

## 7 Week 7. Thermonuclear fusion: an overview, with Ambrogio Fasoli

### 7.1 The basics

#### 7.1.1 Why fusion?

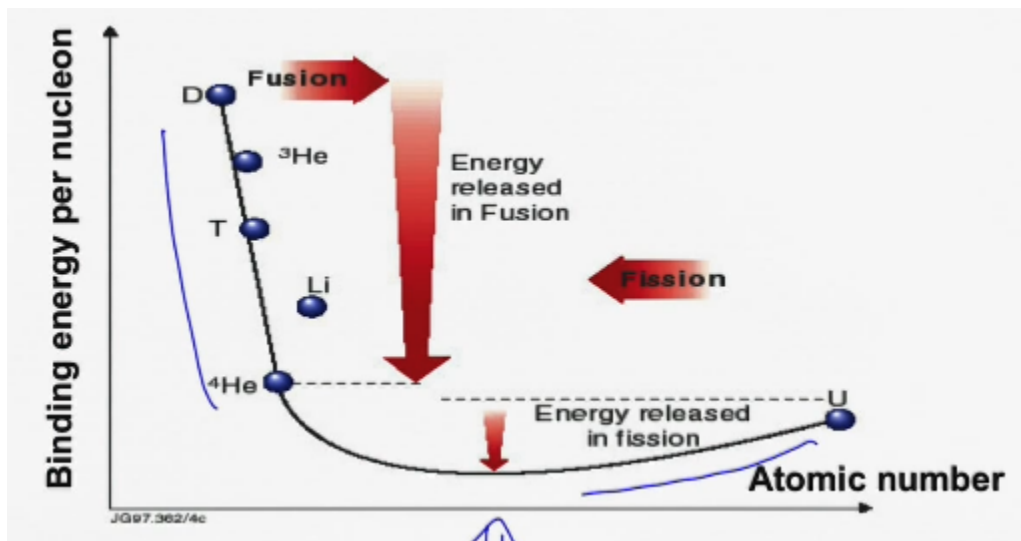
HDI correlated with electricity use per capita (though Canada is using almost three times as much as the Netherlands so there's room for improvement there). So to develop humankind we need better power sources.

We have to fight global warming, stop using fossil fuels and save polar bears.

We also have to make sure the power sources are safe for the population. Energy sources that endanger people are **bad**.

And fusion fulfils all of those criteria!

#### 7.1.2 How does it work



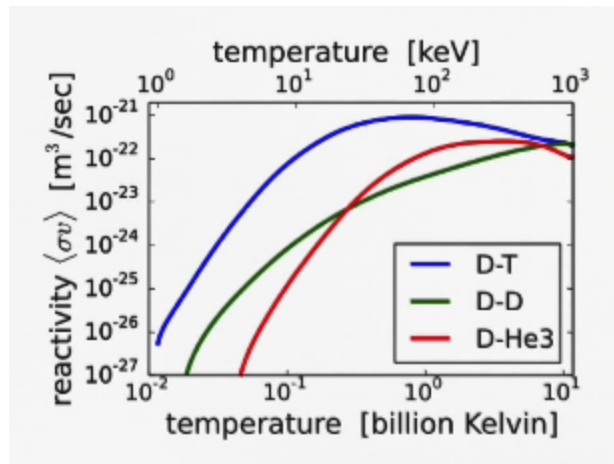
Two possible approaches to nuclear energy generation - split really heavy nuclei, like uranium, or merge small nuclei. Note that as the slope is sharp on the left, so you get a lot of energy per each fusion.

#### 7.1.3 Fusion reactions for a terrestrial reactor

Three possibilities: DT, DHe3, DD fuel cycles.

At low kinetic energies coulomb repulsion deflects particles. This prevents fusion from occurring (essentially interception by atomic forces). High kinetic energies are equivalent to saying we need high temperature.

You don't actually need  $380\text{keV}$  to get into the potential well on the left - you can do with  $10\text{keV}$  due to tunneling (a quantum effect).



Cross section times average velocity over distribution functions (AKA **reactivity**) - this is a parameter that describes how easy it is for particles to fuse. The DT reaction has the largest one at that for the smallest temperatures, so we're focusing on that for now.

At these temperatures (10keV or so), pretty much all matter is in the plasma state! So we're gonna need to go through this to understand that.

**Thermonuclear** fusion - as Coulomb collisions have a humongous cross section, any monoenergetic beam would be scattered in a plasma before the particles have time to fuse. Thus, an important condition is that our plasma is in a **thermalized** Maxwellian distribution (this would mean that the plasma is kind of in a stationary state).

#### 7.1.4 The DT cycle

This is the current approach to terrestrial fusion. Deuterium is ubiquitous in the oceans. Tritium has a half-life of 12.5 years and has to be **bred** from lithium. That can also be found in the ocean.

We look into the  $Li - 6$  reaction due to its large cross-section. Smack a neutron into the  $Li - 6$  atom and one  $T$  and one  $He - 4$  pops out. Lithium can be supplied by using a **blanket** mounted on the inside wall of the vacuum vessel.

#### 7.1.5 Schematic of a fusion reactor

We inject deuterium into the reactor and lithium into the blanket. The tritium stays in a closed system. Helium can be bottled and made into balloons or sold at high prices. The energy is removed from the system by a conventional fluid based system.

#### 7.1.6 Why fusion is neat

Fusion energy density:  $3.5e8 MJ/kg$ . Compare to  $30 MJ/kg$  for coal,  $50 MJ/kg$  for oil,  $8.5e7 MJ/kg$  for uranium fission.  $9e10 MJ/kg$  for  $E = mc^2$  total conversion.

The fuels are virtually inexhaustible. Lithium specifically can be mined (one example - Nevada), both lithium and deuterium can be obtained from sea water. This is so uniformly spread over the Earth that you could almost call it open source.

The environmental impact? No greenhouse gases whatsoever, helium is neat. The radioactive elements, depending on the materials used, take a short time to become safe.

It is impossible to weaponize a fusion reactor. You can't lose control of the reaction (no criticality) and there's very little fuel in the reactor at any given time (about 1g) .

### 7.2 Power balance in reactors

Each DT reaction gives  $3.5 MeV$  to the  $\alpha$  and  $14.1 MeV$  to the neutron, for a total of  $17.6 MeV$ .

Fusion power density for a particular relative velocity (or energy).  $n_D, n_T$  are densities of deuterium and tritium.  $\sigma$  is the reaction cross-section.

$$R_{DT}(v)\Delta E_f = (n_D n_T \sigma_{DT}(v)v)\Delta E_f \quad (1)$$

We can integrate that over velocity distributions for both D and T.

Assuming neutrality ( $n_D = n_T = n_e/2 = n/2$ ), neglecting impurities and density of alpha particles:

$$\text{Power density} = n_D n_T \langle \sigma v \rangle_{DT} \Delta E_f = \frac{1}{4} n^2 \langle \sigma v \rangle_{DT} \Delta E_f \quad (2)$$

To produce  $1GW$  of fusion power at  $T = 20keV$ ,  $n = 5e20m^{-3}$ , use tabularized data for the average  $\langle \sigma v \rangle$ . Divide desired fusion power by fusion power density. The result:  $14m^3$  - apparently a reasonable volume.

### 7.3 Generalities

For steady state operation - conservation of energy. The losses include radiation, convection and conduction ('direct losses'). Our power input is whatever we do to heat the plasma and the fusion power produced inside.

The radiative losses include bremsstrahlung - electrons are accelerated in an electric field, producing x-ray radiation (which can pass through the plasma and the containment vessel - thus escaping). Power density of that  $\sim n^2 T_e^{1/2}$ .

The direct losses are described by the energy confinement time  $\tau_E$ . Power density of that  $= 3nT/\tau_E$ .

### 7.4 Calculation of power balance - breakeven

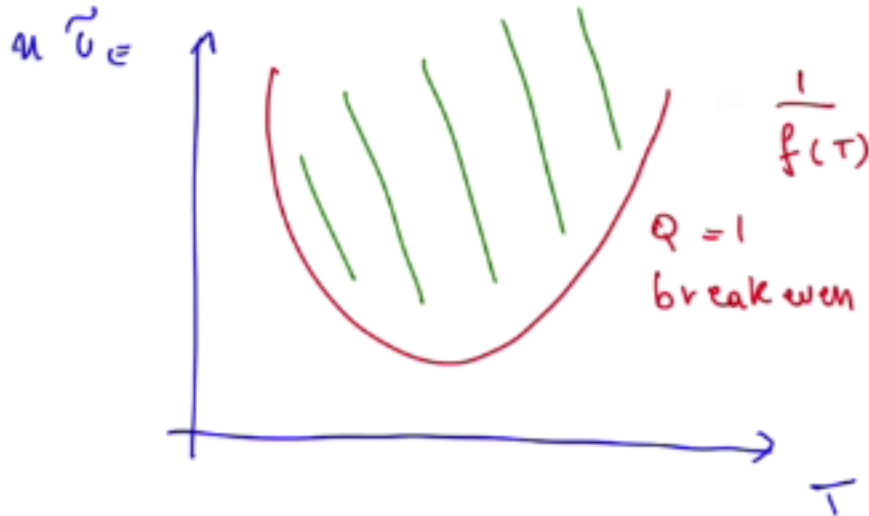
Def. physics fusion gain  $Q = \text{Fusion power}/\text{Input power} = P_f/P_i n$ . A reactor needs  $Q > 1$ . At  $Q = 1$  we have a breakeven situation.  $P_i n + P_\alpha = P_{losses} = P_{directlosses} + P_{bremsstrahlung}$ , so we get  $Q = \frac{P_f}{P_{directlosses} + P_{bremsstrahlung} - P_\alpha}$ .

Inserting the previously derived expressions:

$$n\tau_E \geq \frac{12T}{\frac{6}{5} \langle \sigma v \rangle_{DT} \Delta E_F + 4AZ_{eff} T^{1/2}} \quad (3)$$

At large temperatures (over  $1keV$ ), we can neglect the second term to get

$$\frac{1}{n\tau_E} = f(T) = \frac{\langle \sigma v \rangle_{DT} \Delta E_F}{10T} \quad (4)$$



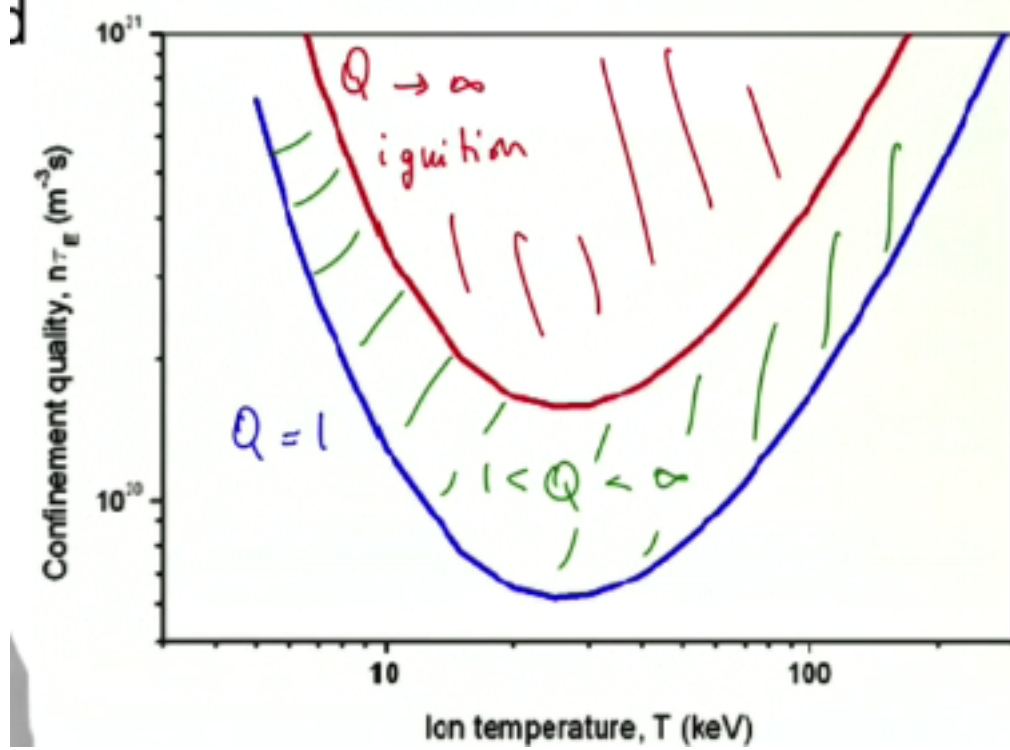
Reactors can work in the green area above the breakeven curve.

### 7.4.1 Ignition

This is the limit where all the heating the plasma needs comes from the fusion reactors. This would be the Sun on Earth scenario.  $P_F/P_{in} = +\infty$  as  $P_{in} \rightarrow 0$

$P_i n = 0$  means that  $P_\alpha \geq P_{losses} \sim P_{dl}$  (we neglect bremsstrahlung).

$n\tau_E$  at ignition  $\geq \frac{12T}{\langle \sigma v \rangle_{DT} \Delta E_\alpha} = 6/f(t)$ . It's just a higher parabola!



The burning plasma regime is when alpha particle heating is greater than the input heating,  $Q \geq 5$ .

### 7.4.2 Engineering fusion gain

Def.  $Q_E$  - ratio of electric power output to input,  $\eta_e$  - efficiency of plasma heating electrical power supply,  $\eta_t$  - efficiency of thermal fusion power conversion into energy.  $\eta = \eta_e \eta_t$ ,  $Q_E = \frac{\eta(P_F + P_{in}) - P_{in}}{P_{in}} = \eta Q - (1 - \eta)$ .  $Q \gg Q_E$ , sadly.

## 7.5 Plasma confinement approaches

We need both  $n\tau_E = 10^{20} m^{-3}s$  and  $T \geq 10 keV$ . Magnetic confinement has  $\tau_E$  about 1s. Inertial confinement goes with  $10^{-9}s$  and much higher densities.

## 7.6 Inertial confinement

We consider the ion confinement time  $\tau_i = R/c_s$ ,  $R$  being the pellet radius and  $c_s$  the speed of sound, at which the ions tend to move.

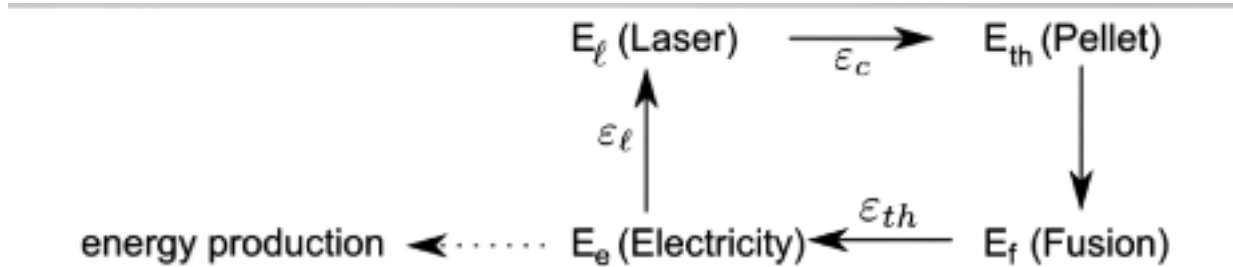
We also define the heating time  $\tau_h = \frac{\text{energy in pellet}}{\text{power deposited on surface}} = \frac{3nT4\pi R^3/3}{F4\pi R^2} = \frac{nTR}{F}$ ,  $F$  being the energy flux

We must have heating faster than the confinement time.  $\tau_h < \tau_i \rightarrow F > mT^{3/2}/\sqrt{m_i}$ . The size of the pellet cancels out. Note that we neglected all compressional effects (shock waves...).

For a standard example,  $F$  is about  $5e19 W/m^2$ . This means lasers. Really big lasers.

### 7.6.1 ICF energy balance

Laser inputs energy  $E_L$  - I took the liberty of changing notation. Thermal energy is scaled by efficiency  $\epsilon_c$ :  $E_{th} = E_L \epsilon_c$ . This produces fusion energy  $E_f$  and that process also has a finite efficiency in conversion to electricity,  $\epsilon_{th}$ . We can also recirculate some of the fusion energy to drive the reactor.



It turns out there's a simple condition for the laser energy sufficient for fusion. For typical values (solid deuterium), and relatively low efficiencies (5%, 10%) it turns out to give  $10^{15} J$  - 50 times the Hiroshima bomb. We can currently achieve  $2 MJ$ . This requires a density of  $3000 n_0$ ,  $n_0$  being solid deuterium density. This means we have to have a pressure of  $3000 n_0 T$ , which is  $2e17$  Pascal. That's a lot. Can't do that just by firing lasers.

### 7.6.2 The rocket effect

Generate an implosion by launching a lot of energy into the external surface. Conservation of momentum will cause an inward motion of the external layers and this will compress the plasma.

Pressure  $P_{rocket} = \frac{d(mv)}{dt_{surface}} \sim \frac{V}{S} \frac{dm}{dt}$  at constant  $v$

The energy flux  $F$ , at constant  $v$ , turns out to be  $\frac{v^2 dm}{2S dt}$  and we can derive the  $\frac{dm}{dt} = \frac{2SF}{v^2}$ .

Cancelling the terms, we get  $\frac{P_{rocket}}{P_{laser}} = \frac{2c}{v}$ . This is much bigger than 1 and thus a considerable boost

### 7.6.3 Additional ICF effects

The shockwave, if it goes symmetrically into the center, helps increase pressure.

Alpha production can also help heat the core, but the pellet must be larger than alpha mean free path.

However, instabilities, as always, occur and the laser light can't be too intense. Also, too much density means the laser beam actually gets reflected (plasma frequency cutoff occurs)

### 7.6.4 Summary of physics

- Radiation phase

Surface heated by laser, gets plasmified

- Blowoff phase

This is where the surface is blown off and the rest is pushed inside.

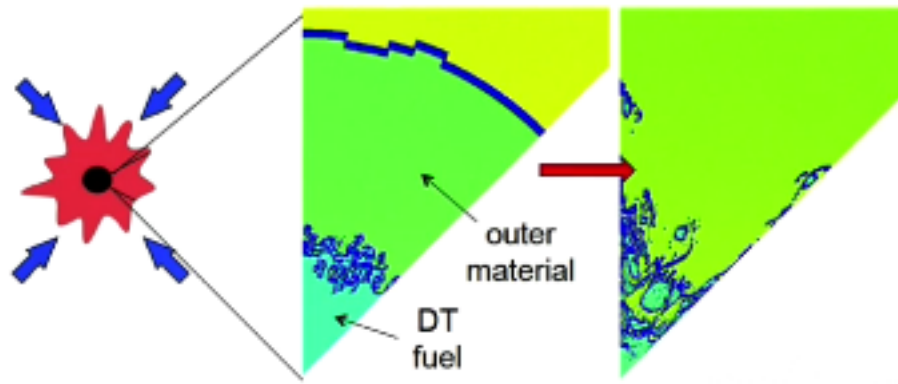
- Implosion phase

Thermal energy is transported to the center, the core is heated

- Thermonuclear burning

The challenges here are as follows: The core cannot be heated before the shock wave hits it (separate it from the rest somehow?). Imposes constraints on laser pulse timing and pellet design.

Also, there's the question of hydrodynamic stability:



The pellet must be very symmetric and the beams have to be really carefully aligned. Shown above is a mixing of the DT fuel and core material due to the Rayleigh Taylor instability - we don't really want that.

### 7.6.5 Direct and indirect drive

The direct drive is the simpler one - we just smack the pellet with lasers, as symmetrically as possible. There's multiple layers - a low density ablator on the outside, a ball of solid or liquid fuel and a gas layer.

The indirect drive is intended to lower the symmetry requirements. We have a small chamber - the so called hohlraum - around the pellet. It receives laser beams through two holes and acts as a black body source of x-rays. This is inherently more symmetric than injecting the pellet with laser energy.

The Lawrence Livermore Laboratory's National Ignition Facility uses 192 lasers to achieve pulse energy of  $2MJ$ , with about  $500TW$  of power. These fire UV light ( $352nm$ ).

### 7.6.6 Engineering challenges

Cost, efficiency, reliability. Difficult to find materials for first vacuum chamber wall. Capsules are really complex (3d printing, perhaps?).

A reactor would need to fire a few times a second to function as a power generator.

## 7.7 Magnetic confinement

Confine particles by making them rotate around magnetic field lines. Macroscopic confinement times allow lower densities.

There's many schemes for this:

- Linear devices - such as magnetic mirrors
- Toroidal configurations
- Stellarators

### 7.7.1 Simple toroidal device example

A simple toroidal field has curvature and an intensity gradient. This causes drifts - positive charges go to the top, negative charges to the bottom. This causes first an E field, then an ambient magnetic field and this means loss of confinement.

What we do is impose a poloidal field. Through their motions particles stay in regions of different drifts directions.

There's the so called hoop force which tends to move field lines to the outer region of the tokamak. To avoid this, we add a vertical field which counters that.

To sum up:

- Toroidal field coils
- Plasma current generates poloidal field
- Outer poloidal field coils that generate the vertical field, can also help positioning

How to drive the current in the plasma? That's a neat idea - it's basically a transformer. The plasma is the secondary circuit and the primary circuit is a coil inside the core - center - of the tokamak. Drive AC through that and you drive AC through the plasma by Faraday's law.

### 7.7.2 Progress

Density times confinement time with respect to temperature - we're steadily moving towards the high energy multiplication and sustained burn region.  $Q = 5$  is getting really close now. ITER should have  $Q$  of about 10 - a reactor needs about 30 or 40. TFTR and JET actually got close to breakeven by significantly increasing the confinement time. ITER aims to push that to several hundreds of second.

ITER is being built in Cadarache in the south of France. A really global project.<sup>1</sup>

And now, back to our regularly scheduled programme on magnetic confinement. 5

## 7.8 Simple design of a magnetic fusion reactor

Constraints on the design result from nuclear physics and engineering.

We put lithium in the blanket, which also collects the heat (usual water energy transfer system).

Tritium is produced in the blanket - has to be kept in a closed system, as it's used in the DT plasma for fusion purposes.

Around the blanket we need magnets for B field generation.

We want  $1 < Q < \infty$ , between breakeven and ignition.  $n\tau_E$  is thus between  $1e20$  and  $6e20m^{-3}s$ .  $T \geq 10keV$ .

We want to minimize the cost of the reactor and the requirements on plasma performance. This means minimizing  $\tau_E$  and  $\beta$  - ratio of plasma pressure and magnetic pressure.

We'll be optimizing:

- the size of the reactor
- reactor geometry
- magnitude of  $B$  field
- $n$
- $T$
- $\tau_E$

Our constraints are those resulting from both physics and engineering.

### 7.8.1 Simplified geometry for magnetic fusion reactor

Assume toroidal general shape, circular cross section. Plasma on the inside, radius  $a$ . Blanket of thickness  $b$ . Magnet thickness  $c$ .

From engineering:

We want to produce  $P_E = 1GW$  for now.

Wall loading  $L_W$  - this tells us how much power we can be depositing on the wall. Usually about  $5MW/m^2$ .

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<sup>1</sup>Really makes you wonder why the victory condition in Civilization games is the space race to Alpha Centauri, huh?

The magnets have to be superconducting in a certain region in BJT space - in practice this says that we can only have 13T at the coil.

From nuclear physics:

Value of fusion rate determined by fusion cross-section.

Processes in the blanket also have their cross-sections:

Neutron multiplication (has to occur before breeding tritium, has to occur at the same rate).

Neutron slowing down - at high neutron energies tritium production cross-section decreases.

Note that we only consider lithium 6 - at low temperatures the cross section for Li-7 is negligibly small. We don't really have to worry about that, it seems (at least at this basic level).

We must also shield the coils from the neutrons - don't want those to become radioactive, of course!

### 7.8.2 Neutron physics - blanket and shield thickness

For each process, the thickness required is the inverse of number density of targets times cross section.

Neutron multiplication can be done on beryllium. Thickness comes out to be 13cm.

Neutron slowing down can be done on lithium itself - that's 20cm.

Tritium breeding at room temperature 0.025eV with 7.5%Li<sup>6</sup> turns out to be 0.2cm. Note that the 7.5% purity of lithium comes into the number density. So we don't have to add more than necessary for slowing down, it seems.

For coil shielding, say we want to reduce the flux 100 times. Assume only the slowdown layer (which is the largest) - we need 1m.

Total thickness about 1.2m - it's not direct addition as layers overlap in function.

### 7.8.3 Minimisation of cost of electricity

Minimising reactor cost over power is equivalent in this kind of work to minimizing volume of complex systems over power. This means - everything but the plasma, that's not something you build.

Volume of torus  $2\pi R_0 \times \pi((a+b+c)^2 - a^2)$ . Electrical power produces is 1/4 efficiency of power input times sum of energies in alpha, neutron and tritium (in blanket!) production, times density squared, times DT cross-section, times plasma volume ( $2\pi R_0 \pi a^2$ )

We use the wall loading constraint to get  $R_0$  - we take the 5MW/m<sup>2</sup> value times plasma surface area.

Neutron power is fusion power over efficiency in transformation into electric energy, times fraction of neutron energy over total produced energy.

From the equality between the latter two pops out (upon assuming 40% =  $\eta_t$ ) an expression for  $R_0$ .

We can now take the complex system volume over power fraction - the electrical power actually drops out - and we're left with an expression:

$$\frac{V}{P} = 0.8 \frac{(a+b+c)^2 - a^2}{a L_W^{max}}$$

Thus the better materials we have (more wall loading), the lower the volume and cost.  $b$  is already set - due to blanket physics - so we're optimizing in 2-space, over  $a$  and  $c$ .

### 7.8.4 Magnet thickness

That should also be minimum. There's the  $\mathbf{J} \times \mathbf{B}$  force on the magnet (superconducting coil) and it has to be able to withstand the stress. A calculation with tensile stress gives  $c = \frac{2\alpha}{1-\alpha}(a+b)$ , alpha being  $\frac{B_{coil}^2}{4\mu_0 \sigma_{max}}$ ,  $\sigma_{max}$  being the maximum stress the magnet can endure.

It's difficult to reach high  $\beta$  in the plasma. So we need the maximum field at the coil - 13T (at higher values they stop superconducting, at least with today's technology).

Maximum stress is about 200MPa.  $\alpha \sim 0.1$ . So  $c = 0.22(a+b)$  - it simplifies out of our optimization problem! It's a one variable problem, in this simplified version.



$$\frac{V}{P} = 0.8 \frac{(a+b+c)^2 - a^2}{aL_W^{max}}$$

Taking the derivative we get  $a = \sqrt{3}b$ .  $c = 0.22(\sqrt{3} + 1)b = 0.6b$ .  $b = 1.2m$ ,  $a = 2.1m$ ,  $c = 0.7m$ .

### 7.8.5 Resulting reactor geometry and plasma parameters

Taking  $L_W^{max} = 4.5MW/m^2$  we get  $R_0 = 4.2m$ . The plasma surface becomes  $350m^2$ , volume  $365m^3$ . The power density in that - power of alphas and neutrons over volume -  $6MW/m^3$ .  $B_0$  being the magnetic field in the plasma, at the major radius.  $B \propto 1/r$  and at the coil it's  $13T$ , so we'll get  $2.7T$  at the major radius, assuming coil located at  $R_0 - a - b$ .

Assuming  $\tau_E = 2s$ ,  $n = 1e20m^{-3}$ ,  $T = 15keV$ ,  $\beta$  turns out to be  $= \frac{nT}{B_0^2/2\mu_0} = 8\%$ .

## 7.9 Plasma-wall interactions

### 7.9.1 The first wall

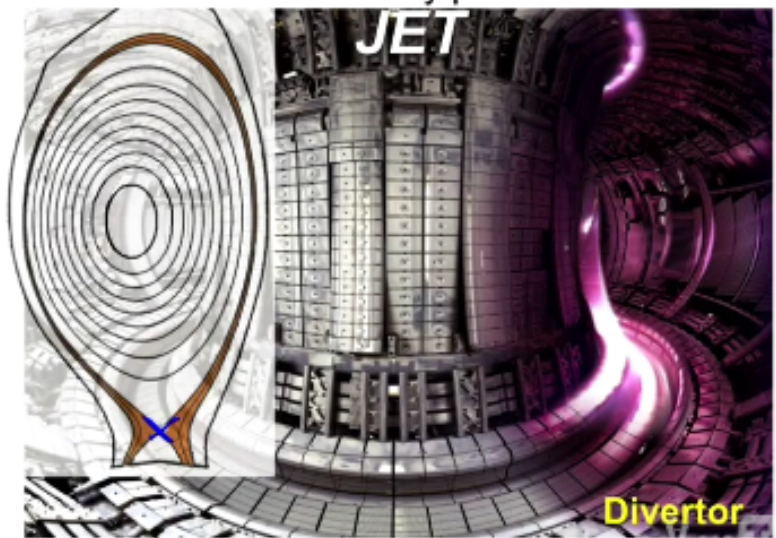
We now turn our investigation to the requirements faced<sup>2</sup> by the reactor wall. This has to take  $10MW/m^2$  of heat!  $1MW/m^2$  is felt by reentering spacecraft.  $80MW/m^2$ ? A live rocket engine. What we also want to make sure is that the plasma is not contaminated - impurities are bad in that they increase ignition temperature, and radiate energy away (line and brehmstrahlung radiation, which goes as  $Z_e f f \sqrt{T_e}$ ). Note that this means we have to use low Z materials, it seems.

We also have to stop retention of T in the wall. Tritium slowly accumulates during the discharge. We can tolerate up to 700g of T in the wall. Carbon materials accumulate it much quicker than if you made it all out of tungsten, which seems like a neat solution.

The walls must also minimize dust production, that leads to instabilities, allow containment, refueling and removal of helium ashes - low energetic alpha particles. We have to remove those as fast as possible.

### 7.9.2 Limiters and divertors

Direct contact should be avoided, but is ultimately necessary (helium, energy removal, etc). People use limiters and divertors for that. Limiters stick out into the plasma and intercept particles. Divertors use an X shaped magnetic field configuration which has a null point for the poloidal field.



Plasma

<sup>2</sup>Sorry.

### 7.9.3 The scrape off layer

This is the layer of the plasma which is in direct contact with the wall. The parallel flow in that strikes the limiter. Using a diffusion argument (Fick's law), we get a  $1\text{cm}$  thickness for ITER - a lot of power striking a small surface!

### 7.9.4 Divertor advantages

Particles move along field lines. The 'connection length' for the ITER divertor will be about  $150\text{m}$ , this slows down the flux of particles and reduces parallel power flux. There is also a temperature gradient - the plasma is colder towards the divertor while being thermonuclear near the core.

The slowdown of particles means that there's less sputtering and charge exchange collisions. This also means that there's less impurities in the main plasma chamber - it seems to be mostly flowing outside to the divertor chamber.

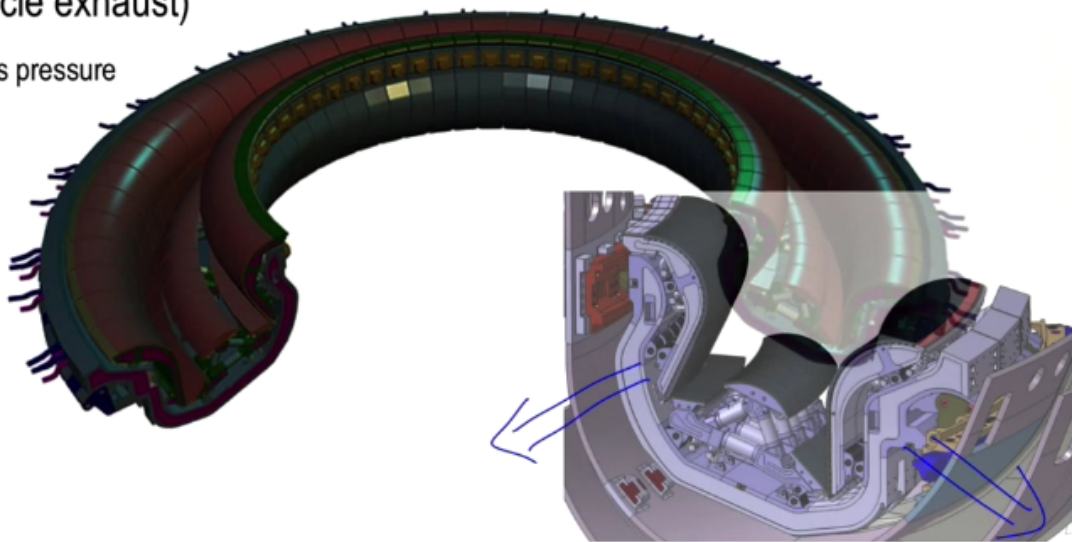
High confinement regimes are allowed by the divertor concept - they have low turbulence and are easier to confine. Transport barrier - a calm layer of plasma that stops transport.

L-mode, AKA low confinement mode - turbulent edge. Small amplitude pressure gaussian, particles sorta tunnel through.. In the H-mode, the pressure gaussian is much taller and this effectively stops particle transport.

Due to the high pressure region, pumps can exhaust particles easier from there. Cryopumps can also be used to exhaust material.

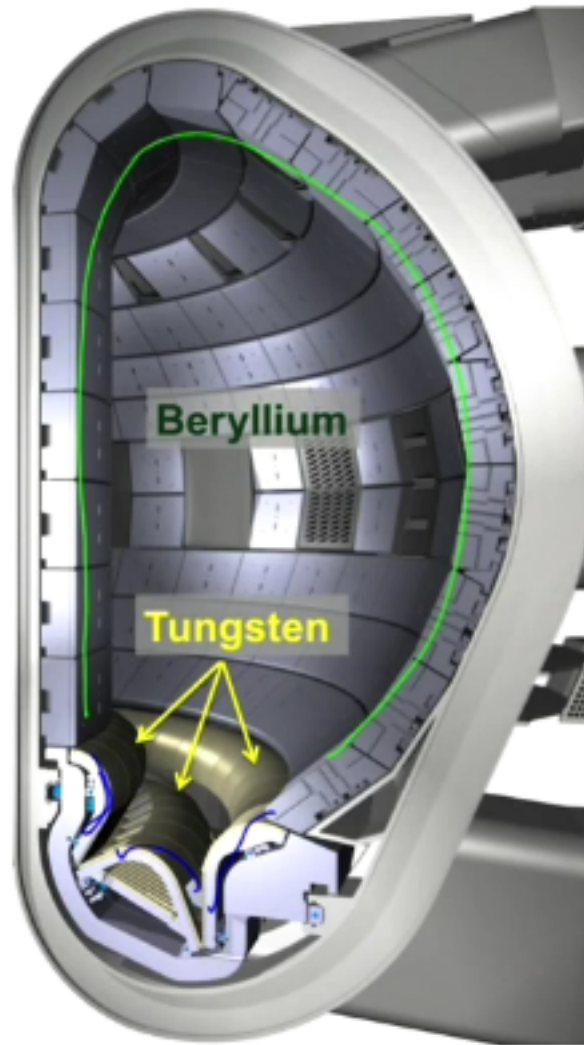
- **Pumping (particle exhaust)**

- Higher neutral gas pressure
- Cryopumps



The divertor allows the plasma to cool down to about  $5\text{ eV}$ . This lowers ionisation cross-section. Allows easier charge exchange collisions. This transfers the energy from ions to neutrals, forming a so-called neutral cushion in the divertor chamber. Energy is mostly dissipated here in radiation and not through wall interactions, which is neat! Otherwise this would hit the wall.

In ITER, the divertor will be made of tungsten. The walls will be made of beryllium. Both have low tritium retention. Tungsten has high  $Z$  but that's okay because the sputtering threshold is high, so not much of it will be injected into the plasma. These materials are also resistant to neutron irradiation.



Transients include ELMs - Edge Localised Modes. Steep gradients of pressure near the edge - large thermodynamic potential - can drive violent instabilities -  $15MJ$  for ITER in a single burst over  $0.2ms$  deposited over  $6m^2$  -  $10GW/m^2$  of power. Heats wall to  $6000C$  - melts the metal.

Another issue are disruptions. A disruption is a sudden loss of plasma control. Deposits energy on the wall, on very specific areas. In ITER, if we have disruptions the divertor will fail after about 300 of them.

We can't fix this right now via materials! We have to control the plasma itself. DEMO will need a lot of work. There's plenty of alternative concepts, some of them: liquid metal walls - a thin layer of melted lithium circulating around the plasma. Super-X divertor designs - expand the divertor to get more volume for the plasma chamber, from which we can radiate energy. Snowflake divertor - tested at EPFL - give the divertor four legs instead of two. Longer connection length. Can change plasma stability, must be investigated. Not sure if this will work better than what current divertors.

## 7.10 Structural materials

Large fluxes of  $14.1MeV$  neutrons in reactors (results of fusion). We must thus minimize activation of materials. Fusion is at least an order of magnitude lower in thermal power than fission. Ferritic steel is nice, but silicon carbide and vanadium alloys are really amazing - they get cold relatively quickly.

Use Carnot efficiency,  $\eta_{Carnot} = 1 - T_{cold}/T_{hot}$ . Higher plasma temperatures and lower coolant temperatures lead to increased  $\eta$ .

Fission reactors go steady state quickly, but temperatures are relatively low in our terms. Fusion reactors would have much higher temperatures, but violent transfers. Main difference: fission has average neutron energy  $2MeV$ . In fusion  $14.1MeV$ . Causes requirement for more complex materials devices.

### 7.10.1 Physical picture of energetic neutron collisions

A  $E_n$  neutron hits mass  $M$  atom, resting in a lattice. Maximum transferred energy  $E_{max} = E_n \frac{4m_n M}{(m_n + M)^2} \sim \frac{E_n 4m_n}{M}$ . For iron,  $M = 56 amu$ .  $E_{max} = 1 MeV$ . Much higher than the so called Wigner energy - threshold value for energy, at which atom is displaced in the lattice (usually about  $25 eV$ ). The iron atom gets bumped out of the lattice! What's worse, the process cascades. The pair between the hole and the displaced atom is called a Frenkel pair. We define displacements per atom as measure of damage.

Not as simple as that though - the neutron also causes nuclear transmutation. It generates hydrogen and helium atoms - this causes the grain boundaries to become brittle. Measured in atomic pair per million (appm) of He or H.

This causes changes in the material microstructure. Degrades macroscopic properties. Defects migrate across the lattice. For a  $3GW$  fusion reactor, we expect  $20 - 30 dpa/y$  in steels. ITER is expected to have  $3 dpa$  at end of life cycle. Appm/dpa is much higher than in fission! This causes:

- chemical composition changes
- changes in physical properties such as electrical and thermal conductivity for functional materials
- changes in mechanical properties for structural materials
- changes in mechanical dimensions - stuff starts deforming, swelling, growing in length... wacky.

Thus we are constrained to considering low activation elements. Currently we're looking into:

- Reduced activation ferritic or martensitic steels - least advanced option
- Vanadium alloys
- Tungsten alloys
- SiC or its composites

We have to test the materials! We know little experimentally about the behavior of those materials under such strong neutron fluxes<sup>3</sup>. We know much more about the plasma parameters at fusion regimes than about material behavior in those regimes. Have to extrapolate from small samples.

Current ideas for testing those are low fusion gain tokamaks and accelerator based irradiation test facilities, such as the IFMIF project - uses a deuterium ion beam at  $10 MW$  to hit a liquid lithium target, which produces lots of neutrons to strike a small solid material target. Can extrapolate results for larger samples.

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<sup>3</sup>High energy neutron gun recently developed at IFPILM may help, go Poland, can into fusion project!