics, San Diego, CA 92121, United States of America

^c Present address: ITER Organization, 13067 St Paul Lez Durance, France

^d Author to whom any correspondence should be addressed.

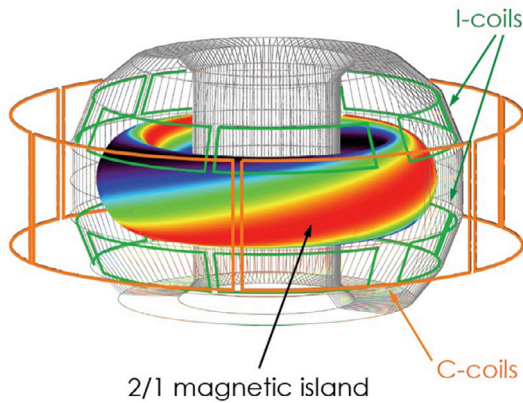


Figure 1. Perturbed current pattern, sinusoidal in helical angle $m\theta - n\phi$, of a $m/n = 2/1$ magnetic island mapped onto a thin surface. Also depicted are the internal I-coils (green) and external C-coils (red) used to generate 3D fields on DIII-D.

A number of approaches have been studied to mitigate the deleterious effects of locked modes, including: avoid seeding the initial mode [5], suppression during the rotating phase [1], or suppression of the locked mode itself [6]. It has been demonstrated that islands can be suppressed by replacing the deficit current [1, 7, 8], using electron cyclotron current drive (ECCD) [9], given proper alignment. In a system with a fixed launcher location, the deposition region is constrained in major radius by resonance location, in the toroidal coordinate by purpose (current drive or heating-only), and in height by the flux surface on which power is desired. Thus the intersection of the LM O-point with the deposition region can only be accomplished by controlling the phase of the mode.

One common technique for controlling the mode phase is to apply resonant magnetic perturbations (RMPs) by means of non-axisymmetric coils. They can be either internal or external to the vessel, like the I- and C-coils at DIII-D, respectively, shown in figure 1. As predicted by theory [10], such coils can be used to apply an electromagnetic torque to the mode and thus control its toroidal phase. The simplest usage would be to apply a pre-determined RMP, either static or rotating, with the expectation that the mode will align to it or, more precisely, to the resultant of the applied RMP and error field. This feedforward method has previously been shown to entrain modes at DITE [11], TEXTOR [12], and DIII-D [6]. Experiments at TEXTOR were conducted on tearing modes driven by error field penetration (where the error field in question was the RMP itself) [13]. Experiments at DIII-D were carried out on pressure-driven NTMs [6, 14, 15]. Other methods include using a feedback algorithm where the measured mode amplitude is multiplied by a complex gain to obtain the applied RMP [16]. At DIII-D, tuning the complex gain yielded control of the mode rotation frequency up to 50 Hz [17], while feedforward reached rotation frequencies up to 300 Hz [18].

Both the feedforward and feedback approaches described forced the mode to rotate despite drag from the wall. However, neither approach had fine control of the mode *phase*. Such a feature is highly desirable when controlled mode rotation is combined with modulated ECCD, in order to suppress the mode and restore good confinement. This is because mode

rotation alone, without ECCD, would only be stabilizing (via rotation shear and wall shielding) if very fast, well above the inverse wall-time [19]. Otherwise, ECCD is necessary to stabilize the rotating mode. Preferably, the ECCD should be modulated in phase with the transit of the island O-point in the deposition region. This poses the need to either measure the island phase and adjust the ECCD modulation accordingly, in real time [20] or, as shown in the present work, to prescribe the island phase in advance, as a function of time, and pre-program the ECCD modulation accordingly.

The theory of a feedback controller of the island phase was laid out in earlier work [21]. Assuming a saturated island of fixed width, its toroidal rotation was modeled under various conditions and compared to experimental observations. Using this representation, the system's response to an arbitrary control scheme was tested and optimized. A modified control scheme was ultimately implemented, the results of which are presented in section 3 of the present paper.

The paper is organized as follows: section 2 shows a new technique of preemptive entrainment in feedforward. Section 3 describes the feedback phase controller as implemented on DIII-D. Section 4 presents the experimental results. Section 5 then extends the model to predict entrainment capabilities for ITER.

2. Feedforward preemptive entrainment

A novel technique of preemptive entrainment is studied here, where a rotating RMP is applied early in the current flattop period, before the mode decelerates significantly and locks. When the $m/n = 2/1$ rotating precursor forms, initially in the kHz range, and decelerates due to wall drag, the mode is expected to lock onto the existing rotating RMP instead of locking to the wall. In theory, the prevention of complete mode locking would limit the island growth, as it is partially stabilized by rotation in the presence of a conducting wall [22] and by rotation shear [23–25].

2.1. Experimental result

An experiment was performed with the preemptive entrainment technique at DIII-D. The discharges had a flat top plasma current of 0.9 to 1.0 MA and a toroidal field of 1.7 T, giving the plasma a safety factor q_{95} between 4.8 to 5.8 during the flat top. After current flat top was reached at 700 ms, a RMP rotating at 70 Hz was applied starting at 1300 ms, with 4.3 kA of current in the internal I-coils. An average of 3 MW of neutral beam power pushed the plasma into H-mode at approximately 1760 ms, after which NBI power was increased up to a peak of 9 MW. In the discharge shown in figure 2, the 2/1 NTM appeared at around 2392 ms, initially rotating at about 7 kHz. Neutral beam power was reduced to roughly 5.8 MW near this time, which provided approximately 0.4 Nm of torque to the plasma for the rest of the discharge.

As the island rotation was reduced to near zero at 2514 ms, it locked to the 70 Hz rotating RMP without locking to the intrinsic residual DC error field. To be more precise, the island tends to align to the resultant of the rotating RMP and static

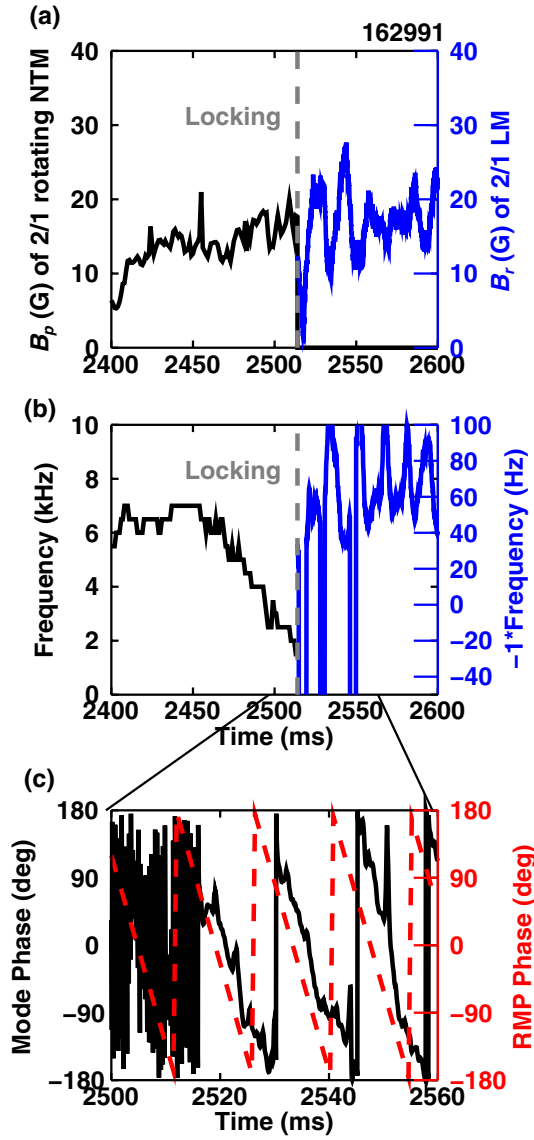


Figure 2. (a) Amplitude, (b) frequency, and (c) phase of the NTM, before and after locking. Note that instead of dropping to zero, the frequency after locking is around 70 Hz (with some noise), showing the island (black) is locked to the applied RMP (red) rotation frequency. Here, negative frequency is in the plasma rotation direction.

error field. This explains why its rotation is non-uniform (figure 2(c)) and why the frequency oscillates around 70 Hz on a sub-period timescale (figure 2(b)). Note that the *instantaneous* rotation frequency plotted in figure 2(b) is defined as the time-derivative of the toroidal phase signal in figure 2(c), where negative frequency is in the direction of natural plasma rotation. Also, the frequency oscillations can be so large that the mode rotation can temporarily change direction (figure 2(c)) [18], and the rotation frequency can temporarily vanish, for a small fraction of the 14.3 ms rotation period. Four instances of vanishing rotation frequency can be noticed in figure 2(b), but they should not be interpreted as mode locking. Other shots from the same experiment exhibit similar behaviour.

The measured amplitude and phase of the mode were corroborated between measurements of the perturbed poloidal

and radial fields, as well as using the electron cyclotron emission diagnostic [26]. Here and in the remainder of the paper, mode phase is defined as the toroidal position of peak perturbed radial magnetic field as measured at the outboard mid-plane. The poloidal field measurements come from an $n = 1$ fit of a toroidal array of Mirnov probes, located inside the vessel. The perturbed radial field was measured by external saddle loops differenced (ESLD), where sensors 180° apart are subtracted to obtain odd- n signals. Further discussion of the magnetic sensors on DIII-D and their capabilities to measure 3D phenomenon can be found in [27, 28]. As the applied RMP and the mode rotate at the same frequency (except for the oscillations just discussed), it was important to isolate the actual mode measurement and remove any coil-sensor pickup of the applied fields. This was realized by a.c. compensation [29].

In summary, this preemptive entrainment technique traded complete mode locking for a slowly rotating entrained island. This technique requires a rough prediction of when the mode might appear, and with what amplitude, so that the rotating RMP is only applied when needed, and with an amplitude just sufficient for entrainment, so that its negative impact on confinement can be minimized.

2.2. Discussion and simulation

By definition, a preemptive rotating RMP is applied before the rotating mode has appeared. Therefore, preemptive entrainment can only be applied in feedforward as there is no signal to feed back on.

Even if the applied fields rotate uniformly, the island will very likely rotate non-uniformly, due to electromagnetic torques from residual error fields, fluctuating island widths, and changing plasma conditions. An extreme case and a typical example are illustrated in figures 3(a) and (b), respectively. Both cases are taken from the same experiment reported in figure 2, although at different times. Here the amount of deviation from uniform rotation is quantified by the root-mean-square of the difference between the actual mode phase and a fixed-frequency trajectory. These r.m.s. phase-differences are evaluated over one period, and are denoted by $\Delta\tilde{\Phi}$. For the 76 periods in which the mode was entrained, the values of $\Delta\tilde{\Phi}$ ranged between 3.5° and 19° (figure 3(a)), with a mean of 9.5° (figure 3(b)).

It is interesting to note that, in the experiment, the phase deviation appears to be a mix of $n = 1$ and $n = 2$ perturbations, exemplified by the asymmetric deviations of the measured phase in black from the ideal red dashed trace. As both the mode and applied RMP are $n = 1$, this behaviour might be the result of an $n = 2$ error field interacting with the mode.

The dynamics of the island are governed by the equation of motion:

$$I \frac{d^2\phi}{dt^2} = T_{\text{wall}} + T_{\text{EF}} + T_{\text{RMP}} \quad (1)$$

where I and ϕ are the moment of inertia and toroidal phase of the island, respectively. On the right-hand-side of the equation are the electromagnetic torques exerted by the wall, static

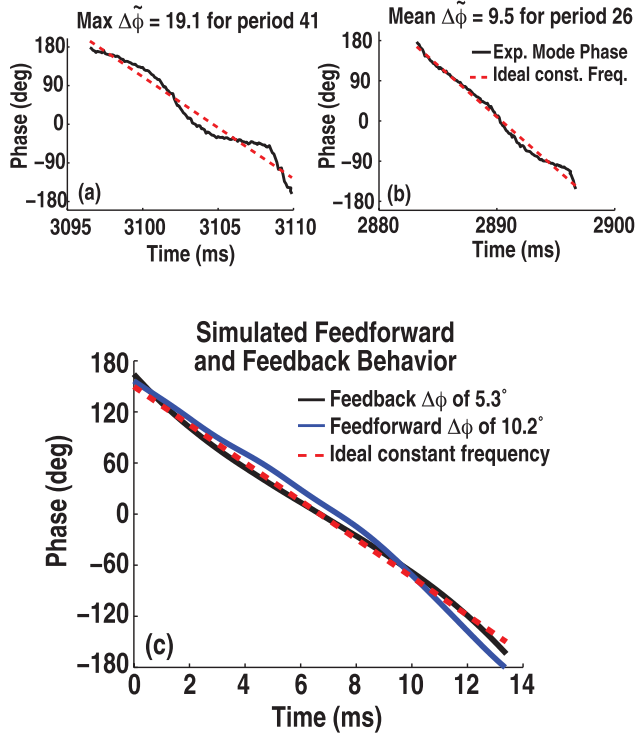


Figure 3. Nonuniform island rotation (black, solid) in response to an applied RMP rotating uniformly (red, dashed), within a single period. (a)–(b) Measurements in the case of maximum and mean nonuniformity, respectively, as measured by $\Delta\tilde{\Phi}$. (c) Simulation illustrating reduced nonuniformity when switching from a feedforward scheme (blue) to feedback control (black).

error field, and applied RMP onto the island. These torques are calculated from a numerical model [21] which treats the island as a single $m/n = 2/1$ Fourier mode in cylindrical geometry, and the perturbed or induced currents and applied or intrinsic magnetic fields are represented by phasors. This is the equation used in time-dependent simulations herein.

A few simplifying assumptions have been made: no other tearing modes exist or can interact with the $2/1$ mode of interest, the plasma around the island has low rotation and is not imparting momentum on the island, and torque from the neutral beams on the island is negligible. This last assumption is valid for either balanced co- and counter-injection, or for beam deposition away from the island location. In this experiment, the NBI torque deposited directly into the island, estimated based on ratio of the poloidal cross-sectional areas of the island and the plasma, is approximately 0.04 Nm, which can be neglected when compared to electromagnetic torques on the order of 1 Nm.

In simulations of the preemptive feedforward entrainment, an uncorrected $n = 1$ error field of 0.5 G at the rational surface causes an $n = 1$ perturbation to the rotation with an average phase deviation of $\Delta\tilde{\Phi} = 10.2^\circ$. This error field amplitude was chosen to approximately match the experimental mean value of $\Delta\tilde{\Phi} = 9.5^\circ$ in simulation. Simulations performed under the same plasma conditions and for the same error field indicate that the deviation is reduced to $\Delta\tilde{\Phi} = 5.3^\circ$ when feedback is used (figure 3(c)), which is described in the next section.

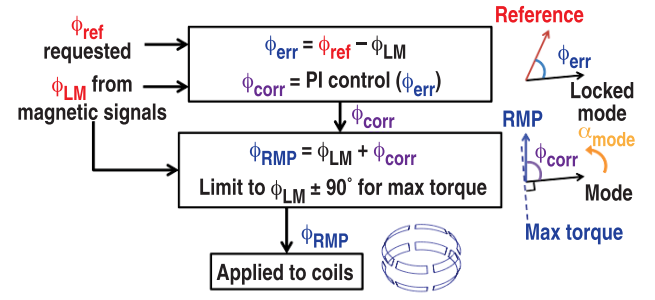


Figure 4. A schematic diagram of the proportional-integral controller implemented on DIII-D.

3. Implementation of feedback phase controller

A feedback controller of the $2/1$ mode phase was implemented in the DIII-D plasma control system, as shown in figure 4. Using the external saddle loops differenced (ESLD) signal as input, the algorithm calculates the amplitude and phase of the $n = 1$ locked or slowly rotating mode. The issue of drifts often associated with time-integrated signals is addressed by subtracting a baseline from each measured value. Such a baseline is evaluated 50 ms before mode locking, when the mode is still rotating rapidly (faster than 100 Hz) and, consequently, is not detectable by the ESLDs, due to wall shielding.

The operation of the controller is described here. The calculated phase of the island ϕ_{LM} is compared to a pre-determined reference phase ϕ_{ref} . The discrepancy or error angle between the two, ϕ_{err} , is used as input to a proportional-integral (PI) controller. Note that the subscript err is not related to (and should not be confused with) the error field. The PI controller accounts for the present error angle as well as its time history with:

$$\phi_{\text{corr}}(t) = K_p \phi_{\text{err}}(t) + K_i \int_{t_0}^t \phi_{\text{err}}(t') dt' \quad (2)$$

where K_p and K_i are the proportional and integral gains respectively, and t_0 is the time at which the controller is turned on. The output of the optimal correction angle ϕ_{corr} advances the phase of the applied perturbation, ϕ_{RMP} , with respect to ϕ_{LM} . This correction is then clipped to $\phi_{\text{LM}} \pm 90^\circ$ for maximum torque. Ultimately, a.c. currents of constant amplitude are delivered to the internal 3D field coils (I-coils) in order to apply an $n = 1$ RMP at the desired phase $\phi_{\text{RMP}}(t)$. This algorithm was implemented on a CPU with a cycle time of 50 μs , much shorter than the wall time of about 3 ms. This fast timescale allows rapid adjustments of $\phi_{\text{RMP}}(t)$, in order for $\phi_{\text{LM}}(t)$ to faithfully match the slowly evolving, desired $\phi_{\text{ref}}(t)$. In the experiments presented here $\phi_{\text{ref}}(t)$ is either static or evolving at 20 Hz.

The present approach is satisfactory at frequencies well below the inverse wall-time (about 50 Hz at DIII-D). For faster entrainment and associated stabilizing effects, the amplitudes of the coil-currents should be increased as a function of the requested rotation frequency to overcome the effect of the increasing wall shielding. This is left as a future improvement.

3.1. Mode phase control in experiment

An experiment was performed on DIII-D with the goal of using the new feedback algorithm to achieve smooth entrainment of the mode, combined with synchronized deposition of ECCD to suppress the island amplitude. Here ‘smooth’ refers to uniform rotation, both within a single rotation period, as well as from period to period.

The DIII-D discharges presented in the remainder of this paper have the following parameters: a plasma current of 1.0 MA and toroidal field of 1.7 T gave a safety factor between 4.3 and 4.5. H-mode was obtained at approximately 1500 ms and subsequently lost around 2200 ms, when an initially rotating 2/1 NTM appears, quickly decelerates and becomes locked within 200 ms. The locking event triggers a response in the controller and, for later shots discussed in section 4, gyrotron power.

The capability of the controller was tested with a pre-programmed reference phase for fixed-phase and 20 Hz rotation. Figure 5 shows the results of a shot where only proportional feedback was used, with the proportional gain K_p stepped over four different values in each type of request.

The agreement between the requested LM phase ϕ_{ref} and the actual, magnetically measured LM phase ϕ_{LM} is qualitatively evident from figure 5(a). The discrepancy between the two, ϕ_{err} , averaged over the duration of each fixed proportional gain, is used as a quantitative figure of merit. This averaged discrepancy $\langle \phi_{err} \rangle$ is calculated for both the experimental measurements (black symbols in figure 6) and the equivalent simulated scenario (blue symbols). All simulations in this section used the same parameters as in shot 166560, including the same radial position of the island, and same RMP strength, corresponding to 2.7 kA of current in the I-coils.

In the fixed-phase case (figure 6(a)), it is clear that the controller can control the LM phase as desired, as long as the proportional gain K_p exceeds a threshold located in the interval $0.5 < K_p < 1$.

The entrainment tests (figure 6(b)) exhibit larger errors. In fact, the lowest proportional gain K_p of 0.5 in the scan was unable to entrain the mode and, at $t \approx 3040$ ms, it allowed the mode to ‘slip’ backwards and catch the next rotation (figure 5(a)). This slippage resulted in a brief period in which $\phi_{LM}(t)$ matched $\phi_{ref}(t)$, which may have reduced the apparent tracking error for several periods that followed (into the next proportional gain value). Higher gain, $K_p \geq 1$, yielded continuous entrainment without such slipping events, but with some significant discrepancy ϕ_{err} . In reality, finite, relatively large discrepancies were actually expected (see blue symbols in figure 6(b)) as a result of how the phase-controller [21] is expected to perform at 20 Hz. This is due to currents induced in the wall not yet being completely accounted for in the controller. As a matter of fact, experiments exhibited even larger tracking errors (black symbols in figure 6(b)), but this was also reasonable, as it is well-known that a proportional-only controller cannot track a ramped reference without error [30]. Adjustments of controller gains achieved the desired smooth rotation and brought average ϕ_{error} to within acceptable levels ($\lesssim 30^\circ$) during later shots. The performance of a simple proportional-integral controller is satisfactory for this first attempt of feedback control of mode

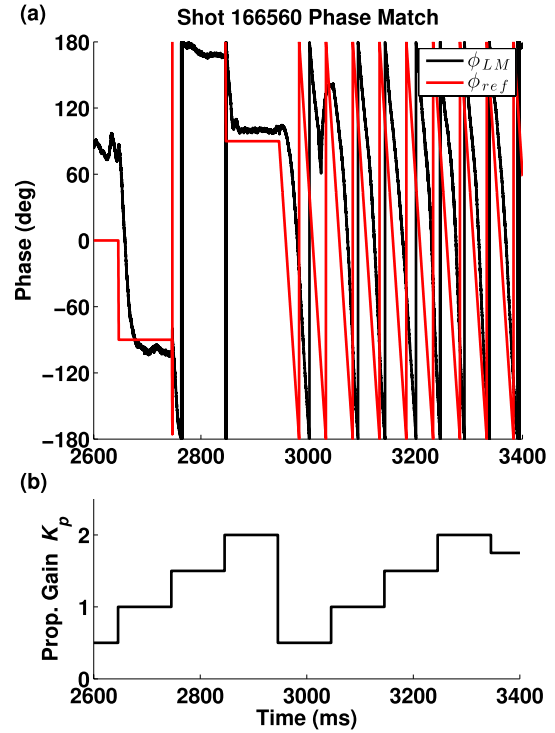


Figure 5. The measured and reference phase of a locked mode, as gain is varied every 100 ms. In both fixed and rotating reference cases, higher gain results in better alignment as expected.

phase. Advanced techniques such as iterative learning control [31, 32] can be adopted for further improvement.

3.2. Simulation of proportional-only control

Other major factors contributing to the tracking error in this proportional-only scheme are the island width and its proximity to the wall, or equivalently, the position of the $q = 2$ surface in normalized minor radius ρ .

For these *steady-state* simulations, the effect of the error field must be dropped from equation (1), to prevent it from adding a sub-period perturbation to an otherwise uniform entrainment. Experimentally, this can be achieved if the error field is well-corrected and made negligible compared with the applied, slowly rotating RMP. The resultant motion will be a balance between the wall torque and the applied RMP torque only:

$$0 = T_{wall} + T_{RMP}. \quad (3)$$

Figure 7 shows the toroidal tracking errors expected from simulations, as described by equation (3), where the phase of the RMP is determined by the proportional control. Each panel presents three colour contours with different thicknesses for changing (a) island width or (b) normalized radial position of the $q = 2$ surface. For example, the contour with medium thickness lines in panel (a) shows that a 5 cm island entrained at 30 Hz with K_p of 2 has a predicted tracking error of 20° , whereas a 4 cm (thin lines) or 6 cm (thick lines) island using the same parameters would have errors of 17° and 22° respectively. This expected lag increases for larger islands or shorter distance from the wall, due to the different dependencies of the RMP and wall torques on these parameters. A higher

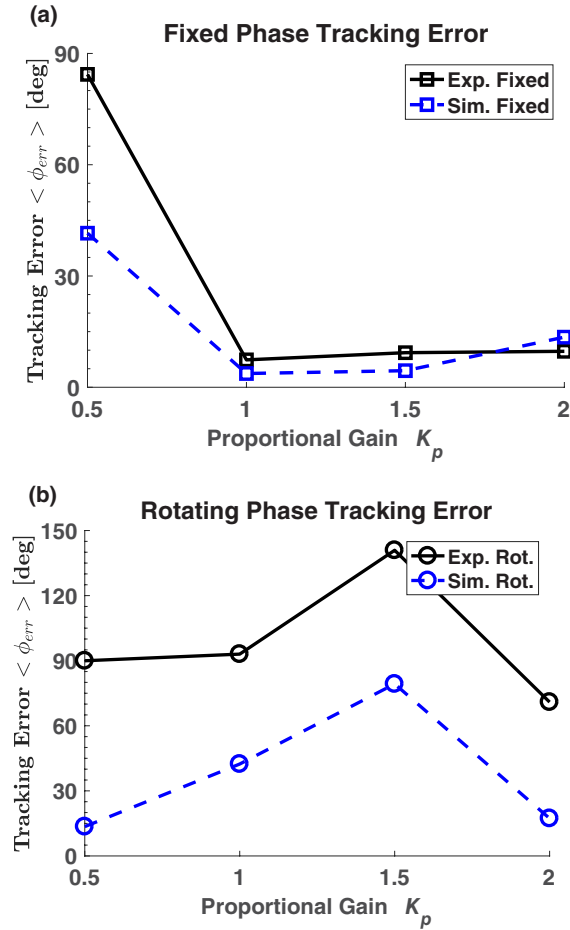


Figure 6. Experimental and simulated tracking error (defined as discrepancy between reference phase and actual mode phase averaged over each 200 ms period) for (a) fixed phase and (b) 20 Hz rotating phase references respectively.

entrainment frequency also increases the error due to stronger wall shielding of the applied RMP and increased wall torque (peaks at the inverse wall time) on the mode. Simulations show that increasing the gain reduces the tracking error, but at the risk of overall system stability in transient evolution [30].

As it currently stands, using saddle loops *external* to the vessel as sensors prevents the controller from detecting modes rotating much faster than the inverse wall time. Even at frequencies yielding measurable signals (comparable with the inverse wall time, or approaching it), shielding presents issues: the measured mode phase lags behind the true mode phase. A future combination of *internal* sensors (whether Mirnov or saddle loops) and *real-time* a.c. compensation for coil-sensor pickup will contribute to deploying the controller at frequencies above the present limit.

4. Combining mode entrainment and modulated ECCD

4.1. Experimental results

After mode entrainment was achieved, modulated ECCD with a 50% duty cycle was deposited in synchronization with the mode rotation at 20 Hz, corresponding to a period

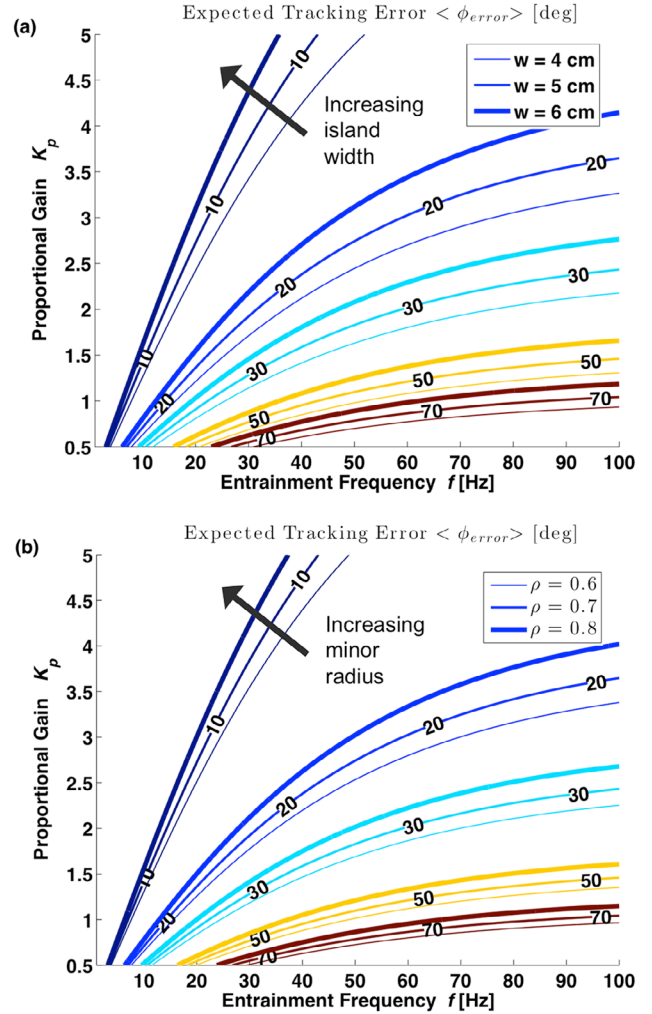


Figure 7. Simulated tracking error in a proportional only controller, as dependent on the gain used, the requested frequency, as well as (a) the island width and (b) position of the $q = 2$ surface. Three contours are presented in each panel, where the colours indicate levels of tracking error in each case and thickness differentiates the scenarios.

of 50 ms. For comparison, typical NTM evolution timescale ($w/(dw/dt)$) in the present experiment is between 40 and 200 ms. Note that in earlier experiments at DIII-D and AUG [20], ECCD was modulated in feedback with island phase measurements, though it is a challenge to do this in real-time with available diagnostics. More simply, here the island phase is prescribed by a feedback controller as a function of time, and the ECCD modulation is simply programmed in advance. Average phase error of $\lesssim 30^\circ$ was considered negligible compared to the toroidal 180° for which the ECCD was turned on. Six gyrotrons were used in this experiment to provide up to 3.5 MW of heating and drive about 22.5 kA of current in the co-plasma current direction [33]. It should be noted that the EC current is established on an electron-electron collisional timescale, which is significantly shorter than the evolution of interest and can be considered instantaneous. However, externally driven current effect changes in the current profile much more slowly, as discussed later.

The four plasma discharges used for analysis follow a similar trajectory in their overall discharge evolution. After

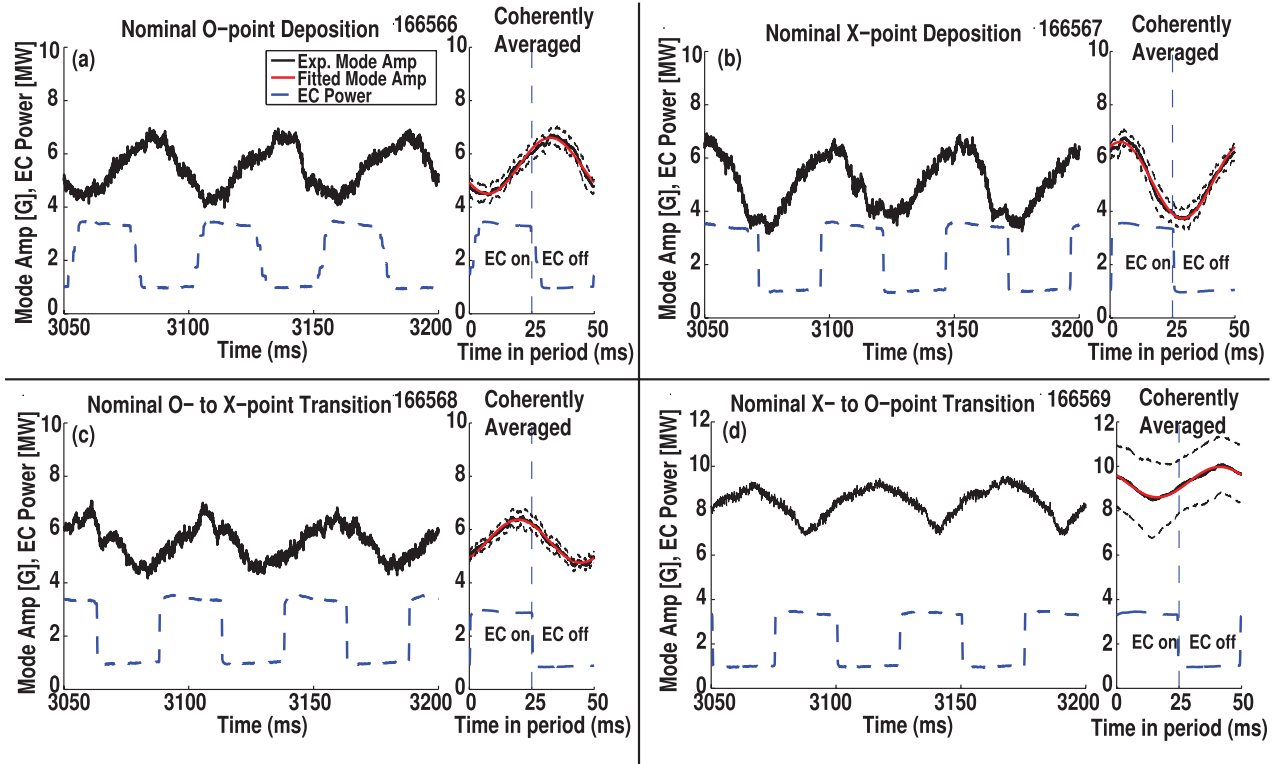


Figure 8. Plots of measured mode amplitude, in black, and electron cyclotron power, in blue. A coherent average of the mode amplitude over 25 periods is shown for each shot on the right, where the dotted lines mark one standard deviation away from mean black line. A sinusoidal fit to each averaged mode amplitude is shown in red. (a) Mode amplitude is increased when ECCD is thought to be deposited in the O-point, against expectation. (b) X-point deposition correlates with mode amplitude decrease. (c) Mode appears to first grow, then be suppressed as deposition transits from O- to X-point, while (d) the deposition transition from X- to O-point has opposite effect.

reaching current flat-top and entering H-mode, a $m/n = 2/1$ NTM appeared with a rotation frequency in the high kHz range. The NTM was allowed to grow and decelerate without intervention. Modulated ECCD and rotating RMPs were applied upon detection of mode locking. From here, the measured mode amplitudes oscillated until programmed I_p ramp-down. In contrast, an earlier reference shot with no ECCD disrupted shortly after locking.

A coherent average of the mode amplitude over 25 periods was performed for each shot, shown on the right subpanels in figure 8. The thin dotted black lines represent one standard deviation away from the mean mode amplitude. This averaging process reduces the noise in the signal for the following analysis. Given the periodic nature of this behaviour, the 20 Hz frequency used in entrainment and synchronized deposition was adopted to fit a sinusoid (red) to the averaged mode amplitude (black). This single-frequency fitting gave a R^2 between 0.974 and 0.981 for the four shots, while fitting the 2nd and 3rd harmonics—with normalized amplitudes up to 0.13 and 0.04 respectively—only improved the R^2 to about 0.995. Higher harmonics were found to be progressively smaller, thus analysis proceeded with the fundamental 20 Hz fit.

The mode amplitude response to the driven current was found to be against the expectation of O-point deposition suppressing the mode. It was actually observed that the mode amplitude had increased when the O-point was believed to be in phase with the deposition location (figure 8(a)). Similarly,

current nominally driven in the X-point was correlated with a decrease in mode amplitude (figure 8(b)). These peculiar results were further corroborated by two additional shots where the gyrotrons were turned on half-way between the island O- and X-points, as in figure 8(c) and (d).

4.2. Effect of radial misalignment

Upon closer analysis, it was found that the radial location of ECCD deposition, r , differed from the rational surface location r_s by an amount $|r - r_s|/w_{\text{ECCD}} = 1.66$, where w_{ECCD} is the ECCD deposition full width at half maximum (FWHM). This was due to a poor between-shot equilibrium reconstruct based on magnetics alone and gyrotron mirrors that could only be aimed in feedforward, lacking real-time tracking of the radial position of the rational surface. The current was driven outside of the island (solid color box in figure 9(a)) and redistributed, due to parallel transport, over the ‘intercepted’ flux-surfaces, external to the island (shaded lighter color). X-point phasing yields figure 9(b), where the current is driven in flux-surfaces farther away from the island separatrix.

When modulated ECCD is entirely deposited outside the island, it does not result in a helical current-*filament* that can compensate for the bootstrap current-deficit in the island. Rather, it results in a current-*sheet* or toroidal annulus being driven in a range of minor radii, but at all poloidal locations. At most, the current-sheet will be kink-deformed on the flux-surfaces, which are rippled due to proximity with the island.

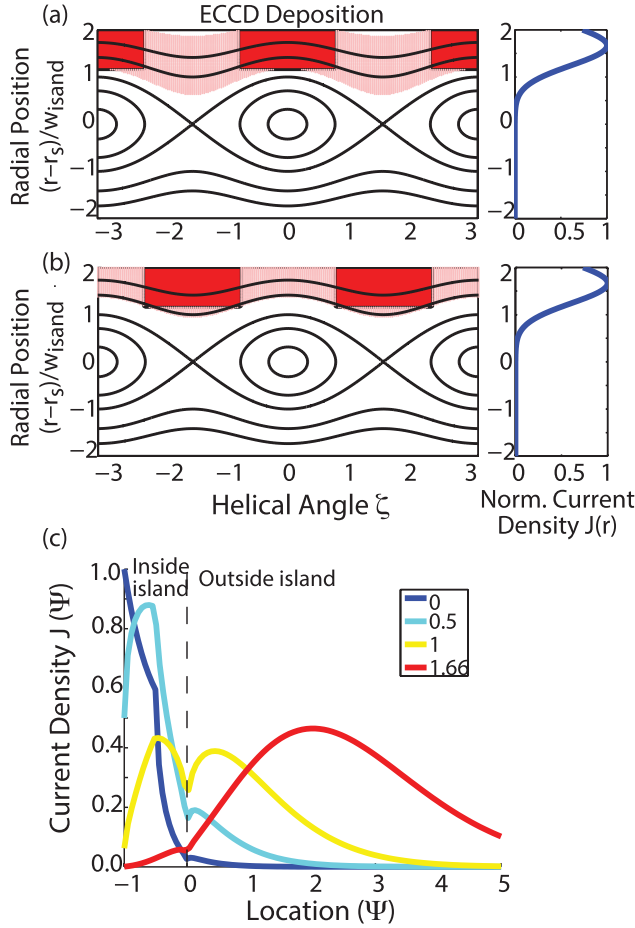


Figure 9. The radial and toroidal deposition location of ECCD in shots (a) 166566 and (b) 166567, based on post-shot analysis from EFIT (mode location), TORAY-GA (ECCD location), and magnetics (island width). The solid box shows the locations near the island where EC power was deposited, which spreads over the flux surface due to parallel transport, as depicted by the shaded region. (c) shows the current drive distribution in terms of perturbed flux surface for 50% duty cycle deposition centered on the O-point toroidally, with the color axis representing normalized radial misalignment $|r - r_s|/w_{\text{ECCD}}$.

In these cases, scanning the phase of the time-modulation does not affect the helical phase of ECCD deposition relative to the island O-point. Rather, it is equivalent to *radially* scanning the ECCD deposition relative to the island separatrix.

A combination of different tools were used to obtain the current deposition location relative to island position plotted in figure 9. TORAY-GA [34] provided the radial deposition profile of the ECCD, fitted with a Gaussian. The mode's radial position is given by an EFIT equilibrium reconstruction, and its toroidal phase and width are given by magnetic measurements. Finally, the toroidal deposition of ECCD relative to the island phase is given by known timing.

The expected current density profile for a toroidally O-point centered 50% duty cycle is plotted in figure 9(c). This is a function of the normalized flux-surface coordinate, perturbed by the presence of the island, introduced in [35]: $\Psi = (r - r_s)^2/w^2 - (1 + \cos(m\alpha))/2$, where w is the island width, and $\alpha = \theta - n\phi/m$ is the helical angle. Here $\Psi = -1$

corresponds to the center of the island, $\Psi = 0$ to its separatrix, and $\Psi > 0$ to the region outside the island. Distinction between the core and edge sides of the island is not needed in this direct current replacement model. Zero (dark blue) to small (light blue) radial misalignment still results in the majority of the current being driven inside the island O-point as desired. However, increased radial misalignments places a large fraction of the current either onto separatrix (vertical dashed line), or outside of the island altogether. Current driven at these locations reinforces the initial perturbation, causing the mode to grow rather than be suppressed. This reversal of effectiveness has been previously predicted by [35] at sufficiently large radial misalignment $|r - r_s|/w_{\text{ECCD}} \gtrsim 0.75$.

4.3. Comparison with theory

The modified Rutherford equation (MRE) is given below [36]:

$$1.22^{-1} \frac{\tau_R}{r} \frac{dw}{dt} = \Delta'(w)r + \epsilon^{1/2} \left(\frac{L_q}{L_p} \right) \beta_\theta \left[\frac{rw}{w^2 + w_d^2} - \frac{rw_{\text{pol}}^2}{w^3} - K_1 \left(\frac{j_{\text{ECCD}}}{j_{\text{BS}}} \right) \right] \quad (4)$$

where τ_R is the local resistive time and r is the minor radius of the rational surface. The right hand side includes the classical stability index $\Delta'(w)$ and neoclassical terms for small island effects w_d and polarization w_{pol} . The last term describes the effect of electron cyclotron current drive on the growth of the island dependent on an efficiency K_1 and the ratio of ECCD and bootstrap currents.

An early model by Perkins [35] calculates the efficiency K_1 of ECCD mode suppression by direct replacement of the missing bootstrap current, averaged over the modulation period.

$$K_1 \left(\frac{w}{w_{\text{ECCD}}}, \frac{\Delta R}{w_{\text{ECCD}}} \right) = \int_{-1}^{\infty} d\Psi W(\Psi) j_{\text{ECCD}}(\Psi, w_{\text{ECCD}}, \Delta R) \quad (5)$$

where the efficiency is a function of island size (affects perturbed flux Ψ), a weighting function $W(\Psi)$, deposition width w_{ECCD} , radial misalignment ΔR , as well as duty cycle and toroidal phasing. Figure 10 shows contours of this efficiency, where the white marks in panels (b) and (c) show estimated experimental values. In both the cw and O-point centered modulation cases, the highest efficiency occurs for well-aligned deposition, as expected. O-point deposition yields higher efficiencies everywhere, compared to cw deposition. In the 50% X-point modulation case (figure 10(c)), a well-aligned deposition results in a moderately negative effect, which diminishes for larger misalignment or island size.

More recent work by Westerhof [37] demonstrates that the MRE can be interpreted to also include the effects of current deposited outside the island that changes the local current profile, which was neglected in this work. Related work by

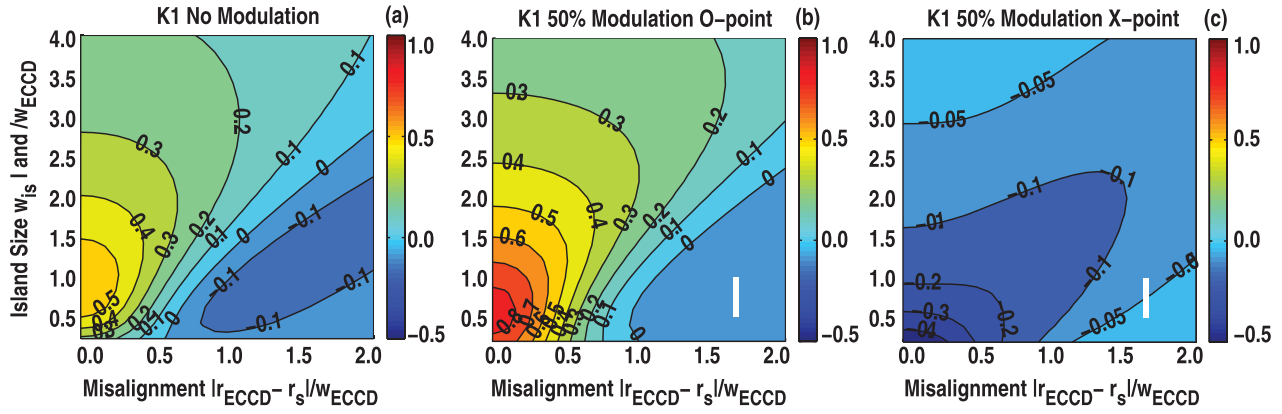


Figure 10. Contours of ECCD deposition efficiency in terms of relative island size, radial misalignment, and duty cycle being (a) full-on, (b) 50% centered on O-point, and (c) 50% centered on X-point. Here, w_{island} is the island half-width, w_{ECCD} is the FWHM of the gaussian current drive profile, and misalignment is the absolute difference between peak CD location and the $q = 2$ surface. The white regions in panels (b) and (c) mark approximately the experimental values for shot 166566 and 166567 respectively.

Ayten and Westerhof [38] further includes the effects of mode rotation, resulting in a constant cw ECCD having a time-dependent, periodic effect on stability due to being deposited at different phases relative to the island O-point. However, the current profile can only be perturbed on an L/R timescale, where L and R are the local inductance and resistivity respectively. In this particular experiment, the L/R time of approximately 60 ms allows us to assume current profile just outside the island does not change significantly during each 25 ms half-period of the ECCD turning on or off.

De Lazzari [39] further refined the model by including the effect of local heating on the local resistivity, and thus on the local current. The relative strengths of this term depend on the ratio of island and deposition widths, and on local temperature, bootstrap current, and perpendicular heat conductivity. This was evaluated to be roughly an order of magnitude smaller than the direct current replacement effect in this experiment. Lastly, the change in local pressure profile gradient produces another perturbed current, but is numerically evaluated to be yet another order of magnitude smaller and therefore neglected in further analysis. Thus, the strongest contributing term of current replacement became the focus of further analysis, neglecting the other effects.

These works have been extended to calculating the change in saturated island width by using the MRE and accounting for the time-evolution of the ECCD term in the equation, due to the power being modulated and the island being entrained, changing the deposition location. This is accomplished by evaluating the effect on the efficiency K_1 of a thin slice of current deposited in a toroidally localized position, and the K_1 contours in figure 10 can be interpreted as the weighted averages over one period. As the deposition location is moved by rotating the island, the effect of the EC suppression will change in time, resulting in a gradual shift of the saturated island width. Figure 11(b) and (c) depict a radially well-aligned blip of current in the O- and X-points of the island, and panel (a) shows the drastic difference between suppression efficiencies for radially well-aligned and misaligned depositions. While this is consistent with mode destabilization shown in

figure 8(a), panel (b) suggests that the mode is somewhat suppressed by the ECCD instead of the expected destabilization. Further investigation is required to explain this discrepancy. In fact, sufficiently radially misaligned deposition is expected to cause mode growth regardless of phase. This is due to the small fraction of helical current deposited near the island X-point having a stronger negative impact on stability than the even smaller amount of current deposited near the island O-point. This highlights a need, in future work, for robust real-time aiming of the EC mirrors towards the $q = 2$ surface in order to achieve maximum efficiency.

5. Modeling of feedback control on ITER

The predictive model for mode dynamics was adapted to ITER to estimate the entrainment capabilities for 2/1 islands. In its present design, ITER will have two sets of 3D coils: the internal three rows of nine ELM control coils [40] and the external three rows of six correction coils [41]. Continuing the phasor representation used by Olofsson [21], each set of coils was modeled by an amplitude and phase appropriate for the applied RMP, optimized for maximum coupling (in vacuum) to the 2/1 island. For now, the simulation assumed that the power supplies used for these 3D coils are capable of delivering the necessary currents at low frequencies (< 20 Hz). ITER will also have a beryllium first wall and two layers of stainless steel vacuum vessel walls [42], treated here as the source of the braking torque on the island. Using a thin wall approximation, they are reduced to a simple time constant that affects the mode dynamics and applied RMP penetration. Other factors such as test blanket modules and divertor were not included at this stage.

5.1. Fixed phase

A basic step reference, feedforward simulation was performed as a first check, using a fixed-width 5 cm island, with peak current of 10 kA/turn driven in the external coils as the RMP, and a 1 G residual error field. The phase of the RMP is advanced

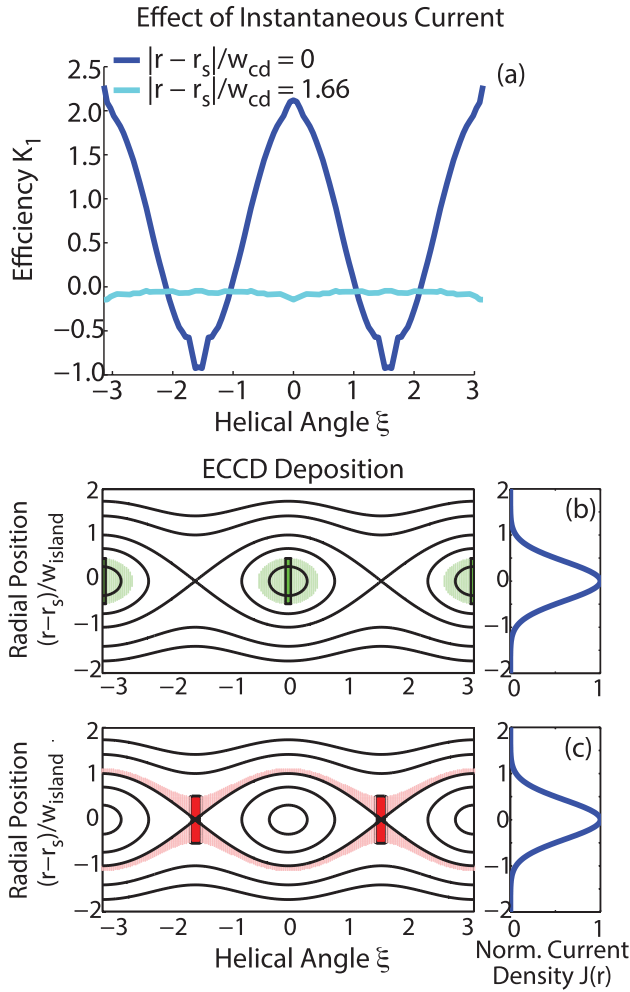


Figure 11. (a) shows the instantaneous efficiency K_1 for current deposited at a particular helical angle, for both radially aligned and misaligned case. (b) and (c) depicts a toroidally localized slice of current deposited in the O- and X-points of the island, respectively, where the solid box shows actual deposition location (5% duty cycle used for clarity), and the shaded region shows the effect of current being spread over the associated flux surfaces by fast parallel transport.

by 60° every 3 s. Figure 12 shows the mode aligning to the RMP as expected, with an offset caused by the error field.

5.2. Entrainment

Similar to section 2, feedforward preemptive entrainment was simulated using the same conditions as above, except now with a rotating RMP. Figure 13 shows the mode slowing down and locking to the 5 Hz entraining RMP. As entrainment is approached, the phase of the mode follows that of the RMP with some oscillations caused by the induced currents in the wall. After these currents have fully decayed, smooth entrainment is observed for $t \gtrsim 10$ s. Equation (1) was used in both of the time domain simulations, which includes the effects of error field and slow decay of shielding currents in the wall.

To predict the maximum entrainment frequencies possible in ITER, we searched for a steady-state torque-balance between the wall drag and the applied RMP, neglecting error

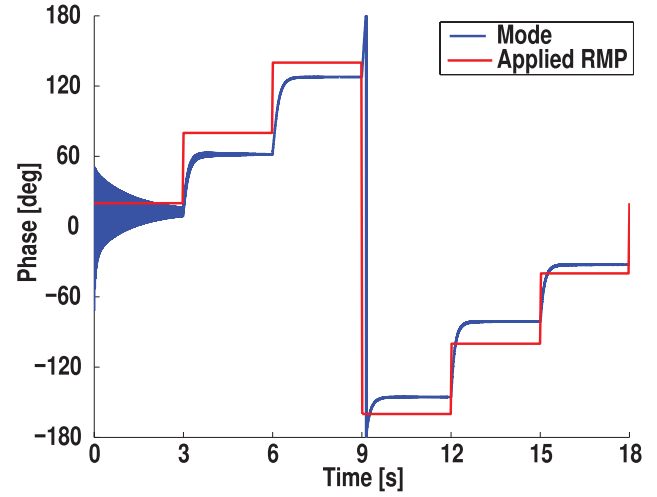


Figure 12. Simulation for ITER of a 5 cm island (blue) following a 10 kA/turn static RMP (red), advancing in steps of 60° every 3 s. There is an offset at each time due to the residual error field of 1 G. The initial oscillations in mode phase are simply an artifact of the magnetic field from the mode trying to penetrate the wall, which was initialized at zero.

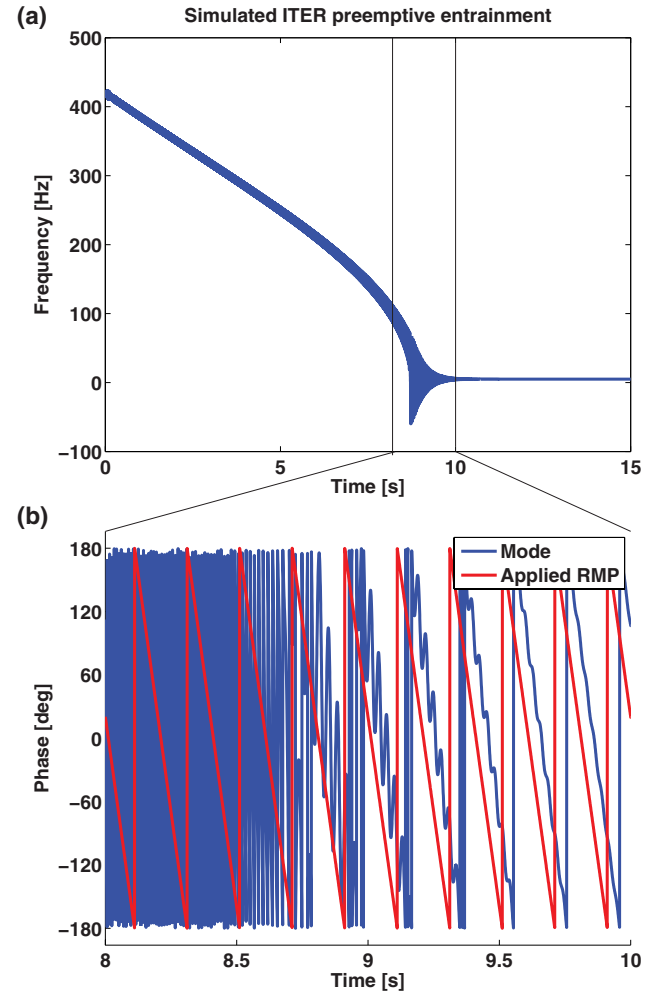


Figure 13. Simulated preemptive feedforward entrainment at 5 Hz of a 5 cm island in ITER, with the rotating 10 kA RMP in the external coils turned on from the beginning. (a) shows the rotation frequency decrease and settle to 5 Hz, (b) gives the details of mode phase behaviour around locking.

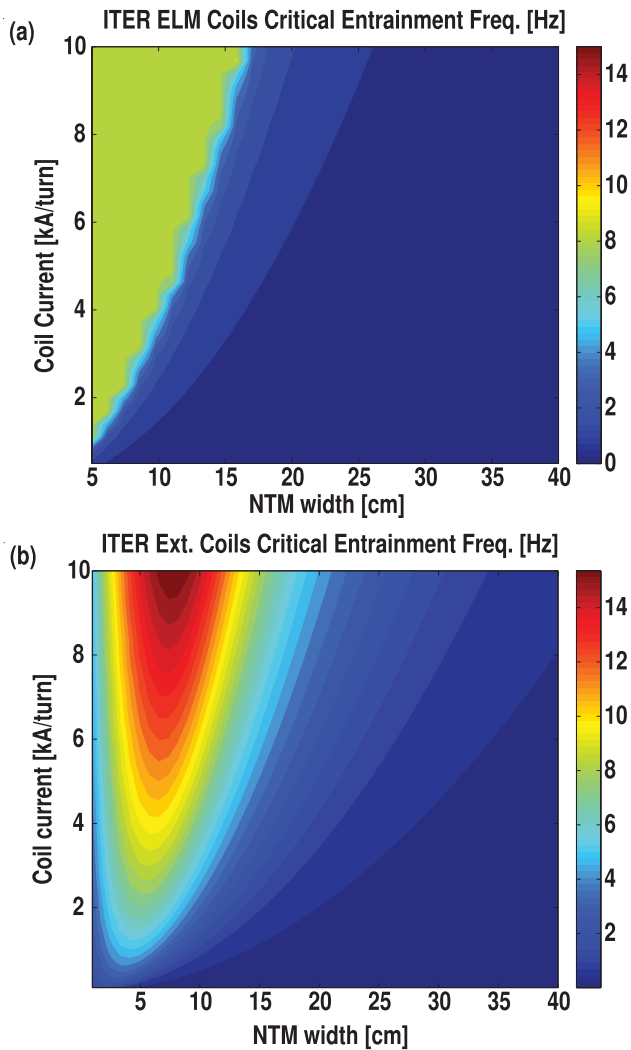


Figure 14. Predicted maximum stable entrainment frequency contours for ITER using (a) internal ELM coils or (b) external correction coils.

fields and possible time history effects. Equation (3) was used in this simplified scenario to calculate the critical entrainment frequency, dependent on island width and applied current. Figure 14 suggests that small 2/1 islands can be entrained in the sub-10 Hz range. Curiously, although an externally applied RMP torque must penetrate through the walls in order to affect the mode, the higher number of turns in the external coils balances out the disadvantage in location. As a result, external coils exhibit entrainment capabilities similar to the internal coils, based on otherwise identical simulation parameters.

6. Summary and conclusions

Following previous work that simulated the dynamics of magnetic islands interacting with applied resonant magnetic perturbations (RMPs), error fields and conducting wall on DIII-D, a feedback controller was developed to control the phase of 2/1 islands. Preemptive feedforward entrainment of these modes was experimentally demonstrated at DIII-D and presented here. However, we also modeled and experimentally

confirmed that feedback control enables more uniform, smoother entrainment. Initial testing of this controller shows promising results in prescribing fixed phase to the mode, as quantified by a low tracking error. Entrainment studies using proportional-only capabilities resulted in a higher tracking error than expected, depending on the proportional gain used and requested frequency. Simulation suggests larger island size or radial position also increases the tracking error.

Stable low frequency (20 Hz) entrainment was achieved in this experiment. Future work includes improving the controller to operate at rotation frequencies higher than the inverse wall-time, about 50 Hz at DIII-D. This is expected to provide additional benefits for confinement, stability, and disruption avoidance.

Modulated electron cyclotron current drive (ECCD) was deposited in synchronization with the entrained, rotating mode for a detailed study of how it affects island stability, as a function of the phase relative to the island O-point. At first sight, the relation seemed counter-intuitive, with O- and X-point deposition respectively destabilizing and slightly stabilizing the mode. The deposition was radially misaligned, resulting in a large fraction of current being driven outside the island separatrix. Theoretical considerations suggest that this should be destabilizing, when the ECCD is modulated with O-point phasing, consistent with experimental observations. However, X-point phasing should also be destabilizing, albeit to a smaller degree. This is not consistent with the observations presented, and will be the subject of further theoretical work. A well-aligned deposition in both radial and toroidal position would offer the most effective suppression, reducing the required input power. The effect of modulated, misaligned heating on the temperature and pressure profiles are secondary corrections not included here, and will be investigated in future work.

The code used to model mode behaviour was adapted to ITER, where static and rotating RMPs are predicted to be able to control the island's toroidal phase. The model also predicts a capability to entrain small (that is, recently formed, not yet saturated) islands in the sub-10 Hz range. However, other possible effects, such as the plasma response to the applied RMP, are not studied here and are left as future work.

Applied 3D magnetic fields have long been available as a tool in studying MHD stability. This paper presents the first instance of direct feedback phase control of 2/1 islands in real time, which can be used to prevent locking and its associated risks or to position the island favourably in order to assist its suppression by ECCD or its characterization by toroidally sparse diagnostics.

Acknowledgments

The authors would like to thank R. Prater and X. Chen for their help in the feedforward experiment, as well as the DIII-D team for making the experiment possible. This work was realized under DOE Grants DE-SC0008520, DE-SC0016372, and DE-FC02-04ER54698.

DIII-D data shown in this paper can be obtained in digital format by following the links at https://fusion.gat.com/global/D3D_DMP

References

- [1] La Haye R.J. 2006 Neoclassical tearing modes and their control *Phys. Plasmas* **13** 055501
- [2] Nave M.F.F. and Wesson J.A. 1990 Mode locking in tokamaks *Nucl. Fusion* **30** 2575
- [3] Chang Z. and Callen J.D. 1990 Global energy confinement degradation due to macroscopic phenomena in tokamaks *Nucl. Fusion* **30** 219
- [4] Morris A.W. 1992 Mhd instability control, disruptions, and error fields in tokamaks *Plasma Phys. Control. Fusion* **34** 1871
- [5] La Haye R.J., Buttery R., Guenter S., Huysmans G., Maraschek M. and Wilson H. 2000 Dimensionless scaling of the critical beta for onset of a neoclassical tearing mode *Phys. Plasmas* **7** 3349–59
- [6] Volpe F.A. *et al* 2015 Avoiding tokamak disruptions by applying static magnetic fields that align locked modes with stabilizing wave-driven currents *Phys. Rev. Lett.* **115** 175002
- [7] Gantenbein G. *et al* 2000 Complete suppression of neoclassical tearing modes with current drive at the electron-cyclotron-resonance frequency in asdex upgrade tokamak *Phys. Rev. Lett.* **85** 1242
- [8] Hennen B.A. *et al* 2010 Real-time control of tearing modes using a line-of-sight electron cyclotron emission diagnostic *Plasma Phys. Control. Fusion* **52** 104006
- [9] Prater R. 2004 Heating and current drive by electron cyclotron waves *Phys. Plasmas* **11** 2349–76
- [10] Fitzpatrick R. and Hender T.C. 1991 The interaction of resonant magnetic perturbations with rotating plasmas *Phys. Fluids B: Plasma Phys.* **3** 644–73
- [11] Morris A.W. *et al* 1990 Feedback stabilization of disruption precursors in a tokamak *Phys. Rev. Lett.* **64** 1254
- [12] Liang Y. *et al* 2007 Observations of secondary structures after collapse events occurring at the $q = 2$ magnetic surface in the textor tokamak *Nucl. Fusion* **47** L21
- [13] Koslowski H.R. *et al* 2006 Tearing mode physics studies applying the dynamic ergodic divertor on textor *Plasma Phys. Control. Fusion* **48** B53
- [14] Volpe F.A. *et al* 2009 Advanced techniques for neoclassical tearing mode control in DIII-D a *Phys. Plasmas* **16** 102502
- [15] Shiraki D. *et al* 2014 Error field detection in DIII-D by magnetic steering of locked modes *Nucl. Fusion* **54** 033006
- [16] Okabayashi M. 2014 *General Atomics Report* GA-A27926
- [17] Okabayashi M. *et al* 2017 Avoidance of tearing mode locking with electro-magnetic torque introduced by feedback-based mode rotation control in DIII-D and rfx-mod *Nucl. Fusion* **57** 016035
- [18] Volpe F. 2014 Using 3D fields to control islands, aid ECCD-stabilization and measure error-fields in DIII-D http://pl.apam.columbia.edu/files/seaspplab/presentations/2014_eps_volpe_3Dfields.pdf.
- [19] De Vries P.C. *et al* 1996 Mhd-mode stabilization by plasma rotation in textor *Plasma Phys. Control. Fusion* **38** 467
- [20] Maraschek M. *et al* 2007 Enhancement of the stabilization efficiency of a neoclassical magnetic island by modulated electron cyclotron current drive in the asdex upgrade tokamak *Phys. Rev. Lett.* **98** 025005
- [21] Olofsson K.E.J. *et al* 2016 Electromechanical modelling and design for phase control of locked modes in the DIII-D tokamak *Plasma Phys. Control. Fusion* **58** 045008
- [22] Brennan D.P. and Finn J.M. 2014 Control of linear modes in cylindrical resistive magnetohydrodynamics with a resistive wall, plasma rotation, and complex gain *Phys. Plasmas* **21** 102507
- [23] Chen X.L. and Morrison P.J. 1990 The effect of viscosity on the resistive tearing mode with the presence of shear flow *Phys. Fluids B: Plasma Phys.* **2** 2575–80
- [24] Buttery R.J. *et al* 2008 The influence of rotation on the β_N threshold for the $2/1$ neoclassical tearing mode in DIII-D a *Phys. Plasmas* **15** 056115
- [25] La Haye R.J. and Buttery R.J. 2009 The stabilizing effect of flow shear on $m/n = 3/2$ magnetic island width in DIII-D *Phys. Plasmas* **16** 022107
- [26] Austin M.E. and Lohr J. 2003 Electron cyclotron emission radiometer upgrade on the DIII-D tokamak *Rev. Sci. Instrum.* **74** 1457–9
- [27] Strait E.J. 2006 Magnetic diagnostic system of the DIII-D tokamak *Rev. Sci. Instrum.* **77** 023502
- [28] King J.D. *et al* 2014 An upgrade of the magnetic diagnostic system of the DIII-D tokamak for non-axisymmetric measurements *Rev. Sci. Instrum.* **85** 083503
- [29] Hanson J.M. *et al* 2016 Validation of conducting wall models using magnetic measurements *Nucl. Fusion* **56** 106022
- [30] Ogata K. 2010 *Modern Control Engineering* (Upper Saddle River, NJ: Pearson Education Inc.)
- [31] Felici F. and Oomen T. 2015 Enhancing current density profile control in tokamak experiments using iterative learning control *IEEE 54th Annual Conf. on Decision and Control* (IEEE) pp 5370–7
- [32] Ravensbergen T. *et al* 2018 Density control in ITER: an iterative learning control and robust control approach *Nucl. Fusion* **58** 016048
- [33] Lohr J. *et al* 2005 The electron cyclotron resonant heating system on the DIII-D tokamak *Fusion Sci. Technol.* **48** 1226–37
- [34] Matsuda K. 1989 Ray tracing study of the electron cyclotron current drive in DIII-D using 60 GHz *IEEE Trans. Plasma Sci.* **17** 6–11
- [35] Perkins F.W. *et al* 1997 Prospects for electron cyclotron current drive stabilization of neoclassical tearing modes in ITER *24th EPS Conf. Proc.* p 1017
- [36] La Haye R.J. *et al* 2002 Control of neoclassical tearing modes in DIII-D *Phys. Plasmas* **9** 2051–60
- [37] Westerhof E., de Blank H.J. and Pratt J. 2016 New insights into the generalized rutherford equation for nonlinear neoclassical tearing mode growth from 2D reduced mhd simulations *Nucl. Fusion* **56** 036016
- [38] Ayten B. and Westerhof E. 2012 Consequences of plasma rotation for neoclassical tearing mode suppression by electron cyclotron current drive *Phys. Plasmas* **19** 092506
- [39] De Lazzari D. and Westerhof E. 2009 On the merits of heating and current drive for tearing mode stabilization *Nucl. Fusion* **49** 075002
- [40] Neumeyer C. 2011 Design of the ITER in-vessel coils *Fusion Sci. Technol.* **60** 95–9
- [41] Foussat A. *et al* 2010 Overview of the ITER correction coils design *IEEE Trans. Appl. Supercond.* **20** 402–6
- [42] Ioki K. *et al* 1998 Design and material selection for ITER first wall/blanket, divertor and vacuum vessel *J. Nucl. Mater.* **258** 74–84