

Database of Prompt Gamma Rays from Slow Neutron Capture for Elemental Analysis

$$\begin{aligned}\kappa_0(E\gamma) &= \kappa\zeta(E\gamma) / \kappa_{H(2223)} \\&= [\sigma\gamma\zeta(E\gamma) / A\rho(Z)] / [\sigma\gamma H(2223)/ A\rho(H)] \\&= 3.03 \xi [\sigma\gamma\zeta(E\gamma) / A\rho(Z)]\end{aligned}$$

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**INTERNATIONAL ATOMIC ENERGY AGENCY
VIENNA, 2007**

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FOREWORD

The increasing importance of prompt gamma ray activation analysis (PGAA) in a broad range of applications is evident, and has been emphasized at many meetings related to this topic. Furthermore, an Advisory Group Meeting (AGM) for the Coordination of the International Network of Nuclear Structure and Decay Data Evaluators concluded that there is a need for a complete library of gamma ray and cross-section data from cold and thermal neutron capture (the AGM was held in Budapest, 14–18 October 1996); this AGM also recommended the organization of an IAEA Coordinated Research Project (CRP) on this subject.

The nuclear data programmes of the IAEA arose as a consequence of the advisory reviews of the International Nuclear Data Committee (INDC). At a biennial meeting in 1997, the INDC strongly recommended that the IAEA support new measurements and update the database on the analysis of prompt gamma ray activation induced by neutrons.

As a consequence of the various recommendations, a CRP, entitled Development of a Database for Prompt Gamma Ray Neutron Activation Analysis (PGAA), was initiated in 1999. Prior to this project, several consultants had defined the scope, objectives and tasks of this CRP, as approved subsequently by the IAEA. Each CRP participant assumed responsibility for the execution of specific tasks. The results of their work and of other research were discussed and approved by the participants in Research Coordination Meetings (RCMs) held in 2000, 2001 and 2003.

Prompt gamma ray activation analysis is a non-destructive radioanalytical method capable of rapid or simultaneous *in situ* multielement analyses across the entire periodic table, from hydrogen to uranium. However, inaccurate and incomplete data have been a significant hindrance in the qualitative and quantitative analyses of complicated neutron capture gamma spectra by means of PGAA. Therefore, the main goal of the CRP was to improve the quality and quantity of the required data in order to make possible the reliable application of PGAA in fields such as materials science, chemistry, geology, mining, archaeology, the environment, food analysis and medicine. This aim was achieved due to the dedicated work and effort of the participants. The CD-ROM included with this publication contains the database, retrieval system, three RCM reports, and other important electronic documents related to the project (see also Chapter 8).

The IAEA wishes to thank all CRP participants who contributed to the success of this project and to the formulation of this publication. The IAEA is grateful to R.B. Firestone for his leading role in the evolution of this CRP and for his comprehensive compilation, analysis and provision of the adopted database, and to V. Zerkin for the software developments associated with the retrieval system. An essential component of this data compilation is the extensive set of new measurements of neutron capture gamma ray energies and intensities undertaken at the Institute of Isotope and Surface Chemistry, Budapest. The IAEA is also grateful to S.C. Frankle and M.A. Lone for their active involvement as consultants at some of the meetings. The technical officer responsible for the CRP, this publication and the resulting database was R. Paviotti-Corcuera of the Division of Physical and Chemical Sciences.

Contributors:

Choi, H.D.	Seoul National University, Republic of Korea
Firestone, R.B.	Lawrence Berkeley National Laboratory, United States of America
Lindstrom, R.M.	National Institute for Standards and Technology, United States of America
Molnár, G.L.	Institute of Isotope and Surface Chemistry, Hungarian Academy of Sciences, Hungary
Mughabghab, S.F.	Brookhaven National Laboratory, United States of America
Paviotti-Corcuera, R.	International Atomic Energy Agency
Révay, Z.	Institute of Isotope and Surface Chemistry, Hungarian Academy of Sciences, Hungary
Trkov, A.	International Atomic Energy Agency
Zerkin, V.	International Atomic Energy Agency
Zhou, Chunmei	China Nuclear Data Centre, China

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Chapter 1

INTRODUCTION

R.M. Lindstrom

Prompt gamma activation analysis (PGAA) from neutron capture is especially valuable as a non-destructive nuclear method in the measurement of elements that do not form neutron capture products with delayed emissions of gamma rays. Furthermore, the elemental coverage of PGAA complements that of conventional (delayed) instrumental neutron activation analysis (INAA). The list of measurable elements emphasizes the low Z and high abundance elements in organic and geological materials, and the high cross-section elements: B, Cd, Sm and Gd. The analysis for hydrogen and boron is especially important because of the paucity of other reliable analytical techniques for trace levels of these elements. Prompt gamma activation analysis is, compared with destructive chemical techniques, extremely sensitive in the quantitative determination of boron, particularly since boron is such an important element over a wide range of situations from meteorites to human tissue [1.1–1.4]. Together, PGAA and INAA can measure all elements except oxygen in most common materials. Conveniently, in silicate rocks and similar oxidized materials, the completeness of the analysis can be tested by expressing the elements as oxides and comparing their sum with 100% [1.5]. Because nearly every neutron capture is an (n, γ) reaction, the yield of prompt gamma rays per neutron is greater than that of delayed gamma rays [1.6]. Unfortunately, PGAA usually has a poorer sensitivity compared with that of INAA because the neutron flux is some five orders of magnitude lower in an external reactor beam than for an irradiation position near the core.

Many review articles have been published on PGAA and its applications [1.7–1.12], and two extensive bibliographies have been compiled [1.13, 1.14]. Reference [1.14] lists 522 references, up to and including 1983. A dedicated book has also appeared [1.15], and an extensive handbook has been published recently [1.16]. Prompt gamma ray analysis developed slowly after the first reports of gamma radiation from neutron capture by Lea [1.17] and the Fermi group [1.18]. The first published tabulation of gamma ray energies and intensities [1.19] and plots of spectra [1.20] led to a

number of applications during the era of NaI scintillation counters, from borehole logging [1.21] to planetary exploration [1.22]. Applications involving coincidence counting were first reported at the second international conference on Modern Trends in Activation Analysis (MTAA-2) [1.23].

The first measurements using reactor based PGAA were published in 1966 [1.6, 1.24, 1.25]. Chopped (pulsed) beams were used in one of the first applications to separate prompt gamma rays from delayed activation products [1.26]. Neutron guides were also first reported in the same year [1.27], and soon afterwards pioneering PGAA work at Saclay with thermal guides and Ge(Li) detectors was reported [1.28, 1.29].

A major breakthrough in the late 1960s was the introduction of germanium semiconductor gamma ray detectors, with energy resolutions 20 or more times better than those of the best NaI scintillators. This development was a considerable aid in the interpretation of complex spectra resulting from neutron capture [1.30]. Diffraction spectrometers used by the nuclear physics community have still better resolutions [1.31], but their efficiency is far too low for practical analysis of materials. Application of germanium detectors to INAA [1.32] and PGAA [1.33] was rapid, and their superior resolution gave improved detection limits [1.34], which led to germanium replacing NaI wherever liquid nitrogen was available to cool the detector.

Early in the application of germanium detectors, a group at the Massachusetts Institute of Technology (MIT) measured the neutron-capture gamma spectra of every element systematically [1.35, 1.36]. Compilations of these data were published in the open literature, with analytical sensitivities and spectral contrasts tabulated [1.37, 1.38]. At that time, the combination of high power research reactors and large gamma ray detectors of high resolution was pursued in parallel at several reactor centres in the USA, Japan and Canada [1.5, 1.39–1.42]. Each of these laboratories compiled tables of analytical gamma rays and their interferences. For example, at the University of Maryland 28 gamma rays from 20 elements were found to be potential interferences with the sulphur line at

841.1 keV (from the $^{32}\text{S}(\text{n}, \gamma)^{33}\text{S}$ reaction) [1.43]. An evaluation directed at the spectrometry of planetary surfaces was also published [1.22].

A major advance was the comprehensive Chalk River compilation of more than 10 000 neutron-capture gamma rays of the elements [1.44], with their energies, abundances and cross-sections drawn chiefly from the MIT measurements. The completeness of the data and their convenient format made the ‘Lone table’ indispensable at the desk of every PGAA researcher for twenty years, despite some inadequacies inherent in these early measurements. A substantial and computer readable subset of these data was made available on diskette with an IAEA Technical Report [1.45], and the complete table has been circulated informally in spreadsheet form among many researchers.

A carefully evaluated table of neutron-capture gamma rays from the elements hydrogen through to zinc has been published [1.46] more recently. The present work incorporates this evaluation and adds recently measured energies and intensities of neutron-capture gamma rays of the elements from the PGAA facility at the Budapest Research Reactor, as well as data from other Coordinated Research Project (CRP) participants and elsewhere. As discussed in detail in Chapter 6, these data are combined and compared with nuclear levels and other information from the Evaluated Nuclear Structure Data File (ENSDF) to produce a comprehensive and self-consistent set of neutron-capture gamma rays.

In the past decade, the application of PGAA has increased because of the availability of high flux thermal and cold beams from neutron guides [1.47]. Guided beams can be entirely free of fast neutrons and tramp gamma rays, and therefore signal to background ratios can be much improved. Thermal guide studies at Kyoto University research reactor have also shown that the spectral quality is perhaps as important as flux in performing high sensitivity analyses [1.4]. Fifteen years after the pioneering work at Grenoble using a flux that is still the highest ever used for PGAA [1.48], there has been a flowering of applications at several neutron sources [1.49–1.55].

Analysis of prompt gamma rays from neutron activation has become a well established analytical method with applications in many areas. The new data compilation presented here should encourage the further use of PGAA in the future.

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Chapter 2

NOMENCLATURE, WESTCOTT g_w FACTORS AND NEUTRON SPECTRAL SHAPE DEPENDENT FORMALISM

H.D. Choi, A. Trkov

2.1. INTRODUCTION

A wide range of neutron source facilities are used for the implementation of PGAA that can be divided into two groups: one group uses thermal or cold neutrons from nuclear reactors, while the other group utilizes smaller mobile systems that involve moderated neutrons from isotopic sources, neutron generators or accelerator driven systems. Reactor based systems use an internal target [2.1, 2.2] or external direct beam [2.3] to take advantage of the large neutron flux. At present, the trend is towards building facilities around guided thermal beams [2.4–2.6] or guided cold beams [2.4, 2.7–2.9] in order to prepare a very clean beam that is free of epithermal neutrons and background gamma rays. Another possibility is to use external filtered beams [2.10] or diffracted beams [2.11, 2.12], which are also characterized by a low background.

Among the many differences between the facilities, the neutron energy spectrum and the epithermal neutron fraction have an important influence on the measured capture rate, particularly for large samples and non- $1/v$ absorber nuclides. Even for some nuclides that are commonly considered good $1/v$ absorbers, slight deviations from $1/v$ capture may exist. Inhomogeneous flux profiles also affect measurements. Precise measurements and standardization can only be achieved by investigating the impact of these effects before k_0 values from different facilities can be compared for consistency. Hence in the present chapter, the definition of nomenclature and a general formalism are reviewed in the context of k_0 standardization in order to accommodate the various forms of neutron spectra.

2.2. DEFINITIONS AND NOMENCLATURE

2.2.1. Prompt k_0 factors

Co-irradiating in a neutron field an analyte (x) and a comparator (c) element contained in a sample

results in the composite nuclear constant (k_0 factor) defined as [2.13–2.15]:

$$k_0 = \frac{P_x(E_{\gamma,x})}{P_c(E_{\gamma,c})} \frac{\sigma_{0,x}}{\sigma_{0,c}} \frac{\theta_x/M_x}{\theta_c/M_c} \quad (2.1)$$

where the subscripts x and c refer to the analyte and comparator element, respectively, θ is the isotopic abundance, M the atomic weight of the element, $P(E_\gamma)$ the absolute gamma emission probability (gammas emitted per capture) of the prompt gamma ray of energy E_γ , and σ_0 is the capture cross-section for 2200 m/s neutrons. It is implicitly assumed that the specific isotope that captures a neutron will decay promptly by emitting a gamma ray of energy E_γ .

The evolution of k_0 methodology has resulted in different definitions (e.g. by using either an effective capture cross-section or an effective thermal capture cross-section instead of the 2200 m/s cross-section [2.16]). Use of σ_0 is emphasized in the present definition in order to keep the k_0 factor as an absolute constant measurable in a facility independent manner.

2.2.2. Elemental cross-sections

The neutron speed-dependent capture cross-section, $\sigma_\gamma(v)$, and the 2200 m/s value (σ_0) are defined for a nucleus of an isotope. The partial capture cross-section for the nucleus, $\sigma_\gamma(E_\gamma)$, is defined by the product $P(E_\gamma)\sigma_0$; the differential form $P(E_\gamma)\sigma_\gamma(v)$ is also used in physics studies. An elemental cross-section is defined for practical convenience in terms of a sample with isotopic natural abundance, and this parameter should be distinguished from the nuclear capture cross-section and the partial nuclear capture cross-section. A partial elemental capture cross-section for the element Z is defined by:

$$\sigma_\gamma^Z(E_\gamma) = \theta P(E_\gamma) \sigma_0 \quad (2.2)$$

where the same notation is used as described previously. This term is the cross-section per elemental atom to produce a particular gamma ray of energy E_γ from irradiation with thermal neutrons. Different names are frequently used, such as ‘gamma ray production cross-section’ [2.17] or ‘partial (elemental) cross-section’ [2.18], both implying the partial elemental capture cross-section.

2.2.3. Effective capture cross-sections

The effective capture cross-section is defined, as the cross-section averaged over the neutron spectrum, by the following equation:

$$\hat{\sigma} = \frac{1}{v_0} \frac{\int_0^\infty n(v)\sigma_\gamma(v)v dv}{\int_0^\infty n(v)dv} = \frac{1}{n_t v_0} \int_0^\infty n(v)\sigma_\gamma(v)v dv = \frac{1}{v_0} \int_0^\infty \rho(v)\sigma_\gamma(v)v dv \quad (2.3)$$

where v is the neutron speed and v_0 equals 2200 m/s, $n(v)dv$ is the number density of neutrons with speed between v and $v + dv$, $\sigma_\gamma(v)$ is the neutron speed-dependent capture cross-section of the nuclide under consideration, n_t is the total neutron density including both thermal and epithermal neutrons, and $\rho(v)$ is the neutron speed distribution function after normalization. These are:

$$n_t = \int_0^\infty n(v)dv \text{ and } \int_0^\infty \rho(v)dv = 1 \quad (2.4)$$

in which the Westcott convention is adopted [2.19]. However, when the Stoughton and Halperin convention is used [2.20], thermal neutron density appears in the denominator of Eq. (2.3). A different convention is used in Ch. 4 for the effective cross-section $\langle\sigma\rangle$ to characterize the neutron beam:

$$\langle\sigma\rangle = \frac{\int_0^\infty n(v)\sigma_\gamma(v)v dv}{\int_0^\infty n(v)v dv} \quad (2.5)$$

where the integrated total flux is used in the denominator. The average cross-section is related to the effective cross-section in Eq. (2.3) by

$\langle\sigma\rangle = \hat{\sigma}v_0/\langle v \rangle$, where $\langle v \rangle$ is the average speed calculated using neutron density $n(v)$ as the weighting function. Equations (2.3)–(2.5) are applicable to any neutron spectrum.

2.2.4. Thermal and epithermal fluxes

As a consequence of the importance of thermal neutrons in capture reactions and the very large differences in the spectral shapes and the fractions of epithermal neutrons in different irradiation facilities, the neutron density per unit speed interval is split into thermal and epithermal components:

$$n(v) = n_{\text{th}}(v) + n_{\text{ep}}(v) \quad (2.6)$$

The reactor thermal neutron spectrum is well represented by the Maxwellian speed distribution, and the integrated thermal neutron density is given by:

$$n_{\text{th}} = \int_0^\infty n_{\text{th}}(v)dv = n_{\text{th}} \int_0^\infty \rho_M(v)dv \quad (2.7)$$

where $\rho_M(v)$ is the normalized Maxwellian function. Different definitions for the thermal flux can be found in the literature [2.20]. The definition widely used in activation analysis is the ‘conventional’ thermal flux given by:

$$\phi_{\text{th}} = n_{\text{th}} v_0 \quad (2.8)$$

while the ‘true (integrated)’ or ‘mean’ thermal flux is the most convenient in reactor physics calculations and is defined as:

$$F_{\text{th}} = \int_0^\infty n_{\text{th}}(v)v dv = n_{\text{th}} \int_0^\infty \rho_M(v)v dv = n_{\text{th}} \bar{v} \quad (2.9)$$

where \bar{v} is the average speed of the Maxwellian distribution. Hence, the relationship between the two fluxes ($F_{\text{th}}/\phi_{\text{th}} = \bar{v}/v_0 = (4T/\pi T_0)^{1/2}$) holds true for the Maxwellian thermal spectrum (where T is the Maxwellian temperature, $T_0 = 293.6$ K). The thermal capture rates for $1/v$ absorbers are the same for either flux representation, so long as the correct

2.2. DEFINITIONS AND NOMENCLATURE

cross-section is used; for example, $R_{\text{th}} = n_{\text{th}}v_0\sigma_0 = n_{\text{th}}\bar{v}\bar{\sigma}$, where $\bar{\sigma}$ is the capture cross-section at neutron speed \bar{v} . The neutron flux ϕ_{ep} is more convenient in the case of epithermal neutrons, and represents the product of neutron speed and density ($\phi_{\text{ep}} = v n_{\text{ep}}$). This approach describes the neutron flux spectrum in terms of energy and is based on theoretical considerations that ideally the distribution follows a $1/E$ shape. Since the flux integrals over neutron speed and energy must be the same, the following relationship is obtained between the epithermal neutron density and the flux:

$$n_{\text{ep}}(v)vdv = \phi_{\text{ep}}(E)dE = \phi_{\text{ep}}dE/E \quad (2.10)$$

Slight deviations from $1/E$ can be described by $1/E^{1+\alpha}$, where α is the epithermal shape parameter widely used in instrumental neutron activation analysis (INAA) [2.13, 2.21]. However, most PGAA facilities prepare a clean thermal or cold beam by means of neutron guide tubes or short wavelength filters. These beams are free from epithermal neutrons as indicated by the cadmium ratio, being typically larger than 10^4 [2.22]. Hence, the need to consider epithermal neutrons is obviated in facilities capable of producing a clean thermal neutron beam.

2.2.5. Westcott g factors

The effective cross-section in Eq. (2.3) is equal to the 2200 m/s cross-section σ_0 for a perfect $1/v$ absorber or even a realistic $1/v$ absorber nuclide irradiated in neutron fields with a negligible epithermal neutron fraction in the resonance region of the nuclide. When the nuclide is a non- $1/v$ absorber (^{113}Cd , ^{124}Xe , ^{149}Sm , most Eu isotopes, $^{155,157}\text{Gd}$, $^{175,176}\text{Lu}$, ^{180}Ta , etc.) or the neutron spectrum contains a significant epithermal component, the effective cross-section is no longer equal to σ_0 .

Westcott approached this problem for the case of a Maxwellian thermal spectrum and a $1/E$ epithermal spectrum [2.19]. Adopting the Westcott convention, the effective cross-section is given by:

$$\hat{\sigma} = \sigma_0(g_w + rs) \quad (2.11)$$

where g_w is the Westcott g factor, r is an index for epithermal fraction in the neutron density and s is a parameter related to the reduced resonance integral. The parameter r for $1/E$ epithermal neutrons can be obtained by measuring the

cadmium ratio with a thin $1/v$ detector or an activation foil [2.19].

Since the Maxwellian shape depends on the temperature, both g_w and s are dependent on the Maxwellian temperature. Hence, the Westcott g factor is given by the ratio of the effective cross-section for a pure Maxwellian spectrum ($\hat{\sigma}_M$) to the 2200 m/s cross-section:

$$\begin{aligned} g_w(T) &= \frac{\hat{\sigma}_M(T)}{\sigma_0} = \frac{1}{\sigma_0 v_0} \int_0^\infty \rho_M(v, T) \sigma_\gamma(v) v dv \\ &= \frac{1}{\sigma_0 v_0} \int_0^\infty \frac{4}{\sqrt{\pi}} \left(\frac{v}{v_T} \right)^3 e^{-(v/v_T)^2} \sigma_\gamma(v) dv \end{aligned} \quad (2.12)$$

where v_T is the most probable speed of the Maxwellian function, which is related to the temperature (T) by $mv_T^2/2 = kT$ or $v_T = v_0(T/T_0)^{1/2}$.

The latest published values of the Westcott g factors are given by Holden [2.23] for nuclides with Westcott g factors that deviate significantly from unity and for temperatures between 0 and 400°C. A series of new g factor calculations has been carried out for this CRP using the capture cross-sections from the EAF-99 library [2.24] over an extended temperature range of 20–600 K. Almost all isotopes up to ^{257}Fm have been considered in these calculations. Two sets of calculated data have been generated using different codes:

- (1) The Evaluated Nuclear Data File (ENDF) utility code INTER was used to generate the Westcott g factors by direct integration.
- (2) A new code GRUPINT was developed to deal with the general neutron spectrum (e.g. a sum of Maxwellian functions of different temperatures, which is typically adopted to describe the spectrum of a guided neutron beam). Instead of using direct integration, GRUPINT reads in fine group cross-sections in a 685-group structure, and calculates the Westcott g factors by group condensation.

GRUPINT was validated by comparing the results from both codes for a pure Maxwellian spectrum. The g factors agree to within considerably less than 1% for all the isotopes considered, although a few exceptional cases are noted:

- (a) Terbium-153 exhibits an anomalous jump in the tabulated cross-sections at the thermal energy, although the overall trend is $1/v$. The

INTER result reflects the anomalous behaviour, and the final GRUPINT g value is produced assuming a smooth $1/v$ shape.

- (b) The $^{187}\text{Re}(n, \gamma)$ reaction has different shapes for the cross-sections of the final activation products ^{188}Re (ground state) and $^{188\text{m}}\text{Re}$, in which only the excitation cross-section for the ground state exhibits a non- $1/v$ behaviour. Even although the reasons for such cross-sectional behaviour need closer investigation,

this example indicates that explicit consideration of cross-sections for the final production state could be important, depending on the nature of activation detection.

The Westcott g factors are listed in Tables 2.1–2.3 for those stable isotopes in which they deviate from unity by more than 1% at some temperature in the range specified.

TABLE 2.1. WESTCOTT g FACTORS FOR ELEMENTS WITH NUCLEON NUMBER $A \leq 143$

T (K)	E (eV)	^{30}Si	^{36}S	^{36}Ar	^{38}Ar	^{83}Kr	^{87}Sr	^{103}Rh	^{105}Pd	^{109}Ag	^{111}Cd
20	0.0017	1.000	0.799	1.135	1.266	1.011	0.990	0.964	1.008	0.991	1.009
40	0.0034	1.000	0.842	1.104	1.242	1.010	0.991	0.968	1.008	0.992	1.008
60	0.0052	1.000	0.871	1.078	1.197	1.009	0.992	0.972	1.007	0.993	1.008
80	0.0069	1.000	0.894	1.060	1.161	1.008	0.994	0.976	1.006	0.994	1.006
100	0.0086	1.000	0.912	1.049	1.133	1.006	0.995	0.981	1.005	0.995	1.005
120	0.0103	1.001	0.928	1.040	1.111	1.005	0.996	0.985	1.004	0.996	1.004
140	0.0121	1.001	0.942	1.035	1.095	1.004	0.997	0.989	1.003	0.997	1.003
160	0.0138	1.003	0.954	1.030	1.082	1.003	0.998	0.993	1.002	0.998	1.002
180	0.0155	1.003	0.965	1.026	1.072	1.001	0.999	0.998	1.001	0.999	1.001
200	0.0172	1.003	0.975	1.023	1.064	1.000	1.000	1.002	0.999	1.000	0.999
220	0.0190	1.004	0.984	1.021	1.057	0.999	1.001	1.007	0.999	1.001	0.999
240	0.0207	1.005	0.993	1.020	1.051	0.998	1.003	1.011	0.998	1.003	0.998
260	0.0224	1.006	1.001	1.018	1.046	0.996	1.004	1.015	0.997	1.003	0.996
280	0.0241	1.007	1.009	1.016	1.043	0.996	1.005	1.020	0.996	1.005	0.996
293	0.0253	1.007	1.014	1.016	1.040	0.995	1.006	1.023	0.995	1.005	0.995
300	0.0258	1.007	1.017	1.016	1.039	0.994	1.006	1.025	0.995	1.005	0.994
320	0.0276	1.008	1.023	1.015	1.036	0.993	1.007	1.029	0.994	1.006	0.993
340	0.0293	1.008	1.030	1.014	1.033	0.992	1.008	1.034	0.993	1.007	0.992
360	0.0310	1.009	1.036	1.013	1.031	0.991	1.010	1.039	0.992	1.008	0.991
380	0.0327	1.009	1.042	1.012	1.029	0.989	1.011	1.044	0.991	1.009	0.990
400	0.0345	1.010	1.047	1.012	1.027	0.988	1.012	1.048	0.990	1.010	0.989
420	0.0362	1.010	1.053	1.011	1.025	0.987	1.013	1.053	0.989	1.011	0.988
440	0.0379	1.011	1.058	1.011	1.024	0.986	1.014	1.059	0.988	1.012	0.987
460	0.0396	1.012	1.063	1.010	1.023	0.985	1.015	1.064	0.987	1.013	0.986
480	0.0414	1.012	1.068	1.010	1.021	0.984	1.017	1.069	0.986	1.015	0.985
500	0.0431	1.013	1.072	1.010	1.020	0.982	1.018	1.074	0.985	1.015	0.984
520	0.0448	1.013	1.077	1.010	1.019	0.981	1.019	1.079	0.984	1.017	0.983
540	0.0465	1.014	1.081	1.010	1.018	0.980	1.020	1.085	0.983	1.018	0.982
560	0.0482	1.014	1.086	1.009	1.018	0.979	1.022	1.090	0.983	1.019	0.980
580	0.0500	1.015	1.090	1.009	1.017	0.978	1.023	1.096	0.982	1.020	0.979
600	0.0517	1.015	1.094	1.009	1.016	0.976	1.024	1.101	0.981	1.021	0.979

2.2. DEFINITIONS AND NOMENCLATURE

TABLE 2.1. WESTCOTT g FACTORS FOR ELEMENTS WITH NUCLEON NUMBER $A \leq 143$ (cont.)

T (K)	E (eV)	^{113}Cd	^{113}In	^{115}In	^{121}Sb	^{123}Te	^{124}Xe	^{132}Ba	^{133}Cs	^{138}Ce	^{143}Nd
20	0.0017	0.780	0.979	0.969	0.994	0.980	0.994	1.000	0.995	0.936	1.007
40	0.0034	0.802	0.982	0.973	0.995	0.983	0.994	1.000	0.996	0.952	1.006
60	0.0052	0.826	0.984	0.976	0.995	0.985	0.995	1.000	0.997	0.962	1.005
80	0.0069	0.852	0.986	0.979	0.996	0.987	0.996	0.999	0.997	0.969	1.005
100	0.0086	0.880	0.988	0.984	0.997	0.989	0.997	0.998	0.998	0.974	1.004
120	0.0103	0.911	0.991	0.987	0.997	0.992	0.997	0.997	0.998	0.978	1.003
140	0.0121	0.945	0.993	0.990	0.998	0.994	0.999	0.995	0.999	0.981	1.002
160	0.0138	0.982	0.996	0.994	0.999	0.996	0.999	0.993	0.999	0.983	1.002
180	0.0155	1.023	0.998	0.998	0.999	0.998	1.000	0.991	1.000	0.985	1.001
200	0.0172	1.068	1.000	1.002	1.000	1.000	1.000	0.989	1.000	0.986	1.000
220	0.0190	1.118	1.003	1.005	1.001	1.003	1.001	0.987	1.001	0.988	0.999
240	0.0207	1.173	1.005	1.009	1.002	1.005	1.003	0.984	1.001	0.989	0.998
260	0.0224	1.231	1.008	1.012	1.002	1.008	1.003	0.983	1.002	0.990	0.997
280	0.0241	1.294	1.010	1.016	1.003	1.010	1.004	0.980	1.002	0.991	0.997
293	0.0253	1.337	1.012	1.019	1.003	1.011	1.004	0.979	1.002	0.991	0.996
300	0.0258	1.361	1.013	1.021	1.003	1.013	1.004	0.979	1.003	0.992	0.996
320	0.0276	1.429	1.015	1.025	1.004	1.015	1.005	0.977	1.003	0.992	0.995
340	0.0293	1.501	1.018	1.028	1.005	1.017	1.006	0.975	1.004	0.993	0.994
360	0.0310	1.575	1.021	1.033	1.005	1.019	1.007	0.973	1.004	0.993	0.994
380	0.0327	1.649	1.023	1.037	1.006	1.022	1.008	0.971	1.005	0.994	0.993
400	0.0345	1.724	1.026	1.041	1.007	1.024	1.008	0.969	1.005	0.994	0.992
420	0.0362	1.799	1.029	1.045	1.007	1.027	1.009	0.967	1.006	0.995	0.991
440	0.0379	1.873	1.031	1.049	1.008	1.029	1.010	0.966	1.006	0.995	0.990
460	0.0396	1.947	1.034	1.053	1.009	1.031	1.011	0.964	1.007	0.995	0.990
480	0.0414	2.018	1.037	1.057	1.009	1.034	1.011	0.962	1.007	0.996	0.989
500	0.0431	2.088	1.040	1.062	1.010	1.036	1.012	0.961	1.008	0.996	0.988
520	0.0448	2.158	1.042	1.066	1.011	1.039	1.013	0.960	1.008	0.996	0.987
540	0.0465	2.223	1.045	1.071	1.011	1.041	1.014	0.958	1.009	0.996	0.987
560	0.0482	2.287	1.048	1.075	1.012	1.044	1.015	0.957	1.009	0.997	0.986
580	0.0500	2.349	1.051	1.080	1.013	1.047	1.015	0.955	1.010	0.997	0.985
600	0.0517	2.408	1.054	1.084	1.013	1.049	1.016	0.954	1.010	0.997	0.985

CHAPTER 2. NOMENCLATURE AND FORMALISM

TABLE 2.2. WESTCOTT g FACTORS FOR ELEMENTS WITH $149 \leq A \leq 176$

T (K)	E (eV)	^{149}Sm	^{152}Sm	^{151}Eu	^{153}Eu	^{155}Gd	^{157}Gd	^{156}Dy	^{158}Dy	^{160}Dy	^{161}Dy
20	0.0017	0.622	0.994	1.273	1.088	0.838	0.794	0.986	1.021	0.985	1.016
40	0.0034	0.656	0.995	1.251	1.078	0.865	0.824	0.988	1.019	0.987	1.014
60	0.0052	0.696	0.995	1.223	1.068	0.887	0.850	0.990	1.017	0.988	1.013
80	0.0069	0.743	0.996	1.193	1.057	0.904	0.871	0.992	1.015	0.990	1.011
100	0.0086	0.800	0.997	1.161	1.048	0.914	0.887	0.993	1.012	0.992	1.009
120	0.0103	0.867	0.997	1.129	1.038	0.919	0.898	0.994	1.010	0.994	1.007
140	0.0121	0.947	0.998	1.097	1.029	0.920	0.904	0.996	1.007	0.995	1.005
160	0.0138	1.036	0.999	1.067	1.020	0.918	0.905	0.997	1.005	0.997	1.003
180	0.0155	1.135	0.999	1.038	1.012	0.911	0.904	0.999	1.002	0.999	1.001
200	0.0172	1.239	1.000	1.010	1.003	0.903	0.899	1.001	1.000	1.000	0.999
220	0.0190	1.345	1.001	0.984	0.994	0.892	0.891	1.002	0.998	1.002	0.998
240	0.0207	1.452	1.002	0.959	0.986	0.880	0.882	1.004	0.995	1.004	0.996
260	0.0224	1.556	1.002	0.936	0.979	0.867	0.872	1.006	0.993	1.006	0.994
280	0.0241	1.656	1.003	0.914	0.971	0.853	0.860	1.008	0.991	1.008	0.992
293	0.0253	1.718	1.003	0.900	0.966	0.843	0.852	1.009	0.989	1.009	0.991
300	0.0258	1.749	1.003	0.893	0.963	0.838	0.847	1.009	0.988	1.009	0.991
320	0.0276	1.838	1.004	0.874	0.956	0.823	0.834	1.011	0.986	1.011	0.989
340	0.0293	1.918	1.005	0.856	0.949	0.808	0.821	1.013	0.984	1.013	0.987
360	0.0310	1.992	1.005	0.840	0.942	0.793	0.807	1.014	0.982	1.015	0.985
380	0.0327	2.058	1.006	0.825	0.935	0.778	0.793	1.016	0.979	1.016	0.984
400	0.0345	2.119	1.007	0.811	0.928	0.763	0.779	1.018	0.977	1.018	0.982
420	0.0362	2.172	1.007	0.799	0.922	0.749	0.765	1.019	0.975	1.020	0.980
440	0.0379	2.219	1.008	0.787	0.916	0.734	0.751	1.021	0.973	1.022	0.979
460	0.0396	2.260	1.009	0.777	0.910	0.720	0.737	1.023	0.971	1.024	0.977
480	0.0414	2.294	1.009	0.769	0.903	0.706	0.723	1.025	0.969	1.026	0.975
500	0.0431	2.325	1.010	0.761	0.897	0.692	0.710	1.026	0.966	1.028	0.974
520	0.0448	2.349	1.011	0.755	0.892	0.678	0.697	1.028	0.964	1.030	0.972
540	0.0465	2.370	1.011	0.750	0.886	0.665	0.684	1.030	0.962	1.031	0.970
560	0.0482	2.387	1.012	0.746	0.880	0.653	0.671	1.032	0.960	1.033	0.969
580	0.0500	2.400	1.013	0.744	0.875	0.640	0.659	1.033	0.958	1.035	0.967
600	0.0517	2.409	1.013	0.743	0.870	0.628	0.647	1.036	0.956	1.037	0.965

2.2. DEFINITIONS AND NOMENCLATURE

TABLE 2.2. WESTCOTT g FACTORS FOR ELEMENTS WITH $149 \leq A \leq 176$ (cont.)

T (K)	E (eV)	^{162}Dy	^{163}Dy	^{164}Dy	^{167}Er	^{169}Tm	^{168}Yb	^{174}Hf	^{176}Hf	^{175}Lu	^{176}Lu
20	0.0017	0.991	1.003	1.023	0.917	0.992	0.925	1.028	0.995	1.065	0.716
40	0.0034	0.993	1.002	1.021	0.926	0.993	0.933	1.025	0.996	1.057	0.744
60	0.0052	0.993	1.002	1.018	0.936	0.994	0.942	1.022	0.996	1.050	0.774
80	0.0069	0.994	1.001	1.015	0.945	0.995	0.951	1.019	0.997	1.042	0.808
100	0.0086	0.995	1.002	1.013	0.955	0.996	0.960	1.016	0.998	1.035	0.847
120	0.0103	0.996	1.001	1.010	0.965	0.997	0.969	1.012	0.998	1.028	0.892
140	0.0121	0.997	1.001	1.008	0.975	0.998	0.978	1.010	0.999	1.021	0.945
160	0.0138	0.998	1.001	1.005	0.986	0.999	0.987	1.006	0.999	1.015	1.010
180	0.0155	0.999	1.001	1.002	0.998	1.000	0.997	1.003	1.000	1.008	1.086
200	0.0172	1.000	1.001	0.999	1.008	1.001	1.007	1.000	1.000	1.003	1.176
220	0.0190	1.001	1.001	0.997	1.020	1.001	1.017	0.997	1.001	0.996	1.280
240	0.0207	1.002	1.002	0.994	1.033	1.003	1.028	0.994	1.001	0.991	1.395
260	0.0224	1.003	1.002	0.992	1.046	1.004	1.039	0.992	1.002	0.985	1.523
280	0.0241	1.004	1.003	0.989	1.059	1.005	1.050	0.988	1.002	0.980	1.658
293	0.0253	1.005	1.003	0.988	1.069	1.005	1.057	0.986	1.002	0.976	1.752
300	0.0258	1.005	1.003	0.987	1.073	1.005	1.061	0.985	1.003	0.975	1.802
320	0.0276	1.006	1.003	0.984	1.089	1.007	1.073	0.983	1.003	0.969	1.949
340	0.0293	1.007	1.004	0.982	1.104	1.008	1.086	0.980	1.004	0.964	2.099
360	0.0310	1.008	1.004	0.979	1.120	1.008	1.098	0.977	1.004	0.960	2.250
380	0.0327	1.009	1.005	0.976	1.138	1.010	1.111	0.974	1.005	0.955	2.399
400	0.0345	1.010	1.006	0.974	1.157	1.010	1.125	0.971	1.005	0.950	2.545
420	0.0362	1.011	1.006	0.972	1.177	1.012	1.139	0.968	1.006	0.946	2.688
440	0.0379	1.012	1.007	0.969	1.199	1.013	1.154	0.965	1.006	0.941	2.826
460	0.0396	1.013	1.008	0.967	1.222	1.013	1.170	0.963	1.007	0.937	2.959
480	0.0414	1.014	1.009	0.964	1.248	1.015	1.187	0.960	1.007	0.933	3.085
500	0.0431	1.015	1.010	0.962	1.276	1.016	1.204	0.957	1.008	0.929	3.205
520	0.0448	1.016	1.011	0.960	1.306	1.017	1.222	0.955	1.008	0.925	3.318
540	0.0465	1.017	1.012	0.957	1.339	1.018	1.242	0.952	1.009	0.921	3.424
560	0.0482	1.018	1.013	0.955	1.375	1.019	1.262	0.949	1.010	0.917	3.524
580	0.0500	1.019	1.014	0.952	1.415	1.020	1.283	0.947	1.010	0.914	3.618
600	0.0517	1.020	1.015	0.950	1.458	1.021	1.306	0.944	1.011	0.910	3.704

TABLE 2.3. WESTCOTT g FACTORS FOR ELEMENTS WITH NUCLEON NUMBER $A \geq 177$

T (K)	E (eV)	^{177}Hf	^{178}Hf	^{179}Hf	^{180}Hf	^{180}Ta	^{181}Ta	^{180}W	^{182}W	^{185}Re	^{187}Re
20	0.0017	0.969	0.994	1.006	1.005	0.831	0.993	1.006	0.995	0.991	1.046
40	0.0034	0.973	0.995	1.005	1.005	0.850	0.994	1.005	0.995	0.991	1.040
60	0.0052	0.976	0.996	1.005	1.004	0.869	0.995	1.005	0.996	0.992	1.035
80	0.0069	0.979	0.996	1.004	1.003	0.889	0.996	1.004	0.997	0.993	1.030
100	0.0086	0.983	0.997	1.003	1.003	0.911	0.996	1.003	0.997	0.994	1.025
120	0.0103	0.987	0.997	1.003	1.003	0.935	0.997	1.003	0.997	0.995	1.020
140	0.0121	0.990	0.998	1.002	1.002	0.962	0.998	1.002	0.999	0.996	1.015
160	0.0138	0.994	0.999	1.001	1.001	0.991	0.999	1.002	0.999	0.997	1.011
180	0.0155	0.998	1.000	1.001	1.001	1.026	0.999	1.001	1.000	0.998	1.006
200	0.0172	1.002	1.000	1.000	1.000	1.065	1.000	1.000	1.000	0.999	1.002
220	0.0190	1.006	1.001	0.999	0.999	1.111	1.001	1.000	1.001	1.000	0.997
240	0.0207	1.010	1.002	0.999	0.999	1.166	1.002	0.999	1.002	1.001	0.993
260	0.0224	1.013	1.002	0.998	0.998	1.230	1.002	0.998	1.002	1.002	0.989
280	0.0241	1.017	1.003	0.997	0.997	1.304	1.003	0.998	1.003	1.004	0.985
293	0.0253	1.020	1.003	0.997	0.997	1.358	1.004	0.997	1.003	1.004	0.982
300	0.0258	1.021	1.003	0.996	0.997	1.389	1.004	0.997	1.003	1.004	0.981
320	0.0276	1.025	1.004	0.996	0.996	1.484	1.005	0.996	1.004	1.005	0.977
340	0.0293	1.029	1.005	0.995	0.995	1.589	1.005	0.996	1.004	1.007	0.973
360	0.0310	1.033	1.005	0.994	0.995	1.704	1.006	0.995	1.005	1.008	0.970
380	0.0327	1.038	1.006	0.994	0.994	1.829	1.007	0.994	1.005	1.009	0.966
400	0.0345	1.042	1.007	0.993	0.993	1.961	1.008	0.994	1.006	1.010	0.962
420	0.0362	1.046	1.007	0.992	0.993	2.101	1.008	0.993	1.007	1.011	0.959
440	0.0379	1.051	1.008	0.992	0.992	2.247	1.009	0.993	1.007	1.012	0.956
460	0.0396	1.055	1.008	0.991	0.992	2.398	1.010	0.992	1.008	1.013	0.952
480	0.0414	1.059	1.009	0.990	0.991	2.554	1.010	0.991	1.009	1.015	0.949
500	0.0431	1.064	1.010	0.990	0.990	2.713	1.011	0.991	1.009	1.016	0.946
520	0.0448	1.069	1.010	0.989	0.990	2.874	1.012	0.990	1.010	1.017	0.942
540	0.0465	1.073	1.011	0.988	0.989	3.039	1.013	0.989	1.010	1.018	0.939
560	0.0482	1.078	1.012	0.988	0.989	3.204	1.014	0.989	1.011	1.019	0.936
580	0.0500	1.083	1.013	0.987	0.988	3.370	1.014	0.988	1.012	1.020	0.933
600	0.0517	1.088	1.013	0.987	0.988	3.536	1.015	0.988	1.012	1.022	0.930

2.3. GENERALIZED FORMALISM

2.3.1. Capture rates

The instantaneous neutron capture rate $dR(t)$ of a stable nuclide in a differential volume $d^3\mathbf{r}$ localized at a point \mathbf{r} of a sample in a neutron field is given by:

$$dR(t) = d^3\mathbf{r} n_x(\mathbf{r}) \int_0^\infty n(\mathbf{r}, v, t) \sigma_\gamma(v) v dv \quad (2.13)$$

where $n_x(\mathbf{r})$ is the capturing nuclide density in the sample target and $n(\mathbf{r}, v, t)$ is the neutron density per unit speed interval at location \mathbf{r} and time t . By preparing a target sample of homogeneous nuclide density, the time averaged capture rate by the given nuclide in the sample is given by [2.14]:

2.3. GENERALIZED FORMALISM

TABLE 2.3. WESTCOTT g FACTORS FOR ELEMENTS WITH NUCLEON NUMBER $A \geq 177$ (cont.)

T (K)	E (eV)	^{186}Os	^{187}Os	^{191}Ir	^{193}Ir	^{197}Au	^{196}Hg	^{199}Hg	^{232}Th	^{234}U	^{235}U
20	0.0017	1.005	1.035	1.018	0.973	0.991	1.023	1.021	1.008	1.019	1.173
40	0.0034	1.005	1.032	1.016	0.976	0.992	1.021	1.019	1.007	1.017	1.143
60	0.0052	1.004	1.027	1.014	0.979	0.993	1.018	1.016	1.006	1.015	1.119
80	0.0069	1.003	1.023	1.012	0.983	0.994	1.015	1.015	1.005	1.012	1.100
100	0.0086	1.003	1.020	1.010	0.985	0.995	1.013	1.012	1.005	1.010	1.083
120	0.0103	1.003	1.015	1.008	0.988	0.996	1.010	1.010	1.003	1.008	1.068
140	0.0121	1.002	1.012	1.006	0.992	0.997	1.008	1.007	1.003	1.006	1.054
160	0.0138	1.001	1.008	1.005	0.995	0.998	1.005	1.005	1.002	1.004	1.042
180	0.0155	1.001	1.004	1.003	0.998	0.999	1.002	1.002	1.001	1.001	1.031
200	0.0172	1.000	1.000	1.002	1.001	1.000	0.999	1.000	0.999	0.999	1.021
220	0.0190	1.000	0.996	1.001	1.005	1.001	0.997	0.997	0.999	0.998	1.012
240	0.0207	0.999	0.993	0.999	1.008	1.003	0.994	0.995	0.998	0.995	1.003
260	0.0224	0.998	0.989	0.998	1.011	1.003	0.992	0.993	0.997	0.993	0.995
280	0.0241	0.998	0.985	0.997	1.014	1.005	0.989	0.991	0.996	0.991	0.989
293	0.0253	0.998	0.983	0.996	1.017	1.005	0.988	0.989	0.995	0.990	0.985
300	0.0258	0.997	0.982	0.996	1.018	1.005	0.987	0.988	0.995	0.989	0.983
320	0.0276	0.997	0.978	0.995	1.022	1.006	0.984	0.986	0.994	0.987	0.977
340	0.0293	0.996	0.975	0.995	1.025	1.007	0.982	0.984	0.993	0.985	0.972
360	0.0310	0.996	0.971	0.994	1.029	1.008	0.979	0.981	0.992	0.983	0.967
380	0.0327	0.995	0.967	0.994	1.032	1.009	0.977	0.979	0.991	0.981	0.963
400	0.0345	0.994	0.964	0.994	1.036	1.010	0.974	0.977	0.990	0.979	0.960
420	0.0362	0.994	0.961	0.994	1.039	1.011	0.972	0.975	0.990	0.977	0.957
440	0.0379	0.993	0.957	0.994	1.043	1.012	0.969	0.973	0.989	0.975	0.954
460	0.0396	0.993	0.954	0.994	1.047	1.013	0.967	0.970	0.988	0.973	0.952
480	0.0414	0.992	0.950	0.994	1.051	1.014	0.965	0.968	0.987	0.972	0.950
500	0.0431	0.992	0.947	0.995	1.055	1.015	0.962	0.966	0.986	0.970	0.949
520	0.0448	0.991	0.944	0.996	1.059	1.016	0.960	0.964	0.985	0.968	0.948
540	0.0465	0.990	0.941	0.997	1.062	1.018	0.957	0.962	0.984	0.966	0.947
560	0.0482	0.990	0.937	0.998	1.066	1.018	0.955	0.960	0.983	0.964	0.946
580	0.0500	0.989	0.934	1.000	1.071	1.020	0.953	0.957	0.983	0.962	0.946
600	0.0517	0.989	0.931	1.001	1.075	1.021	0.951	0.955	0.982	0.960	0.946

$$\begin{aligned} \langle R \rangle &= \frac{1}{t_m} \int_0^{t_m} dt \int_V d^3r n_x(r) \int_0^\infty n(r, v, t) \sigma_\gamma(v) v dv \\ &= \frac{1}{V M} N_A \theta \int_V d^3r \int_0^\infty n(r, v) \sigma_\gamma(v) v dv \end{aligned} \quad (2.14)$$

where t_m is the irradiation period, V is the volume of the sample, m is the mass of the relevant element in the target, M is the atomic mass of the element, N_A is Avogadro's number, θ is the abundance of the

capturing isotope in the element and $n(r, v)$ is the time averaged neutron density per unit speed interval at location r given by:

$$n(r, v) = \frac{1}{t_m} \int_0^{t_m} dt n(r, v, t) \quad (2.15)$$

The expressions are greatly simplified for $1/v$ absorbers. Using the relationship $\sigma(v) = \sigma_0 v_0/v$, the

capture rate in Eq. (2.14) becomes proportional to the total neutron density in the sample, and is given by:

$$\begin{aligned}\langle R \rangle_{1/v} &= \frac{1}{V M} N_A \theta \int_V d^3r \int_0^\infty n(\mathbf{r}, v) \sigma_\gamma(v) v dv \\ &= \frac{m}{M} N_A \theta \sigma_0 v_0 \bar{n}_t\end{aligned}\quad (2.16)$$

where \bar{n}_t is the total neutron density averaged over volume in the sample. The result is exact even when the spectrum in the sample is distorted or the neutron beam profile is inhomogeneous. Thus, for an approximate $1/v$ absorber nuclide over the neutron spectral range, Eq. (2.16) is valid to a reasonable degree. Hence, for a PGAA facility in which the neutron beam is free from an epithermal component, no detailed information about the incident beam spectrum nor the spectrum inside the sample is required for $1/v$ absorbers as far as k_0 standardization is concerned.

Capture rates of realistic nuclides with resonances in the epithermal region are composed of contributions by thermal and epithermal neutrons within the sample. This problem has been addressed in numerous INAA studies, in which the underlying assumptions are that the thermal neutron spectrum is Maxwellian and the epithermal flux is characterized by $1/E$ or $1/E^{1+\alpha}$. Since the beam spectrum in PGAA is closely described by a Maxwellian with or without a significant $1/E$ epithermal flux contribution, the existing formalism in INAA is judged to be equally applicable [2.25].

2.3.2. Non- $1/v$ absorbers, effective g factors and cadmium ratios

The capture rate for a non- $1/v$ absorber has been quantified in terms of the Westcott g factor. As the g factor is defined for a Maxwellian thermal spectrum, there arises the problem of treating realistic neutron spectra, which may deviate significantly from the Maxwellian shape in the thermal energy region. Measured time of flight (TOF) spectra for supermirror guided cold beams exhibit large deviations of this kind, which are difficult to parameterize [2.26]. The curved mirror guided thermal beam also has spatial inhomogeneity and results in deviations with respect to spectral correlations as a function of position along the curvature of

the mirror [2.27]. Furthermore, the thermal spectrum deviates from Maxwellian in filtered beam facilities [2.28], where the spectrum form is distinctly non-Maxwellian [2.12, 2.29]. As the capture rate for a non- $1/v$ absorber is highly dependent on the shape of the thermal and epithermal spectra, a generalized approach is described in terms of an effective g factor.

Even when the neutron spectrum is correlated with the neutron density in the sample, the reduction of the capture rate to measurable quantities is possible for a $1/v$ absorber. However, this correlation becomes more complex for a non- $1/v$ absorber because the strong capture process causes spectral hardening at low energies and from self-shielding around the resonances. A thin sample with infinite (or sufficiently realistic) dilution of strong absorber nuclides is an important requirement for ensuring that the neutron spectrum within the sample does not change compared with that of the incident beam. When the neutron density of the incident beam can be separated ($n(\mathbf{r}, v) = n(\mathbf{r})\rho(v)$), this same separation process is valid for dilution of thin samples and simplifies theoretical considerations.

If the thermal spectrum deviates significantly from Maxwellian, the Høgdahl convention can be used to classify the thermal and epithermal neutrons in terms of cadmium cut-off [2.30], and the neutron density separates into two terms:

$$\begin{aligned}n(\mathbf{r}, v) &= n(\mathbf{r})\rho(v) = n_{th}(\mathbf{r})\rho_{th}(v)\Theta(v_{Cd} - v) \\ &\quad + n_{ep}(\mathbf{r})\rho_{ep}(v)\Theta(v - v_{Cd})\end{aligned}\quad (2.17)$$

where $n_{th}(\mathbf{r})$ and $n_{ep}(\mathbf{r})$ are the local thermal and epithermal neutron densities, respectively, $\Theta(x)$ is the step function, which is unity for non-negative arguments x and zero otherwise, and v_{Cd} is the neutron speed corresponding to the cadmium cut-off energy $E_{Cd} \approx 0.5$ eV (and $mv_{Cd}^2/2 \equiv E_{Cd}$). The speed distribution functions $\rho(v)$, $\rho_{th}(v)$ and $\rho_{ep}(v)$ are normalized so that:

$$\int_0^\infty \rho(v)dv = \int_0^{v_{Cd}} \rho_{th}(v)dv = \int_{v_{Cd}}^\infty \rho_{ep}(v)dv = 1\quad (2.18)$$

Hence, the capture rate is given by:

2.3. GENERALIZED FORMALISM

$$\begin{aligned} \langle R \rangle_{\text{non-}1/v} &= \frac{1}{V M} N_A \theta \int_V d^3 r n(r) \int_0^\infty \rho(v) \sigma_\gamma(v) v dv \\ &= \frac{m}{M} N_A \theta \left(\bar{n}_{\text{th}} \int_0^{v_{\text{Cd}}} \rho_{\text{th}}(v) \sigma_\gamma(v) v dv \right. \\ &\quad \left. + \bar{n}_{\text{ep}} \int_{v_{\text{Cd}}}^\infty \rho_{\text{ep}}(v) \sigma_\gamma(v) v dv \right) \end{aligned} \quad (2.19)$$

where \bar{n}_{th} and \bar{n}_{ep} are the volume averaged thermal and epithermal neutron densities in the sample, respectively. A general beam spectrum can be considered by including the epithermal capture rate in parallel.

Accordingly, an effective g factor is defined in Ref. [2.31]:

$$\begin{aligned} \hat{g} &\equiv \frac{1}{\sigma_0 v_0} \frac{\int_0^{v_{\text{Cd}}} \rho_{\text{th}}(v) \sigma_\gamma(v) v dv}{\int_0^{v_{\text{Cd}}} \rho_{\text{th}}(v) dv} \\ &= \frac{1}{\sigma_0 v_0} \int_0^{v_{\text{Cd}}} \rho_{\text{th}}(v) \sigma_\gamma(v) v dv \end{aligned} \quad (2.20)$$

for a realistic thermal neutron spectrum $\rho_{\text{th}}(v)$ of the incident beam. Therefore, the effective g factor for a given non- $1/v$ absorber nuclide is specific to a particular PGAA beam facility, and is unity for an exact $1/v$ absorber, regardless of the spectral shape. If resonances are present above E_{Cd} and if the epithermal neutron contribution to the reaction rates is not negligible, the definition of the effective g factor is still valid, but the second integral in Eq. (2.19) must be accounted for explicitly. Procedures developed for INAA can be applied. In general, the effective g factor depends on E_{Cd} , but this dependence is usually weak, except for a few nuclides (e.g. ^{176}Lu , ^{151}Eu and ^{115}In) with strong resonances near this energy.

If detailed information about the neutron spectral shape is available, the effective g factors can be calculated from the pointwise capture cross-sections (e.g. the JEF-2.2 data set [2.32]). However, there are additional complications that may arise when a cold beam is incident on the target at room temperature. The neutron energy gain by upscattering in the target can lead to spectral distortion, which is difficult to predict and complicates the interpretation of measurements of non- $1/v$ absorbers [2.33].

Effective g factors for a particular PGAA facility can be determined by measuring the k_0 factors (described in Section 2.2.4) and comparing them with reference values from the literature. According to Eq. (2.1), k_0 factors are composite nuclear constants independent of the facility. Therefore, if the k_0 value is known, it is possible to determine the ratio of the effective g factor of the measured nuclide and the comparator, which is normally a $1/v$ absorber with a g factor equal to one.

The epithermal contribution to the capture rate of a nuclide can be estimated from the measured cadmium ratio (R_{Cd}), which is the ratio of the specific activity of this nuclide in a sample irradiated without a cadmium cover to that of one with a cadmium cover. Activity is proportional to the reaction rate, which can be calculated by defining the cadmium transmission function, assuming exponential neutron attenuation through the cadmium cover, as:

$$t(v) = \exp[-dn_{\text{Cd}} \sigma_{\text{Cd}}(v)] \quad (2.21)$$

where d is the cadmium cover thickness, n_{Cd} is the cadmium number density and σ_{Cd} is the cadmium cross-section. The cadmium ratio is given by:

$$R_{\text{Cd}} = \frac{\bar{n} \int_0^\infty \rho(v) \sigma_\gamma(v) v dv}{\bar{n} \int_0^\infty t(v) \rho(v) \sigma_\gamma(v) v dv} \quad (2.22)$$

Owing to the nature of the cadmium cross-section, the transmission function is close to unity above the cadmium resonance at about 0.5 eV and nearly zero below. This parameter can be approximated by an idealized Heaviside function, with a step from zero to one at speed v_{Cd} , to give a greatly simplified expression for the cadmium ratio:

$$\begin{aligned} R_{\text{Cd}} &= \frac{\left(\bar{n}_{\text{th}} \int_0^{v_{\text{Cd}}} \rho_{\text{th}}(v) \sigma_\gamma(v) v dv + \bar{n}_{\text{ep}} \int_{v_{\text{Cd}}}^\infty \rho_{\text{ep}}(v) \sigma_\gamma(v) v dv \right)}{\bar{n}_{\text{ep}} \int_{v_{\text{Cd}}}^\infty \rho_{\text{ep}}(v) \sigma_\gamma(v) v dv} \\ &= 1 + \frac{\bar{n}_{\text{th}} v_0 \hat{g} \sigma_0}{\bar{n}_{\text{ep}} \int_{v_{\text{Cd}}}^\infty \rho_{\text{ep}}(v) \sigma_\gamma(v) v dv} \end{aligned} \quad (2.23)$$

and the capture rate is given by:

$$\langle R \rangle_{\text{non}-1/v} = \frac{m}{M} N_A \theta \bar{n}_{\text{th}} v_0 \hat{g} \sigma_0 \left(\frac{R_{\text{Cd}}}{R_{\text{Cd}} - 1} \right) \quad (2.24)$$

which is a generalized expression for Eq. (2.16). By comparing Eqs (2.22) and (2.23), an effective cadmium cut-off speed (v_{Cd}) can be determined that depends mainly on the thickness of the cadmium cover. The dependence on the shape of the cross-section is weak, except for nuclides with resonances near the cadmium cut-off speed. Cadmium cut-off energies have been determined for various cadmium thicknesses, epithermal neutron components and beam geometries that are applicable to Maxwellian thermal spectra and $1/E$ epithermal spectra above $\approx 5kT$ [2.19, 2.20, 2.34].

When the ratio for cadmium is too large to obtain a statistically meaningful gamma count rate, the terms in Eq. (2.24) that involve the cadmium ratio are not required. The estimated lower limit of the cadmium ratio can be used to assign the error arising from the epithermal neutron contribution.

2.3.3. Prompt capture gamma counting rates

The measured count rate of a prompt gamma ray of energy E_γ emitted from a capturing nuclide is given by:

$$\langle C \rangle = \frac{1}{V} \frac{m}{M} N_A \theta \int_V d^3r \varepsilon(\mathbf{r}, E_\gamma) \times \int_0^\infty P(E_\gamma, v) n(\mathbf{r}, v) \sigma_\gamma(v) v dv \quad (2.25)$$

where $\varepsilon(\mathbf{r}, E_\gamma)$ is the detection efficiency for a prompt gamma ray of energy E_γ emitted at location \mathbf{r} and $P(E_\gamma, v)$ is the absolute gamma ray emission probability (number of gamma rays emitted per capture) of the prompt gamma ray of energy E_γ emitted from a nucleus capturing a neutron of speed v .

Using a small sample, the detection efficiency $\varepsilon(\mathbf{r}, E_\gamma)$ is assumed to have the same shape over the sample volume and is separable into $f(\mathbf{r})\varepsilon(E_\gamma)$, where $f(\mathbf{r})$ is a geometrical factor independent of the gamma ray energy, unless attenuated [2.14]. A high resolution gamma ray spectroscopy system is assumed for the detection, consisting of a semiconductor detector of single or Compton suppressed type and associated electronics. Typically, the sample should be as small as practicable (point

source) and located 15–20 cm or more from the detector so that the effects of the gradient of the detection efficiency through the sample are negligible [2.22]. Gamma ray attenuation within the sample is insignificant due to the small sample size and high prompt gamma ray energy (greater than 200 keV). Typical correction factors arise from sum coincidence, random coincidence and dead time losses, and are introduced during or after measurement. Typical corrections for saturation, cooling and decay before and during the counting period are not required.

The absolute gamma ray emission probability $P(E_\gamma, v)$ is dependent on the captured neutron speed (energy) [2.28]. This parameter is related to the partial capture cross-section and partial radiative width, which fluctuates from resonance to resonance (Porter–Thomas fluctuations [2.35]). Neutron capture models based on statistical theory [2.36] or simple direct (potential) capture [2.37–2.39] predict that the energy dependence for $P(E_\gamma, v)$ in the thermal region is negligible. However, the neutron energy dependence can only be appreciable when interference occurs [2.40, 2.41], either between different resonance amplitudes [2.42] or between resonance and direct capture amplitudes [2.43].

Such experimental studies are difficult to perform and are scarce, especially in the thermal and cold energy ranges. Some signatures have been determined for a few transitions from $^{238}\text{U}(n, \gamma)$ [2.44], $^{197}\text{Au}(n, \gamma)$ [2.45], $^{195}\text{Pt}(n, \gamma)$ [2.42], $^{169}\text{Tm}(n, \gamma)$ [2.46] and $^{149}\text{Sm}(n, \gamma)$ [2.47] resonances that influence the thermal region. Even though there is some experimental evidence and theoretical models that support the energy variation in P , quantitative prediction of this phenomenon requires further study beyond the scope of the current book.

For most nuclides, the slow neutron energy region (<0.1 eV) is far from the lowest positive energy resonance (see, e.g., Table 2.4 [2.48]), while the negative energy resonance is closest to the neutron threshold. Hence, the absolute gamma ray emission probability $P(E_\gamma)$ is assumed to be independent of the neutron energy for slow neutron capture. Data for absolute gamma ray emission probabilities are based on the incident neutron energy being thermal, as specified in the current PGAA database [2.49].

By combining Eqs (2.24) and (2.25), the specific count rate (the count rate per mass of element in the sample, or the so-called analytic sensitivity) is given by:

2.4. CONCLUDING REMARKS

TABLE 2.4. ENERGY (eV) ORDERED RESONANCES
(extracted from Appendix A of Ref. [2.48])

E_0	Isotope								
0.031	Gd-157	0.178	Am-242	0.307	Am-241	0.546	Ir-192	0.653	Ir-191
0.084	Xe-135	0.192	Eu-154	0.321	Eu-151	0.574	Am-241	0.702	Cf-249
0.097	Sm-149	0.195	Bk-249	0.400	Pa-231	0.584	Er-167	0.807	Yb-169
0.141	Lu-176	0.200	Ta-180	0.435	Ta-180	0.597	Yb-168	0.872	Sm-149
0.148	Ta-182	0.256	Ir-192	0.460	Eu-151	0.603	Eu-155	0.884	Eu-152
0.169	Pm-148	0.258	Pu-241	0.460	Er-167	0.609	Th-229	1.000	Cf-252
0.178	Cd-113	0.296	Pu-239	0.489	Np-237	0.615	Am-242	1.060	Pu-240

$$A = \left\langle \frac{C}{m} \right\rangle = \frac{N_A}{M} \theta P(E_\gamma) \epsilon(E_\gamma) \bar{n}_{\text{th}} v_0 \hat{g} \sigma_0 \left(\frac{R_{\text{Cd}}}{R_{\text{Cd}} - 1} \right) \quad (2.26)$$

2.3.4. Experimental k_0 factors

The same irradiation conditions for analyte (x) and comparator (c) elements are achieved by co-irradiating a homogeneous mixture of analyte and comparator element in a neutron field, and measuring the signature of prompt gamma rays in parallel. Hence, the experimental prompt k_0 factor is given from Eqs (2.1) and (2.26) by:

$$k_0 \equiv \frac{P_x(E_{\gamma,x}) \sigma_{0,x} \theta_x/M_x}{P_c(E_{\gamma,c}) \sigma_{0,c} \theta_c/M_c} = \frac{A_x/\epsilon(E_{\gamma,x}) \hat{g}_x \left(\frac{R_{\text{Cd}}}{R_{\text{Cd}} - 1} \right)_x}{A_c/\epsilon(E_{\gamma,c}) \hat{g}_c \left(\frac{R_{\text{Cd}}}{R_{\text{Cd}} - 1} \right)_c} \quad (2.27)$$

This general expression contains two correction factors: \hat{g} for non- $1/v$ absorption and R_{Cd} for epithermal absorption. Typical comparator elements hydrogen and chlorine are both good $1/v$ absorbers with effective g factors close to unity in most facilities. The last term in parentheses deviates from unity by about $(1/R_{\text{Cd}})_c - (1/R_{\text{Cd}})_x$ and is therefore closer to unity for a clean beam. Guided or filtered neutron beams result in conditions that do not require epithermal correction.

Accurately determined k_0 factors permit the generation of precisely measured data sets of partial cross-sections by normalization to the well defined comparator element hydrogen. Data sets of partial cross-sections are known to be considerably more precise than either the isotopic cross-section (σ_0) or the absolute gamma ray emission probability (P) [2.49]. Hence, by measuring the ratio of gamma ray emission rates for two selected elements and using the known k_0 factors, the concentration ratio of the two elements can be precisely determined. Furthermore, the absolute elemental concentrations can be obtained if all the elements in the sample are observed in the measured gamma ray spectrum (elemental analysis of a sample).

2.4. CONCLUDING REMARKS

Typical spectra of the neutron beams used for PGAA deviate appreciably from the ideal Maxwellian function. Although analysis in terms of k_0 standardization has been expanded to non- $1/v$ absorbers, the resulting deviation is neglected and the thermal spectrum has been approximated by the Maxwellian with or without a $1/E$ epithermal contribution so that developments in INAA apply. Since the majority of nuclides exhibit $1/v$ absorption in the thermal energy region and even the non- $1/v$ absorbers behave asymptotically as $1/v$ absorbers in the cold region (below 5 eV), the analytical solution is relatively simple in most cases. Quantification of the various effects becomes important as the accuracy in the measured k_0 factors is reported to be less than 3% (typically around 1%). Therefore, highly accurate PGAA requires well defined experimental conditions and procedures, along with the analytical data and the assumptions underlying the

final result. Applications of PGAA are very diverse in terms of the sample composition and size, neutron beam characteristics, analysis method and procedure, and therefore the validity and limitations of the present approach need to be considered in greater detail.

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Chapter 3

CHARACTERISTICS OF PGAA FACILITIES

H.D. Choi

3.1. THE SNU-KAERI PGAA FACILITY AND DIFFRACTED POLYCHROMATIC NEUTRON BEAMS

The SNU-KAERI PGAA facility was developed through the joint efforts of Seoul National University (SNU) and the Korea Atomic Energy Research Institute (KAERI), and has been operational since May 2001. A detailed layout of the facility is shown in Fig. 3.1. The PGAA system is installed on a platform located at the exit of the 4 m long ST1 tangential beam port of the Hanaro reactor [3.1]. Pyrolytic graphite (PG) crystals are used to extract the thermal beam by the method of Bragg diffraction, with the Bragg angle set at 45° so that most of the beam flux originates from diffraction orders two, three and four. The diffracted beam is diverted vertically to the first collimator positioned downstream from the PG crystals, and is controlled further by a second collimator made of ^6LiF positioned on the beam

shutter. The neutron flux and cadmium ratio for gold at the sample location are $7.9 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ and 266, respectively. A flux uniformity of within 12% is achieved in the central area of $1 \times 1 \text{ cm}^2$ of the total beam cross-section (of $2 \times 2 \text{ cm}^2$).

The neutron beam spectrum has been characterized both experimentally and theoretically [3.1, 3.2]. A TOF spectrometer was used to measure the spectrum of the diffracted polychromatic beam, as shown in Fig. 3.2. Bragg peaks up to the sixth order of diffraction are recognizable, and hence the measurement is only restricted in the thermal energy region. Higher order diffractions above sixth order and the epithermal region of the spectrum were obtained indirectly by comparing theoretical predictions with the measured effective cross-section for the $^{10}\text{B}(\text{n}, \alpha)$ reaction and cadmium ratios for various nuclides.

The theoretical diffracted beam spectrum was obtained from the reflectivity model of the PG crystal. Lattice vibration effects were included in

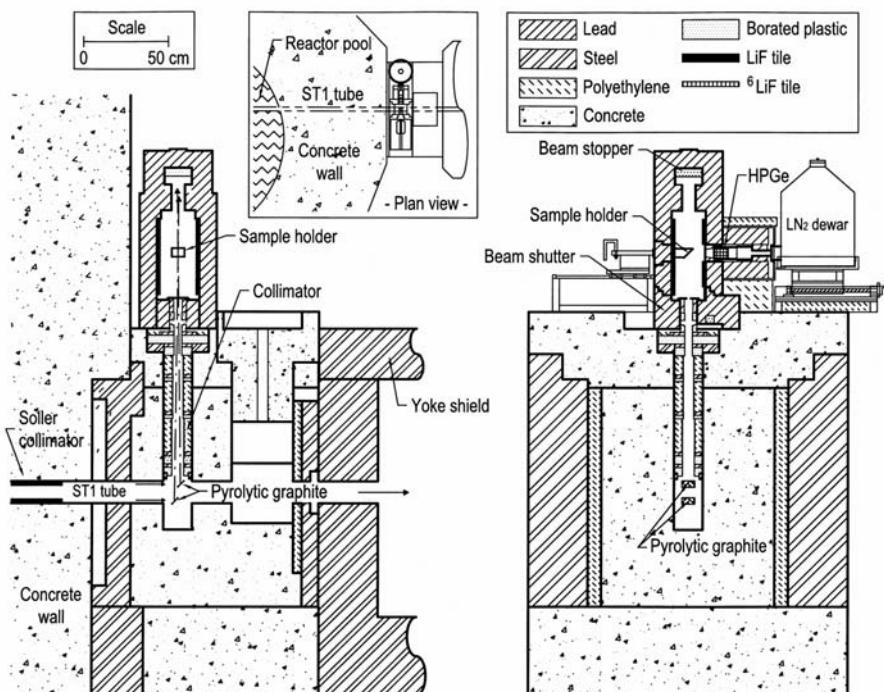


FIG. 3.1. The SNU-KAERI PGAA facility.

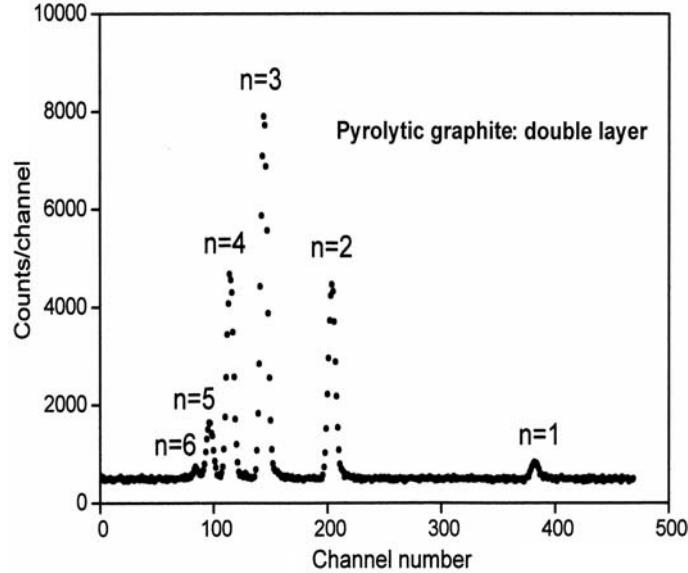


FIG. 3.2. A diffracted neutron TOF spectrum measured by double layered crystals set at a Bragg angle of 45°.

the calculation using the reported vibrational amplitude of the PG crystal and comparing with the measured TOF spectra in the thermal region [3.3]. A continuous spectrum of background neutrons was included as a minor component that originated mainly from the incoherent scattering by the structural materials of the PG crystal mount and goniometer. The calculated neutron spectrum up to 40 eV is shown in Fig. 3.3, while the neutron flux and energy width of each diffraction order up to $n = 15$ was compared with the TOF measurements in Table 3.1. The energy width was determined theoretically considering the mosaic spread of the

PG crystal and the angular divergence of the white neutron beam. Cadmium ratios for Au, Cl, Cd, Sm, Eu and Gd as well as the effective cross-section of the $^{10}\text{B}(n, \alpha)$ reaction were measured and compared with theoretical calculations based on the spectrum and pointwise neutron cross-sections. These theoretical predictions were consistent with the measured quantities, even though the agreement was not perfect.

The measured effective wavelength and velocity of the beam are $1.87 \pm 0.02 \text{ \AA}$ and $2117 \pm 21 \text{ m/s}$, respectively. All of the measured cadmium ratios except that for gold are in the range 340–410,

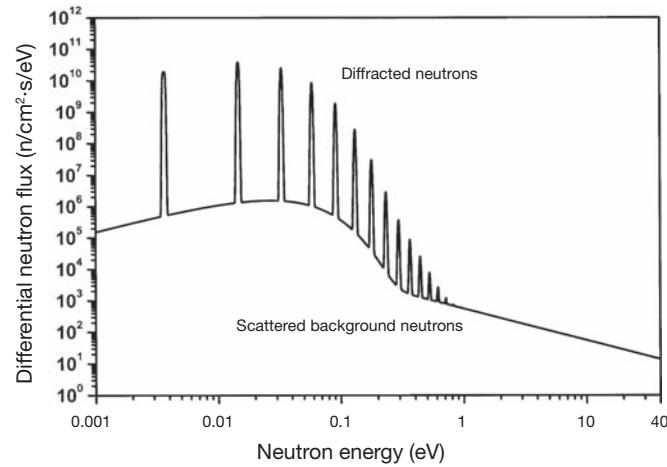


FIG. 3.3. Neutron spectrum at the sample position of the SNU-KAERI PGAA facility.

3.1. THE SNU-KAERI PGAA FACILITY

TABLE 3.1. RELATIVE FRACTION OF THE DIFFRACTED NEUTRON FLUX AS A FUNCTION OF DIFFRACTION ORDER

Diffraction order, n	Energy (meV)	Width (meV)	Relative flux (%)	
			TOF measurement	Theoretical calculation
1	3.6	0.2	4.4 ± 0.2	5.2
2	14.6	0.7	25.9 ± 0.2	29.6
3	32.8	1.5	39.3 ± 0.3	36.4
4	58.3	2.6	22.9 ± 0.2	20.4
5	91.0	4.1	6.2 ± 0.1	6.7
6	131.1	5.9	1.3 ± 0.1	1.4
7	178.4	8.0	n/d*	2.1 × 10 ⁻¹
8	233.0	10.4	n/d	2.5 × 10 ⁻²
9	294.9	13.2	n/d	4.1 × 10 ⁻³
10	364.1	16.3	n/d	1.2 × 10 ⁻³
11	440.5	19.7	n/d	4.0 × 10 ⁻⁴
12	524.3	23.4	n/d	1.3 × 10 ⁻⁴
13	615.3	27.5	n/d	4.0 × 10 ⁻⁵
14	713.6	31.9	n/d	1.1 × 10 ⁻⁵
15	819.1	36.6	n/d	3.0 × 10 ⁻⁶

* n/d: not detected.

and hence the epithermal neutrons have negligible impact on the capture rate. Details of the method of analysis and the results are reported in Refs [3.2, 3.3].

A gamma ray detector (n type/HPGe, with a relative efficiency of 43%) is normally placed at a distance of 25 cm from the sample. The pulse processing system consists of a preamplifier with a resistive feedback, amplifier, 16 000 analog to digital converter (ADC), multichannel buffer and personal computer with Ethernet connection to the buffer. Data collection and on-line analysis of the spectra are undertaken using commercial software, while off-line analysis is carried out by the program HYPERMET [3.4]. The total background counting rate for a neutron beam incident on a blank target is approximately 3000 counts/s, while the ADC dead time is less than a few per cent. Most of the background gamma ray peaks identified are nitrogen and germanium capture lines, along with gamma rays originating from the inelastic excitation of germanium isotopes. Several methods have been proposed to reduce the background in a future upgrade. Radiation levels around the lead wall and sample position are kept low to ensure safety, with measured gamma ray and neutron dose rates of 10

and 30 μ Sv/h, respectively. Both the efficiency and energy calibration of the detection system are determined according to the procedures adopted by the Budapest group [3.5, 3.6]. Full energy peak efficiency is determined by fitting polynomials to the measured data; the relative standard uncertainty is less than 3% over the low energy region and less than 5% for the complete spectrum. Non-linearity of the spectrometer is determined in a similar manner by fitting a polynomial function to the observed data for accurately known gamma ray lines [3.7].

The facility was first used to determine the sensitivity for boron. Dilute boric acid was used to prepare the solid samples, and a sensitivity of 2131 counts/(s · mg-B) was derived from the 478 keV Doppler broadened peak. The sensitivities for various elements are listed in Table 3.2, along with the detection limits for a counting period of 10 000 s [3.1]. Since the neutron spectrum is simple and well defined, k_0 standardization can be applied in the study of non-1/v absorbers. The k_0 factors and relative gamma ray emission intensities have been measured for ^{113}Cd , ^{149}Sm , ^{151}Eu and $^{155, 157}\text{Gd}$ [3.7].

Thus, diffracted polychromatic neutrons can be successfully used in a PGAA facility. Even

TABLE 3.2. MEASURED SENSITIVITIES AND DETECTION LIMITS FOR SOME ELEMENTS

Element	Energy (keV)	Sensitivity (counts · s ⁻¹ · mg ⁻¹)	Detection limit (μ g)
H	2223	4.322 ± 0.005	11.500 ± 0.001
B	478	2131 ± 40	0.067 ± 0.001
Cl	1165	4.170 ± 0.020	11.500 ± 0.001
K	770	0.532 ± 0.010	105.00 ± 0.07
Ti	1382	2.023 ± 0.010	23.600 ± 0.001
Cd	558	452 ± 10	0.165 ± 0.001
Sm	333	2663 ± 40	0.043 ± 0.001
Gd	182	3071 ± 40	0.057 ± 0.001

though the purity of the resulting thermal neutrons is inferior to that of a thermal beam guided by a mirror, a higher flux and detection sensitivity have been achieved at considerably lower cost and effort. For example, quantification of sub-ppm boron content is feasible in a non-destructive manner within 30 min for a small sample of 0.1 g. Future upgrading of the facility to reduce the background is expected to enhance the performance further.

3.2. CHARACTERIZATION OF PROMPT GAMMA NEUTRON ACTIVATION ANALYSIS AT THE DALAT RESEARCH REACTOR

The principle of extraction of the neutron beam, and the design of the beam shutter, beam catcher, detector shielding and gamma ray spectrometer are briefly described below for the PGAA facility at the Dalat reactor. Neutron flux, cadmium ratio, gamma dose rate and absolute efficiency are also quantified.

3.2.1. Experimental configuration

3.2.1.1. Neutron beam

The beam emerging from the reactor beam port consists mainly of fast and thermal neutrons and high energy gamma rays. The peak to background ratio of the gamma ray spectrum depends upon the background gamma radiation within the thermal neutron beam. Thermal neutrons are extracted from the beam port for PGAA by slowing down the fast neutrons to thermal energy

and filtering out the high energy gamma rays. Radiation beam port No. 4 was selected for the installation of the PGAA facility. The average neutron flux inside the reactor is of the order of 10^{13} n · cm⁻² · s⁻¹, from which a neutron flux level of 10^{12} n · cm⁻² · s⁻¹ is required at the base of the collimator for PGAA. Graphite was selected as the moderator because of its availability and the large diffusion length (a 40 cm thick moderator, placed 85 cm from the end side wall of the reactor). A 20 cm thick block of bismuth is used as a beam filter to minimize the high energy gamma radiation at the sample position and to reduce the need for additional shielding outside the biological shield. The beam aperture consists of two boron carbide sheets (each 3 mm thick) to give an aperture diameter of 25 mm. A hollow graphite block 15 cm thick separates the aperture from the moderator block in order to obtain a uniform neutron beam, with the outer diameter of the divergent beam collimator being 30 mm. Streaming of the radiation is eliminated by using bismuth and lead as beam stoppers that intercept all the radiation coming from the core of the reactor, gamma rays that arise from radiative capture of the neutrons, and scattered radiation from the sample and sample holder.

The beam shutter ensures the safe operation of the facility while positioning the sample. This shutter system consists of two parts:

- (1) The first segment is made of borated paraffin, cadmium carbide, boron carbide and cadmium sheets, and is enclosed in aluminium casing; thermalized neutrons are attenuated and

3.2. THE DALAT RESEARCH REACTOR

- absorbed by the borated paraffin, cadmium and boron carbide sheets.
- (2) The second part consists of a 15 cm thick shutter made from lead bricks and boron carbide sheets, and is enclosed in a steel casing.

The shutter is mounted on a trolley, and is moved into position by means of an overhead crane. The beam catcher is fabricated from borated paraffin, lead, boron carbide and steel, while an enclosure of concrete blocks provides additional shielding from the scattered gamma rays and neutron radiation. Figure 3.4 shows the layout of the PGAA facility.

3.2.1.2. Detector shield and sample arrangement

A horizontal HPGe detector of volume 90 cm³ manufactured by the Intertechnique company is used to count the prompt gamma rays (with a resolution of 2.5 keV at 1332 keV). The multichannel analyser (MCA) has been calibrated from 0.121 to 8 MeV by means of the delayed gamma rays from ¹⁵²Eu and prompt gamma rays from ³⁵Cl(n, γ) and ¹⁴N(n, γ), using the energies and intensities recommended by Molnár et al. [3.8].

Samples are sealed in a film of 25 μm thick fluorinated ethylene-propylene resin (FEP), and placed on the sample holder using a 0.3 mm diameter PTFE string. The spectrometer system is directly shielded from the neutrons by a layer of

3 mm thick boron carbide, and on all sides by 10 m borated paraffin. A 10 cm layer of lead is placed within the borated paraffin to protect the detector from undesired gamma rays that originate from the filtered neutron beam or neutron capture reactions on the shielding materials (Fig. 3.4). The prompt gamma rays are detected through a Li₂CO₃ window of 32 mm diameter located in the upper lead layer.

3.2.2. Characteristics of the system

3.2.2.1. Neutron flux, cadmium ratio and gamma dose rate

The beam position was determined by neutron radiography, and the neutron flux and flux distribution were measured by means of activated gold foils. The cadmium ratio was also determined by activating gold foils with and without a cadmium cover. The neutron flux and cadmium ratio are $2.1 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ and 21, respectively. Flux variations at the sample position during one reactor operation cycle of 100 hours were measured every five hours by means of 0.025 mm thick gold foils, and found to be 1.2%. The gamma dose rate at the sample position was determined by thermoluminescence dosimetry to be 200 mR/h.

3.2.2.2. Efficiency calibration

Efficiency measurements have been described by many authors: the curve of full energy peak

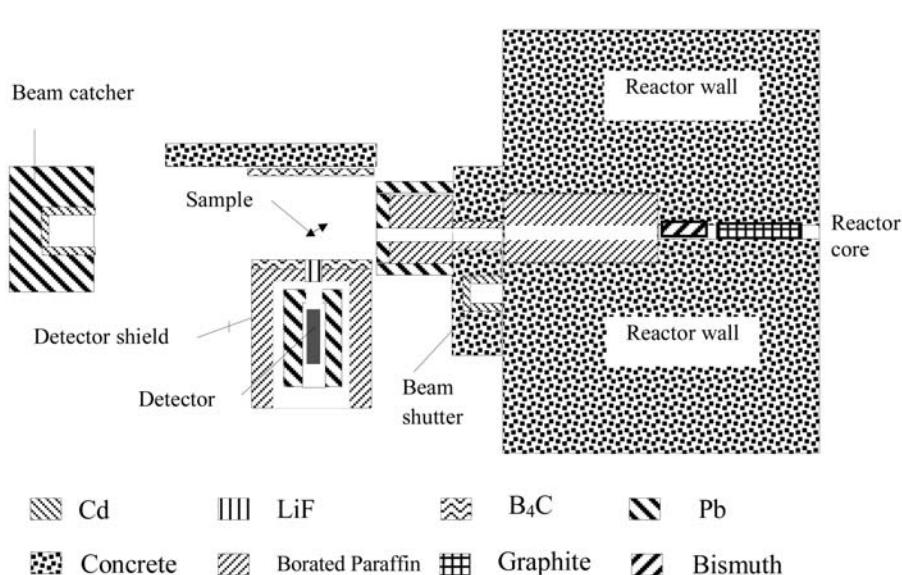


FIG. 3.4. Configuration of the PGAA facility at the Dalat research reactor.

efficiency is divided into three energy regions: 100–658 keV, 447–2754 keV and 1262–10 829 keV. Gamma ray sources of ^{24}Na , ^{54}Mn , ^{57}Co , ^{60}Co , ^{65}Zn , ^{88}Y , ^{137}Cs , ^{152}Eu and ^{241}Am were used for the absolute efficiency calibration from 100 to 2754 keV (calibrant emission probabilities from all of these sources have been recommended in Ref. [3.9]). Prompt gamma rays from the $^{14}\text{N}(\text{n}, \gamma)$, $\text{Cl}(\text{n}, \gamma)$ and $\text{Ti}(\text{n}, \gamma)$ reactions cover a wide energy span from 0.5 to 10 829 MeV, and are sufficiently well spaced to cover the efficiency curve from the low to the high energy region; their intensity values (I_γ) were accurately defined at the fourth international symposium on Neutron-Capture Gamma-Ray Spectroscopy and Related Topics, 1981. The resulting absolute efficiency curve is shown in Fig. 3.5.

3.3. PROMPT GAMMA ACTIVATION ANALYSIS AT NIST

The NIST Center for Neutron Research (NCNR) of the National Institute of Standards and Technology (NIST) at Gaithersburg, Maryland, is centred on a 20 MW research reactor that is cooled and moderated by D_2O [3.10]. This reactor operates on a seven week cycle, with about 38 days of continuous operation between refuelling. Among the experimental facilities are two instruments for PGAA.

The thermal neutron system was developed jointly by the University of Maryland and NIST, and has been in regular operation since 1978 [3.11, 3.12]. A vertical collimator extends 7 m down from the

top of the reactor to the reactor midplane, with an external beam tube, beam stop and germanium detector with Compton suppressor; a 5 cm sapphire filter has been added recently to reduce the background from fast neutrons and gamma rays. With the filter, the neutron fluence rate is $3.0 \times 10^8 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ and the cadmium ratio is 160. All components of the system outside the reactor have recently been replaced, with a large reduction in the background for H, B, C and N [3.13]. Furthermore, the titanium sensitivity for the capture line at 1382 keV is 1120 counts $\cdot \text{s}^{-1} \cdot \text{g}^{-1}$ in the current configuration (with a detector efficiency of 40% when located about 45 cm from the irradiated sample).

A second system has been developed for cold neutron PGAA (CPGAA) and has been operational since December 1990 [3.14]. Significant modifications have been made to this system [3.15]: the CPGAA spectrometer is located 41 m from the liquid hydrogen cold neutron source at the end of the lower half of the neutron guide NG7. Neutrons are filtered through a 127 mm thick sheet of beryllium and a 203 mm thick single crystal of bismuth (both at 77 K), before emerging through a 0.25 mm thick magnesium alloy window. The upper half of this neutron beam continues past the prompt gamma ray station to a 30 m small angle neutron scattering (SANS) instrument. Walls of steel shot 30 cm thick surround the guide tube, and a shutter composed of glass enriched with ^6Li can be opened to admit neutrons to the prompt gamma ray station [3.16]. The neutron beam is collimated to 20 mm or smaller, as required, by apertures of ^6Li glass

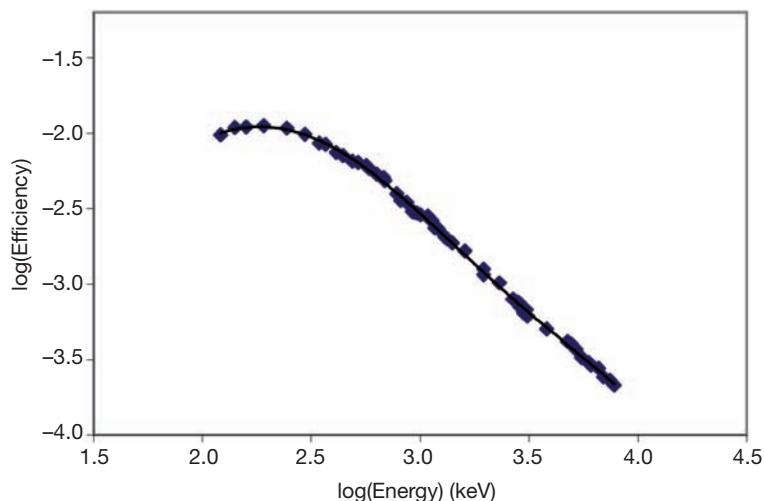


FIG. 3.5. The absolute efficiency curve.

3.3. THE NIST PGAA FACILITY

located upstream from the sample, and unused neutrons are absorbed by a fixed beam stop of ${}^6\text{Li}$ glass. Samples can be irradiated in air, or within a 120 mm cubic magnesium alloy box that can be evacuated or purged with helium. The CPGAA spectrometer is shown in Fig. 3.6, with the detectors in position.

Prompt gamma rays are measured by a high purity germanium detector (with a relative efficiency of 35% and a resolution of 1.7 keV) positioned vertically inside a horizontal BGO Compton suppression detector at a distance of 35 cm from the sample. The detectors and their shielding are located on an aluminium plate carried on rails perpendicular to the neutron guide. Both the sample holder and the neutron collimator are mounted on the same plate at a fixed position in front of the detector. Exchangeable lead apertures of different sizes placed between the detector and the sample allow variable collimation of the gamma ray signal in order to balance detector efficiency with the field of view. A cold neutron source of the third generation was installed in early 2002 to give a thermal equivalent neutron fluence rate (a reaction

rate per atom divided by the 2200 m/s cross-section) at the sample position of $9.5 \times 10^8 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$, and a titanium sensitivity of 7700 counts $\cdot \text{s}^{-1} \cdot \text{g}^{-1}$ at 1382 keV.

Spectra up to 11 MeV can be measured in both the thermal and cold neutron PGAA systems, using a digital signal processor on the cold neutron system with Compton suppression electronics and Ethernet 16 384 channel pulse height analysers. Data reduction and spectral manipulation are accomplished by means of standard Canberra nuclear data software, the HYPERMET program [3.4, 3.17], and the interactive algorithm SUM written at NIST [3.18].

Cold neutrons gain energy by scattering in hydrogenous samples at room temperature, and therefore the cross-section for absorption depends on the sample temperature [3.19]. The thermal PGAA system is preferred for the analysis of materials such as biological tissues and foods, while the greater sensitivity and lower hydrogen background make the cold neutron system advantageous for small samples and low concentrations.

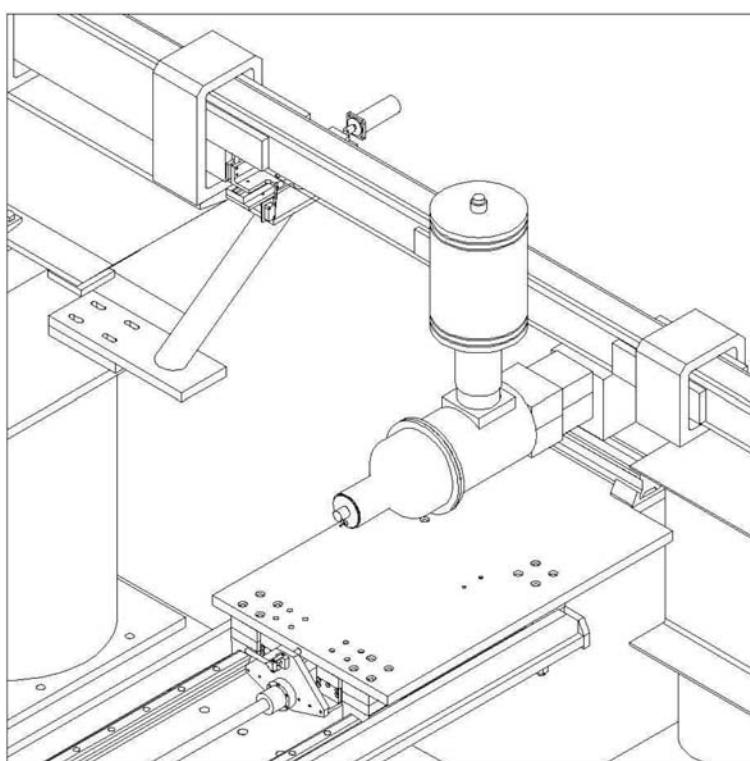


FIG. 3.6. Isometric view of detectors in position with the shielding removed. The sample position is hidden by the gamma ray collimator (the rectilinear block in front of the horizontal bismuth germanate (BGO) Compton detector), and the plate carrying the final neutron collimator, sample support, detectors and associated shielding is movable on the rails perpendicular to the neutron beam.

3.4. NEUTRON CAPTURE GAMMA RAY FACILITIES AT THE BUDAPEST RESEARCH REACTOR

The Budapest research reactor is a light water moderated and light water cooled reactor operating at 10 MW thermal power. Three neutron guides serve the external neutron beam facilities, and a liquid hydrogen cold source was commissioned in early 2001.

The thermal neutron PGAA analysis facility has been rebuilt and includes a neutron induced prompt gamma ray spectrometer (NIPS) for a variety of experiments involving prompt and delayed gamma rays (including $\gamma\gamma$ coincidences) induced by nuclear reactions [3.20–3.22]. A pneumatic beam shutter at the end of the guide tube allows the neutrons to enter the 3 m long evacuated aluminium tube that extends across the experimental area ($3 \times 5 \text{ m}^2$) to the beam stop at the rear wall of the guide hall (Fig. 3.7). This neutron beam can be divided into two separate beams of smaller diameter by appropriate collimation: the upper beam is used for PGAA measurements, while the lower beam is directed to the NIPS station.

The PGAA target chamber is located at a distance of 1.5 m from the end of the guide tube, and targets are suspended on a thin aluminium frame by fine Teflon strings. A vacuum, ${}^4\text{He}$, or

other gaseous atmospheres, can be maintained inside the sample box to reduce the background radiation induced by the neutrons. Furthermore, a neutron absorber layer can be placed in the horizontal plane to prevent scattering from the lower beam to the PGAA sample.

The NIPS chamber is positioned a further 1 m from the PGAA station, and is shielded with lead bricks to minimize the background radiation that originates from other measurements. The aluminium tubing and the NIPS chamber are sufficiently narrow for several detectors to be placed close to the irradiated sample.

All three sections of aluminium tube can be easily removed if necessary so that samples larger than the target chamber can be studied. A beam chopper is also provided for specific experimental investigations.

3.4.1. Beam characteristics

The thermal-equivalent neutron flux achieved at the old PGAA facility was $2 \times 10^6 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ [3.22]; fluxes at the sample positions of the new cold neutron PGAA and NIPS facilities are 5×10^7 and $3 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$, respectively [3.20]. Both beams are individually collimated to give a cross-section of 2×2 or $1 \times 1 \text{ cm}^2$. The neutron flux profile at the PGAA sample position is shown in Fig. 3.8.

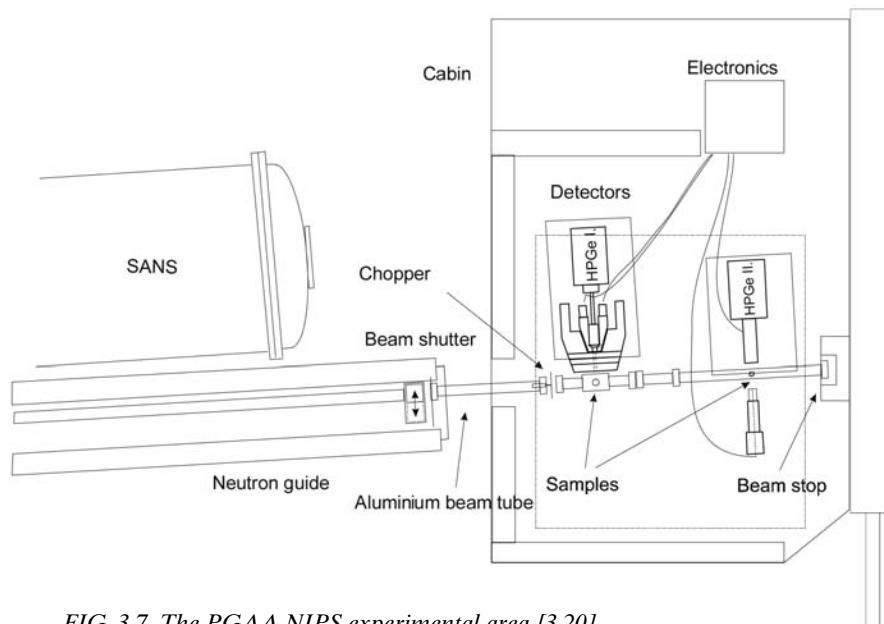


FIG. 3.7. The PGAA NIPS experimental area [3.20].

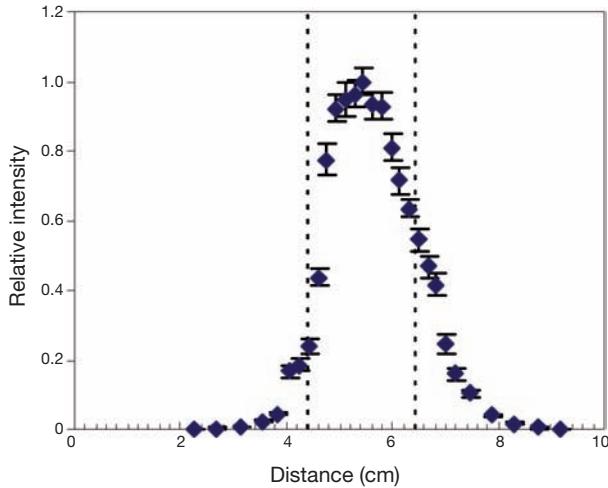


FIG. 3.8. Neutron flux profile at the sample position of the PGAA facility [3.21].

3.4.2. Instrumentation at the Budapest PGAA facility

An n type high-purity germanium (HPGe) detector with closed end coaxial geometry is normally used in the PGAA facility, along with a BGO scintillator guard detector annulus surrounded by lead shielding of 10 cm thickness [3.21, 3.22]. This complete system is positioned on a movable table. By removing the three lead discs in front of the detector, the HPGe detector can be placed 12 cm from the target, and as close as 3 cm by simply using the bare detector. The BGO annulus and catchers around the HPGe device detect most of the scattered gamma photons. Connection of the HPGe and BGO in anticoincidence mode results in the accumulation of Compton suppressed spectra.

With appropriate electronic gating, the HPGe BGO gamma ray spectrometer can also be used in

annihilation pair mode to simplify the spectra at high energies [3.22]. A 16 000 multichannel PC based analyser collects the resulting data. The HPGe BGO detector assembly is shown in Fig. 3.9, and the operational characteristics of the PGAA system are listed in Table 3.3. A Compton suppression ratio of about 5 can be achieved for the 1332 keV gamma ray emission of ^{60}Co (although this ratio is much larger for higher energy gamma rays, as can be seen in Fig. 3.10).

3.4.3. Detection efficiency and system non-linearity

The energy and intensity calibration of the gamma ray spectrometer system is important for both nuclear spectroscopic and analytical experiments. However, this essential procedure becomes problematic when the energy of interest is higher

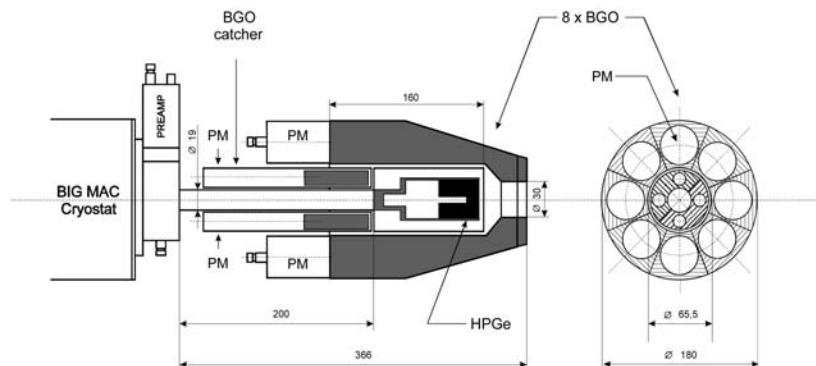


FIG. 3.9. Cross-section of the HPGe BGO gamma ray spectrometer at the Budapest research reactor (PM: photomultiplier).

TABLE 3.3. MAIN SPECIFICATIONS OF THE PGAA FACILITY AT THE BUDAPEST RESEARCH REACTOR [3.20]

Beam tube	NV1 guide, end position
Distance from guide end	1.5 m
Beam cross-section	$1 \times 1 \text{ cm}^2$ or $2 \times 2 \text{ cm}^2$
Thermal-equivalent flux at target	$\approx 5 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$
Vacuum in target chamber (optional)	$\approx 1 \text{ mbar}$ (10^2 Pa)
Target chamber aluminium window thickness	0.5 mm
Form of target at room temperature	Solid/powder/liquid/gas (pressurized chamber)
Target packing at atmospheric pressure	Sealed FEP Teflon bag or vial
Activity of target after irradiation	Negligible
Largest target dimensions	$4 \times 4 \times 10 \text{ cm}^3$
Gamma ray detector	Coaxial HPGe of n type with BGO shield
Distance from the target to the detector window	23.5 cm
HPGe window	0.5 mm Al
Relative efficiency	25% at 1332 keV (Co-60)
Full width at half-maximum (FWHM)	1.8 keV at 1332 keV (Co-60)
Compton suppression enhancement	≈ 5 (1332 keV) to ≈ 40 (7000 keV)

than the highest gamma ray energy of the ^{56}Co calibrant source. The counting efficiency has been accurately determined over the energy range from 50 keV to 10 MeV using several multiple gamma ray sources and (n, γ) reactions in order to avoid this difficulty. The accuracy of the efficiency function is better than 1% from 500 keV to 6 MeV [3.22]. Figure 3.11 shows the absolute peak efficiency at full energy for a distance from the target to the detector of 23.5 cm, with the single and double escape peak efficiencies also included.

When constructing the non-linear energy function, long term instabilities of the system may result in peak shifts and create inconsistencies between independent measurements. Therefore, a non-linear calibration procedure that uses radioactive sources and capture gamma rays with well known energies has been introduced to overcome this problem [3.6]. When the non-linear function is combined with the normal linear energy calibration for strong gamma ray peaks, an energy precision of between 0.01 and 0.1 keV can be

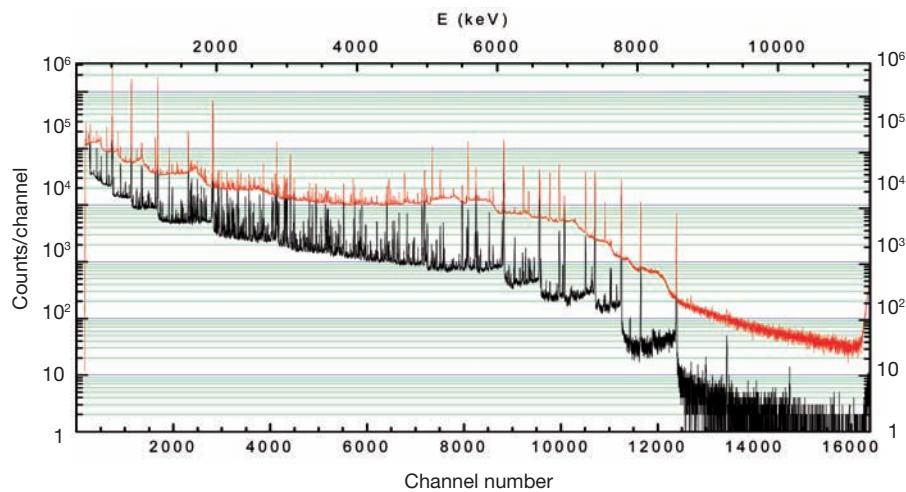


FIG. 3.10. Normal (red trace) and Compton suppressed (black trace) spectra of a CCl_4 sample.

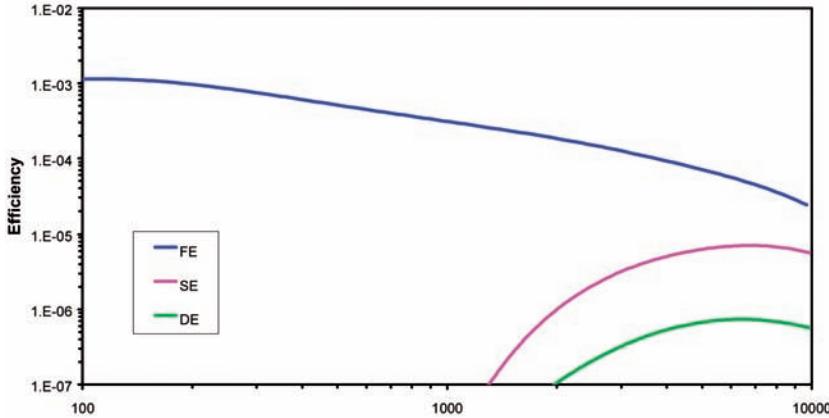


FIG. 3.11. Efficiency of the PGAA spectrometer in Compton suppressed mode (FE: full energy peak, blue curve; SE: single escape peak, purple curve; DE: double escape peak, green curve).

achieved, depending on the statistics. The non-linearity functions are regularly determined at the beginning of each period of reactor operation.

3.4.4. Data acquisition and analysis

A Canberra S100 type single-input PC based MCA has been used to collect PGAA spectra. However, a digital spectrum analyser will soon be installed to achieve a much higher input rate without any substantial deterioration in spectral resolution.

Gamma ray spectra from neutron capture are extremely complex, and therefore a high quality fitting code has been developed for the data analysis [3.23]. The HYPERMET-PC program is an interactive non-linear fitting code that evolved from the spectrum evaluation program HYPERMET. The PC version has user friendly graphics and a database to store the fitted regions, as well as quality assurance, calibration and nuclide identification modules. Peak energies and intensities that result from the fitting process can be corrected within the program for non-linearity and detector efficiency, respectively. Element identification on the basis of peak energies is also possible with the help of the built-in library.

3.5. PROMPT GAMMA RAY NEUTRON ACTIVATION ANALYSIS AT THE BHABHA ATOMIC RESEARCH CENTRE

Initial PGAA studies at the Bhabha Atomic Research Centre (BARC) were carried out using a guided beam facility, with subsequent improvements including the installation of a reflected beam. A dedicated beamline is currently being developed. Brief descriptions of these systems are given in Sections 3.5.1–3.5.5 below.

3.5.1. PGAA systems

A facility with a thermal guided beam has been used for PGAA in the 100 MW Dhruva reactor at BARC, Trombay. A beam tube was used to guide and transport the neutrons about 30 m away from the reactor core to a temporary experimental facility (a beam of cross-section $2.5 \times 10 \text{ cm}^2$). A boron carbide sheet of 1 cm thickness minimized scattering of neutrons towards the detector, except when boron was contained within the sample for analysis. The gamma ray detector was located about 40 cm from the irradiated sample, and was provided with a 30 cm thick lead shield to reduce the background radiation. A lead collimator (with a diameter of 3 cm and a length of 30 cm) was placed in front of the detector to control the gamma rays emitted from the sample. The layout of this PGAA system is shown in Fig. 3.12.

The effective thermal neutron flux at the sample irradiation position has been measured by

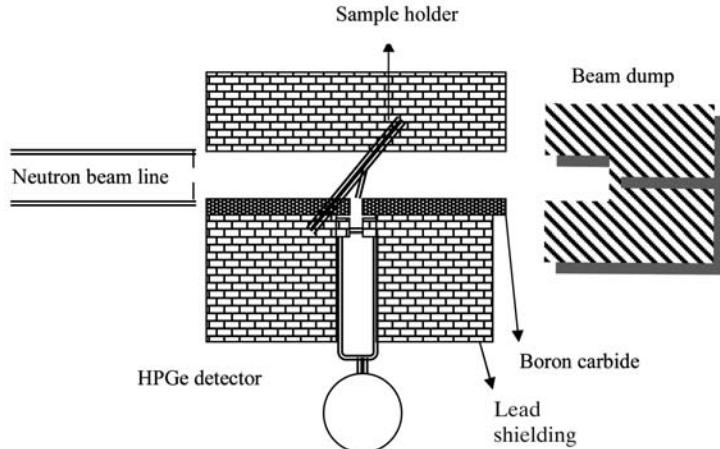


FIG. 3.12. The PGAA arrangement used at BARC.

means of indium foils, while the cadmium ratio method was used to determine the subcadmium to epithermal flux ratio. An indium foil (110 mg/cm^2) was irradiated with and without a covering of cadmium (0.8 mm thick), followed by off-line counting of ^{116m}In by means of an HPGe detector with a relative efficiency of 15% coupled to a 4000 MCA. The subcadmium to epithermal neutron flux ratio was found to be 3.45×10^4 , indicating that at the irradiation position more than 99.99% of the neutron beam consisted of thermal neutrons. A $Q_o(I_0/\sigma_0)$ value of 16.8 was derived from ^{116m}In gamma rays (E_γ of 1097 and 1293 keV) and used to estimate a total neutron flux of $(1.4 \pm 0.1) \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ [3.24]. The indium foil was estimated to attenuate the beam by as much as 8%, which affected the cadmium ratio. However, this effect does not have any impact on the k_0 values or elemental analyses based on this method.

3.5.2. Sample irradiation and data acquisition

Samples weighing between 100 and 500 mg were wrapped in thin Teflon tape and placed at 90° with respect to the beam direction. Care was taken to ensure that the sample dimensions were significantly less than the beam dimensions. An HPGe detector with a relative efficiency of 22% connected to a PC based 8000 MCA was used to assay the prompt gamma rays, with a resolution of 2.4 keV at 1332 keV.

3.5.3. Energy calibration and peak area analysis

The MCA has been calibrated from 0.1 to 8.5 MeV by means of the delayed gamma rays of ^{152}Eu and ^{60}Co , and prompt gamma rays of ^{36}Cl and ^{49}Ti . Non-linearity over this energy range was not significant, and therefore a second order polynomial was used for the energy calibration. The compilation of Lone et al. for capture gamma rays was used to identify the prompt gamma ray emissions of the different elements [3.25].

The photopeak areas in the gamma ray spectra were determined using the PHAST-2.6 code developed at the electronics division of BARC [3.26]. This software can be used to derive energy calibrations and determine spectral shape parameters. A second order polynomial was used to calibrate the full width at half-maximum (FWHM) of the photopeaks, and the measured FWHM and shape parameters as functions of energy were subsequently used to identify multiplets and undertake their deconvolution.

3.5.4. Efficiency calibration

Delayed gamma rays from ^{152}Eu and prompt gamma rays from ^{36}Cl and ^{49}Ti were used for absolute/relative efficiency calibrations of the detector over a wide energy range from 100 keV to 10 MeV. The absolute gamma ray abundances of ^{36}Cl and ^{49}Ti were obtained from the literature [3.9, 3.27]. Ammonium chloride packed in Teflon was irradiated for about 12 hours, and capture gamma ray spectra were accumulated. Absolute full energy peak efficiencies were determined for the lower

energy region (i.e. up to 1500 keV) using the gamma ray spectrum of ^{152}Eu , and the relative efficiency plot from 0.5 to 8 MeV was obtained from the prompt gamma ray spectra of ^{36}Cl and ^{49}Ti . Relative efficiencies were converted to absolute values using the overlap with equivalent ^{152}Eu data.

Efficiencies as a function of gamma ray energy (E_γ) were fitted to a fifth order polynomial using Eq. (3.1):

$$(\ln \varepsilon)_{E_\gamma} = k_j + \sum_{i=0}^5 a_i (\ln E_\gamma)^i \quad (3.1)$$

where a_i are the coefficients of the polynomial and k_j is the normalization constant for the j th gamma ray emitting nuclide used in the efficiency calibration. The number of free parameters used to fit the efficiency data is $6 + (n - 1)$, where n is the number of radionuclides whose gamma ray emissions have been used in the fitting procedure. A standard non-linear least squares program was used in which the peak areas of the gamma rays from each specific nuclide are fitted with a particular constant k_j so that the relative efficiency curves from different radionuclides are normalized with respect to the absolute efficiency determined from ^{152}Eu . The efficiency of the PGAA system at BARC is shown in Fig. 3.13 (the inset shows the efficiency on a logarithmic scale).

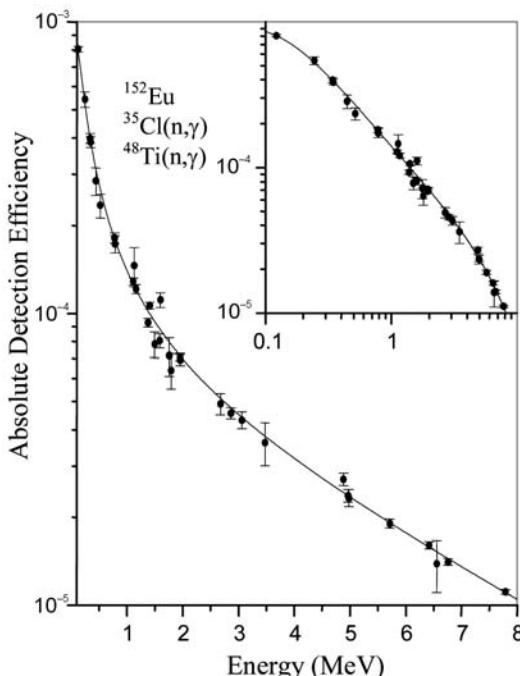


FIG. 3.13. Absolute detection efficiency of the PGAA system at BARC.

3.5.5. The new beam facility at Dhruva reactor

Another PGAA system has been established at the Dhruva reactor (BARC), using a reflected neutron beam that is normally applied to neutron diffraction experiments. The tangential beam of neutrons is reflected by a graphite crystal towards the PGAA experimental facility (neutron energy of 0.05 eV, and composed mainly of first-order reflection). Neutron beam characteristics have been determined in terms of dimensions, homogeneity and thermal equivalent flux. A gadolinium loaded neutron radiographic film was held in the beam path to measure a neutron beam of area $2.5 \times 3.5 \text{ cm}^2$. The neutron flux profile was obtained by irradiating a gold foil ($40 \text{ mm} \times 40 \text{ mm}$) for 48 hours in the beam, cutting the foil into 64 squares (of $5 \text{ mm} \times 5 \text{ mm}$), and then measuring the activity.

Separate shielding has been placed in front of the detector: an $8 \text{ cm} \times 8 \text{ cm} \times 30 \text{ cm}$ collimator was located inside a $30 \text{ cm} \times 30 \text{ cm} \times 60 \text{ cm}$ lead shield. Graded shielding was also used around the detector. Samples are held in quartz containers placed in front of the collimator and within the path of the neutron beam. Compared with the earlier PGAA system, the background in the newer facility has been reduced by a factor of two. The same data acquisition system is used as previously, and the procedures followed for the energy and efficiency calibrations are identical. Figure 3.14 shows the efficiency calibration of the new facility presented on both logarithmic and linear scales.

3.6. SUMMARY OF EXPERIMENTAL FACILITIES

The most important performance characteristics of any PGAA facility are the thermal equivalent neutron flux and the associated neutron spectrum, gamma ray detection sensitivity and achievement of a low background. Other essential features include the method and the quality of the calibrations and spectral analyses. The main characteristics of the facilities associated with the present CRP are summarized in Table 3.4. These comparative data show that the development of an excellent performance feature for a particular facility is usually achieved at the expense and degradation of other features. While improved characteristics can be achieved in various ways, the best performance is often achieved by considering conditions at the site and tailoring the design of the

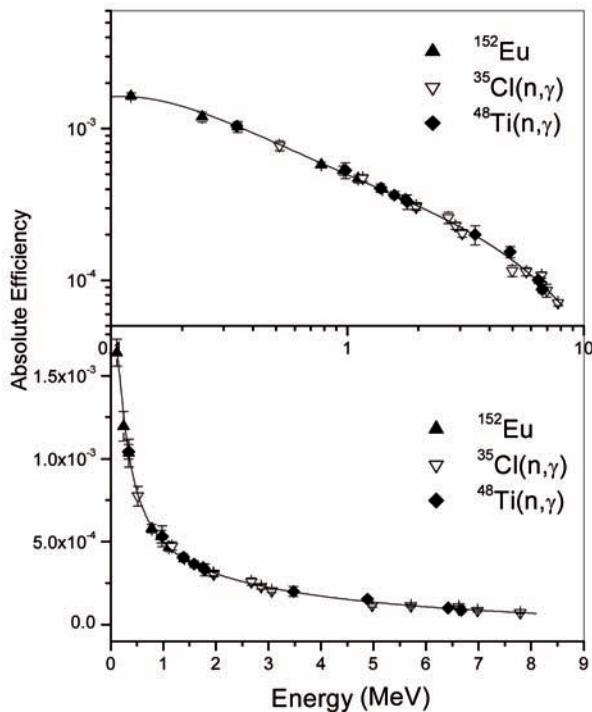


FIG. 3.14. Detection efficiency as a function of energy, PGAA system, BARC.

facility accordingly, and by improving the operational characteristics gradually during the course of the various work programmes.

3.7. EXPERIMENTS

The largest amount of new PGAA data has come from the Institute of Isotope and Surface Chemistry, Budapest, Hungary. Neutron capture reactions on all naturally occurring elements except for the four noble gases (i.e. 79 elements from hydrogen to uranium) have been studied by means of the guided thermal neutron beam PGAA facility at the Budapest research reactor. The $^{10}\text{B}(\text{n}, \alpha\gamma)$ reaction on natural boron has also been measured. These results are described below.

A thermal guided beam was used for PGAA experiments at the Bhabha Atomic Research Centre (BARC), India. Activities were concentrated on the experimental determination of prompt

k_0 factors with respect to the 1951 keV gamma ray emission from the $^{35}\text{Cl}(\text{n}, \gamma)^{36}\text{Cl}$ reaction using a mixture of ammonium chloride and other stoichiometric compounds [3.28, 3.29]. The emission probabilities of capture gamma rays from ^{60}Co have also been determined [3.29, 3.30].

The Seoul National University-KAERI PGAA system was used in the Republic of Korea to measure the prompt k_0 factors for the major non- $1/\nu$ nuclides, and to determine the corresponding effective g factors for their polychromatic diffracted neutron beam [3.7].

The Vietnam Atomic Energy Commission has supported the measurements of prompt k_0 factors at the Dalat research reactor for a number of elements with respect to the 1951 keV gamma ray emission from chlorine, using a filtered thermal neutron beam [3.31]. The reliability of these k_0 factors has been tested on all facilities for a number of applications.

The Budapest group has measured partial cross-sections for the elements. As the other CRP participants have measured only k_0 factors with respect to the 1951 keV chlorine line, comparison with the adopted set and the new Budapest data is only possible for the similar inferred k_0 factors. Available data are compared in Table 3.5 with the adopted set from the CRP and the new Budapest data [3.32]. Data from the NIST/University of Maryland thermal beam facility [3.33], as well as recent data obtained in thermal and cold guided beams at the Japan Atomic Energy Research Institute (JAERI) [3.34, 3.35], are also included in order to assess the possible dependence on neutron beam characteristics.

The data in Table 3.5 show that the agreement is generally good for $1/\nu$ nuclides at the quoted uncertainty level. Furthermore, it is especially gratifying to observe that the very precise JAERI data corroborate the adopted values, as do the new Budapest data. Moreover, the cold neutron data from JAERI agree well with similar data from NIST and with the thermal data, supporting the $1/\nu$ form of the cross-sections. The only exceptions are the well known cases discussed in Chapter 2: ^{113}Cd , ^{149}Sm , ^{155}Gd and ^{157}Gd , for which the g factor deviates strongly from unity.

3.7. EXPERIMENTS

TABLE 3.4. MAIN CHARACTERISTICS OF THE PGAA FACILITIES IN THE CRP

Facility	Characteristics
SNU-KAERI	Thermal beam extraction: diffraction (PG) Beam flux: $8.2 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (thermal equivalent) Beam size: $2 \times 2 \text{ cm}^2$ Cadmium ratio: 266 (for gold) Effective temperature: 269 K Titanium (1382 keV) sensitivity: $2020 \text{ counts} \cdot \text{s}^{-1} \cdot \text{g}^{-1}$ Detection system: single HPGe with pulse processing system Total background counting rate: 3000 counts/s
Dalat research reactor	Thermal beam extraction: moderation (graphite) and filtering (Bi) Beam flux: $2.1 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ Beam size: 2.5 cm Cadmium ratio: 21 (for gold) Detection system: single HPGe with pulse processing system
NIST (thermal)	Thermal beam extraction: filtering (sapphire) Beam flux: $3.0 \times 10^8 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ Cadmium ratio: 160 Effective temperature: 300 K Titanium (1382 keV) sensitivity: $890 \text{ counts} \cdot \text{s}^{-1} \cdot \text{g}^{-1}$ Detection system: HPGe and Compton suppression electronics
NIST (cold)	Cold beam extraction: filtering (Be, Bi) and mirror guide Beam flux: $9.5 \times 10^8 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (thermal equivalent) Beam size: 2 cm or smaller Effective temperature: 14 K Titanium (1382 keV) sensitivity: $7700 \text{ counts} \cdot \text{s}^{-1} \cdot \text{g}^{-1}$ Detection system: HPGe and Compton suppression electronics
Budapest research reactor	Cold beam extraction: mirror guide Beam flux: $5 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (thermal equivalent) Beam size: 1×1 or $2 \times 2 \text{ cm}^2$ Effective temperature: ≈ 60 K Titanium (1382 keV) sensitivity: $750 \text{ counts} \cdot \text{s}^{-1} \cdot \text{g}^{-1}$ Detection system: HPGe and Compton suppression electronics
BARC (thermal 1)	Thermal beam extraction: mirror guide Beam flux: $1.4 \times 10^7 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (total) Beam size: $2.5 \times 10 \text{ cm}^2$ Cadmium ratio: 3.4×10^4 (for indium) Detection system: single HPGe with pulse processing system
BARC (thermal 2)	Thermal beam extraction: diffraction (graphite) Beam flux: $1.6 \times 10^6 \text{ n} \cdot \text{cm}^{-2} \cdot \text{s}^{-1}$ (thermal equivalent) Beam size: $2.5 \times 3.5 \text{ cm}^2$ Detection system: single HPGe with pulse processing system

TABLE 3.5. COMPARISON OF LIBRARY $k_{0,\text{Cl}}$ FACTORS WITH OTHER MEASUREMENTS FOR THE MOST PROMINENT GAMMA RAYS OF SELECTED ELEMENTS

Z	Target isotope	$E(\text{d}E)$	Adopted	Dalat thermal beam [3.31]	BARC thermal guide [3.28]	SNU diffraction beam [3.7]	NIST thermal beam [3.33]	JAERI thermal guide [3.34, 3.35]	NIST cold guide [3.33]	JAERI cold guide [3.34, 3.35]	Budapest thermal guide [3.32]
1	H-1	2223.25	1.848(11)	1.800(16)	2.00(10)	1.80(6)	2.05(11)	1.86(6)	1.803(10)		
3	Li-7	2032.30(4)	0.0307(8)	0.0230(5) ^a	312(22)		371(31)		380(32)	360(3)	
5	B-10	477.595(3)	369.5(23)	0.00041(1) ^a			0.000573(5)		0.000551(6)	0.000546(9)	
6	C-12	1261.765(9)	0.000579(15)				0.00124(3)		0.001160(17)	0.001192(13)	
	C-12	4945.301(3)	0.001218(25)				0.005800(13)		0.005890(18)	0.00569(4)	
7	N-14	1884.821(16)	0.00588(8)	0.00567(11)			0.105(4)	0.11600(41)	0.105(4)	0.1160(25)	0.1181(13)
11	Na-23	472.202(9)	0.1165(11)				0.0065(2)		0.0064(3)		
12	Mg-25	585.00(3)	0.0072(3)				0.0467(18)	0.0440(4)	0.0463(21)	0.0433(14)	0.0472(9)
13	Al-27	1778.92(3)	0.0482(10)								
14	Si-28	2092.902(18)	0.00660(13)	0.00603(11)							
	Si-28	3538.966(22)	0.0237(4)				0.0214(7)	0.02180(10)	0.0216(9)	0.02110(11)	0.0231(5)
15	P-31	636.663(21)	0.0056(3)				0.00572(9)		0.00570(9)	0.0055(3)	
16	S-32	840.993(13)	0.0606(11)	0.0603(15)			0.0558(18)	0.0554(10)	0.0562(23)	0.0570(12)	0.0608(13)
17	Cl-35	786.302(10)	0.540(3)		1.30(3) ^b		1.28(6) ^b	1.330(45) ^b	1.26(7) ^b	1.350(44) ^b	
	Cl-35	788.428(10)	0.856(9)		1.30(3) ^b		1.28(6) ^b	1.330(45) ^b	1.26(7) ^b	1.350(44) ^b	
	Cl-35	1951.140(20)	1	1		1		1	1	1	1
19	K-39	770.305(20)	0.1294(18)		0.116(4)		0.126(4)	0.127(4)	0.122(5)	0.128(4)	0.127(3)
20	Ca-40	1942.67(3)	0.0492(10)		0.045(2)		0.0461(16)	0.047(2)	0.0459(19)	0.0464(16)	0.0463(14)
22	Ti-48	341.706(5)	0.215(3)		0.187(6) ^a		0.211(3)		0.2250(16)		
	Ti-48	1381.745(5)	0.606(15)	0.433(10) ^a	0.604(13)		0.582 ^c	0.582(6)	0.591 ^c	0.591(6)	0.591(7)
	Ti-48	1585.941(5)	0.0730(10)		0.056(3) ^a						
24	Cr-50	749.09(3)	0.0614(10)		0.065(8)			0.0562(20)		0.0601(25)	
	Cr-50	834.849(22)	0.149(3)		0.138(8)		0.141(5)		0.142(5)	0.145(2)	
	Cr-50	7998.46(23)	0.0457(11)		0.048(3)						
25	Mn-55	314.398(20)	0.1488(22)				0.152(5)		0.149(8)	0.150(3)	

TABLE 3.5. COMPARISON OF LIBRARY $k_{0,\text{cl}}$ FACTORS WITH OTHER MEASUREMENTS FOR THE MOST PROMINENT GAMMA RAYS OF SELECTED ELEMENTS (cont.)

Z	Target isotope	$E(\text{d}E)$	Adopted	Dalat thermal beam [3.31]	BARC thermal guide [3.28]	SNU diffraction beam [3.7]	NIST thermal beam [3.33]	JAERI thermal guide [3.34, 3.35]	NIST cold guide [3.33]	JAERI cold guide [3.34, 3.35]	Budapest thermal guide [3.32]
26	Fe-56	352.347(12)	0.0274(3)				0.0253(9)	0.0273(10)	0.0248(10)	0.0269(11)	0.0537(27) ^a
	Fe-56	7631.136(14)	0.0654(13)				0.0588(24) ^a	0.058(24) ^a	0.0537(27) ^a	0.0676(14)	0.0537(27) ^a
27	Co-59	229.879(17)	0.682(8)		0.58(4)		0.67(2)	0.67(2)	0.664(22)	0.702(8)	0.664(22)
	Co-59	277.161(17)	0.643(8)		0.55(4) ^a		0.619(21)	0.619(21)	0.615(21)	0.615(21)	0.615(21)
Co-59	555.972(13)	0.547(6)		0.46(3) ^a	0.46(3) ^a		0.516(18)	0.460(12) ^a	0.509(20)	0.509(20)	0.509(20)
	Co-59	1515.720(25)	0.165(3)	0.186(6) ^a	0.186(6) ^a		0.19(1) ^a	0.19(1) ^a	0.19(1) ^a	0.19(1) ^a	0.19(1) ^a
Co-59	1830.800(25)	0.1616(24)		0.185(15) ^a	0.185(15) ^a		0.156(6) ^a	0.156(6) ^a	0.156(6) ^a	0.156(6) ^a	0.156(6) ^a
	Co-59	6485.99(3)	0.220(6)		0.131(3)	0.131(3)		0.075(3)	0.074(3)	0.074(3)	0.0781(9)
28	Co-59	7214.42(3)	0.131(3)		0.0804(10)		0.068(4)	0.077(3)	0.0762(25)	0.0762(25)	0.0762(25)
	Ni-58	464.978(12)	0.0804(10)		0.0787(14)	0.0787(14)	0.019(1) ^b	0.0174(7) ^b	0.0166(6) ^b	0.0166(6) ^b	0.0166(6) ^b
29	Cu-63	278.250(14)	0.0787(14)		0.00617(13)	0.00617(13)	0.019(1) ^b	0.0174(7) ^b	0.0166(6) ^b	0.0166(6) ^b	0.0166(6) ^b
	Cu-63	384.45(5)	0.00617(13)		0.01155(18)	0.01155(18)	0.0261(14)	0.0261(14)	0.0261(14)	0.0261(14)	0.0261(14)
37	Cu-63	385.77(3)	0.01155(18)		0.0283(15)	0.0283(15)					
	Rb-85	7306.93(4)	0.0283(15)		0.00599(17)	0.00599(17)					
38	Rb-85	556.82(3)	0.00599(17)		0.00210(5) ^a	0.00210(5) ^a					
	Sr-87	898.055(11)	0.0449(8)		0.0449(8)	0.0449(8)		0.042(2)	0.042(2)	0.042(2)	0.0434(6)
49	Sr-87	1836.067(21)	0.0658(12)		0.0658(12)	0.0658(12)		0.132(7) ^a	0.132(7) ^a	0.132(7) ^a	0.0634(7)
	Cd-113 ^d	558.32(3)	92.6(16)		41(2) ^a	41(2) ^a	90(6)	81(2)	81(2)	81(2)	90.7(11)
55	Cs-133	116.3749(20)	0.059(6)		0.059(6)	0.059(6)		0.172(6) ^b	0.172(6) ^b	0.172(6) ^b	0.172(6) ^b
	Cs-133	116.612(4)	0.061(6)		0.061(6)	0.061(6)		0.172(6) ^b	0.172(6) ^b	0.172(6) ^b	0.172(6) ^b
56	Cs-133	307.015(4)	0.0612(13)		0.0612(13)	0.0612(13)		0.0692(25) ^a	0.0692(25) ^a	0.0692(25) ^a	0.0692(25) ^a
	Ba-138	627.29(5)	0.01200(25)		0.0106(3)	0.0106(3)	0.0106(3)	0.0111(4)	0.0111(4)	0.0111(4)	0.0111(4)
62	Ba-135	818.514(12)	0.00865(17)		0.012(2) ^a	0.012(2) ^a	0.012(2) ^a	0.0118(4)	0.0118(4)	0.0118(4)	0.0118(4)
	Ba-137	1435.77(4)	0.0126(3)		0.011(1)	0.011(1)	0.011(1)	0.0118(4)	0.0118(4)	0.0118(4)	0.0118(4)
62	Sm-149 ^d	333.97(4)	178.4(24)		188(4)	188(4)	172(14)	1339(18) ^a	131(9) ^a	111(7) ^a	116(1) ^a
											178(2)

For footnotes see end of table.

TABLE 3.5. COMPARISON OF LIBRARY $k_{0,\text{Cl}}$ FACTORS WITH OTHER MEASUREMENTS FOR THE MOST PROMINENT GAMMA RAYS OF SELECTED ELEMENTS (cont.)

Z	Target isotope	$E(\text{d}E)$	Adopted	Dalat thermal beam [3.31]	BARC thermal guide [3.28]	SNU diffraction beam [3.7]	NIST thermal beam [3.33]	JAERI thermal guide [3.34, 3.35]	NIST cold guide [3.33]	JAERI cold guide [3.34, 3.35]	Budapest thermal guide [3.32]
63	Eu-151 ^d	89.847(6)	52.7(11)		46(3)				236(13)	214(1) ^a	
64	Gd-157 ^d	181.931(4)	257(11)		277(15)		222(12)	255(3)			267(6)
	Gd-155 ^d	199.2130(10)	71.9(23)				68(5)				
	Gd-157 ^d	944.174(10)	110.0(25)	162(3)							
	Gd-155 ^d	1187.120(21)	12(4)		111(4) ^{a,b}		105(6) ^{a,b}				
	Gd-157 ^d	1187.122(9)	51(3)		111(4) ^{a,b}		105(6) ^{a,b}				
73	Ta-181	402.623(3)	3.29(8)	0.156(3) ^a							
80	Hg-199	367.947(9)	7.00(15)	5.8(3)			7.11(26)		7.01(14)		6.82(12)
	Hg-199	1693.296(11)	1.57(5)	1.37(8)			1.41(5)		1.40(5)		
	Hg-199	5967.02(4)	1.74(4)					1.43(6) ^a			
82	Pb-207	7367.78(7)	0.00370(8)		0.00338(6)		0.00329(3)		0.00361(8)		

^a Value deviates significantly from the adopted value.

^b Doublet line.

^c Normalizing transition: set equal to corresponding JAERI value.

^d Non-1/ ν nuclide.

CHAPTER 3. CHARACTERISTICS OF PGAA FACILITIES

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Chapter 4

BENCHMARKS AND REFERENCE MATERIALS

R.M. Lindstrom

4.1. INTRODUCTION

Two sets of sample materials were sent to experimentalists within the CRP to aid in characterizing each neutron beam and detector system, and to analyse an unknown sample.

The first set of samples comprised the following:

- (a) 99.65% titanium foil, 0.25 mm thick: a 2.5 cm² square, and 6 and 13 mm discs;
- (b) Gold foil, 0.025 mm thick by 5 mm diameter;
- (c) Borophosphosilicate glass on silicon: $\approx 5 \times 10^{16}$ atoms/cm² of ¹⁰B (surface density measured by neutron depth profiling);
- (d) ¹⁰B-aluminium alloy sheet, 1.3 mm thick and 4.5 wt% ¹⁰B as two ≈ 2.5 cm² squares;
- (e) Approximately 2 g of an ‘unknown’ mixture of aluminosilicate and graphite.

The titanium foil was used to measure the sensitivity of the PGAA system (i.e. the product of neutron flux and detector efficiency, expressed as the count rate per milligram of titanium of the 1381.5 keV capture gamma ray of ⁴⁸Ti). The effective velocity or wavelength of the beam can be measured by means of the boron samples, as described below. Excel spreadsheets for flux and wavelength were also developed and made available on the IAEA server, as illustrated below.

The unknown sample was distributed in order to demonstrate the ability of CRP participants to perform quantitative analysis. This material was made by blending dried and weighed quantities of two NIST fly ash standard reference materials (SRMs 1633a and 1633b) with spectroscopic graphite as a diluent in a mixer mill. The participants received no information about the constituents, or their proportions. The known values of eleven elements were calculated from their SRM certificates or from published consensus numbers. Unfortunately for the comparison, the concentrations of hydrogen and boron reported by all three participants are not known in SRM 1633b, so the ‘correct’ value of these elements is unknown as well.

4.2. CHARACTERIZATION OF NEUTRON BEAMS

Foil activation is the simplest and perhaps the most accurate method of measuring the neutron flux [4.1]. A known mass of a monitor element is irradiated for a known time and the resulting radioactivity measured with a detector of known efficiency. If the reaction rate per atom ($R = \sigma\phi$) is calculated with the 2200 m/s thermal cross-section (e.g., $\sigma_0 = 98.65$ b for ¹⁹⁸Au production), the thermal equivalent flux (ϕ_0) can be determined. Epithermal flux is often measured by irradiating a bare monitor and another specimen of the same monitor under 1 mm shielding of cadmium, as described in Section 2.2.2. Fast neutron (MeV) monitoring is similar, using threshold reactions that cannot be induced by slow neutrons, such as ⁵⁴Fe(n, p)⁵⁴Mn [4.2].

The effective temperature (or wavelength) is a useful single parameter that has been devised to characterize a neutron beam in the thermal and subthermal energy regions where most analytically useful reactions take place. This basic concept involves measurement of the reaction rate of a thin sample (proportional to the temperature sensitive effective cross-section) and comparison with the total flux incident on a ‘black’ sample [4.3]. One approach involves the adoption of the same element for both samples, negating the need to determine the detector efficiency, but resulting in a large difference in count rate.

When the effects of neutron absorption and scattering can be neglected, the neutron capture rate (R) of a given element in an irradiated sample is proportional to the product of the number of atoms in the beam (N) and the neutron flux (ϕ), defined as the number of neutrons entering the sample per unit area per unit time:

$$R = N\phi\langle\sigma\rangle \quad (4.1)$$

where the effective cross-section ($\langle\sigma\rangle$) is the constant of proportionality.

For a thin sample of area S with a known surface density D atoms/cm² of the target species,

$N = DS$, and therefore the counting rate C for a detection efficiency ε counts per capture is given by the following equation:

$$C_{\text{thin}} = \varepsilon R_{\text{thin}} = \varepsilon S D \phi \langle \sigma \rangle \quad (4.2)$$

However, for a thick ‘black’ sample of the same material, every neutron is captured, and the reaction rate is:

$$C_{\text{thick}} = \varepsilon S \phi \quad (4.3)$$

If the thick and thin samples are identically irradiated (same sample area (S) and capture gamma detection efficiency (ε)), the ratio of counting rates is given by:

$$\frac{C_{\text{thin}}}{C_{\text{thick}}} = \frac{\varepsilon S D \phi \langle \sigma \rangle}{\varepsilon S \phi} \quad (4.4)$$

from which the effective cross-section can be derived:

$$\langle \sigma \rangle = \frac{C_{\text{thin}}}{D C_{\text{thick}}} \quad (4.5)$$

For a $1/v$ absorber for which the cross-section is inversely proportional to the neutron velocity, the effective velocity $\langle v \rangle$ is defined as:

$$\langle v \rangle = v_0 \frac{\sigma_0}{\langle \sigma \rangle} \quad (4.6)$$

where by convention $v_0 = 2200$ m/s. The corresponding effective wavelength is defined as

$$\langle \lambda \rangle = \frac{h}{m \langle v \rangle} \quad (4.7)$$

where h is Planck’s constant and m is the neutron mass. A spreadsheet in which these calculations can be performed is displayed in Table 4.1.

TABLE 4.1. SPREADSHEET FOR NEUTRON BEAM WAVELENGTH MEASUREMENT

Sample	Live time (s)	Clock time (s)	Dead time (%)	Count/s B @478	1 σ uncertainty (%)									
Thick boron	340.4	391.5	13.1	6330.6	0.08									
Thin boron	29 989.6	30 409.8	1.4	5.96	0.84									
Input data		SI units												
Thick source thickness	1.3 mm													
^{10}B content	4.5%													
Density	2.70 g/cm ³													
Thin deposit thickness D	4.83E+16 at. $^{10}\text{B}/\text{cm}^2$													
Angle with beam	45.0°													
Thickness in beam direction	6.83E+16 at. $^{10}\text{B}/\text{cm}^2$													
<i>Results</i>														
$\sigma(\text{eff})$	13.792 b													
$\sigma(\text{eff})/\sigma_0$	3.6													
$v(\text{eff})$	612 m/s													
$\lambda(\text{eff})$	6.5 Å													
$E(\text{eff}) = mv^2/2$	0.0020 eV													
$T(\text{eff}) = E/k$	22.7 K													
Calculated absorption of thick source: 99.9998%														
Calculated absorption of thin source: 9.42E-08 (boron only)														

4.3. ANALYSIS OF THE UNKNOWN SAMPLES

4.3. ANALYSIS OF THE UNKNOWN SAMPLES

Three participants reported measurements of the composition of the unknown mixture of silicate and graphite. Some adjustment was necessary to compare results because the Budapest measurements were forced to add up to 100% and the BARC measurements were normalized to an assumed (and incorrect) iron concentration. Both sets of results were renormalized to the known iron concentration of 5.35%. Table 4.2 summarizes the comparisons. Eight to ten elements were reported: about half of the elements of known concentrations in the mixture (not hydrogen or boron) were measured correctly to within $\pm 25\%$. A weak comparison can be made by taking into account the uncertainties in the measurements (reported by two participants). About a third of the measured concentrations agreed with the expected values to within the stated uncertainties. If the true uncertainties of the expected values had been known and taken into account, this measure of PGAA performance would have been considerably better.

4.4. CROSS-SECTION MEASUREMENTS

A second set of materials was distributed to assist in the resolution of a discrepancy in the

thermal cross-section of carbon. These materials were as follows:

- (a) ≈ 2 g of urea ($(\text{NH}_2)_2\text{CO}$) (NIST SRM 912, 99.7%);
- (b) ≈ 1.2 g of deuterourea ($(\text{ND}_2)_2\text{CO}$) (Aldrich 176087, 98+ at.% deuterium);
- (c) ≈ 2.5 g of melamine ($\text{C}_3\text{N}_3(\text{NH}_2)_3$) (Fisher ACROS 220481, assay $\geq 99\%$);
- (d) Spectroscopic graphite (Union Carbide UCAR L4100, palletizing grade).

No results from these materials have been reported to NIST.

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TABLE 4.2. MEASUREMENTS MADE BY THE DIFFERENT LABORATORIES

Laboratory	BARC	IISC ^a	NIST	SNU	VAEC ^b	Units
Sensitivity	0.031	0.54	6.2	2.0		Counts/s at 1382/mg Ti
Neutron flux		4.3×10^7	8.3×10^8	7.9×10^7		$\text{cm}^{-2} \cdot \text{s}^{-1}$, thermal equivalent
Effective neutron velocity		473	610	2120		m/s
<i>Unknown sample analysis</i>						
Elements reported	8	11		10		
Number within 25%	4	6		5		
Number within stated uncertainty		3		4		

^a IISC: Institute of Isotope and Surface Chemistry (Budapest).

^b VAEC: Vietnam Atomic Energy Commission.

Chapter 5

THERMAL NEUTRON CAPTURE CROSS-SECTIONS AND NEUTRON SEPARATION ENERGIES

R.B. Firestone, S.F. Mughabghab, G.L. Molnár

5.1. INTRODUCTION

Thermal radiative neutron capture cross-sections have been re-evaluated [5.1] as part of an ongoing project at the National Nuclear Data Center at Brookhaven National Laboratory at Upton, New York, to update the Neutron Cross-sections compendia, Vol. 1, Parts A and B, Neutron Resonance Parameters and Thermal Capture Cross-sections, published by Academic Press in 1981 and 1984, respectively [5.2, 5.3]. Neutron separation energies are evaluated as part of an ongoing project at the Atomic Mass Data Center in Orsay, France [5.4]. The adopted data are compared with new results derived from this evaluation.

5.2. THERMAL CROSS-SECTION EVALUATION METHODOLOGY

A brief description of the evaluation procedure is given in the following. As an initial step in the evaluation procedure, CINDA retrievals were carried out on nuclear parameters such as thermal capture, scattering and total cross-sections, as well as coherent scattering amplitudes for measurements since 1979, the cut-off date of the publication of Neutron Cross Sections, Vol. 1, Part A. The search engines of the American Physical Society and Elsevier Science web sites were utilized for the most recent publications that may not be referenced in CINDA.

Since the present evaluated capture cross-sections are applied to test the validity of the k_0 methodology described elsewhere in this report, the capture cross-sections derived by this technique were not included in the present evaluation. As in other previous evaluation studies [5.2, 5.3], various factors were considered in evaluating the thermal capture cross-sections (the cross-sections under consideration were normalized to the recently recommended standard cross-sections (of ^1H , ^{14}N , ^{35}Cl , ^{55}Mn , ^{59}Co , ^{197}Au and ^{235}U):

- (a) Half-lives of the product nuclei, branching ratios and conversion coefficients;
- (b) Measurement accuracy;
- (c) Measurement method, as to whether it is specific or non-specific, such as an absorption measurement by a pile oscillator method as compared with quantification by an activation method;
- (d) Sample characteristics, which include information regarding the isotopic enrichment, impurities, chemistry and sample thickness;
- (e) Measurer's experience and general consistency;
- (f) Characterization of the neutron spectrum;
- (g) Paramagnetic scattering cross-sections of rare earth nuclei in dealing with total cross-sections;
- (h) Accurate total cross-section measurements, from which capture cross-sections can be obtained if the scattering cross-sections are well known.

In some cases, measured reactor capture cross-sections can be converted to 2200 m/s values if the thermal reactor index and the integrals for capture resonances are known.

For low and medium weight nuclides, as well as near-magic nuclides, the direct capture cross-section is computed within the framework of the Lane-Lynn theory [5.5–5.8] following the Mughabghab procedure outlined in Ref. [5.7], and can shed some light on the measured capture cross-section.

In the final step of the evaluation procedure, the contribution of positive energy resonances to the thermal capture cross-section is computed and subsequently compared with measurements. For the majority of nuclides, negative energy resonances are postulated to achieve consistency between calculations and measurements. However, in some cases, the computed thermal capture cross-section can be accounted for in terms of positive energy resonances, such as ^{162}Dy [5.3].

Finally, consistency between the isotopic and elemental cross-sections is sought. Several

iterations in the evaluation procedure may be necessary for this objective to be realized.

5.3. ADOPTED THERMAL NEUTRON CROSS-SECTIONS

The resulting evaluated thermal neutron capture cross-sections for the elements $Z = 1\text{--}92$ are summarized in column 3 of Table 5.1 for 395 naturally abundant isotopes and isomers [5.1–5.3]. The quoted natural abundances, listed in column 2, are representative isotopic compositions (at.%) from the 1997 values of the International Union of Pure and Applied Chemistry (IUPAC) published by Rosman and Taylor [5.9]. The uncertainties of the capture cross-sections evaluated here have been substantially reduced for the following nuclides:

^{14}N , ^{24}Mg , ^{25}Mg , ^{28}Si , ^{29}Si , ^{30}Si , ^{32}S , ^{33}S , ^{36}S , ^{47}Ti , ^{49}Ti , ^{51}V , ^{55}Mn , ^{58}Fe , ^{66}Zn , ^{71}Ga , ^{73}Ge , ^{74}Ge , ^{75}As , ^{79}Br , ^{81}Br , ^{82}Kr , ^{83}Kr , ^{105}Pd , ^{108}Cd , ^{117}Sn , ^{128}Xe , ^{136}Ba , ^{137}Ba , ^{146}Nd , ^{148}Nd , ^{150}Nd , ^{144}Sm , ^{156}Gd , ^{174}Yb , ^{174}Hf , ^{182}W , ^{187}Os , ^{192}Os , ^{190}Pt and ^{232}Th .

Also, in the cases of

^9Be , ^{33}S , ^{36}S , ^{49}Ti , ^{104}Ru , ^{117}Sn , ^{128}Xe , ^{137}Ba , ^{144}Sm , ^{187}Os , ^{192}Os , ^{190}Pt , ^{196}Pt , ^{206}Pb , ^{207}Pb and ^{208}Pb ,

the most recent recommended capture cross-sections [5.1] are not consistent with previous evaluations [5.2, 5.3], lying outside the sum of the uncertainties of previous and present recommendations. Of particular importance is the significant change of the capture cross-section of ^{207}Pb from $0.712 \pm 0.010 \text{ b}$ to $0.620 \pm 0.014 \text{ b}$.

5.4. EXPERIMENTAL THERMAL NEUTRON CROSS-SECTIONS

Thermal neutron cross-sections have been derived from the evaluated gamma ray production cross-sections discussed in Chapter 7, and are shown in column 4 of Table 5.1. These values are derived from the sum of cross-sections for primary gamma rays de-exciting the capture state and/or cross-sections for secondary gamma rays populating the ground state and isomers, as indicated in columns 5 and 6 of Table 5.1, and from selected gamma ray decay cross-sections. The cross-sections for primary gamma rays are typically incomplete

due to large unobserved statistical feedings, except for the light nuclei. Total cross-sections derived from neutron activation decay gamma ray cross-sections as observed in PGAA measurements are expected to be reliable after correction for the neutron irradiation time and decay gamma ray emission probabilities. All other cross-sections may be considered as lower limits, depending on the completeness of the data.

Inspection of the measured cross-sections shows that agreement with the experimentally deduced values is fairly good, especially for light nuclides, and the precision has been improved in many cases. One notable discrepancy is the cross-section for ^{12}C , where the new value of $3.89 \pm 0.06 \text{ mb}$ exceeds the adopted value of $3.53 \pm 0.07 \text{ mb}$ by $11 \pm 3\%$. A summary of the eleven measurements [5.10–5.19] (including French measurements cited in Ref. [5.16]) considered in deriving the adopted value is given in Table 5.2. Four measurements agree with the new value within one standard deviation, and five measurements disagree by more than two standard deviations.

In view of the importance of the carbon cross-section, new experiments were performed at Budapest on four different compounds containing carbon with a well defined stoichiometry to test the accuracy of the new value. These measurements yielded a cross-section of $3.87 \pm 0.05 \text{ mb}$, in excellent agreement with the earlier value. Other recent values deduced from JAERI k_0 factors [5.20, 5.21] are $3.63 \pm 0.13 \text{ mb}$ for their cold neutron guide and $4.01 \pm 0.15 \text{ mb}$ for their thermal neutron guide, which appear to corroborate the new value. All of the measurements discussed in Table 5.2 were performed with external comparator standards and may be susceptible to error due to neutron scattering, so we recommend that the new internally calibrated value should be adopted in the future.

Nitrogen-14 is an important standard for thermal neutron capture cross-section and gamma ray spectra measurements. The measured capture cross-sections for this nuclide [5.17, 5.22, 5.23] are presented in Table 5.3. The adopted value of $79.8 \pm 1.4 \text{ mb}$ [5.1] agrees well with the new value of $79.0 \pm 0.9 \text{ mb}$ from this work. All of the measured values except one of Islam et al. [5.22] agree within their uncertainties. The discrepant value is based on a ^{207}Pb standard that in turn was based on the adopted ^{12}C standard which we have shown to be too low. Adjusting this value to the new ^{12}C measurement gives $76.4 \pm 1.9 \text{ mb}$, which is in reasonable agreement with all the other values.

5.5. NEUTRON SEPARATION ENERGIES

Neutron separation energies (S_n) have been evaluated as part of an ongoing effort at the Atomic Mass Data Center at Orsay [5.4]. The most recent S_n values are shown in column 7 of Table 5.1. The gamma ray energies from this evaluation have undergone least squares fits to the level scheme to derive ‘best’ level energies including S_n for the capture state. The energies are corrected for the nuclear recoil and uncertainties are adjusted for outliers as described in Chapter 6. The new S_n values are shown in column 8 of Table 5.1; agreement is generally good, and greater precision has been achieved in most cases.

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TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION

(Total isotopic (n, γ) cross-sections are shown except when the cross-section populating a specific level or reaction is indicated. The numbers in parentheses are the uncertainties in the final decimal places. The adopted neutron separation energies were calculated from least squares fits of the primary gamma ray energies to the level scheme, and the adopted cross-sections are based on primary, secondary and/or decay gamma ray cross-sections. In many cases the decay scheme may be incomplete, so the adopted cross-sections should be considered as lower limits.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at.%)	σ_γ (mb or b)			S_n (keV)
		Mughabghab [5.1-5.3]	This work	Secondary	
H-1	99.9885(70)	332.6(7) mb	Standard		2224.5725(22)
H-2	0.0115(70)	0.519(7) mb	0.492(25) mb		6257.2482(24)
He-3	0.000137(3)	0.031(9) mb			20577.62
He-4	99.999863(3)				
Li-6	7.59(4)	39(3) mb	52.6(22) mb	52.7(21) mb	52.5(22) mb
$^6\text{Li}(\text{n}, \alpha)$		940(4) b			7249.96(9)
Li-7	92.41(4)	45(3) mb	45.7(9) mb	45.7(9) mb	7249.94(4)
B-9	100	8.8(4) mb	8.8(6) mb	8.8(6) mb	2032.57(4)
B-10	19.9(7)	500(200) mb	303(20) mb	306(16) mb	6812.33(6)
$^{10}\text{B}(\text{n}, \alpha)$		3837(9) b	3820(135) b	298(15) mb	11454.12(20)
B-11	80.1(7)	6(3) mb			11454.15(14)
C-12	98.93(8)	3.53(7) mb	3.89(6) mb	3.90(6) mb	4946.3111(3)
C-13	1.07(8)	1.37(4) mb	1.22(6) mb	1.21(11) mb	8176.61(18)
N-14	99.632(7)	79.8(14) mb	79.0(9) mb	78.8(9) mb	10833.230(10)
$^{14}\text{N}(\text{n}, \text{p})$		1.83(3) b			10833.317(12)
N-15	0.368(7)	24(8) mb			
O-16	99.757(16)	0.190(19) mb	0.177(11) mb	0.194(7) mb	2490.8(23)
O-17	0.038(1)	0.54(7) mb	0.54(11) mb	0.49(7) mb	4143.33(21)
$^{17}\text{O}(\text{n}, \alpha)$		235(10) mb			8044.4(8)
O-18	0.205(14)	0.16(1) mb			8043.5(10)
F-19	100	9.6(5) mb	9.50(11) mb	9.51(14) mb	4143.06(10)
Ne-20	90.48(3)	37(4) mb	36.9(5) mb	37(3) mb	6601.344(16)
Ne-21	0.27(1)	670(110) mb	670(190) mb	580(100) mb	6761.11(4)
					10363.96(23)
					10363.9(4)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Ne-22	9.25(3)	45(6) mb	44(6) mb	44(2) mb	5200.62(12)
Na-23	100	530(5) mb	527(7) mb	527(7) mb	5200.64(17)
Na-23 (472)		400(30) mb	478(4) mb		
Mg-24	78.99(4)	53.6(15) mb	53.7(14) mb	53.9(14) mb	7330.67(4)
Mg-25	10.00(1)	200(5) mb	197(5) mb	192.8(22) mb	11093.157(21)
Mg-26	11.01(3)	38.6(6) mb	37.7(13) mb	38.3(14) mb	6443.35(3)
Al-27	100	231(3) mb	232(3) mb	187.2(17) mb	7725.170(4)
Si-28	92.2297(7)	177(5) mb	186(3) mb	185.2(23) mb	8473.537(23)
Si-29	4.6832(5)	119(3) mb	118(3) mb	120(3) mb	10609.18(3)
Si-30	3.0872(5)	107(2) mb	116(3) mb	117(7) mb	6587.40(5)
P-31	100	172(6) mb	167(5) mb	159.1(22) mb	7935.596(23)
S-32	94.93(31)	548(10) mb	536(8) mb	543(8) mb	8641.809(25)
S-33	0.76(2)	454(25) mb	461(15) mb	383(14) mb	11417.219(16)
S-34	4.29(28)	235(5) mb	277(8) mb	278(19) mb	6986.091(15)
S-36	0.02(1)	230(20) mb	230(25) mb	247(21) mb	4303.58(9)
Cl-35	75.78(4)	43.6(4) b	43.84(17) b	41.89(20) b	8579.672(18)
Cl-37	24.22(4)	430(6) mb	553(23) mb	550(40) mb	6107.78(10)
Ar-36	0.3365(30)	5.2(5) b	5.2(8) b	4.1(7) b	8788.9(4)
Ar-38	0.0632(5)	800(200) mb			8789.9(9)
Ar-40	99.6003(30)	660(10) mb	710(50) mb	660(40) mb	6099.1(4)
K-39	93.2581(44)	2.1(2) b	2.19(3) b	1.737(14) b	7799.558(14)
K-40	0.0117(1)	30(4) b	76(3) b	96(15) b	10095.18(10)
K-41	6.7302(44)	1.46(3) b	1.64(6) b	1.37(5) b	7533.77(15)
Ca-40	96.94(16)	410(20) mb	415(7) mb	378(6) mb	8363.7(3)
Ca-42	0.647(23)	680(70) mb	740(40) mb	670(80) mb	7933.0(3)
Ca-43	0.135(10)	6.2(6) b	7.3(5) b	3.3(2) b	7932.73(16)
					11131.54(18)
					11132.0(7)

TABLE 5.1

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Ca-44	2.09(11)	880(50) mb	1055(25) mb	990(70) mb	7414.8(3)
Ca-46	0.004(3)	720(30) mb	730(70) mb	750(60) mb	7276.1(5)
Ca-48	0.187(21)	1090(70) mb	1050(120) mb	1050(120) mb	5146.6(4)
Sc-45	100	27.2(2) b	26.28(23) b	19.29(24) b	8760.62(11)
Sc-45(143)		9.8(11) b	7.78(11) b		8760.745(20)
Ti-46	8.25(3)	590(180) mb	310(16) mb	229(19) mb	8877.7(10)
Ti-47	7.44(2)	1.52(11) b	1.63(4) b	1.177(11) b	11626.59(4)
Ti-48	73.72(3)	7.88(25) b	8.6(3) b	8.32(16) b	8142.36(5)
Ti-49	5.41(2)	1.79(12) b	1.88(4) b	1.675(18) b	10939.13(4)
Ti-50	5.18(2)	179(3) mb	172(3) mb	142(2) mb	6372.3(9)
V-50	0.250(4)	21(4) b	20.4(8) b	13.5(3) b	11051.142(24)
V-51	99.750(4)	4.92(4) b	5.18(18) b	5.18(18) b	7311.273(15)
Cr-50	4.345(13)	15.9(2) b	15.73(21) b	16.0(5) b	9260.63(8)
Cr-52	83.789(18)	760(60) mb	871(14) mb	855(17) mb	7939.17(16)
Cr-53	9.501(17)	18.2(15) b	19.0(4) b	18.2(6) b	9719.01(25)
Cr-54	2.365(7)	360(40) mb	440(40) mb	390(40) mb	6246.28(17)
Mn-55	100	13.36(5) b	11.33(9) b	11.36(10) b	7270.419(25)
Fe-54	5.845(35)	2.25(18) b	2.44(6) b	2.31(10) b	9297.9(3)
Fe-56	91.754(36)	2.59(14) b	2.49(5) b	2.447(24) b	7646.0954(6)
Fe-57	2.119(10)	2.5(3) b	1.9(5) b	1.5(5) b	10044.5(3)
Fe-58	0.282(4)	1.30(3) b	1.30(5) b	1.20(2) b	6580.90(20)
Co-59	100	37.18(6) b	38.4(3) b	32.4(5) b	7491.93(8)
Co-59(59)		20.4(8) b	20.76(20) b		7492.05(3)
Ni-58	68.077(9)	4.5(2) b	4.36(5) b	4.30(5) b	8999.44(14)
Ni-60	26.223(8)	2.9(2) b	2.42(3) b	2.36(3) b	7820.04(10)
Ni-61	1.1399(6)	2.5(8) b	1.65(12) b	1.28(11) b	10597.2(4)
-	-	-	-	-	10595.6(3)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Ni-62	3.6345(17)	14.5(3) b	14.99(22) b	14.97(22) b	6837.85(7)
Ni-64	0.9256(9)	1.63(7) b	2.2(3) b	2.1(4) b	6098.01(20)
Cu-63	69.17(3)	4.52(2) b	4.75(4) b	4.74(11) b	7915.96(11)
Cu-65	30.83(3)	2.17(3) b	2.134(18) b	1.81(3) b	7065.93(11)
Zn-64	48.6(6)	1100(100) mb	843(20) mb	627(7) mb	7979.28(7)
Zn-66	27.9(3)	620(60) mb	376(6) mb	360(20) mb	7052.2(4)
Zn-67	4.10(13)	9.5(14) b	11.44(14) b	11.44(15) b	10198.2(5)
Zn-68 (0)	18.8(5)	1000(100) mb	790(50) mb	660(40) mb	6482.2(5)
Zn-69 (439)	0.62(3)	72(4) mb	68(9) mb	5834(10)	
Zn-70 (0)	0.62(3)	83(5) mb			
Zn-70 (158)		8.7(5) mb			
Ga-69	60.108(9)	1.68(7) b	1.753(16) b	0.373(11) b	7655.1(8)
Ga-71	39.892(9)	4.73(15) b	4.29(17) b	2.61(4) b	6521.0(10)
Ga-71 (120)		150(50) mb	429(9) mb		
Ge-70	20.8(9)	3.45(16) b	3.69(7) b	3.69(7) b	7415.90(5)
Ge-70 (198)		280(70) mb	400(30) mb		
Ge-72	27.54(34)	950(110) mb	770(80) mb	770(80) mb	6782.90(5)
Ge-72 (67)		460(40) mb			
Ge-73	7.73(5)	14.4(4) b	16.5(3) b	16.5(3) b	7415.925(23)
Ge-74	36.3(7)	530(50) mb	505(10) mb	505(10) mb	6505.22(8)
Ge-75 (140)		170(30) mb	164(5) mb	164(5) mb	6505.45(4)
Ge-76 (0)	7.61(38)	140(20) mb	140(30) mb	140(30) mb	6072.6(11)
Ge-76 (160)		100(10) mb	155(21) mb	155(21) mb	6072.3(4)
As-75	100	4.23(8) b	4.01(5) b	4.01(5) b	7328.44(7)
Se-74	0.89(4)	51.8(12) b	49(3) b	49(3) b	8027.53(8)
Se-76	9.37(29)	85(7) b	84.3(8) b	84.3(8) b	7418.81(7)

TABLE 5.1

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Se-76 (162)		22(1) b	17.2(4) b		
Se-77	7.63(16)	42(4) b	36.3(7) b	18.4(5) b	10498.0(3)
Se-78 (0)	23.77(28)	50(10) mb	98(15) mb	9 mb	6962.9(7)
Se-78 (96)		380(20) mb	135(30) mb		6700.9(5)
Se-80 (0)	49.61(41)	530(50) mb	441(17) mb	280(60) mb	6701.0(6)
Se-80 (103)		80(10) mb	104(7) mb		
Se-82 (0)	8.73(22)	5.2(4) mb			5818(3)
Se-82 (228)		39(3) mb			
Br-79	50.69(7)	10.32(13) b	8.97(14) b	1.035(13) b	7892.19(20)
Br-79 (86)		2.4(6) mb	2.16(6) b		7892.41(8)
Br-81	49.31(7)	2.36(5) b	2.40(10) b	0.50(2) b	7592.90(20)
Br-81 (46)		2.4(4) b	2.32(10) b		7593.017(22)
Kr-78	0.35(2)	4.7(7) b			8355(8)
Kr-78 (130)		170(20) mb			
Kr-80	2.28(6)	11.5(5) b			7872(3)
Kr-80 (190)		4.6(7) b			
Kr-82	11.58(14)	19(4) b			7464(4)
Kr-82 (42)		14.0(25) b			
Kr-83	11.49(6)	202(10) b			
Kr-84	57.00(4)	111(15) mb			
Kr-84 (305)		90(13) mb			
Kr-86	17.3(2)	3(2) mb			
Rb-85 (0)	72.17(2)	427(11) mb	426(7) mb	2.8(4) mb	5515.4(8)
Rb-85 (556)		53(5) mb	57.4(14) mb	94(2) mb	8651.2(10)
Rb-87	27.83(2)	120(30) mb	122(4) mb	44(2) mb	6080(3)
Sr-84	0.56(1)	620(60) mb	630(80) mb	300(50) mb	6082.52(11)
-	-	-	-	-	8529(4)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Sr-84 (239)		600(60) mb	300(50) mb		
Sr-86 (0)	9.86(1)	200(30) mb	124(10) mb	1090(30) mb	910(17) mb
Sr-86 (389)		840(60) mb	970(30) mb		
Sr-87	7.00(1)	17(3) b	15.0(3) b	15.0(3) b	8.31(9) b
Sr-88	82.58(1)	5.8(4) mb	4.1(4) mb	4.1(4) mb	8.9(11) mb
Y-89	100	1.28(2) b	1.282(13) b	1.282(13) b	1.22(4) b
Y-89 (682)		1.0(2) mb	1.8(5) mb		
Zr-90	51.45(40)	11(5) mb	470(40) mb	470(40) mb	5.6(25) mb
Zr-91	11.22(5)	1240(250) mb	1210(40) mb	1210(40) mb	405(21) mb
Zr-92	17.15(8)	220(60) mb	101(5) mb	101(5) mb	46(3) mb
Zr-94	17.38(28)	49.9(24) mb	110(9) mb	110(9) mb	32(4) mb
Zr-96	2.80(9)	22.9(10) mb	920(30) mb	920(30) mb	82(14) mb
Nb-93	100	1.15(5) b	1.138(14) b	1.138(14) b	0.828(8) b
Nb-93 (41)		783(13) mb			7227.631(13)
Mo-92	14.84(35)	19 mb	82(9) mb	82(9) mb	31(4) mb
Mo-94	9.25(12)	15 mb	340(30) mb	340(30) mb	42(4) mb
Mo-95	15.92(13)	13.4(3) b	13.6(4) b	13.6(4) b	2.30(6) b
Mo-96	16.68(2)	500(200) mb	780(40) mb	780(40) mb	220(20) mb
Mo-97	9.55(8)	2.5(2) b	2.20(7) b	2.20(7) b	0.50(11) b
Mo-98	24.13(31)	137(5) mb	160(30) mb	160(30) mb	28 mb
Mo-100	9.63(23)	199(3) mb	150(13) mb	150(13) mb	50(4) mb
Ru-96	5.54(14)	220(20) mb	270(30) mb	270(30) mb	0
Ru-98	1.87(3)	<8 b	>480 mb	480(90) mb	0
Ru-99	12.76(14)	7.1(10) b	13.7(10) b	13.7(10) b	3.03(14) b
Ru-100	12.60(7)	5.0(6) b	0.93(5) mb	0.93(5) mb	0.69(3) b
Ru-101	17.06(2)	3.4(9) b	6.4(5) b	6.4(5) b	1.34(7) b

TABLE 5.1

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Ru-102	31.55(14)	1.21(7) b	2.5(1) mb	0.49(3) b	6232.4(3)
Ru-102 (238)		120(13) mb			6232.00(11)
Ru-104	18.62(27)	470(20) mb	860(40) mb	570(90) mb	5910.07(19)
Rh-103	100	145(2) b	156(5) b	7.69(10) b	6999.05(6)
Rh-103 (129)		10(1) b	9.7(8) b		6998.946(24)
Pd-102	1.02(1)	3.4(3) b	1.11(22) b	0	7625.6(9)
Pd-104	11.14(8)	600(300) mb	373(25) mb	0	7094.1(7)
Pd-105	22.33(8)	21.0(15) b	19.95(18) b	0.55(3) b	9561.5(3)
Pd-106 (0)	17.33(8)	290(30) mb	197(12) mb	44(11) mb	6536.4(5)
Pd-106 (242)		13(2) mb			
Pd-108	26.46(9)	7.6(4) b	7.01(6) b	2.76(9) b	6153.3(3)
Pd-108 (189)		180(30) mb	185(10) mb		6153.54(12)
Pd-110 (0)	11.72(9)	190(30) mb	160(30) mb	175(25) mb	5750(40)
Pd-110 (172)		36(6) mb			5726.3(4)
Ag-107	51.839(8)	37.6(12) b	38.2(5) b	3.08(9) b	7269.6(6)
Ag-107 (109)		330(80) mb	170(40) mb		7271.41(8)
Ag-109 (0)	48.161(8)	86(3) b	78(3) b	10.21(11) b	6808.20(9)
Ag-109 (118)		4.7(2) b	8.82(16) b		
Cd-106	1.25(6)	~1 b			7926(9)
Cd-108	0.89(3)	720(130) mb			7324(6)
Cd-110	12.49(18)	11(1) b			6975.84(19)
Cd-110 (396)		140(50) mb	780(70) mb		6975.1(4)
Cd-111	12.80(12)	24(3) b			9398.1(22)
Cd-112	24.13(21)	2.2(5) b			6540.2(6)
Cd-113	12.22(12)	20600(400) b	19560(250) b	1970(30) b	9042.7(3)
Cd-114 (0)	28.73(42)	300(20) mb			9043.18(6)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Cd-114 (181)		36(7) mb			5777.2(10)
Cd-116 (0)	7.49(18)	50(8) mb			
Cd-116 (136)		25(10) mb			
In-113 (0)	4.29(5)	3.9(4) b	6.2(12) b	15.0(18) b	7274.4(12)
In-113 (190)		8.1(8) b	8.2(13) b		7273.83(23)
In-113 (502)		3.1(7) b	0.63(21) b		
In-115 (0)	95.71(5)	40(2) b	42(3) b	190(7) b	6784.3(8)
In-115 (127)		162.3(7) b	88(4) b		6784.72(17)
In-115 (290)		81(8) b	60(4) b		
Sn-112	0.97(1)	860(90) mb			7742.9(18)
Sn-112 (77)		300(40) mb			
Sn-114	0.66(1)	120(30) mb			7545.7(16)
Sn-115	0.34(1)	30(7) b	58.0(8) b	12.5(4) b	9563.41(11)
Sn-116 (0)	14.54(9)	130(30) mb	154(3) mb	154(3) mb	6944.5(11)
Sn-116 (314)		6(2) mb			6942.9(5)
Sn-117	7.68(7)	1.32(18) b	1.045(18) b	1.045(18) b	9326.3(14)
Sn-118	24.22(9)	220(50) mb	83(3) mb	83(3) mb	6585.2(14)
Sn-118 (90)		10(6) mb			6483.3(6)
Sn-119	8.59(4)	2.2(5) b	1.134(16) b	1.134(16) b	9107.2(22)
Sn-120 (0)	32.58(9)	140(30) mb	118(8) mb	118(8) mb	6170.8(6)
Sn-120 (6)		1(1) mb	1.9(4) mb		6170.1(4)
Sn-122 (0)	4.63(3)	1(1) mb			5946.0(12)
Sn-122 (25)		138(15) mb			
Sn-124 (0)	5.79(5)	4(2) mb	126(4) mb	79(6) mb	0
Sn-124 (28)		130(5) mb	13(2) mb	13(2) mb	0
Sn-121	57.21(5)	5.9(2) b	8.0(11) b	8.0(11) b	6806.6(10)
					6806.36(7)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Sb-121 (164)		60(10) mb	49(10) mb		
Sb-123 (0)	42.79(5)	4.1(1) b	3.14(25) b	0.68(3) b	6467.45(7)
Sb-123 (11)		37(10) mb	740(80) mb		6467.58(5)
Sb-123 (37)		19(10) mb	310(16) mb		
Te-120 (0)	0.09(1)	2.0(3) b			7230(30)
Te-120 (294)		340(60) mb			
Te-122	2.55(12)	3.9(5) b	1.49(9) b	0.88(10) b	6939.4(25)
Te-122 (248)		1.1(5) b	300(30) mb		6929.16(10)
Te-123	0.89(3)	418(30) b	339(18) b	49(2) b	9424.1(12)
Te-124	4.74(14)	6.8(13) b	7.73(25) b	4.18(20) b	6575.9(14)
Te-124 (145)		40(25) mb	770(70) mb		6569.39(14)
Te-125	7.07(15)	1.55(16) b	0.70(7) b	0.70(7) b	9113.8(4)
Te-126 (0)	18.84(25)	900(150) mb	28(7) mb	28(7) mb	6291(3)
Te-126 (88)		135(23) mb			6287.8(4)
Te-128 (0)	31.74(8)	200(8) mb	195(9) mb	195(9) mb	6082.36(14)
Te-128 (106)		15(1) mb	29.0(22) mb		
Te-130 (0)	34.08(62)	270(6) mb	132(10) mb	79(9) mb	5929.7(5)
Te-130 (182)		20(10) mb			5930.16(15)
I-127	100	6.2(2) b	4.4(3) b	0.98(5) b	6826.07(5)
Xe-124	0.09(1)	165(20) b	11(2) b	0	7603.3(4)
Xe-124 (253)		28(5) b	5.0(5) b		
Xe-126	0.09(1)	3.8(5) b			7223(6)
Xe-126 (297)		450(130) mb			
Xe-128	1.92(3)	5.2(13) b	1.23(15) b	0.57(12) b	6907.6(16)
Xe-128 (236)		480(100) mb	190(40) mb		
Xe-129	26.44(24)	21(5) b	7.2(9) b	1.95(14) b	9255.2(9)
					9255.57(23)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Xe-130	4.08(2)	4.8(12) b	0.76(9) b	0.23(6) b	6605.2(19)
Xe-130 (164)		450(100) mb			
Xe-131	21.18(3)	85(10) b	35.7(24) b	10.7(9) b	8936.0(9)
Xe-132	26.89(6)	415(50) mb			6440(4)
Xe-132 (233)		50(10) mb			
Xe-134	10.44(10)	265(20) mb			8548(4)
Xe-134 (527)		3.0(3) mb			
Xe-136	8.87(16)	260(20) mb	130(30) mb	102(16) mb	4025.53(8)
Cs-133	100	30.3(11) b	23.3(7) b	3.58(8) b	6891.540(10)
Cs-133 (139)		2.5(2) b	2.47(4) b		6891.3909(23)
Ba-130 (0)	0.106(1)	8.7(9) b			6493.5(3)
Ba-130 (187)		2.5(3) b			
Ba-132 (0)	0.101(1)	6.5(8) b			7189.9(4)
Ba-132 (288)		500(200) mb			
Ba-134	2.417(18)	1.5(3) b	1.07(4) b	0.457(17) b	6971.97(12)
Ba-134 (268)		158(24) mb	46(3) mb		
Ba-135	6.592(12)	5.8(9) b	4.02(7) b	0.69(6) b	9107.74(4)
Ba-135 (2030)		13.9(7) mb	35(15) mb		9107.73(4)
Ba-136	7.854(24)	680(170) mb	735(24) mb	613(19) mb	6905.76(3)
Ba-136 (662)		10(1) mb	20(4) mb		6905.74(8)
Ba-137	11.232(24)	3.6(2) b	4.06(8) b	2.05(3) b	8611.72(4)
Ba-138	71.698(42)	400(40) mb	435(12) mb	366(10) mb	4723.43(4)
La-138	0.090(1)	57(6) b	57(6) b	10(3) b	8778(3)
La-139	99.910(1)	9.04(4) b	6.13(24) b	5.76(5) b	5161.004(6)
Ce-136 (0)	0.185(2)	6.5(10) b	3.8(4) b	0.070(6) b	7480.7(4)
Ce-136 (254)		950(250) mb	200(60) mb		7481.58(9)

TABLE 5.1

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_j [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_{γ} (mb or b)			S_n (keV)		
		Mughabghab [5.1-5.3]	This work	Secondary	Primary	Audi and Wapstra [5.4]	This work
Ce-138 (0)	0.251(2)	1.00(24) b	6.1(4) b	6.1(4) b	0.87(12) b	7456(12)	
Ce-138 (754)		15(5) mb					
Ce-140	88.450(51)	580(20) mb	284(17) mb	250(10) mb	5428.6(7)	5428.19(6)	
Ce-142	11.114(51)	970(20) mb	732(23) mb	422(20) mb	5145.1(3)	5144.81(6)	
Pr-141	100	11.5(3) b	7.72(15) b	7.72(15) b	3.65(4) b	5843.06(10)	5843.155(5)
Pr-141 (3.7)		3.9(3) b	3.45(13) b				
Nd-142	27.2(5)	18.7(7) b	17.6(15) b	17.6(15) b	7.8(4) b	6123.59(13)	6123.41(7)
Nd-143	12.2(2)	325(10) b	288(19) b	288(19) b	38(2) b	7817.02(7)	7816.94(17)
Nd-144	23.8(3)	3.6(3) b	5.3(3) b	5.3(3) b	2.02(18) b	5755.5(6)	5755.26(22)
Nd-145	8.3(1)	42(2) b	39.9(10) b	39.9(10) b	18.8(6) b	7565.25(14)	7565.05(9)
Nd-146	17.2(3)	1.41(5) b	1.21(11) b	1.21(11) b	0.178(6) b	5292.07(15)	5292.19(4)
Nd-148	5.7(1)	2.58(14) b	1.9(3) b	1.9(3) b	0.37(6) b	5038.68(10)	5038.82(3)
Nd-150	5.6(2)	1.03(8) b	1.8(5) b	1.8(5) b	0.6(1) b	5334.43(20)	5334.552(24)
Sm-144	3.07(7)	1.64(10) b				6757.1(3)	
Sm-147	14.99(18)	57(3) b	67(4) b	67(4) b	338(17) b	8141.5(6)	8141.3(3)
Sm-148	11.24(10)	2.4(6) b				5871.6(9)	
Sm-149	13.82(7)	40140(600) b	37970(150) b	37970(150) b	18223(70) b	7986.7(4)	7986.7(7)
Sm-150	7.38(1)	100(4) b	105(8) b	105(8) b	46(2) b	5596.44(10)	5596.44(6)
Sm-152	26.75(16)	206(6) b	167(10) b	167(10) b	36(2) b	5867.73(23)	5868.40(10)
Sm-154	22.75(29)	8.3(5) b			8.4(9) b	5807.2(3)	
Eu-151 (0)	47.81(3)	5900(200) b	6700(300) b	6700(300) b	243(9) b	6306.72(10)	6307.11(6)
Eu-151 (46)		3300(200) b	4500(2200) b				
Eu-151 (148)		4(2) b					
Eu-153	52.19(3)	312(7) b	387(70) b	387(70) b	18(5) b	6442.0(3)	6442.2(4)
Gd-152	0.20(1)	735(20) b	>370 b	734(30) b	46(3) b	6247.3(3)	6247.48(17)
Gd-154	2.18(3)	85(12) b		85(7) b	17(1) b	6435.1(3)	6435.29(19)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]		
		Mughabghab [5.1-5.3]	This work	Secondary	Primary	Audi and Wapstra [5.4]	This work
Gd-154 (122)		49(15) mb					
Gd-155	14.80(12)	60900(500) b	51700(1800) b	51700(1800) b	8680(400) b	8536.37(12)	8536.04(9)
Gd-156	20.47(9)	1.8(7) b				6360.05(15)	
Gd-157	15.65(2)	254000(800) b	210000(5000) b	210000(5000) b	41000(500) b	7937.33(12)	7937.39(5)
Gd-158	24.84(7)	2.2(2) b				5943.29(15)	
Gd-160	21.86(19)	1.4(3) b				5635.4(10)	
Tb-159	100	23.3(4) b				6375.2(3)	6375.13(7)
Dy-156	0.06(1)	33(3) b				6969(6)	
Dy-158	0.10(1)	43(6) b				6831.5(24)	
Dy-160	2.34(8)	55(3) b	2910(200) b	56(4) b	66(4) b	6454.36(9)	6454.34(6)
Dy-161	18.91(24)	600(25) b	560(15) b	560(15) b	9(2) b	8196.95(12)	8193(3)
Dy-162	25.51(26)	194(10) b	154(6) b	154(6) b	44(4) b	6270.93(7)	6271.14(3)
Dy-163	24.90(16)	134(7) b	68(8) b	68(8) b	5.0(4) b	7655.0(9)	
Dy-164 (0)	28.18(37)	1040(140) b	770(50) b	770(50) b	696(15) b	5715.89(10)	5715.95(3)
Dy-164 (108)		1610(240) b	1514(40) b	1514(40) b			
Ho-165 (0)	100	61.2(11) b	52.8(13) b	54.6(13) b	9.82(14) b	6243.640(20)	6243.677(6)
Ho-165 (6)		3.5(4) b	1.85(11) b				
Er-162	0.14(1)	19(2) b				6903(5)	
Er-164	1.61(3)	13(2) b				6650.0(7)	
Er-166	33.61(35)	16.9(16) b	20.8(14) b	20.8(14) b	9.8(8) b	6436.1(4)	6436.46(18)
Er-166 (208)		15(2) b	11.6(13) b				
Er-167	22.93(17)	649(8) b	688(30) b	688(30) b	271(7) b	7771.07(25)	7771.45(3)
Er-168	26.78(26)	2.74(8) b	17.4(24) b	17.4(24) b	8.3(9) b	6003.1(3)	6003.16(14)
Er-170	14.93(27)	8.85(30) b	5.5(10) b	5.5(10) b	4.0(6) b	5681.5(5)	5681.6(5)
Tm-169	100	92(4) b	110.7(12) b	110.7(12) b	16.2(4) b	6593.3(11)	6591.95(11)
Tm-169 (183)		8.2(17) b	2.3(7) b				

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Yb-168	0.13(1)	2300(170) b	1640(160) b	149(18) b	6867.2(3)
Yb-170	3.04(15)	9.9(18) b	18(3) b	1.8(3) b	6614.8(7)
Yb-171	14.28(57)	58(4) b	50(7) b	3.63(18) b	8019.7(3)
Yb-172	21.83(67)	1.3(8) b	0.92(10) b	0.18(2) b	6367.6(5)
Yb-173	16.13(27)	15.5(15) b	25(3) b	0.97(11) b	7464.60(10)
Yb-174	31.83(92)	63.2(15) b	55(8) b	13.5(21) b	5822.33(12)
Yb-175 (515)			40(8) b		5822.5(4)
Yb-176	12.76(41)	2.85(5) b	0.39(4) b	0.24(3) b	5566.40(19)
Yb-176 (332)			300(30) mb		
Lu-175 (0)	97.41(2)	6.9(13) b	2.71(22) b	23.5(10) b	6287.98(15)
Lu-175 (123)		16.2(5) b	20.8(10) b		6289.78(20)
Lu-176	2.59(2)	2090(70) b	1864(30) b	222(6) b	7072.7(7)
Lu-176 (150)		317(58) b	597(17) b		7072.85(9)
Lu-176 (970)		2.8(7) b			
Hf-174	0.16(1)	549(7) b	411(7) b	72(6) b	6708.7(5)
Hf-176	5.26(7)	24(3) b	24.8(15) b	4.4(8) b	6378.8(15)
Hf-177	18.60(9)	373(10) b	450(30) b	450(30) b	7626.3(3)
Hf-177 (1147)		960(50) mb	790(180) mb		7625.80(16)
Hf-177 (2446)		0.2(1) mb			
Hf-178	27.28(7)	84(4) b	105(5) b	34.9(11) b	6099.03(10)
Hf-178 (375)		53(6) b	69(4) b		6098.946(22)
Hf-179	13.629(6)	41(3) b	39.2(21) b	14.7(8) b	7388.2(4)
Hf-179 (1142)		445(3) mb			7387.85(9)
Hf-180	35.08(16)	13.04(7) b	12.2(13) b	8.9(8) b	5695.7(7)
Ta-180	0.012(2)	563(60) b			7577.0(13)
Ta-181 (0)	99.988(2)	20.5(5) b	9.01(22) b	1.54(3) b	6062.96(16)
					6062.89(6)

CHAPTER 5. NEUTRON CAPTURE CROSS-SECTIONS

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Ta-181 (520)		11(2) mb			
W-180	0.12(1)	<150 b	19.3(18) b	0	6681(6)
W-182	26.50(16)	19.9(2) b	12.6(5) b	4.66(20) b	6190.7(10)
W-182 (309)		88(18) mb			6190.89(3)
W-183	14.31(4)	10.3(2) b	7.21(17) b	4.12(11) b	7411.15(7)
W-184	30.64(2)	1.7(1) b	2.0(4) b	1.58(21) b	5753.7(3)
W-184 (197)		2(1) mb			5754.62(21)
W-186	28.42(19)	38.5(5) b	20.3(3) b	14.21(24) b	5466.59(6)
Re-185	37.40(2)	112(2) b	113(12) b	17.6(5) b	6179.34(13)
Re-187	62.60(2)	76.4(5) b	79(10) b	7.16(24) b	5871.75(6)
Re-187 (172)		2.8(1) b	1.73(18) b		
Os-184	0.02(1)	3000(150) b	4410(60) b	4410(60) b	6625.4(9)
Os-186	1.59(3)	80(13) b	16.4(16) b	3.3(5) b	6292.6(13)
Os-187	1.96(2)	245(40) b	169(3) b	45.9(13) b	7989.58(7)
Os-188	13.24(8)	4.7(5) b	5.5(11) b	2.4(3) b	5922.0(4)
Os-189	16.15(5)	25(4) b	25.1(5) b	4.56(18) b	7792.31(11)
Os-189 (1705)		0.26(3) mb			
Os-190 (0)	26.26(2)	3.9(6) b	0.85(4) b	17.5(11) b	5758.67(16)
Os-190 (74)		9.2(7) b	16.6(11) b	3.11(12) b	5758.81(9)
Os-192	40.78(19)	3.12(16) b	2.69(12) b	2.69(12) b	5585.1(9)
Ir-191 (0)	37.3(2)	309(30) b	630(70) b	1080(70) b	6198.08(20)
Ir-191 (57)		645(32) b	450(20) b		
Ir-191 (155)		160(70) mb			
Ir-193 (112 + y)*	62.7(2)	111(5) b	97(17) b	97(17) b	6066.8(4)
Pt-190		5.8(2) b			6437(6)
		122(4) b			

* y in Ir-193 (112 + y) means that the absolute isotope level energy is not known but is above 112 keV by some value y.

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Pt-192	0.782(7)	10.0(25) b			6255.5(19)
Pt-192 (150)		2.2(8) b			
Pt-194	32.967(99)	580(190) mb	745(25) mb	231(22) mb	6105.06(12)
Pt-194 (259)		98(11) mb	65(4) mb		6109.17(4)
Pt-195	33.832(10)	28.5(12) b	22.37(22) b	8.25(21) b	7921.88(15)
Pt-196 (0)	25.242(41)	410(40) mb	550(40) mb	630(30) mb	7921.92(7)
Pt-196 (400)		44(4) mb			5846.4(3)
Pt-198	7.163(55)	3.66(19) b	2.69(12) b		5846.0(7)
Pt-198 (424)		350(40) mb			
Au-197	100	98.65(9) b	108(5) b	12.8(5) b	6512.17(22)
Hg-196 (0)	0.15(1)	3080(180) b	1240(120) b	578(50) b	6785.4(15)
Hg-196 (299)		109(6) b			
Hg-198	9.97(20)	2.0(3) b			6664.0(6)
Hg-198 (532)		18(4) mb			
Hg-199	16.87(22)	2150(50) b	2215(30) b	1571(14)	8028.26(25)
Hg-200	23.10(19)	<60 b			8028.37(4)
Hg-201	13.18(9)	5.7(12) b	4.9(6) b	2.17(13) b	6230.2(6)
Hg-202	29.86(26)	4.42(7) b			7754.31(23)
Hg-204	6.87(15)	430(100) mb			5992.9(17)
Tl-203	29.524(14)	11.4(2) b	12.09(12) b	10.58(9) b	5668(4)
Tl-205	70.476(14)	104(17) mb	101(3) mb	44(4) mb	6654.88(4)
Pb-204	1.4(1)	660(70) mb	397(11) mb	388(7) mb	6503.7(4)
Pb-206	24.1(1)	26.6(12) mb	29.2(8) mb	29.5(8)	6731.80(9)
Pb-206 (1633)		6.3(13) mb			6737.74(10)
Pb-207	22.1(1)	620(14) mb	622(14) mb	622(14) mb	7367.82(9)
Pb-208	52.4(1)	0.23(3) mb			7367.92(7)
-	-	-	-	-	3935.9(13)

TABLE 5.1. COMPARISON OF ADOPTED NEUTRON CROSS-SECTIONS σ_γ [5.1-5.3] AND NEUTRON SEPARATION ENERGIES S_n [5.4] WITH THE RESULTS OF THIS EVALUATION (cont.)

Isotope and (E), (mode)	Percentage abundance [5.9] (at. %)	σ_γ (mb or b)			S_n (keV) [5.4]
		Mughabghab [5.1-5.3]	This work	Secondary	
Bi-209 (0)	100	24.2(4) mb	21.3(23) mb	61(3) mb	4604.58(13)
Bi-209 (271)		9.6(8) mb	17(6) mb		4604.63(5)
Th-232	100	7.35(3) b	9.5(12) b	0.91(2) b	4786.35(25)
U-234	0.0055(5)	99.8(13) b			4786.34(3)
U-235	0.7200(51)	98.3(8) b	28 b	0.44(6) b	5297.84(23)
U-238	99.274(11)	2.68(19) b	2.34(4) b	2.3(4) b	6544.8(5)
				0.491(12) b	4806.26(21)

TABLE 5.1

TABLE 5.2. COMPARISON OF THERMAL NEUTRON CAPTURE CROSS-SECTION MEASUREMENTS ON ^{12}C WITH THE VALUE ADOPTED BY MUGHABGHAB [5.1] AND THE RESULTS OF THIS EVALUATION

Measurement method	^{12}C cross-section (mb)	Reference
Diffusion length	3.44 ± 0.8	Hendrie et al. [5.10]
Mass spectrometry	3.30 ± 0.15	Henning [5.11]
Pile oscillator	3.5 ± 0.3	Muehlhause et al. [5.12]
Pile oscillator	3.65 ± 0.15	Muehlhause et al. [5.12]
Pile oscillator	3.85 ± 0.15	Koechlin et al. [5.13]
Pulsed neutrons	3.72 ± 0.15	Sagot [5.14]
Pulsed neutrons	3.83 ± 0.06	Starr and Price [5.15]
Reactivity	3.57 ± 0.03	Nichols [5.16]
Capture	3.8 ± 0.4	Jurney and Motz [5.17]
Capture	3.53 ± 0.07	Jurney et al. [5.18]
Capture	3.50 ± 0.16	Prestwich et al. [5.19]
Adopted value	3.53 ± 0.07 mb	Mughabghab [5.1]
This work	3.89 ± 0.06 mb	

TABLE 5.3. NITROGEN THERMAL NEUTRON CAPTURE CROSS-SECTIONS MEASURED BY THE CAPTURE GAMMA RAY LEVEL SCHEME INTENSITY BALANCE

(column 1: comparator standards used; column 2: reported capture cross-sections; column 3: cross-sections renormalized to the new adopted standard value [5.1])

Cross-section, σ_γ			
Standard	Measured (mb)	Renormalized (mb)	Reference
C-12 (3.53 ± 0.07 mb)	79.7 ± 2.4	79.7 ± 2.4	Islam et al. [5.22]
Cl-35 (43.6 ± 0.4 b)	80.1 ± 2.0	80.0 ± 2.0	Islam et al. [5.22]
Pb-207 (712 ± 10 mb)	79.6 ± 1.6	69.3 ± 1.4	Islam et al. [5.22]
Al-27 (230 ± 3 mb)	76.7 ± 2.7	77.0 ± 2.7	Islam et al. [5.23]
Cl-35 (43.6 ± 0.5 b)	79.7 ± 2.4	79.6 ± 2.4	Islam et al. [5.23]
H-1 (332 ± 2 mb)	75.0 ± 7.5	75.1 ± 7.5	Jurney and Motz [5.17]
Adopted value	79.8 ± 1.4 mb		Mughabghab [5.1]
This work	79.0 ± 0.9 mb		

Chapter 6

DATA SOURCES AND EVALUATION METHODOLOGY

R.B. Firestone, G.L. Molnár, Z. Révay

6.1. PROMPT GAMMA RAY SOURCE DATABASES

Four primary databases were used in this evaluation.

6.1.1. The database of Lone et al.

The database of Lone et al. [6.1] was based primarily on measurements of elemental spectra made by Rasmussen et al. [6.2] and Orphan et al. [6.3] using small Ge(Li) detectors. These data were not constrained by nuclear structure information, so that the gamma ray assignments were often unreliable.

6.1.2. The ENSDF database

The Evaluated Nuclear Structure Data File (ENSDF) is a comprehensive nuclear structure and decay database evaluated internationally under the auspices of the IAEA Nuclear Structure and Decay Data Evaluators Network [6.4]. Contained in ENSDF are experimental data compiled from literature sources and organized by isotope, with separate data sets for each reaction type including thermal neutron capture. Intensity data are generally normalized per 100 neutron captures. The primary emphasis of ENSDF evaluations is the determination of nuclear structure properties, i.e. these data sets were not evaluated for use in applications. Capture gamma ray data sets using ENSDF are often intermixed with information from epithermal reactions, and sometimes the gamma ray intensity scale has multiple normalization factors for different energy regions. Updated ENSDF data sets for $A = 1\text{--}44$ and some nuclides with $A > 190$ were provided by Chunmei and Firestone [6.5–6.8]. The primary ENSDF thermal neutron capture gamma ray literature references are listed in Appendix II.

6.1.3. The Reedy and Frankle database

The database of Reedy and Frankle encompasses essentially the same literature as ENSDF for the isotopes of elements from $Z = 1\text{--}30$

[6.9, 6.10]. These data are normalized per 100 neutron captures, but have been carefully evaluated for use in various important applications.

6.1.4. The Budapest database

The largest amount of new data and the only complete source of radiative neutron capture gamma ray cross-sections came from the Institute of Isotope and Surface Chemistry, Budapest, Hungary. Neutron capture reactions on all naturally occurring elements except four noble gases (He, Ne, Ar, Kr), i.e. 79 elements from hydrogen to uranium, were studied on the PGAA guided beam facility for thermal neutrons of the Budapest Research Reactor.

Capture gamma ray spectra were measured with natural targets using a Compton suppression spectrometer [6.11]. All elemental targets were measured together with a chlorine target in order to achieve a consistent energy calibration. The precise energies of two of the peaks from the $^{35}\text{Cl}(n, \gamma)$ reaction [6.12] were used to determine the energies of two distinct peaks, which were then used for the energy calibration of elemental spectra after correction for non-linearity. The accurate new energy and intensity data were sufficient to identify over 13 000 gamma rays from 79 elements. The data for transitions with cross-sections greater than 5% of the largest cross-section for each element are reported in Appendix I, and the complete Budapest measurements are included on the accompanying CD-ROM.

Measurements with composite targets (stoichiometric compounds, mixtures or solutions) yielded accurate normalizing factors, with respect to the $\text{H}(n, \gamma)$ cross-section, by means of internal k_0 standardization [6.13]. Thus, very accurate determinations of the partial gamma ray production cross-sections and related k_0 factors became possible. Energies and k_0 factors for the most important gamma lines have been published [6.14, 6.15], and the data library has been discussed in Refs [6.16–6.18]. Partial cross-sections and k_0 factors for the best lines for each element were remeasured [6.19], often with several targets, and complemented with gamma rays from short lived decay products [6.20], as summarized in Table 6.1.

CHAPTER 6. DATA SOURCES AND EVALUATION METHODOLOGY

TABLE 6.1. PARTIAL GAMMA RAY CROSS-SECTIONS FOR THE ELEMENTS AS MEASURED BY INTERNAL STANDARDIZATION AT THE BUDAPEST THERMAL GUIDE [6.19]

(*Decay gamma rays are denoted by a ‘d’ in the energy column. The numbers in parentheses are the uncertainties in the final decimal places.*)

Z	Element	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
1	H	2223.2590(10)	0.3326(7)
3	Li	2032.300(20)	0.038(1)
4	Be	6809.58(10)	0.0054(5)
5	B	478(3)	713(5)
6	C	1261.71(6)	0.00120(2)
		4945.30(7)	0.00262(3)
7	N	1884.85(3)	0.01458(6)
8	O	870.68(3)	0.000175(8)
9	F	1633.53(3)d	0.0093(3)
11	Na	472.222(13)	0.497(5)
12	Mg	584.936(24)	0.0327(7)
13	Al	1778.92(3)d	0.233(4)
14	Si	3538.98(5)	0.119(2)
15	P	636.570(17)	0.031(1)
16	S	841.013(14)	0.357(7)
17	Cl	1951.150(15)	6.51(4)
19	K	770.325(23)	0.91(2)
20	Ca	1942.68(3)	0.34(1)
21	Sc	584.80(3)	1.83(3)
22	Ti	1381.74(3)	5.18(5)
23	V	1434.10(3)d	5.2(1)
24	Cr	834.80(3)	1.38(2)
25	Mn	846.829(1)d	13.3(2)
26	Fe	7631.05(9)	0.68(1)
27	Co	229.811(12)	7.18(7)
28	Ni	464.972(18)	0.843(9)
29	Cu	277.993(25)	0.893(9)
30	Zn	1077.336(17)	0.358(4)
31	Ga	690.943(24)	0.26(3)
32	Ge	595.879(20)	1.59(4)
33	As	165.09(3)	1.00(1)
34	Se	6600.67(12)	0.57(3)
35	Br	1248.78(12)	0.054(1)
37	Rb	556 + 557	0.132(2)
38	Sr	1836.05(3)	1.02(1)
39	Y	6080.12(7)	0.85(2)
40	Zr	213 + 214	0.125(6)

TABLE 6.1. PARTIAL GAMMA RAY CROSS-SECTIONS FOR THE ELEMENTS AS MEASURED BY INTERNAL STANDARDIZATION AT THE BUDAPEST THERMAL GUIDE [6.19] (cont.)

Z	Element	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
41	Nb	499.48(3)	0.065(5)
42	Mo	778.221(10)	2.04(5)
44	Ru	539.522(11)	1.5(1)
45	Rh	470.41(3)	2.50(7)
46	Pd	616.219(15)	0.638(6)
47	Ag	657.741(22)	1.93(4)
48	Cd	558.32(3)	1866(21)
49	In	5892.38(15)	2.1(2)
50	Sn	1293.53(6)	0.134(2)
51	Sb	921.04(4)	0.086(4)
52	Te	602.723(12)	2.4(2)
53	I	133.59(4)	1.42(5)
54	Xe	667.87(9)	6.9(10)
55	Cs	5505.46(20)	0.306(4)
56	Ba	1435.65(6)	0.308(6)
57	La	567.413(23)	0.333(7)
58	Ce	662.03(5)	0.233(18)
59	Pr	176.95(3)	1.06(2)
60	Nd	696.487(20)	33.2(7)
62	Sm	334.02(5)	4900(60)
63	Eu	89.97(8)	1450(20)
64	Gd	182.12(6)	7680(170)
65	Tb	74.89(8)	0.35(4)
66	Dy	184.34(7)	146(3)
67	Ho	136.67(4)	14.5(7)
68	Er	184.301(25)	57(2)
69	Tm	204.41(5)	8.7(1)
70	Yb	639.73(3)	1.5(1)
71	Lu	150.34(6)	13.7(4)
72	Hf	213+214	1.97(4)
73	Ta	270.48(6)	2.60(4)
74	W	145.74(9)	0.97(2)
75	Re	207.92(4)	4.5(2)
76	Os	186.85(3)	2.08(4)
77	Ir	351.59(5)	2.42(8)
78	Pt	355.54(4)	6.17(5)
79	Au	215.01(3)	7.77(5)
80	Hg	5967.00(10)	53(2)
81	Tl	873.16(8)	0.168(6)
82	Pb	7367.83(12)	0.137(3)

TABLE 6.1. PARTIAL GAMMA RAY CROSS-SECTIONS FOR THE ELEMENTS AS MEASURED BY INTERNAL STANDARDIZATION AT THE BUDAPEST THERMAL GUIDE [6.19] (cont.)

Z	Element	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
83	Bi	319.83(4)	0.017(2)
90	Th	256.25(11)	0.093(4)
92	U	4060.35(5)	0.186(3)

6.2. EVALUATION DATABASES

Two data sets in ENSDF format were created for each isotope, one from the Budapest experimental data and another combining isotopic data from the above sources. The Budapest measurements were elemental, and gamma rays were assigned to an isotope and placed in the level scheme by comparing the energies and relative intensities with those in ENSDF. Additional new placements of gamma rays were determined for some transitions by comparing the experimental data with the ENSDF adopted levels and the Gammas data set. The gamma ray energies and intensities taken from the literature and experimental data sets were then averaged to determine the adopted energies and cross-sections.

The isotopic ENSDF database combines data from the ENSDF and the Reedy and Frankle database, as well as additional references retrieved from the Nuclear Sciences Reference (NSR) file [6.21]. This data set was evaluated further for the consistency of the normalization factors and the completeness of the data. Additional gamma ray branches, internal conversion coefficients and other data were added from the ENSDF adopted levels and the gammas data set.

6.3. ADOPTED GAMMA RAY ENERGIES

Gamma ray energies were determined by a weighted least squares fit of both the isotopic and experimental database gamma ray energies to the level energies. Since the adopted gamma ray

energies are the level energy differences after correction for recoil, weak transitions could be determined to good precision. A chi squared analysis was performed by comparing the input with the adopted data, and the uncertainties of individual outliers with $\chi^2/f > 4$ and/or all data in data sets with $\chi^2/f > 1$ were increased and the fit repeated until $\chi^2/f = 1$. Badly discrepant outliers were discarded, particularly when more accurate data were available. A typical fit of gamma ray energies is shown in Table 6.2 for $^{24}\text{Mg}(n, \gamma)$.

6.4. ADOPTED GAMMA RAY CROSS-SECTIONS

Measured experimental gamma ray intensities were reported as elemental cross-sections, whereas the corresponding literature values were typically compiled for 100 neutron captures per isotope. These data were averaged by one of two methods:

- (1) If a well defined gamma ray cross-section existed in the literature, the gamma ray intensities in the literature data set were renormalized to that value, converted to an elemental cross-section by means of the isotopic abundance [6.22], and averaged with the experimental values.
- (2) If no precise normalization factor existed for most cross-sections, the intensities in the literature data set were renormalized by a factor chosen to minimize the weighted average difference between the literature and experimental intensity data. The renormalized intensities were then averaged with the experimental data to obtain the adopted cross-sections.

A similar chi squared analysis to that described for the energies was performed to deal with outliers and discrepant data. The skew in the chi squared distribution as a function of energy was used to probe systematic differences in the underlying efficiency curves, and discrepant data were adjusted or removed as necessary. A typical fit of gamma ray intensities is shown in Table 6.3 for $^{24}\text{Mg}(n, \gamma)$.

CHAPTER 6. DATA SOURCES AND EVALUATION METHODOLOGY

TABLE 6.2. FIRST ITERATION OF A LEAST SQUARES FIT OF THE GAMMA RAY ENERGIES FOR THE LEVEL SCHEME FOR $^{24}\text{Mg}(n, \gamma)$

(Numbers in parentheses represent the discrepancy in the number to the right, compared with the adopted value, expressed in terms of the number of standard deviations. The uncertainties in each data set were increased and additional iterations were performed until $\chi^2/f = 1$.)

Fitted level energies: ^{24}Mg					
ENSDF			Budapest	Adopted	
	Level 1	Level 2			
1.	0.0			7.	3413.341 23
2.	585.001 16			8.	4276.32 3
3.	974.689 18			9.	4358.2 5
4.	1964.69 9			10.	5116.36 14
5.	2563.32 3			11.	7330.52 3
6.	2801.53 9				
(2)	389.69 5	(1)	389.64 3	389.685 18	3
(2)	585.06 3	(2)	584.936 24	584.994 16	2
(1)	611.8 10			611.80 9	7
(1)	836.95 10		836.75 8	836.82 6	6
(2)	849.9 3		849.93 16	850.01 3	7
(2)	863.09 5	(2)	862.88 4	862.962 23	8
(3)	974.84 5	(1)	974.61 3	974.669 18	3
	989.7 4			989.98 9	4
	1379.7 3		1379.69 19	1379.65 9	4
	1448.7 10			1448.61 9	7
	1474.8 10			1474.74 9	8
	1588.65 9	(1)	1588.40 9	1588.58 3	5
	1702.6 7			1702.96 14	10
	1713.05	(1)	1712.85 6	1712.94 3	8
	1964.7 4		1964.63 25	1964.61 9	4
	1978.25 5	(1)	1978.14 8	1978.24 3	5
	2213.8 5		2214.29 25	2214.05 14	11
	2216.5 6		2216.8 4	2216.42 9	6
(1)	2438.48 4	(1)	2438.42 9	2438.524 22	7
	2553.7 8			2552.90 14	10
	2563.6 5			2563.18 3	5
(1)	2801.0 3		2801.5 4	2801.36 9	6
(1)	2828.21 4		2828.12 10	2828.168 22	7
	2972.4 8			2972.2 5	11
	3053.99 4	(1)	3053.85 12	3054.00 3	8
	3301.42 5		3301.29 13	3301.40 3	3
(1)	3413.15 5		3413.04 14	3413.091 23	7
	3691.07		3690.98 18	3691.03 3	8
	3916.86 4	(1)	3916.65 16	3916.85 3	11
	4141.4 3		4141.38 24	4141.31 14	10
			4357.9 6	4357.8 5	9
	4528.47		4528.66 22	4528.55 9	11
					6

6.4. ADOPTED GAMMA RAY CROSS-SECTIONS

TABLE 6.2. FIRST ITERATION OF A LEAST SQUARES FIT OF THE GAMMA RAY ENERGIES FOR THE LEVEL SCHEME FOR $^{24}\text{Mg}(\text{n}, \gamma)$ (cont.)

ENSDF	Budapest	Adopted	Level 1	Level 2
4766.86 23	4766.68 25	4766.71 4	11	5
6355.02	6354.9 3	6354.96 3	11	3
(1) 6744.9 3		6744.54 3	11	2
(1) 7330.6 9		7329.37 3	11	1

Note: ENSDF: $\chi^2/f = 1.561, f = 25$; Budapest: $\chi^2/f = 1.907, f = 17$.

Total $\chi^2/f = 1.429$ (fit of 61 gamma transitions to ten levels).

TABLE 6.3. FIRST ITERATION OF A LEAST SQUARES FIT OF GAMMA RAY INTENSITIES FOR $^{24}\text{Mg}(\text{n}, \gamma)$

(Numbers between asterisks represent the discrepancy in the data to the left expressed in terms of the number of standard deviations. The uncertainties in each data set were increased, and additional iterations were performed until $\chi^2/f = 1$. The fitted cross-sections from the Budapest reactor measurements were adopted.)

E_γ	I_γ (ENSDF) ^a		σ_γ (Budapest) ^b		Relative I_γ
	Input	Fit	Input	Fit	
389.670 21	7.5 4	7.4 3	0.0058 3	0.00585 24 ^c	18.3 7
585.00 3	39.8 12	39.9 11	0.0316 15	0.0314 11	98.1 25
611.81 9	0.015 15	0.015 15		1.2E-05 12	0.04 4
836.83 6	0.21 3	0.200 19	1.52E-04 18	1.57E-04 15 ^d	0.49 5
849.99 4	0.070 20	0.084 14	7.2E-05 15	6.6E-05 11	0.21 4
862.96 3	0.48 5	0.52 3	0.000420 25	0.000410 21	1.28 7
974.66 3	8.3 4	8.4 3	0.0067 3	0.00662 24	20.7 7
989.99 10	0.050 10	0.050 10		3.9E-05 8	0.123 25
1379.64 9	0.100 20	0.107 14	8.8E-05 14	8.4E-05 11	0.26 3
1448.62 10	0.015 15	0.015 15		1.2E-05 12	0.04 4
1474.75 10	0.015 15	0.015 15		1.2E-05 12	0.04 4
1588.61 4	0.37 4	0.316 22*1*	2.22E-04 19	2.49E-04 17*1*	0.78 5
1702.95 15	0.040 10	0.040 10		3.1E-05 10	0.098 25
1712.92 4	1.5 3	1.50 10	0.00118 7	0.00118 7	3.69 21
1964.61 10	0.060 20	0.092 18*1*	8.5E-05 20	7.2E-05 14	0.23 4
1978.25 3	1.42 11	1.41 7	0.00110 6	0.00111 5	3.46 15
2214.06 15	0.40 5	0.36 4	2.3E-04 4	0.00029 3*1*	0.89 9
2216.42 9	0.25 4	0.22 3	1.3E-04 3	1.75E-04 23*1*	0.55 7
2438.54 3	6.3 4	6.0 3	0.00459 22	0.00472 19	14.8 6
2552.88 15	0.030 10	0.030 10		2.4E-05 9	0.074 25
2563.21 4	0.070 20	0.070 20		5.5E-05 16	0.17 5
2801.37 9	0.170 20	0.158 17	8.2E-05 20	1.24E-04 14*2*	0.39 4
2828.172 25	30.5 10	30.5 9	0.0239 11	0.0240 8	74.9 20
2972.2 5	0.090 20	0.090 20		7.1E-05 17	0.22 5
3054.00 3	10.4 5	10.5 4	0.0083 4	0.0082 3	25.8 9
3301.41 3	7.7 4	7.9 3	0.0063 3	0.00619 24	19.3 7
3413.10 3	5.1 3	5.09 21	0.00400 20	0.00400 16	12.5 5

For footnotes see end of table.

TABLE 6.3. FIRST ITERATION OF A LEAST SQUARES FIT OF GAMMA RAY INTENSITIES FOR $^{24}\text{Mg}(\text{n}, \gamma)$ (cont.)

E_γ	I_γ (ENSDF) ^a		σ_γ (Budapest) ^b		Relative I_γ
	Input	Fit	Input	Fit	
3691.02 3	0.90 8	0.86 5	0.00065 5	0.00067 4	2.11 12
3916.84 3	41.0 13	40.7 11	0.0314 15	0.0320 11	100 3
4141.31 14	0.21 3	0.195 20	1.42E-04 20	1.53E-04 16	0.48 5
4528.55 9	0.46 4	0.44 3	0.00029 5	0.00035 3*1*	1.09 8
4766.69 4	0.41 4	0.42 3	0.00033 3	0.000326 22	1.02 7
6354.98 3	1.31 9	1.35 7	0.00109 8	0.00106 6	3.31 17
6744.54 3	0.18 3	0.18 3		1.42E-04 25	0.44 7
7329.38 4	0.018 4	0.018 4		1.4E-05 3	0.044 10

^a ENSDF: $\chi^2/f = 0.266$, skew = -0.214, $f = 35$.^b Budapest: $\chi^2/f = 0.595$, skew = -1.780, $f = 25$.^c 0.00585 24 means 0.00585 ± 0.00024 .^d 1.57E-04 15 means $(1.57 \pm 0.15)\text{E-04}$.

Gamma ray intensity balances through the level scheme were used to determine the quality and completeness of the evaluated data. The total gamma ray cross-section feeding the ground state was compared with the corresponding values from Mughabghab et al. [6.23–6.25], and the ratio of the total primary gamma ray cross-section to the cross-section feeding the ground state indicated the completeness of the data set. The intensity balances

through intermediary levels indicate missing or anomalous intensities; such problems were corrected whenever possible. An example of an intensity balance analysis with no important discrepancies is shown in Table 6.4. Level schemes are complete for the more abundant isotopes of the light nuclei, but significant inconsistencies in the intensity balance may arise for heavier nuclei and those in the continuum remain unresolved.

TABLE 6.4. CROSS-SECTION BALANCE FOR $^{24}\text{Mg}(\text{n}, \gamma)$ ADOPTED DATA
(The numbers in parentheses are the uncertainties in the final decimal places.)

E (level)	σ (in)	σ (out)	$\Delta\sigma$
0	0.0536(14)	0.0	0
585.01(3)	0.0406(11)	0.0398(14)	0.0008(18)
974.68(3)	0.0157(4)	0.0158(4)	0.0001(6)
1964.69(10)	0.00022(2)	0.00026(3)	0.00004(4)
2563.35(4)	0.00202(10)	0.00179(7)	0.00023(12)
2801.54(9)	0.00047(4)	0.00061(5)	0.00013(6)
3413.35(3)	0.0411(14)	0.0416(11)	0.0005(18)
4276.33(4)	0.0105(4)	0.0107(3)	0.0002(5)
4358.2(5)	0.00009(2)	0.0	0.00009(2)
5116.37(15)	0.00038(4)	0.00027(3)	0.00011(5)
7330.53(4)	0.0	0.0539(14)	0.0539(14)

Note: σ (Mughabghab [6.23]) = 0.0536(15) b; σ (measured, average) = 0.0538(14) b.

6.5. RADIOACTIVE DECAY DATA

6.5. RADIOACTIVE DECAY DATA

Gamma rays emitted by radioactive decay from isomers and activation products were observed simultaneously with the prompt gamma rays and have been included in this evaluation. Decay data were taken from the relevant ENSDF data sets and renormalized using the total cross-sections from Mughabghab et al. [6.23–6.25], other literature or the Budapest experimental data (only used when corrections for bombardment time were negligible). These data must be corrected for decay and saturation as described in Chapter 7.

Several naturally abundant isotopes emit gamma rays that can be used for quantitative analysis. Data are included for ^{40}K (half-life: 1.265×10^9 a), ^{50}V (1.4×10^{14} a), ^{138}La (1.05×10^{11} a), ^{176}Lu (4.00×10^{10} a), ^{232}Th (1.405×10^{10} a) and ^{235}U (7.038×10^8 a). These gamma ray intensities are provided in units of disintegrations per second per gram of the element.

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Chapter 7

ADOPTED DATABASE AND USER TABLES

R.B. Firestone

7.1. INTRODUCTION

The Evaluated Gamma Ray Activation File (EGAF) is a database of $\approx 32\,000$ adopted prompt gamma rays and ≈ 3000 gamma rays emitted by radioactive decay, and has been created for all stable isotopes of the elements from hydrogen to uranium. This complete EGAF database is available on the CD-ROM accompanying this book, in both tabulated and ENSDF format [7.1]. Selected gamma rays with partial cross-sections greater than 1% of the most intense transitions are presented in Tables 7.1–7.4, in which at least one prompt gamma ray and at least one decay gamma ray (when applicable) are listed for each isotope regardless of intensity. Gamma rays ordered by energy are given for each element with isotopic identification, energy and uncertainty in kiloelectronvolts, as well as partial elemental cross-sections and k_0 , and their uncertainties.

7.2. PRESENTATION OF NUMERICAL UNCERTAINTY

Uncertainties in the tables are contained within parentheses, and expressed in terms of the last digit or digits of the recommended value without a decimal point. These uncertainties are defined as standard deviations corresponding to the 1σ confidence level, for example:

$$1234.5(12) \equiv 1234.5 \pm 1.2$$

$$1.234(5) \equiv 1.234 \pm 0.005$$

$$1.23(4) \times 10^{-5} \equiv (1.23 \pm 0.04) \times 10^{-5}$$

7.3. ISOTOPIC DATA

The isotopic data are presented in Table 7.1. The first three columns give the atomic number Z , element symbol El. and mass number A , respectively. The natural abundances (θ) quoted in column 4 are representative isotopic compositions

(at.%) from the 1997 IUPAC values listed by Rosman and Taylor [7.2]. Thermal radiative cross-sections (σ_γ) are listed in column 5 and discussed in Chapter 5 [7.3–7.5], while Trkov calculated the Westcott g factors for 293 K as listed in column 6 [7.6]. The number of prompt gamma rays reported for each isotope (N_γ) is given in column 7, and the most intense prompt capture gamma rays for some elements are quantified in column 8.

7.4. RADIOACTIVE DECAY DATA

Gamma rays emitted by the radioactive decay of isomers and activation products are observed simultaneously with the prompt gamma rays and have been included in this evaluation. Decay data were taken from the ENSDF file and renormalized to the total radiative cross-sections of Mughabghab [7.3–7.5] or to Budapest experimental data if corrections for the bombardment time were negligible. Radioactive decay data are presented in Table 7.2. The first column gives the mass number A and element symbol El. The decay mode is given in column 2 and the half-life in column 3. Column 4 indicates the percentage branching ratio (%BR) for the indicated decay mode, and column 5 gives the number of decay gamma rays (N_γ) reported for each parent and decay mode. Column 6 shows the energies E_γ and partial elemental gamma ray cross-sections $\sigma_\gamma^Z(E_\gamma)$ for the principal decay gamma rays. The naturally abundant radioisotopes ^{40}K , ^{50}V , ^{138}La , ^{176}Lu , ^{232}Th and ^{235}U are indicated by '(nat)' next to the element symbol, and the principal decay gamma ray activity in disintegrations per second per gram of the element is shown instead of the partial elemental gamma ray cross-section $\sigma_\gamma^Z(E_\gamma)$.

7.5. THE k_0 FORMULATION

The k_0 formulation is commonly used in activation analysis because the product of the yield and cross-section can usually be measured with greater accuracy than either parameter alone. A value of k_0 for a gamma ray emitted from isotope i

is defined relative to the hydrogen standard on a mass scale:

$$\begin{aligned} k_0(E_\gamma) &= k_Z(E_\gamma)/k_{\text{H}}(2223) \\ &= [\sigma_\gamma^Z(E_\gamma)/A_r(Z)]/[\sigma_\gamma^{\text{H}}(2223)/A_r(\text{H})] \\ &= 3.03 \times [\sigma_\gamma^Z(E_\gamma)/A_r(Z)] \end{aligned}$$

where $\sigma_\gamma^Z(E_\gamma)$ is the partial elemental cross-section in barns for the production of gamma rays of energy E_γ from element Z , assuming the natural abundance, and $A_r(Z)$ is the relative atomic weight of element Z . The partial elemental cross-section for neutron capture by hydrogen is $\sigma_\gamma^{\text{H}}(2223) = 0.3326(7)$ and $A_r(\text{H}) = 1.00794$, while $k_0(2223) \equiv 1$ by definition. For example, consider the 841.0 keV gamma ray from $^{32}\text{S}(\text{n}, \gamma)$ with $\sigma(841) = 0.347$ b and $A_r(\text{S}) = 32.066$:

$$k_0(841) = 3.03 \times 0.347/32.066 = 0.0328$$

7.6. PGAA DATA TABLES

The adopted PGAA database of prompt and delayed gamma rays is presented in Table 7.3.

7.6.1. Prompt gamma rays

Only k_0 values that are greater than 1% of the largest value for each element are listed in Table 7.3, while those that are greater than 10% are shown in bold type. Gamma rays with k_0 less than 1% of the largest value are included in the full database on the CD-ROM. Both the $\sigma_\gamma^Z(E_\gamma)$ and $k_0(E_\gamma)$ values presented in this evaluation have the same percentage uncertainties because they are measured with respect to the very precise hydrogen value.

The 477.6 keV gamma ray from the $^{10}\text{B}(\text{n}, \alpha)$ reaction is uniquely identified in Table 7.3 because this emission undergoes Doppler broadening to a width of ≈ 15 keV.

The IUPAC atomic weight values [7.7] were used in the calculation of k_0 , and the elemental cross-sections are shown for each element in the column heading in Table 7.3.

7.6.2. Radioactive decay gamma rays

Gamma rays from radioactive decay are denoted in Table 7.3 by a ‘d’ immediately after the energy and uncertainty. Saturation values for k_0 are

listed, but many half-lives are too long for saturation to occur under normal experimental conditions. Per cent saturation has been calculated, assuming irradiation for one hour:

$$\% \text{ Saturation} = 100 \times [1.0 - (1.0 - e^{-\lambda t})/\lambda t]$$

where $\lambda = (\ln 2)/t_{1/2}$ and $t = 3600$ s. The values of percent saturation are given in parentheses after the $k_0(E_\gamma)$ decay values in Table 7.3. Only decay gamma rays with $k_0(E_\gamma) > 10\%$ of the largest k_0 values or the most intense gamma ray are listed in Table 7.3.

Gamma rays from several naturally abundant radioisotopes are included in Table 7.3 and indicated as ‘abundant’ in the k_0 column. Instead of k_0 and $\sigma_\gamma^Z(E_\gamma)$, the gamma emission rate per second per gram of the element is given as calculated by:

$$\text{Gamma emission rate } (\text{s}^{-1} \cdot \text{g}^{-1})$$

$$= \lambda N P_\gamma$$

$$= [(\ln 2)/t_{1/2}] \times [N_A/A_r(Z)] \times \theta \times P_\gamma$$

where $t_{1/2}$ is the half-life, $N_A = 6.022 \times 10^{23}$ mol⁻¹, θ is the isotopic abundance (at.%) and P_γ is the absolute gamma ray intensity per decay.

7.6.3. Energy ordered gamma ray table

Table 7.4 presents a list of energy ordered gamma rays with $\sigma_\gamma^Z(E_\gamma)$ and $k_0(E_\gamma)$ values, as well as the most intense gamma rays associated with these transitions. This table has been abbreviated to include only those gamma rays with $k_0(E_\gamma) > 10\%$ of the largest value for each element (giving a total of about 1300 transitions). Radioactive decay transitions are also included, and have been appended with a ‘d’ immediately after the gamma ray energy and uncertainty.

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TABLE 7.1. ISOTOPIC DATA

(Abundances are from Rosman and Taylor [7.2], σ_γ from Mughabab et al. [7.3–7.5] and g factors from Trkov [7.6]. The number of prompt gamma rays (N_γ) reported for each isotope and the most intense gamma rays for each element are also shown.)

Z	El.	A	Abundance (at.%)	σ_γ (total)	g(293 K)	N_γ	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
1	H	1	99.9885(70)	0.3326(7)	0.999	1	2223.24835(0.3326)
	H	2	0.0115(70)	0.000519(7)	1.000	1	
2	He	3	0.000137(3)	0.000031(9)	1.000	1	
	He	4	99.999863(3)	0	1.000	0	
3	Li	6	7.59(4)	0.039(4)	1.000	3	
	Li	7	92.41(4)	0.045(3)	1.000	3	2032.30(0.0381), 980.53(0.00415), 1051.90(0.00414)
4	Be	9	100	0.0088(4)	1.000	13	6809.61(0.0058), 3367.448(0.00285), 853.630(0.00208)
	B	10	19.9(7)	0.5(1)	1.000	10	477.595(716)
5	B	11	80.1(7)	0.005(3)	1.000	0	
	C	12	98.93(8)	0.00353(5)	1.000	6	4945.301(0.00261), 1261.765(0.00124), 3683.920(0.00122)
6	C	13	1.07(8)	0.00137(4)	0.998	7	
	N	14	99.632(7)	0.0798(14)	1.000	60	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
7	N	15	0.368(7)	0.000024(8)	1.003	12	
	O	16	99.757(16)	0.000190(19)	1.000	4	
8	O	17	0.038(1)	0.00054(7)	0.999	20	
	O	18	0.205(14)	0.00016(1)	1.000	13	
9	F	19	100	0.0096(5)	1.000	168	1633.53(0.0096)d, 583.561(0.00356), 656.006(0.00197)
10	Ne	20	90.48(3)	0.037(4)	1.000	27	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
	Ne	21	0.27(1)	0.67(11)	1.000	11	
	Ne	22	9.25(3)	0.045(6)	1.000	15	1979.89(0.00306), 1017.00(0.0030)
11	Na	23	100	0.530(5)	1.000	240	1368.66(0.530)d, 2754.13(0.530)d, 472.202(0.478)d
12	Mg	24	78.99(4)	0.0536(15)	1.001	35	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
	Mg	25	10.00(1)	0.200(5)	1.001	206	1808.668(0.0180), 1129.575(0.00891), 3831.480(0.00418)
	Mg	26	11.01(3)	0.0386(6)	1.001	44	
13	Al	27	100	0.231(3)	1.000	216	1778.92(0.232)d, 30.6380(0.0798), 7724.027(0.0493)
14	Si	28	92.2297(7)	0.177(5)	1.001	46	3538.966(0.1190), 4933.889(0.1120), 2092.902(0.0331)
	Si	29	4.6832(5)	0.119(3)	1.003	99	
	Si	30	3.0872(5)	0.107(2)	1.007	39	
15	P	31	100	0.172(6)	1.001	158	512.646(0.079), 78.083(0.059), 636.663(0.0311)
16	S	32	94.93(31)	0.548(10)	1.000	101	840.993(0.347), 5420.574(0.308), 2379.661(0.208)
	S	33	0.76(2)	0.454(25)	1.001	249	
	S	34	4.29(28)	0.235(5)	1.001	55	
	S	36	0.02(1)	0.23(2)	1.014	22	
17	Cl	35	75.78(4)	43.5(4)	1.000	384	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
	Cl	37	24.22(4)	0.430(6)	1.000	71	
18	Ar	36	0.3365(30)	5.2(5)	1.016	10	
	Ar	38	0.0632(5)	0.8(2)	1.040	0	
	Ar	40	99.6003(30)	0.66(1)	1.002	40	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
19	K	39	93.2581(44)	2.1(2)	1.001	308	29.8300(1.380), 770.3050(0.903), 1158.887(0.1600)

TABLE 7.1

TABLE 7.1. ISOTOPIC DATA (cont.)

<i>Z</i>	El.	<i>A</i>	Abundance (at.%)	σ_γ (total)	<i>g</i> (293 K)	<i>N</i> _{γ}	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
20	K	40	0.0117(1)	30(4)	1.000	490	
	K	41	6.7302(44)	1.45(3)	1.001	638	
	Ca	40	96.94(16)	0.41(2)	1.001	49	1942.67(0.352), 6419.59(0.176), 4418.52(0.0708)
	Ca	42	0.647(23)	0.68(7)	1.001	44	
	Ca	43	0.135(10)	6.2(6)	1.001	129	
	Ca	44	2.09(11)	0.88(5)	1.001	41	
	Ca	46	0.004(3)	0.72(3)	1.000	10	
21	Ca	48	0.187(21)	1.09(14)	1.001	15	
	Sc	45	100	27.2(2)	1.002	440	227.773(7.13), 147.011(6.08), 142.528(4.88)d
	Ti	46	8.25(3)	0.59(18)	1.001	23	
	Ti	47	7.44(2)	1.52(11)	1.001	175	
	Ti	48	73.72(3)	7.88(25)	1.002	92	1381.745(5.18), 6760.084(2.97), 6418.426(1.96)
	Ti	49	5.41(2)	1.79(12)	1.001	88	
	Ti	50	5.18(2)	0.179(3)	1.001	19	
23	V	50	0.250(4)	21(4)	0.999	328	
	V	51	99.750(4)	4.92(4)	1.001	309	1434.10(4.81)d, 125.082(1.61), 6517.282(0.78)
	Cr	50	4.345(13)	15.9(2)	1.000	64	749.09(0.569), 8510.77(0.233), 8482.80(0.169)
	Cr	52	83.789(18)	0.76(6)	1.000	16	7938.46(0.424)
	Cr	53	9.501(17)	18.2(15)	1.000	90	834.849(1.38), 8884.36(0.78), 9719.06(0.260)
	Cr	54	2.365(7)	0.36(4)	1.000	38	
	Mn	55	100	13.36(5)	1.000	126	846.754(13.10)d, 1810.72(3.62)d, 26.560(3.42)
26	Fe	54	5.845(35)	2.25(18)	1.001	33	9297.68(0.0747)
	Fe	56	91.754(36)	2.59(14)	1.000	193	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
	Fe	57	2.119(10)	2.5(3)	1.001	35	
	Fe	58	0.282(4)	1.30(3)	1.002	67	
	Co	59	100	37.18(6)	1.000	340	229.879(7.18), 277.161(6.77), 555.972(5.76)
	Ni	58	68.0769(89)	4.5(2)	1.000	236	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
	Ni	60	26.2231(77)	2.9(2)	1.000	137	7819.517(0.336), 282.917(0.211), 7536.637(0.190)
28	Ni	61	1.1399(6)	2.5(8)	1.000	64	
	Ni	62	3.6345(17)	14.5(3)	1.000	53	6837.50(0.458)
	Ni	64	0.9256(9)	1.63(7)	1.000	35	
	Cu	63	69.17(3)	4.52(2)	1.001	306	278.250(0.893), 7915.62(0.869), 159.281(0.648)
	Cu	65	30.83(3)	2.17(3)	1.002	350	185.96(0.244), 465.14(0.1350), 385.77(0.1310)
	Zn	64	48.63(60)	1.1(1)	1.001	78	115.225(0.167), 7863.55(0.1410), 855.69(0.066)
	Zn	66	27.90(27)	0.62(6)	1.000	17	6958.8(0.043)
30	Zn	67	4.10(13)	9.5(14)	1.000	175	1077.335(0.356), 1883.12(0.0718), 1340.14(0.0457)
	Zn	68	18.75(51)	1.07(10)	1.000	33	1007.809(0.056), 5474.02(0.042), 834.77(0.037)
	Zn	70	0.62(3)	0.091(5)	1.000	79	
	Ga	69	60.108(9)	1.68(7)	1.000	68	508.19(0.349), 690.943(0.305), 187.84(0.1080)
	Ga	71	39.892(9)	4.73(15)	1.001	245	834.08(1.65)d, 2201.91(0.52)d, 629.96(0.490)d
	Ge	70	20.84(87)	3.45(16)	1.000	84	175.05(0.164), 499.87(0.162)
	Ge	72	27.54(34)	0.95(11)	1.000	48	

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.1. ISOTOPIC DATA (cont.)

Z	El.	A	Abundance (at.%)	σ_γ (total)	g(293 K)	N_γ	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
	Ge	73	7.73(5)	14.4(4)	1.000	603	595.851(1.100), 867.899(0.553), 608.353(0.250)
	Ge	74	36.28(73)	0.53(5)	1.000	47	
	Ge	76	7.61(38)	0.14(2)	1.000	196	
33	As	75	100	4.23(8)	1.000	348	559.10(2.00)d, 165.0490(0.996), 86.7880(0.579)
34	Se	74	0.89(4)	51.8(12)	1.001	142	286.5710(0.280)
	Se	76	9.37(29)	85(7)	1.000	456	238.9980(2.06), 520.6370(1.260), 161.9220(0.855)d
	Se	77	7.63(16)	42(4)	1.000	215	613.724(2.14), 694.914(0.443), 1308.632(0.317)
	Se	78	23.77(28)	0.430(22)	1.000	37	
	Se	80	49.61(41)	0.61(5)	1.000	71	
	Se	82	8.73(22)	0.044(3)	1.000	0	
35	Br	79	50.69(7)	10.32(13)	1.000	257	245.203(0.80), 271.374(0.462), 314.982(0.460)
	Br	81	49.31(7)	2.36(5)	1.000	181	776.517(0.990)d, 554.3480(0.838)d, 619.106(0.515)d
36	Kr	78	0.35(1)	4.7(7)	1.000	1	
	Kr	80	2.28(6)	11.5(5)	1.000	1	
	Kr	82	11.58(14)	19(4)	1.000	2	
	Kr	83	11.49(6)	202(10)	0.995	75	881.74(20.8), 1213.42(8.28), 1463.86(7.10)
	Kr	84	57.00(4)	0.111(15)	1.000	7	
	Kr	86	17.30(22)	0.003(2)	1.000	38	
37	Rb	85	72.17(2)	0.48(9)	1.000	90	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)d
	Rb	87	27.83(2)	0.12(3)	1.000	86	196.34(0.00964)
38	Sr	84	0.56(1)	0.62(6)	1.000	5	
	Sr	86	9.86(1)	1.04(7)	1.000	375	
	Sr	87	7.00(1)	17(3)	1.006	210	1836.067(1.030), 898.055(0.702), 850.657(0.275)
	Sr	88	82.58(1)	0.0058(4)	1.000	57	
39	Y	89	100	1.28(2)	1.005	397	6080.171(0.76), 776.613(0.659), 202.53(0.289)
40	Zr	90	51.45(40)	0.011(5)	1.000	15	1465.7(0.037), 1205.6(0.025), 2042.2(0.019)
	Zr	91	11.22(5)	1.24(25)	1.000	81	934.4640(0.0737), 1405.159(0.0178), 560.958(0.0169)
	Zr	92	17.15(8)	0.22(6)	1.000	18	
	Zr	94	17.38(28)	0.0499(24)	1.000	14	
	Zr	96	2.80(9)	0.020(1)	1.000	34	1102.67(0.0139)
41	Nb	93	100	1.15(5)	1.002	535	99.4070(0.211), 255.9290(0.190), 253.115(0.1420)
42	Mo	92	14.84(35)	0.019	1.000	5	
	Mo	94	9.25(12)	0.015	1.001	13	
	Mo	95	15.92(13)	13.4(3)	0.998	139	778.221(2.02), 849.85(0.43), 847.603(0.324)
	Mo	96	16.68(2)	0.5(2)	1.001	36	
	Mo	97	9.55(8)	2.5(2)	0.998	110	
	Mo	98	24.13(31)	0.137(5)	1.000	56	
	Mo	100	9.63(23)	0.199(3)	1.000	332	
44	Ru	96	5.54(14)	0.22(2)	1.001	2	
	Ru	98	1.87(3)	<8.0	1.002	1	
	Ru	99	12.76(14)	7.1(10)	1.002	134	539.538(1.53), 686.907(0.52)
	Ru	100	12.60(7)	5.0(6)	1.000	32	

TABLE 7.1

TABLE 7.1. ISOTOPIC DATA (cont.)

<i>Z</i>	El.	<i>A</i>	Abundance (at.%)	σ_γ (total)	<i>g</i> (293 K)	<i>N</i> _{γ}	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
Ru	101	17.06(2)	3.4(9)	1.001	60	475.0950(0.98), 631.22(0.30), 627.970(0.176)	
	102	31.55(14)	1.21(7)	1.000	173	1959.30(0.210)	
	104	18.62(27)	0.47(2)	1.000	183		
45	Rh	103	100	145(2)	1.023	264	180.87(22.6), 97.14(19.5), 51.50(16.0)
	Pd	102	1.02(1)	3.4(3)	0.997	4	
46	Pd	104	11.14(8)	0.6(3)	1.000	11	
	Pd	105	22.33(8)	21.0(15)	0.995	114	511.843(4.00), 717.356(0.777), 616.192(0.629)
	Pd	106	27.33(3)	0.31(3)	0.999	7	
	Pd	108	26.46(9)	7.6(4)	1.000	140	
	Pd	110	11.72(9)	0.23(3)	1.000	87	
	Ag	107	51.839(8)	37.6(12)	0.998	172	78.91(3.90), 206.46(3.58), 192.90(2.20)
	Ag	109	48.161(8)	91(1)	1.005	130	198.72(7.75), 235.62(4.62), 117.45(3.85)
48	Cd	106	1.25(6)	~1.0	1.000	0	
	Cd	108	0.89(3)	0.72(13)	1.001	0	
	Cd	110	12.49(18)	11(1)	1.000	191	245.3(274)
	Cd	111	12.80(12)	24(3)	0.995	5	
	Cd	112	24.13(21)	2.2(5)	1.000	0	
	Cd	113	12.22(12)	20600(400)	1.337	135	558.32(1860), 651.19(358)
	Cd	114	28.73(42)	0.34(2)	1.000	0	
	Cd	116	7.49(18)	0.075(20)	1.000	0	
	In	113	4.29(5)	15.1(13)	1.012	232	
	In	115	95.71(5)	283(8)	1.019	199	1293.54(131)d, 1097.30(87.3)d, 416.86(43.0)d
50	Sn	112	0.97(1)	0.86(9)	1.000	0	
	Sn	114	0.66(1)	0.12(3)	1.001	0	
	Sn	115	0.34(1)	30(7)	1.000	395	1293.591(0.1340), 972.619(0.0158), 2112.302(0.0152)
	Sn	116	14.54(9)	0.14(3)	1.000	9	158.65(0.0145)
	Sn	117	7.68(7)	1.32(18)	1.000	19	1229.64(0.0673)
	Sn	118	24.22(9)	0.23(5)	1.000	9	
	Sn	119	8.59(4)	2.2(5)	1.000	9	1171.28(0.0879)
	Sn	120	32.58(9)	0.14(3)	1.000	10	
	Sn	122	4.63(3)	0.139(15)	1.000	9	
	Sn	124	5.79(5)	0.134(5)	1.000	25	
51	Sb	121	57.21(5)	5.9(2)	1.003	151	564.24(2.700)d, 61.4130(0.75), 78.0910(0.48)
	Sb	123	42.79(5)	4.1(1)	1.001	175	87.6010(0.212), 40.8040(0.10), 155.1780(0.081)
52	Te	120	0.09(1)	2.3(3)	1.000	0	
	Te	122	2.55(12)	3.9(5)	1.000	113	
	Te	123	0.89(3)	418(30)	1.011	162	602.729(2.46), 722.772(0.52), 645.819(0.263)
	Te	124	4.74(14)	6.8(13)	1.000	280	
	Te	125	7.07(15)	1.55(16)	1.000	8	
	Te	126	18.84(25)	1.0(15)	1.000	2	
	Te	128	31.74(8)	0.215(8)	1.000	23	
	Te	130	34.08(62)	0.29(6)	1.000	258	

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.1. ISOTOPIC DATA (cont.)

<i>Z</i>	El.	<i>A</i>	Abundance (at.%)	σ_γ (total)	<i>g</i> (293 K)	<i>N</i> _{γ}	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
53	I	127	100	6.2(2)	0.999	348	133.6110(1.42), 442.901(0.595)d, 27.3620(0.43)
54	Xe	124	0.09(1)	165(11)	1.004	4	
	Xe	126	0.09(1)	3.8(8)	1.000	0	
	Xe	128	1.92(3)	5.2(13)	0.998	7	
	Xe	129	26.44(24)	21(7)	1.001	59	536.17(1.71)
	Xe	130	4.08(2)	4.8(12)	0.998	13	
	Xe	131	21.18(3)	85(10)	1.002	72	667.79(6.7), 772.72(1.78), 630.29(1.41)
	Xe	132	26.89(6)	0.41(5)	1.000	0	
	Xe	134	10.44(10)	0.265(20)	0.999	0	
	Xe	136	8.87(16)	0.26(2)	1.000	113	
55	Cs	133	100	30.3(11)	1.002	384	176.4040(2.47), 205.615(1.560), 510.795(1.54)
56	Ba	130	0.106(1)	8.7(9)	1.000	2	
	Ba	132	0.101(1)	7.0(8)	0.979	2	
	Ba	134	2.417(18)	1.5(3)	1.000	120	
	Ba	135	6.592(12)	5.8(9)	1.000	87	818.514(0.212), 1261.52(0.095)
	Ba	136	7.854(24)	0.68(17)	1.000	96	283.58(0.0404)
	Ba	137	11.232(24)	3.6(2)	1.000	210	1435.77(0.308), 1444.91(0.0801), 462.78(0.0660)
	Ba	138	71.698(42)	0.40(4)	1.000	48	627.29(0.294), 4095.84(0.155), 454.73(0.0853)
57	La	138	0.090(1)	57(6)	1.003	6	
	La	139	99.910(1)	9.04(4)	0.999	308	1596.21(5.84)d, 487.021(2.79)d, 815.772(1.430)d
58	Ce	136	0.185(2)	6.5(10)	0.999	109	
	Ce	138	0.251(2)	1.02(24)	0.991	1	
	Ce	140	88.450(51)	0.58(2)	0.999	29	661.99(0.241), 4766.10(0.113), 475.04(0.082)
	Ce	142	11.114(51)	0.97(2)	0.998	48	1107.66(0.040), 737.43(0.026), 4336.46(0.0251)
59	Pr	141	100	11.5(3)	0.999	213	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)d
60	Nd	142	27.2(5)	18.7(7)	0.998	208	742.106(3.8)
	Nd	143	12.2(2)	325(10)	0.996	119	696.499(33.3), 618.062(13.4), 814.12(4.98)
	Nd	144	23.8(3)	3.6(3)	1.000	16	
	Nd	145	8.3(1)	42(2)	1.000	123	
	Nd	146	17.2(3)	1.41(5)	0.999	73	
	Nd	148	5.7(1)	2.58(14)	1.000	298	
	Nd	150	5.6(2)	1.03(8)	0.999	581	
62	Sm	144	3.07(7)	1.64(10)	0.999	0	
	Sm	147	14.99(18)	57(3)	1.001	22	
	Sm	148	11.24(10)	2.4(6)	1.000	0	
	Sm	149	13.82(7)	40100(600)	1.718	160	333.97(4790), 439.40(28601), 737.44(597)
	Sm	150	7.38(1)	100(4)	0.998	301	
	Sm	152	26.75(16)	206(6)	1.003	160	
	Sm	154	22.75(29)	8.3(5)	1.000	136	
63	Eu	151	47.81(3)	9200(300)	0.900	148	89.847(1430), 77.23(187), 48.31(181)
	Eu	153	52.19(3)	312(7)	0.966	64	
64	Gd	152	0.20(1)	735(20)	0.998	503	

TABLE 7.1

TABLE 7.1. ISOTOPIC DATA (cont.)

<i>Z</i>	El.	<i>A</i>	Abundance (at.%)	σ_γ (total)	<i>g</i> (293 K)	N_γ	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
	Gd	154	2.18(3)	85(12)	1.000	329	
	Gd	155	14.80(12)	60900(500)	0.843	324	199.2130(2020), 88.9670(1380)
	Gd	156	20.47(9)	1.8(7)	1.001	0	
	Gd	157	15.65(2)	254000(800)	0.852	390	181.931(72003), 79.5100(40101), 944.174(3090)
	Gd	158	24.84(7)	2.2(2)	1.000	20	
	Gd	160	21.86(19)	1.4(3)	1.000	98	
65	Tb	159	100	23.3(4)	1.000	224	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
66	Dy	156	0.06(1)	33(3)	1.009	25	
	Dy	158	0.10(1)	43(6)	0.989	0	
	Dy	160	2.34(8)	55(3)	1.009	100	
	Dy	161	18.91(24)	600(25)	0.991	78	185.19(31.6), 882.27(14.8), 80.64(13.3)
	Dy	162	25.51(26)	194(10)	1.005	328	
	Dy	163	24.90(16)	134(7)	1.003	45	
	Dy	164	28.18(37)	2650(70)	0.988	271	184.257(118), 538.609(55.9), 496.931(36.3)
67	Ho	165	100	64.7(12)	1.002	550	136.6650(14.5), 116.8360(8.1), 80.574(3.87)d
68	Er	162	0.14(1)	19(2)	1.001	1	
	Er	164	1.61(3)	13(2)	1.000	0	
	Er	166	33.61(35)	16.9(16)	1.000	87	
	Er	167	22.93(17)	649(8)	1.069	805	184.2850(56), 815.9890(42.5), 198.2440(29.9)
	Er	168	26.78(26)	2.74(8)	1.000	102	
	Er	170	14.93(27)	8.9(3)	1.000	97	
69	Tm	169	100	105(2)	1.005	303	204.4480(8.72), 149.7180(7.11), 144.4800(5.96)
70	Yb	168	0.13(1)	2300(170)	1.057	233	191.2140(0.22)
	Yb	170	3.04(15)	9.9(18)	1.001	24	
	Yb	171	14.28(57)	58(4)	0.999	266	78.7430(0.67), 181.529(0.53), 1076.246(0.52)
	Yb	172	21.83(67)	1.3(8)	1.000	25	
	Yb	173	16.13(27)	15.5(15)	1.001	47	175.30(0.58), 102.60(0.44), 76.9960(0.40)
	Yb	174	31.83(92)	63.2(15)	0.999	176	514.868(9.0)d, 639.261(1.43), 396.329(1.42)d
	Yb	176	12.76(41)	2.85(5)	1.000	129	
71	Lu	175	97.41(2)	23.1(14)	0.976	304	71.5170(3.96), 225.4030(1.73), 310.1870(1.49)
	Lu	176	2.59(2)	2090(70)	1.752	184	150.392(13.8), 457.944(8.3), 138.607(6.79)
72	Hf	174	0.16(1)	549(7)	0.986	23	
	Hf	176	5.26(7)	24(3)	1.002	5	
	Hf	177	18.60(9)	373(10)	1.020	308	213.439(29.3), 93.182(13.3), 325.559(6.69)
	Hf	178	27.28(7)	137(7)	1.003	347	214.3410(17.7)d, 214.3410(7.2), 303.9880(4.27)
	Hf	179	13.629(6)	41(3)	0.997	339	
	Hf	180	35.08(16)	13.04(7)	0.997	105	
73	Ta	180	0.012(2)	563(60)	1.358	0	
	Ta	181	99.988(2)	20.5(5)	1.004	262	270.4030(2.60), 173.2050(1.210), 402.623(1.180)
74	W	180	0.12(1)	<150	0.997	3	
	W	182	26.50(16)	19.9(2)	1.003	131	6190.78(0.45), 46.4840(0.192), 5164.43(0.19)
	W	183	14.31(4)	10.3(2)	0.999	211	111.216(0.195), 792.059(0.119), 903.274(0.115)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.1. ISOTOPIC DATA (cont.)

<i>Z</i>	El.	<i>A</i>	Abundance (at.%)	σ_γ (total)	<i>g</i> (293 K)	<i>N_γ</i>	$E_\gamma \sigma_\gamma^Z(E_\gamma)$ for the most intense capture gamma rays for each element
	W	184	30.64(2)	1.7(1)	0.999	75	4573.7(0.104)
	W	186	28.42(19)	38.5(5)	1.001	225	685.73(3.24)d, 479.550(2.59)d, 72.002(1.32)d
75	Re	185	37.40(2)	112(2)	1.004	188	59.0100(5.5), 137.157(5.29)d, 214.647(2.53)
	Re	187	62.60(2)	79.2(10)	0.982	218	63.5820(8.0), 155.041(7.16)d, 207.853(4.44)
76	Os	184	0.02(1)	3000(150)	1.000	72	
	Os	186	1.59(3)	80(13)	0.998	38	
76	Os	187	1.96(2)	245(40)	0.983	174	155.10(1.19), 633.14(0.585), 478.04(0.523)
	Os	188	13.24(8)	4.7(5)	1.002	163	272.82(0.242)
76	Os	189	16.15(5)	25(4)	1.004	147	186.7180(2.08), 557.978(0.84), 569.344(0.694)
	Os	190	26.26(2)	13.1(9)	0.997	76	5146.63(0.409), 527.60(0.300)
76	Os	192	40.78(19)	3.12(16)	1.000	95	
	Ir	191	37.3(2)	954(10)	0.996	286	351.689(10.9), 84.2740(7.7), 136.1250(6.5)
77	Ir	193	62.7(2)	111(5)	1.017	303	328.448(9.1)d, 371.5020(2.11), 278.5040(1.8)
	Pt	190	0.014(1)	142(4)	0.998	0	
78	Pt	192	0.782(7)	10.0(25)	1.001	0	
	Pt	194	32.967(99)	0.58(19)	1.000	64	
78	Pt	195	33.832(10)	28.5(12)	1.000	235	355.6840(6.17), 332.985(2.580)
	Pt	196	25.242(41)	0.45(4)	1.000	36	
78	Pt	198	7.163(55)	3.66(19)	1.000	44	
	Au	197	100	98.65(9)	1.005	737	411.8020(94.29)d, 214.9710(9.0), 247.5730(5.56)
80	Hg	196	0.15(1)	3190(180)	0.988	10	
	Hg	198	9.97(20)	2.0(3)	1.001	3	
80	Hg	199	16.87(22)	2150(50)	0.989	425	367.947(251), 5967.02(62.5), 1693.296(56.2)
	Hg	200	23.10(19)	<60	1.000	0	
80	Hg	201	13.18(9)	5.7(12)	1.000	97	
	Hg	202	29.86(26)	4.42(7)	1.000	0	
80	Hg	204	6.87(15)	0.43(10)	1.000	13	
	Tl	203	29.524(14)	11.4(2)	1.000	115	139.94(0.400), 347.96(0.361), 318.88(0.325)
81	Tl	205	70.476(14)	0.104(17)	1.000	13	
	Pb	204	1.4(1)	0.66(7)	1.001	35	
82	Pb	206	24.1(1)	0.0266(12)	1.001	6	
	Pb	207	22.1(1)	0.63(3)	1.001	23	7367.78(0.137)
82	Pb	208	52.4(1)	0.00023(3)	1.003	0	
	Bi	209	100	0.0338(7)	0.999	230	4171.05(0.0131), 4054.57(0.0105), 319.78(0.0088)
90	Th	232	100	7.35(3)	0.995	196	583.27(0.279), 566.63(0.19), 472.30(0.165)
92	U	234	0.0055(5)	99.8(13)	0.990	49	
	U	235	0.7200(51)	98.3(8)	0.985	8	297.00(0.220), 1279.01(0.200), 943.14(0.082)
92	U	238	99.274(11)	2.680(19)	1.002	267	74.6640(1.30000)d, 106.1230(0.723)d, 277.5990(0.382)d

TABLE 7.2

TABLE 7.2. SUMMARY OF DATA FOR RADIOACTIVE ISOTOPES PRODUCED BY THERMAL NEUTRON ACTIVATION

Isotope	Mode	Half-life	%BR	N_{γ}	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for principal decay gamma rays
^{16}N	β^-	7.13(2) s	100	12	$6128.63(5.90 \times 10^{-8})$
^{19}O	β^-	26.88(5) s	100	13	$197.142(3.15 \times 10^{-7}), 1356.843(1.66 \times 10^{-7})$
^{20}F	β^-	11.163(8) s	100	3	$1633.53(0.0096)$
^{23}Ne	β^-	37.24(12) s	100	5	$440.0(0.00140)$
^{24}Na	β^-	14.9590(12) h	100	6	$2754.13(0.530), 1368.66(0.530)$
^{24}Na	IT	20.20(7) ms	99.95(1)	1	$472.202(0.478)$
^{27}Mg	β^-	9.462(11) min	100	3	$843.71(0.00298), 1014.30(0.00117)$
^{28}Al	β^-	2.2414(1) min	100	1	$1778.92(0.232)$
^{31}Si	β^-	157.3(3) min	100	1	$1266.15(2.5 \times 10^{-6})$
^{37}S	β^-	5.05(2) min	100	7	$3103.4(2.8 \times 10^{-5})$
^{38}Cl	β^-	37.24(5) min	100	2	$2166.90(0.0568), 1642.5(0.0427)$
^{38}Cl	IT	715(3) ms	100	1	$671.355(0.0122)$
$^{40}\text{K}(\text{nat})$	EC	$1.265(13) \times 10^9$ a	10.86(13)	1	$1460.822(3.24 \text{ cps/g})$
^{42}K	β^-	12.360(12) h	100	8	$1524.6(0.0200)$
^{49}Ca	β^-	8.718(6) min	100	12	$3084.40(0.00190)$
^{46}Sc	IT	18.75(4) s	100	1	$142.528(4.88)$
^{51}Ti	β^-	5.76(1) min	100	3	$320.076(0.00860)$
$^{50}\text{V}(\text{nat})$	β^-	$1.4(4) \times 10^{17}$ a	17(11)	1	$783.29(8 \times 10^{-7} \text{ cps/g})$
$^{50}\text{V}(\text{nat})$	EC	$1.4(4) \times 10^{17}$ a	83(11)	1	$1553.77(3.8 \times 10^{-6} \text{ cps/g})$
^{52}V	β^-	3.75(1) min	100	13	$1434.10(4.81)$
^{55}Cr	β^-	3.497(3) min	100	7	$1528.00(3.80 \times 10^{-6})$
^{56}Mn	β^-	2.5789(1) h	100	10	$846.754(13.1), 1810.72(3.62), 2113.05(1.91)$
^{60}Co	IT	10.467(6) min	99.76(3)	1	$58.603(0.411)$
^{60}Co	β^-	10.467(6) min	0.24(3)	3	$1332.89(0.068)$
^{65}Ni	β^-	2.51719(3) h	100	10	$1481.84(0.00330), 1115.53(0.00219), 366.27(0.000680)$
^{64}Cu	EC	12.700(2) h	61.0(3)	1	$1345.77(0.0155)$
^{66}Cu	β^-	5.120(14) min	100	3	$1038.97(0.0598)$
^{69}Zn	β^-	13.76(2) h	0.033(3)	1	$573.90(4.2 \times 10^{-6})$
^{69}Zn	β^-	56.4(9) min	100	2	$318.40(2.6 \times 10^{-6}), 871.70(5.5 \times 10^{-7})$
^{69}Zn	IT	13.76(2) h	99.967(3)	1	$438.634(0.0128)$
^{71}Zn	β^-	2.45(10) min	100	23	$511.60(1.60 \times 10^{-4}), 910.30(4.0 \times 10^{-5}), 390.0(1.97 \times 10^{-5})$
^{71}Zn	β^-	3.96(5) h	100	56	$487.34(3.34 \times 10^{-5}), 620.19(3.04 \times 10^{-5}), 511.55(1.52 \times 10^{-5})$
^{70}Ga	β^-	21.14(3) min	99.59(6)	2	$1039.20(0.0070), 176.170(0.0030)$
^{72}Ga	β^-	14.10(1) h	100	82	$834.08(1.65), 2201.91(0.52), 629.96(0.490)$
^{72}Ga	IT	39.68(13) ms	100	2	$103.25(0.0526), 16.43(0.0125)$
^{71}Ge	IT	20.40(17) ms	100	2	$175.05(0.078)$
^{73}Ge	IT	0.499(11) s	100	2	$53.440(0.0134)$
^{75}Ge	β^-	82.78(4) min	100	10	$264.60(0.0180), 198.60(0.00190)$
^{75}Ge	IT	47.7(5) s	99.970(6)	1	$139.68(0.0232)$
^{77}Ge	β^-	11.30(1) h	100	169	$264.44(0.00640), 211.03(0.00367), 215.50(0.00341)$
^{77}Ge	IT	52.9(6) s	19(2)	1	$159.61(0.00100)$
^{77}Ge	β^-	52.9(6) s	81(2)	17	$215.53(0.0025)$

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.2. SUMMARY OF DATA FOR RADIOACTIVE ISOTOPES PRODUCED BY THERMAL NEUTRON ACTIVATION (cont.)

Isotope	Mode	Half-life	%BR	N_{γ}	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for principal decay gamma rays
⁷⁶ As	β^-	26.24(9) h	100	50	559.10(2.00), 657.05(0.279)
⁷⁷ Se	IT	17.36(5) s	100	1	161.9220(0.855)
⁷⁹ Se	IT	3.92(1) min	100	1	95.73(0.0031)
⁸¹ Se	β^-	18.45(12) min	100	10	275.93(0.00160), 290.04(0.00135), 828.27(0.00069)
⁸¹ Se	IT	57.28(2) min	99.949(13)	1	102.89(0.0065)
⁸⁰ Br	β^-	17.68(2) min	91.7(2)	4	616.3(0.39)
⁸⁰ Br	EC	17.68(2) min	8.3(2)	2	665.80(0.0628)
⁸⁰ Br	IT	4.4205(8) h	100	2	37.0520(0.428)
⁸² Br	β^-	35.30(2) h	100	31	776.517(0.990), 554.3480(0.838), 619.106(0.515)
⁸² Br	IT	6.13(5) min	97.6(3)	1	45.949(0.00285)
⁸² Br	β^-	6.13(5) min	2.4(3)	16	776.50(0.00250), 1474.83(0.00090), 698.21(0.00053)
⁷⁹ Kr	IT	50(3) s	100	1	130.010(1.60×10^{-4})
⁸¹ Kr	IT	13.10(3) s	99.9975(4)	1	190.46(0.072)
⁸³ Kr	IT	1.83(2) h	100	2	9.4050(0.122)
⁸⁵ Kr	β^-	4.480(8) h	78.6(4)	6	151.195(0.0385)
⁸⁵ Kr	IT	4.480(8) h	21.4(4)	1	304.870(0.0071)
⁸⁷ Kr	β^-	76.3(6) min	100	28	402.587(0.000257), 2554.80(4.78×10^{-5}), 845.44(3.80×10^{-5})
⁸⁶ Rb	β^-	18.631(18) d	99.9948(5)	1	1076.64(0.0301)
⁸⁶ Rb	IT	1.017(3) min	100	1	555.61(0.0407)
⁸⁸ Rb	β^-	17.78(11) min	100	30	1836.00(0.00714), 898.03(0.00468)
⁸⁵ Sr	EC	67.63(4) min	13.4(4)	1	150.75(0.00046)
⁸⁵ Sr	IT	67.63(4) min	86.6(4)	2	231.68(0.0029)
⁸⁷ Sr	IT	2.803(3) h	99.70(8)	1	388.526(0.0785)
⁹⁰ Y	IT	3.19(6) h	99.9979(2)	2	202.53(0.0018), 479.60(0.0016)
⁹⁷ Zr	β^-	16.744(11) h	100	31	743.36(0.00101)
⁹⁴ Nb	β^-	6.26(1) min	0.50(6)	1	871.1(0.00390)
⁹⁴ Nb	IT	6.26(1) min	99.50(6)	1	40.887(0.000574)
⁹⁹ Mo	β^-	65.94(1) h	100	30	140.5110(0.0276), 739.500(0.00405)
¹⁰¹ Mo	β^-	14.61(3) min	100	163	590.10(0.00380), 191.920(0.00360), 1012.47(0.00258)
¹⁰³ Ru	IT	1.69(7) ms	100	2	210.519(0.033)
¹⁰⁵ Ru	β^-	4.44(2) h	100	84	724.30(0.0760), 469.37(0.0281), 676.36(0.0251)
¹⁰⁴ Rh	β^-	42.3(4) s	99.55	14	555.81(3.14)
¹⁰⁴ Rh	IT	4.34(5) min	99.87(1)	4	51.50(5.2)
¹⁰⁷ Pd	IT	21.3() s	100	1	214.9(0.0024)
¹⁰⁹ Pd	IT	4.69(1) min	100	1	188.9900(0.0273)
¹¹¹ Pd	β^-	23.4(2) min	100	76	580.00(1.90×10^{-4}), 70.43(1.68×10^{-4}), 1459.0(1.25×10^{-4})
¹¹¹ Pd	IT	5.5(1) h	73(3)	1	172.18(0.0015)
¹⁰⁸ Ag	β^-	2.37(1) min	97.15(20)	1	632.98(0.369)
¹⁰⁸ Ag	EC	2.37(1) min	2.85(20)	11	433.96(0.0990), 618.86(0.052)
¹¹⁰ Ag	β^-	24.6(2) s	99.70(6)	13	657.50(1.86)
¹¹⁴ In	β^-	71.9(1) s	99.50(15)	1	1299.83(2.4×10^{-4})
¹¹⁴ In	IT	43.1(6) ms	100	1	311.646(0.13)
¹¹⁶ In	β^-	54.41(6) min	100	30	1293.54(131), 1097.30(87.3), 416.86(43.0)

TABLE 7.2

TABLE 7.2. SUMMARY OF DATA FOR RADIOACTIVE ISOTOPES PRODUCED BY THERMAL NEUTRON ACTIVATION (cont.)

Isotope	Mode	Half-life	%BR	N_{γ}	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for principal decay gamma rays
¹¹⁶ In	IT	2.18(4) s	100	1	162.393(15.8)
¹¹⁶ In	β^-	14.10(3) s	100	10	1293.4(0.470), 463.3(0.0930)
¹²³ Sn	β^-	40.06(1) min	100	5	160.32(0.00580)
¹²⁵ Sn	β^-	9.52(5) min	100	23	331.90(0.00830)
¹²² Sb	β^-	2.7238(2) d	97.59(12)	7	564.24(2.70)
¹²² Sb	IT	4.191(3) min	97.59(12)	3	61.4130(0.0200), 76.0590(0.0081)
¹²⁴ Sb	β^-	93(5) s	25(5)	4	498.40(0.068), 645.82(0.068), 602.72(0.068)
¹²⁴ Sb	IT	93(5) s	75(5)	1	10.8630(1.40×10^{-5})
¹²⁴ Sb	IT	20.2(2) min	100	2	10.8630(6.04×10^{-6}), 25.9820(4.45×10^{-6})
¹³¹ Te	β^-	25.0(1) min	100	78	149.716(0.0630), 452.3230(0.0168)
¹³¹ Te	β^-	30(2) h	77.8(16)	171	773.67(0.00355), 852.21(0.00192), 793.75(0.00129)
¹³¹ Te	IT	30(2) h	22.2(16)	1	182.250(0.00026)
¹²⁸ I	β^-	24.99(2) min	93.1(6)	7	442.901(0.595)
¹²⁸ I	EC	24.99(2) min	6.9(1)	1	743.50(0.0051)
¹²⁵ Xe	IT	56.9(9) s	100	2	111.3(0.0027), 141.4(0.00091)
¹²⁹ Xe	IT	8.88(2) d	100	2	39.578(0.00069), 196.56(0.00042)
¹³⁷ Xe	β^-	3.818(13) min	100	83	455.490(0.00350)
¹³⁴ Cs	IT	2.903(8) h	100	3	127.500(0.310)
¹³¹ Ba	IT	14.6(2) min	100	2	108.45(0.00150)
¹³³ Ba	IT	38.9(1) h	99.99	2	275.925(9.00×10^{-5})
¹³⁵ Ba	IT	28.7(2) h	100	1	268.218(0.00060)
¹³⁶ Ba	IT	0.3084(19) s	100	3	1048.073(0.000919), 818.514(0.000916), 163.920(0.000280)
¹³⁷ Ba	IT	2.552(1) min	100	1	661.657(0.00071)
¹³⁹ Ba	β^-	83.06(3) min	100	28	165.8570(0.074)
¹⁴⁰ Ba	β^-	12.752(3) d	100	16	537.261(0.066), 29.966(0.0381), 162.660(0.0168)
¹³⁸ La(nat)	β^-	$1.05(3) \times 10^{11}$ a	33.6(5)	1	788.7(0.273 cps/g)
¹³⁸ La(nat)	EC	$1.05(3) \times 10^{11}$ a	66.4(5)	1	1435.795(0.539 cps/g)
¹⁴⁰ La	β^-	1.6781(7) d	100	38	1596.21(5.84), 487.021(2.79), 815.772(1.43)
¹³⁷ Ce	EC	9.0(3) h	100	20	447.15(1.30×10^{-4}), 10.61(5.6×10^{-5}), 436.59(1.86×10^{-5})
¹³⁷ Ce	IT	34.4(3) h	99.22(3)	1	254.29(2.0×10^{-4})
¹³⁹ Ce	IT	54.8(10) s	100	1	754.24(3.5×10^{-5})
¹⁴² Pr	β^-	19.12(4) h	99.98	2	1575.6(0.426)
¹⁴⁹ Nd	β^-	1.728(1) h	100	213	211.309(0.0370), 114.314(0.0274), 270.166(0.0153)
¹⁵¹ Nd	β^-	12.44(7) min	100	471	116.800(0.0262), 255.680(0.0099), 1180.890(0.0089)
¹⁵⁵ Sm	β^-	22.3(2) min	100	50	104.320(1.43)
¹⁵² Eu	IT	96(1) min	100	4	89.847(1.30)
¹⁵⁵ Gd	IT	31.97(3) ms	100	3	86.545(0.00074), 13.47(7.6×10^{-5})
¹⁵⁹ Gd	β^-	18.56(8) h	100	20	363.5430(0.063), 58.000(0.0118)
¹⁶¹ Gd	β^-	3.66(5) min	100	98	360.940(0.199), 314.920(0.075), 102.315(0.046)
¹⁵⁷ Dy	EC	8.14(4) h	100	25	326.16(0.018)
¹⁶⁵ Dy	β^-	2.334(6) h	100	55	94.700(10.6), 361.680(2.50), 633.415(1.69)
¹⁶⁵ Dy	β^-	1.257(6) min	2.24(11)	11	515.467(6.93), 361.471(2.42), 153.803(1.10)
¹⁶⁵ Dy	IT	1.257(6) min	97.76(11)	1	108.159(13.6)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.2. SUMMARY OF DATA FOR RADIOACTIVE ISOTOPES PRODUCED BY THERMAL NEUTRON ACTIVATION (cont.)

Isotope	Mode	Half-life	%BR	N_{γ}	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for principal decay gamma rays
¹⁶⁶ Ho	β^-	26.80(2) h	100	14	80.574(3.87), 1379.40(0.537)
¹⁶⁷ Er	IT	2.269(6) s	100	1	207.801(2.15)
¹⁷¹ Er	β^-	7.516(2) h	100	58	308.291(0.559), 295.901(0.251), 111.621(0.178)
¹⁶⁹ Yb	IT	46(2) s	100	1	24.200(5.6×10^{-6})
¹⁷⁵ Yb	β^-	4.185(1) d	100	6	396.329(1.42), 282.522(0.666), 113.805(0.417)
¹⁷⁵ Yb	IT	68.2(3) ms	100	1	514.868(9.0)
¹⁷⁷ Yb	β^-	1.911(3) h	100	24	150.6(0.073), 1080.20(0.0201), 1241.20(0.0125)
¹⁷⁷ Yb	IT	6.41(3) s	100	2	104.50(0.029), 227.02(0.0047)
¹⁷⁶ Lu(nat)	β^-	$4.00(22) \times 10^{10}$ a	100	4	306.84(45.2 cps/g), 201.83(37.9 cps/g)
¹⁷⁷ Lu	β^-	6.73(1) d	100	6	208.366(6.0), 112.9500(3.47)
¹⁷⁸ Hf	IT	4.0(2) s	100	6	426.380(0.175), 325.559(0.170), 213.439(0.1470)
¹⁷⁹ Hf	IT	18.67(4) s	100	2	214.341(16.3)
¹⁸⁰ Hf	IT	5.5(1) h	99.7(1)	6	332.275(0.0586), 443.163(0.0509), 215.426(0.0506)
¹⁸² Ta	IT	15.84(10) min	100	5	171.580(0.00540), 146.7740(0.00408), 184.951(0.00268)
¹⁸³ W	IT	5.2(3) s	100	6	107.932(0.00438), 99.079(0.00189), 52.595(0.00157)
¹⁸⁵ W	IT	1.67(3) min	100	12	65.86(3.44×10^{-5}), 131.550(2.56×10^{-5}), 173.680(1.93×10^{-5})
¹⁸⁷ W	β^-	23.72(6) h	100	74	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁸⁶ Re	β^-	3.7183(11) d	92.53(10)	8	137.157(5.29)
¹⁸⁶ Re	EC	3.7183(11) d	7.47(10)	1	122.640(0.250)
¹⁸⁸ Re	β^-	17.005(4) h	100	51	155.041(7.16)
¹⁸⁸ Re	IT	18.6(1) min	100	5	63.582(0.279), 105.862(0.140), 92.4640(0.066)
¹⁹¹ Os	IT	13.10(5) h	100	1	74.380(0.0032)
¹⁹³ Os	β^-	30.11(1) h	100	63	138.92(0.0467), 460.49(0.0432), 73.040(0.035)
¹⁹² Ir	IT	1.45(5) min	99.9825	1	56.719(0.085)
¹⁹⁴ Ir	β^-	19.28(13) h	100	65	328.448(9.1), 293.541(1.76)
¹⁹⁴ Ir	IT	31.85(24) ms	100	9	112.231(0.302), 84.2840(0.168)
¹⁹⁷ Pt	β^-	19.8915(19) h	100	3	77.35(0.031), 191.437(0.00660)
¹⁹⁷ Pt	IT	95.41(18) min	96.7(4)	2	346.50(0.00132)
¹⁹⁹ Pt	β^-	30.8(4) min	100	42	542.98(0.0390), 493.75(0.0147), 317.03(0.0130)
¹⁹⁹ Pt	IT	13.6(4) s	100	2	391.93(0.0212)
¹⁹⁸ Au	β^-	2.69517(21) d	100	3	411.8(94.29)
¹⁹⁷ Hg	EC	23.8(1) h	8.6(7)	5	279.00(0.00330)
¹⁹⁷ Hg	IT	23.8(1) h	91.4(7)	2	133.98(0.0155)
¹⁹⁹ Hg	IT	42.6(2) min	100	3	158.30(0.000940), 374.10(2.47×10^{-4})
²⁰⁵ Hg	β^-	5.2(1) min	100	13	203.750(0.00064)
²⁰⁶ Tl	β^-	4.200(17) min	100	2	803.30(3.5×10^{-6})
²⁰⁷ Pb	IT	0.806(6) s	100	2	569.7(0.0014), 1063.662(0.0013)
²³² Th(nat)	α	$14.05(6) \times 10^9$ a	100	2	63.810(10.7 cps/g)
²³⁵ U(nat)	α	$7.038(5) \times 10^8$ a	100	49	185.715(329 cps/g), 143.760(63.0 cps/g)
²³⁹ Np	β^-	2.3565(4) d	100	36	106.1230(0.723), 277.5990(0.382), 228.1830(0.286)
²³⁹ U	β^-	23.45(2) min	100	97	74.664(1.30)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0					
Hydrogen (Z=1), At.Wt.=1.00794(7), $\sigma_{\gamma}^z = 0.3326(7)$												
¹ H	2223.24835(9)	0.3326(7)	1.0000(21)	¹⁴ N	9148.98(5)	0.00129(6)	0.000279(13)					
² H	6250.243(3)	0.000519(7)(a)	0.001560(21)	¹⁴ N	10829.120(12)	0.0113(8)	0.00244(17)					
Oxygen (Z=8), At.Wt.=15.9994(3), $\sigma_{\gamma}^z = 1.90E-4(19)$												
¹⁸ O	197.142(4)d	3.15(22)E-7	6.0E-8[99%]	¹⁸ O	197.142(4)	3.15(22)E-7	6.0E-8[99%]					
¹⁶ O	870.68(6)	1.77(11)E-4	3.35(21)E-5	¹⁶ O	1087.75(6)	1.58(7)E-4	2.99(13)E-5					
¹⁷ O	1981.95(9)	2.0(4)E-7	3.8(8)E-8	¹⁶ O	2184.42(7)	1.64(7)E-4	3.11(13)E-5					
¹⁶ O	3272.02(8)	3.53(23)E-5	6.7(4)E-6	Fluorine (Z=9), At.Wt.=18.9984032(5), $\sigma_{\gamma}^z = 0.0096(5)$								
¹⁹ F	166.700(20)	0.000413(18)	6.6(3)E-5	¹⁹ F	325.606(24)	4.0(3)E-5	6.4(5)E-6					
¹⁹ F	556.40(4)	2.01(8)E-4	3.21(13)E-5	¹⁹ F	583.561(16)	0.00356(12)	0.000568(19)					
¹⁹ F	656.006(18)	0.00197(7)	0.000314(11)	¹⁹ F	661.647(21)	2.24(14)E-4	3.57(22)E-5					
¹⁹ F	662.25(10)	1.02(15)E-4	1.63(24)E-5	¹⁹ F	665.207(18)	0.00149(6)	2.38(10)E-4					
¹⁹ F	822.700(19)	2.20(9)E-4	3.51(14)E-5	¹⁹ F	978.19(5)	6.8(6)E-5	1.08(10)E-5					
¹⁹ F	983.538(20)	0.00116(4)	1.85(6)E-4	¹⁹ F	1045.98(3)	1.79(8)E-4	2.86(13)E-5					
¹⁹ F	1056.776(17)	0.00095(3)	1.52(5)E-4	¹⁹ F	1148.077(20)	0.000258(12)	4.12(19)E-5					
¹⁹ F	1187.725(25)	4.5(3)E-5	7.2(5)E-6	¹⁹ F	1282.15(4)	8.5(5)E-5	1.36(8)E-5					
¹⁹ F	1309.126(17)	0.00076(3)	1.21(5)E-4	¹⁹ F	1371.520(24)	1.44(7)E-4	2.30(11)E-5					
¹⁹ F	1387.901(20)	0.00082(3)	1.31(5)E-4	¹⁹ F	1392.191(23)	8.3(5)E-5	1.32(8)E-5					
¹⁹ F	1542.498(20)	0.000271(11)	4.32(18)E-5	¹⁹ F	1633.53(3)d	0.0096(4)	0.00153[100%]					
¹⁹ F	1644.538(25)	7.3(6)E-5	1.16(10)E-5	¹⁹ F	1843.688(20)	0.000600(23)	9.6(4)E-5					
¹⁹ F	1935.52(3)	7.3(5)E-5	1.16(8)E-5	¹⁹ F	1970.726(20)	8.5(6)E-5	1.36(10)E-5					
¹⁹ F	2009.52(6)	4.6(4)E-5	7.3(6)E-6	¹⁹ F	2043.858(20)	7.0(4)E-5	1.12(6)E-5					
¹⁹ F	2143.248(21)	1.95(8)E-4	3.11(13)E-5	¹⁹ F	2179.091(20)	8.9(6)E-5	1.42(10)E-5					
¹⁹ F	2194.159(21)	1.32(6)E-4	2.11(10)E-5	¹⁹ F	2194.159(21)	1.32(6)E-4	2.11(10)E-5					
¹⁹ F	2229.75(9)	5.3(5)E-5	8.5(8)E-6	¹⁹ F	2255.83(3)	8.5(5)E-5	1.36(8)E-5					
¹⁹ F	2309.929(25)	4.5(3)E-5	7.2(5)E-6	¹⁹ F	2324.12(3)	1.18(5)E-4	1.88(8)E-5					
¹⁹ F	2427.82(3)	1.89(8)E-4	3.01(13)E-5	¹⁹ F	2431.084(10)	0.000392(24)	6.3(4)E-5					
¹⁹ F	2431.425(19)	7(3)E-5	1.1(5)E-5	¹⁹ F	2447.574(21)	1.44(7)E-4	2.30(11)E-5					
¹⁹ F	2469.34(3)	1.94(9)E-4	3.09(14)E-5	¹⁹ F	2504.658(25)	3.8(4)E-5	6.1(6)E-6					
¹⁹ F	2519.02(3)	6.8(5)E-5	1.08(8)E-5	¹⁹ F	2529.212(18)	0.00061(3)	9.7(5)E-5					
¹⁹ F	2529.553(18)	9(3)E-5	1.4(5)E-5	¹⁹ F	2623.16(3)	4.5(3)E-5	7.2(5)E-6					
¹⁹ F	2636.09(3)	9.6(5)E-5	1.53(8)E-5									

(a) Total deuterium isotopic cross-section.

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁹ F	2655.70(3)	7.6(6)E-5	1.21(10)E-5	²¹ Ne	2165.9(7)	0.00084(21)	1.3(3)E-4
¹⁹ F	2920.96(3)	9.6(5)E-5	1.53(8)E-5	²² Ne	2203.58(6)	0.00238(23)	0.00036(4)
¹⁹ F	2930.284(21)	8.5(5)E-5	1.36(8)E-5	²⁰ Ne	2437.84(25)	0.00036(7)	5.4(11)E-5
¹⁹ F	2965.854(22)	9.3(5)E-5	1.48(8)E-5	²⁰Ne	2793.94(5)	0.00900(11)	0.001352(17)
¹⁹F	3014.568(10)	0.000405(15)	6.46(24)E-5	²² Ne	2819.22(16)	0.00052(5)	7.8(8)E-5
¹⁹ F	3025.10(3)	8.4(9)E-5	1.34(14)E-5	²⁰Ne	2895.32(10)	0.00252(7)	0.000378(11)
¹⁹ F	3051.435(20)	0.000297(12)	4.74(19)E-5	²¹ Ne	2987.8(5)	0.00086(22)	1.3(3)E-4
¹⁹ F	3074.78(3)	1.86(8)E-4	2.97(13)E-5	²¹ Ne	3181.8(16)	0.00048(12)	7.2(18)E-5
¹⁹ F	3112.693(18)	2.36(9)E-4	3.76(14)E-5	²² Ne	3220.42(16)	0.00057(23)	9(4)E-5
¹⁹ F	3220.00(3)	6.1(4)E-5	9.7(6)E-6	²⁰ Ne	3971.98(15)	0.00039(3)	5.9(5)E-5
¹⁹ F	3293.23(4)	3.8(8)E-5	6.1(13)E-6	²¹ Ne	4018.3(5)	0.00090(23)	1.4(4)E-4
¹⁹ F	3387.58(9)	6.1(5)E-5	9.7(8)E-6	²⁰Ne	4374.13(6)	0.01910(22)	0.00287(3)
¹⁹F	3488.064(18)	0.00073(3)	1.16(5)E-4	²¹ Ne	4634.83	0.00042(11)	6.3(17)E-5
¹⁹ F	3586.186(10)	0.000286(13)	4.56(21)E-5	²¹ Ne	4840.1(5)	0.00038(10)	5.7(15)E-5
¹⁹ F	3589.45(3)	1.79(8)E-4	2.86(13)E-5	²⁰ Ne	5688.97(6)	0.00214(3)	0.000321(5)
¹⁹ F	3679.79(3)	8.7(8)E-5	1.39(13)E-5	²⁰ Ne	6760.06(6)	0.002100(25)	0.000315(4)
¹⁹ F	3741.46(3)	5.7(5)E-5	9.1(8)E-6	²¹ Ne	9087.3(5)	0.00028(7)	4.2(11)E-5
¹⁹ F	3823.093(24)	1.07(6)E-4	1.71(10)E-5	Sodium (Z=11), At.Wt.=22.989770(2), σ_γ^Z=0.530(5)			
¹⁹F	3964.872(20)	0.000435(18)	6.9(3)E-5	²³ Na	90.9920(10)	0.235(3)	0.0310(4)
¹⁹ F	4046.504(23)	6.0(16)E-5	1.0(3)E-5	²³ Na	472.202(9)d	0.478(4)	0.0630[100%]
¹⁹ F	4081.71(3)	5.6(4)E-5	8.9(6)E-6	²³ Na	499.381(5)	0.0143(3)	0.00189(4)
¹⁹ F	4094.85(10)	5.1(17)E-5	8(3)E-6	²³ Na	501.347(13)	0.00314(13)	0.000414(17)
¹⁹ F	4173.527(23)	1.66(7)E-4	2.65(11)E-5	²³ Na	563.1920(20)	0.0085(3)	0.00112(4)
¹⁹ F	4200.68(4)	1.11(6)E-4	1.77(10)E-5	²³ Na	711.967(10)	0.00430(22)	0.00057(3)
¹⁹ F	4245.68(3)	9.5(5)E-5	1.52(8)E-5	²³ Na	778.221(9)	0.0058(3)	0.00076(4)
¹⁹ F	4335.08(4)	4.6(4)E-5	7.3(6)E-6	²³ Na	781.435(11)	0.0175(5)	0.00231(7)
¹⁹F	4556.817(20)	0.000517(23)	8.2(4)E-5	²³ Na	835.292(18)	0.0109(3)	0.00144(4)
¹⁹ F	4708.007(20)	5.1(4)E-5	8.1(6)E-6	²³ Na	869.210(9)	0.1080(13)	0.01424(17)
¹⁹ F	4735.16(4)	5.6(4)E-5	8.9(6)E-6	²³ Na	874.389(6)	0.0760(11)	0.01002(15)
¹⁹ F	4756.957(23)	1.86(9)E-4	2.97(14)E-5	²³ Na	886.749(11)	0.00402(16)	0.000530(21)
¹⁹ F	4951.90(3)	6.2(6)E-5	9.9(10)E-6	²³ Na	1006.23(4)	0.00370(18)	0.000488(24)
¹⁹F	5033.530(23)	0.00063(3)	1.00(5)E-4	²³ Na	1150.002(17)	0.00528(21)	0.00070(3)
¹⁹F	5279.360(20)	0.000421(20)	6.7(3)E-5	²³ Na	1282.764(8)	0.0055(3)	0.00073(4)
¹⁹ F	5291.420(19)	2.35(11)E-4	3.75(18)E-5	²³ Na	1322.262(14)	0.0062(3)	0.00082(4)
¹⁹ F	5360.986(21)	1.17(5)E-4	1.87(8)E-5	²³ Na	1337.73(4)	0.00313(20)	0.00041(3)
¹⁹F	5543.713(10)	0.000407(17)	6.5(3)E-5	²³ Na	1344.607(11)	0.0217(5)	0.00286(7)
¹⁹ F	5554.51(3)	5.1(4)E-5	8.1(6)E-6	²³ Na	1368.66(3)d	0.530(8)	0.0699[2.3%]
¹⁹ F	5616.933(23)	1.41(8)E-4	2.25(13)E-5	²³ Na	1373.751(8)	0.0079(19)	0.00104(25)
¹⁹ F	5935.179(20)	9.1(8)E-5	1.45(13)E-5	²³ Na	1504.92(7)	0.00293(23)	0.00039(3)
¹⁹F	6016.802(16)	0.00094(4)	1.50(6)E-4	²³ Na	1562.470(21)	0.00256(20)	0.00034(3)
¹⁹F	6600.175(16)	0.00096(3)	1.53(5)E-4	²³ Na	1620.49(4)	0.00294(22)	0.00039(3)
Neon (Z=10), At.Wt.=20.1797(6), σ_γ^Z=0.039(4)				²³ Na	1633.080(23)	0.0074(4)	0.00098(5)
²⁰Ne	350.72(6)	0.0198(4)	0.00297(6)	²³Na	1636.293(21)	0.0250(7)	0.00330(9)
²² Ne	439.986d	0.001400(5)	2.102E-4[99%]	²³ Na	1712.43(20)	0.0112(6)	0.00148(8)
²⁰ Ne	768.55(7)	2.5(4)E-4	3.8(6)E-5	²³ Na	1885.421(14)	0.0039(3)	0.00051(4)
²⁰ Ne	964.41(7)	0.00029(11)	4.4(17)E-5	²³ Na	1899.06(4)	0.0081(4)	0.00107(5)
²²Ne	1017.00(20)	0.0030(5)	0.00045(8)	²³ Na	1899.86(3)	0.0036(16)	0.00047(21)
²⁰Ne	1071.34(7)	0.0054(4)	0.00081(6)	²³ Na	1914.44(3)	0.00606(21)	0.00080(3)
²¹ Ne	1274.542(7)	0.0018(5)	0.00027(8)	²³ Na	1928.16(4)	0.00480(19)	0.000633(25)
²² Ne	1364.8(3)	0.00091(12)	1.37(18)E-4	²³ Na	1928.37(4)	0.0055(5)	0.00073(7)
²² Ne	1822.40(20)	0.00052(5)	7.8(8)E-5	²³ Na	1950.112(23)	0.0087(3)	0.00115(4)
²⁰Ne	1931.08(6)	0.00591(22)	0.00089(3)	²³ Na	2019.50(8)	0.0025(3)	0.00033(4)
²²Ne	1979.89(6)	0.00306(17)	0.00046(3)	²³Na	2025.139(22)	0.0341(8)	0.00450(11)
²² Ne	2013.8(4)	0.00040(5)	6.0(8)E-5	²³ Na	2027.104(25)	0.0038(5)	0.00050(7)
²⁰Ne	2035.67(20)	0.0245(25)	0.0037(4)	²³ Na	2030.318(23)	0.0219(7)	0.00289(9)
²¹ Ne	2082.5(4)	0.0011(3)	1.7(5)E-4	²³ Na	2071.78(3)	0.0059(3)	0.00078(4)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	
²³ Na	2208.40(3)	0.0259(9)	0.00341(12)	²⁵ Mg	1896.72(3)	0.00094(4)	1.17(5)E-4	
²³ Na	2361.026(21)	0.0084(3)	0.00111(4)	²⁴ Mg	1978.25(3)	0.00111(5)	1.38(6)E-4	
²³ Na	2397.433(25)	0.0069(4)	0.00091(5)	²⁵ Mg	2132.67(3)	0.00089(4)	1.11(5)E-4	
²³ Na	2414.457(21)	0.0237(5)	0.00312(7)	²⁵ Mg	2189.57(4)	0.000592(22)	7.4(3)E-5	
²³ Na	2505.439(21)	0.0167(5)	0.00220(7)	²⁵ Mg	2353.27(4)	0.000447(21)	5.6(3)E-5	
²³ Na	2517.81(3)	0.0699(15)	0.00921(20)	²⁵ Mg	2426.12(3)	0.000519(20)	6.47(25)E-5	
²³ Na	2595.49(3)	0.0052(3)	0.00069(4)	24 Mg	2438.54(3)	0.00473(19)	0.000590(24)	
²³ Na	2630.66(3)	0.00289(14)	0.000381(18)	²⁵ Mg	2510.02(4)	0.00058(3)	7.2(4)E-5	
²³ Na	2715.87(3)	0.00306(16)	0.000403(21)	²⁵ Mg	2523.65(4)	0.00100(4)	1.25(5)E-4	
²³ Na	2752.271(23)	0.0654(12)	0.00862(16)	²⁵ Mg	2541.21(3)	0.00148(7)	1.85(9)E-4	
²³ Na	2754.13(6)d	0.530(8)	0.0699[2.3%]	24 Mg	2828.172(25)	0.0240(8)	0.00299(10)	
²³ Na	2763.17(7)	0.0053(12)	0.00070(16)	²⁶ Mg	2881.64(3)	0.00272(14)	0.000339(17)	
²³ Na	2808.468(22)	0.0168(7)	0.00221(9)	²⁵ Mg	2938.159(25)	0.00094(4)	1.17(5)E-4	
²³ Na	2860.355(20)	0.0177(5)	0.00233(7)	24 Mg	3054.00(3)	0.0083(3)	0.00103(4)	
²³ Na	2865.534(22)	0.0130(4)	0.00171(5)	²⁵ Mg	3208.97(4)	0.000398(19)	4.96(24)E-5	
²³ Na	2904.89(3)	0.0059(3)	0.00078(4)	24 Mg	3301.41(3)	0.00620(24)	0.00077(3)	
²³ Na	2940.91(3)	0.00347(18)	0.000457(24)	²⁵ Mg	3319.65(3)	0.00100(4)	1.25(5)E-4	
²³ Na	2981.97(3)	0.0142(6)	0.00187(8)	²⁵ Mg	3341.00(4)	0.00046(3)	5.7(4)E-5	
²³ Na	3025.99(4)	0.0146(6)	0.00192(8)	²⁵ Mg	3406.41(16)	0.0014(5)	1.7(6)E-4	
²³ Na	3092.50(5)	0.0025(4)	0.00033(5)	24 Mg	3413.10(3)	0.00401(16)	0.000500(20)	
²³ Na	3093.79(8)	0.00280(20)	0.00037(3)	²⁵ Mg	3551.19(3)	0.00109(4)	1.36(5)E-4	
²³ Na	3096.78(3)	0.0199(7)	0.00262(9)	²⁶ Mg	3561.29(3)	0.00249(12)	0.000310(15)	
²³ Na	3099.99(3)	0.0160(9)	0.00211(12)	²⁴ Mg	3691.02(3)	0.00068(4)	8.5(5)E-5	
²³ Na	3116.97(4)	0.00523(24)	0.00069(3)	²⁵ Mg	3744.00(3)	0.00136(5)	1.70(6)E-4	
²³ Na	3209.59(10)	0.00381(20)	0.00050(3)	²⁵ Mg	3810.13(4)	0.00097(4)	1.21(5)E-4	
²³ Na	3214.22(4)	0.0054(4)	0.00071(5)	25 Mg	3831.480(24)	0.00418(14)	0.000521(17)	
²³ Na	3277.32(10)	0.00377(17)	0.000497(22)	²⁶ Mg	3843.00(5)	0.00033(3)	4.1(4)E-5	
²³ Na	3369.94(4)	0.0133(4)	0.00175(5)	24 Mg	3916.84(3)	0.0320(11)	0.00399(14)	
²³ Na	3409.39(3)	0.00237(11)	0.000312(15)	²⁵ Mg	4216.38(3)	0.00145(5)	1.81(6)E-4	
²³ Na	3413.97(3)	0.00441(18)	0.000581(24)	²⁵ Mg	4410.13(3)	0.00067(4)	8.4(5)E-5	
²³ Na	3504.94(3)	0.00676(23)	0.00089(3)	²⁴ Mg	4528.55(9)	0.00035(3)	4.4(4)E-5	
²³ Na	3546.00(3)	0.00454(22)	0.00060(3)	²⁵ Mg	4602.93(3)	0.000363(17)	4.53(21)E-5	
23 Na	3587.460(25)	0.0596(11)	0.00786(15)	²⁴ Mg	4766.69(4)	0.000327(22)	4.1(3)E-5	
²³ Na	3643.655(20)	0.0067(3)	0.00088(4)	²⁵ Mg	4967.19(3)	0.00162(7)	2.02(9)E-4	
²³ Na	3878.10(3)	0.0218(6)	0.00287(8)	²⁵ Mg	5067.14(3)	0.00096(4)	1.20(5)E-4	
23 Na	3981.450(25)	0.0677(11)	0.00892(15)	²⁵ Mg	5452.025(25)	0.00206(7)	0.000257(9)	
²³ Na	4187.49(3)	0.0073(5)	0.00096(7)	²⁴ Mg	6354.98(3)	0.00106(6)	1.32(8)E-4	
²³ Na	5113.007(16)	0.00250(14)	0.000330(18)	²⁶ Mg	6442.52(3)	0.00039(4)	4.9(5)E-5	
²³ Na	5612.274(16)	0.0026(11)	0.00034(15)	²⁵ Mg	6742.14(3)	0.000411(19)	5.12(24)E-5	
²³ Na	5614.239(18)	0.005(3)	0.0007(4)	²⁵ Mg	8153.448(21)	0.00285(11)	0.000355(14)	
²³ Na	5617.452(17)	0.016(5)	0.0021(7)	²⁵ Mg	9282.642(20)	0.000438(18)	5.46(22)E-5	
23 Na	6395.478(15)	0.1000(20)	0.0132(3)	Aluminum (Z=13), At.Wt.=26.981538(2), $\sigma_{\gamma}^z=0.231(3)$				
Magnesium (Z=12), At.Wt.=24.3050(6), $\sigma_{\gamma}^z=0.0666(13)$								
²⁴ Mg	389.670(21)	0.00586(24)	0.00073(3)	²⁷ Al	30.6380(10)	0.0798(20)	0.00896(22)	
²⁴ Mg	585.00(3)	0.0314(11)	0.00392(14)	²⁷ Al	400.589(25)	0.00141(4)	1.58(5)E-4	
²⁶ Mg	843.71(3)d	0.00298(14)	0.000372[78%]	²⁷ Al	831.426(22)	0.00269(7)	0.000302(8)	
²⁴ Mg	862.96(3)	0.000410(21)	5.1(3)E-5	²⁷ Al	865.84(3)	0.00087(3)	9.8(3)E-5	
²⁴ Mg	974.66(3)	0.00663(24)	0.00083(3)	²⁷ Al	982.951(10)	0.00902(14)	0.001013(16)	
²⁶ Mg	984.88(4)	0.00064(4)	8.0(5)E-5	²⁷ Al	1013.588(10)	0.00555(10)	0.000623(11)	
²⁵ Mg	1003.14(3)	0.00161(6)	2.01(8)E-4	²⁷ Al	1073.94(4)	0.00100(4)	1.12(5)E-4	
²⁵ Mg	1129.575(23)	0.00891(25)	0.00111(3)	²⁷ Al	1102.06(4)	0.00103(4)	1.16(5)E-4	
²⁵ Mg	1411.70(3)	0.00130(5)	1.62(6)E-4	²⁷ Al	1125.289(14)	0.00083(4)	9.3(5)E-5	
²⁶ Mg	1615.11(4)	0.00070(4)	8.7(5)E-5	²⁷ Al	1193.476(22)	0.00097(4)	1.09(5)E-4	
²⁴ Mg	1712.92(4)	0.00118(7)	1.47(9)E-4	²⁷ Al	1283.693(12)	0.00222(6)	2.49(7)E-4	
²⁵ Mg	1775.31(3)	0.00129(5)	1.61(6)E-4	²⁷ Al	1342.320(20)	0.00209(6)	2.35(7)E-4	
²⁵ Mg	1808.668(22)	0.0180(5)	0.00224(6)	²⁷ Al	1408.344(9)	0.00640(13)	0.000719(15)	

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
²⁷ Al	1526.246(12)	0.00339(9)	0.000381(10)	²⁷ Al	6440.650(11)	0.00147(8)	1.65(9)E-4
²⁷ Al	1589.62(3)	0.00247(7)	0.000277(8)	²⁷ Al	6619.73(4)	0.00093(7)	1.04(8)E-4
²⁷Al	1622.877(18)	0.00989(15)	0.001111(17)	²⁷ Al	6710.699(10)	0.00220(12)	2.47(13)E-4
²⁷ Al	1705.509(22)	0.00080(5)	9.0(6)E-5	²⁷Al	7693.397(4)	0.0081(3)	0.00091(3)
²⁷Al	1778.92(3)d	0.232(4)	0.0261[95%]	²⁷Al	7724.027(4)	0.0493(15)	0.00554(17)
²⁷ Al	1864.33(3)	0.00091(4)	1.02(5)E-4	Silicon (Z=14), At.Wt.=28.0855(3), $\sigma_{\gamma}^Z = 0.172(5)$			
²⁷ Al	1927.527(25)	0.00262(7)	0.000294(8)	³⁰ Si	752.215(23)	0.00316(10)	0.000341(11)
²⁷ Al	1983.978(14)	0.00207(8)	2.32(9)E-4	³⁰ Si	1266.15(10)d	2.5(4)E-6	2.7E-7[12%]
²⁷ Al	2108.197(10)	0.00549(11)	0.000617(12)	²⁸Si	1273.349(17)	0.0289(6)	0.00312(7)
²⁷ Al	2138.833(10)	0.00424(9)	0.000476(10)	²⁸ Si	1446.176(22)	0.00134(13)	1.45(14)E-4
²⁷ Al	2170.70(3)	0.00082(5)	9.2(6)E-5	²⁸ Si	1867.32(3)	0.00129(14)	1.39(15)E-4
²⁷ Al	2255.37(3)	0.00109(5)	1.22(6)E-4	²⁸Si	2092.902(18)	0.0331(6)	0.00357(7)
²⁷ Al	2271.686(21)	0.00396(10)	0.000445(11)	²⁹ Si	2235.227(22)	0.00250(11)	0.000270(12)
²⁷Al	2282.794(9)	0.00890(17)	0.001000(19)	²⁸ Si	2425.767(23)	0.00494(15)	0.000533(16)
²⁷ Al	2451.565(11)	0.00106(7)	1.19(8)E-4	³⁰ Si	2780.552(22)	0.00241(13)	0.000260(14)
²⁷ Al	2577.701(12)	0.00412(10)	0.000463(11)	³⁰ Si	3054.321(23)	0.00245(14)	0.000264(15)
²⁷Al	2590.193(9)	0.00807(16)	0.000906(18)	²⁹ Si	3101.19(3)	0.00149(8)	1.61(9)E-4
²⁷ Al	2625.859(14)	0.00264(6)	0.000297(7)	²⁸Si	3538.966(22)	0.1190(20)	0.01284(22)
²⁷ Al	2709.62(3)	0.00140(7)	1.57(8)E-4	²⁸ Si	3660.713(23)	0.00703(21)	0.000759(23)
²⁷ Al	2821.444(7)	0.00752(15)	0.000845(17)	²⁹ Si	3864.900(23)	0.00166(9)	1.79(10)E-4
²⁷ Al	2954.47(7)	0.00098(5)	1.10(6)E-4	²⁸ Si	3954.39(3)	0.00449(19)	0.000484(21)
²⁷Al	3033.896(6)	0.0179(3)	0.00201(3)	²⁸Si	4933.889(24)	0.1120(23)	0.01209(25)
²⁷ Al	3265.538(13)	0.00082(6)	9.2(7)E-5	²⁸ Si	5106.693(22)	0.0064(3)	0.00069(3)
²⁷ Al	3303.146(10)	0.00241(7)	0.000271(8)	²⁸Si	6379.801(21)	0.0207(6)	0.00223(7)
²⁷ Al	3346.970(13)	0.00111(5)	1.25(6)E-4	²⁹ Si	6743.25(3)	0.00170(9)	1.83(10)E-4
²⁷ Al	3391.699(23)	0.00117(5)	1.31(6)E-4	²⁸Si	7199.199(23)	0.0125(4)	0.00135(4)
²⁷Al	3465.058(7)	0.0146(3)	0.00164(3)	²⁸ Si	8472.209(23)	0.00381(18)	0.000411(19)
²⁷ Al	3560.555(8)	0.00206(8)	2.31(9)E-4	Phosphorus (Z=15), At.Wt.=30.973761(2), $\sigma_{\gamma}^Z = 0.172(6)$			
²⁷Al	3591.189(8)	0.01000(21)	0.001123(24)	³¹P	78.083(20)	0.059(3)	0.0058(3)
²⁷ Al	3708.939(14)	0.00088(8)	9.9(9)E-5	³¹P	512.646(19)	0.079(4)	0.0077(4)
²⁷ Al	3789.326(12)	0.00191(8)	2.15(9)E-4	³¹P	558.46(7)	0.0010(3)	1.0(3)E-4
²⁷ Al	3823.909(23)	0.00114(7)	1.28(8)E-4	³¹P	636.663(21)	0.0311(14)	0.00304(14)
²⁷ Al	3849.111(8)	0.00699(17)	0.000785(19)	³¹P	744.99(5)	0.00101(5)	9.9(5)E-5
²⁷ Al	3875.487(8)	0.00618(14)	0.000694(16)	³¹P	1034.16(4)	0.00206(11)	2.02(11)E-4
²⁷ Al	4015.658(13)	0.00166(7)	1.86(8)E-4	³¹P	1071.217(23)	0.0249(12)	0.00244(12)
²⁷Al	4133.407(7)	0.0149(3)	0.00167(3)	³¹P	1149.298(19)	0.00380(19)	0.000372(19)
²⁷Al	4259.534(7)	0.0153(3)	0.00172(3)	³¹P	1244.64(3)	0.00357(17)	0.000349(17)
²⁷ Al	4377.618(12)	0.00103(8)	1.16(9)E-4	³¹P	1322.72(3)	0.00529(25)	0.000518(24)
²⁷ Al	4428.414(13)	0.00185(8)	2.08(9)E-4	³¹P	1353.56(5)	0.00126(7)	1.23(7)E-4
²⁷ Al	4660.043(5)	0.00605(16)	0.000680(18)	³¹P	1508.85(3)	0.00318(16)	0.000311(16)
²⁷Al	4690.676(5)	0.01090(24)	0.00122(3)	³¹P	1676.84(3)	0.00405(20)	0.000396(20)
²⁷Al	4733.844(11)	0.0126(3)	0.00142(3)	³¹P	1739.14(5)	0.00201(10)	1.97(10)E-4
²⁷ Al	4736.92(10)	0.00100(22)	1.12(25)E-4	³¹P	1873.52(4)	0.00320(16)	0.000313(16)
²⁷ Al	4754.377(24)	0.00080(7)	9.0(8)E-5	³¹P	1941.05(3)	0.00413(20)	0.000404(20)
²⁷ Al	4764.477(11)	0.00210(10)	2.36(11)E-4	³¹P	2114.47(3)	0.0115(5)	0.00113(5)
²⁷ Al	4903.113(6)	0.00716(18)	0.000804(20)	³¹P	2151.52(4)	0.0100(5)	0.00098(5)
²⁷ Al	5103.711(8)	0.00097(6)	1.09(7)E-4	³¹P	2156.90(4)	0.0128(6)	0.00125(6)
²⁷ Al	5134.343(8)	0.00722(23)	0.00081(3)	³¹P	2227.50(5)	0.00248(15)	2.43(15)E-4
²⁷ Al	5302.642(11)	0.00124(9)	1.39(10)E-4	³¹P	2229.59(3)	0.00080(9)	7.8(9)E-5
²⁷ Al	5411.077(8)	0.00481(19)	0.000540(21)	³¹P	2234.07(6)	0.00123(8)	1.20(8)E-4
²⁷ Al	5585.651(11)	0.00279(12)	0.000313(13)	³¹P	2426.29(3)	0.00265(13)	0.000259(13)
²⁷ Al	5709.853(13)	0.00148(8)	1.66(9)E-4	³¹P	2514.65(4)	0.00156(9)	1.53(9)E-4
²⁷ Al	5766.296(25)	0.00091(8)	1.02(9)E-4	³¹P	2579.27(6)	0.00082(6)	8.0(6)E-5
²⁷ Al	6101.529(18)	0.00570(21)	0.000640(24)	³¹P	2586.00(4)	0.0089(4)	0.00087(4)
²⁷ Al	6198.143(11)	0.00210(14)	2.36(16)E-4	³¹P	2657.35(6)	0.00252(14)	2.47(14)E-4
²⁷ Al	6316.024(9)	0.00500(20)	0.000562(22)	³¹P	2740.11(5)	0.00085(5)	8.3(5)E-5

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
³¹ P	2863.01(7)	0.00359(18)	0.000351(18)	³² S	4430.60(4)	0.0262(6)	0.00248(6)
³¹ P	2885.99(3)	0.0064(3)	0.00063(3)	³⁴ S	4637.981(14)	0.00734(22)	0.000694(21)
³¹P	3058.17(4)	0.0110(4)	0.00108(4)	³²S	4869.61(3)	0.0650(13)	0.00614(12)
³¹ P	3185.61(3)	0.00326(12)	0.000319(12)	³² S	5047.10(3)	0.0163(4)	0.00154(4)
³¹P	3273.98(4)	0.0083(3)	0.00081(3)	³²S	5420.574(24)	0.308(7)	0.0291(7)
³¹ P	3365.98(5)	0.00112(5)	1.10(5)E-4	³² S	5583.50(3)	0.0086(3)	0.00081(3)
³¹ P	3444.06(5)	0.00121(5)	1.18(5)E-4	³² S	5887.96(3)	0.00373(17)	0.000353(16)
³¹P	3522.59(3)	0.0219(8)	0.00214(8)	³² S	7799.815(24)	0.0144(5)	0.00136(5)
³¹ P	3548.73(4)	0.00135(6)	1.32(6)E-4	³² S	8640.594(25)	0.0098(7)	0.00093(7)
³¹ P	3554.31(5)	0.00084(4)	8.2(4)E-5	Chlorine (Z=17), At.Wt.=35.453(2), $\sigma_{\gamma}^Z = 33.1(3)$			
³¹P	3899.89(3)	0.0294(10)	0.00288(10)	³⁵ Cl	292.177(8)	0.0893(10)	0.00763(9)
³¹ P	3922.87(7)	0.00302(12)	0.000295(12)	³⁵ Cl	436.222(4)	0.3090(20)	0.02641(17)
³¹ P	3926.48(5)	0.00368(14)	0.000360(14)	³⁵ Cl	508.866(4)	0.108(17)	0.0092(15)
³¹ P	3930.52(5)	0.00108(5)	1.06(5)E-4	³⁵ Cl	517.0730(10)	7.58(5)	0.648(4)
³¹ P	3957.10(3)	0.00102(5)	9.98(5)E-5	³⁵ Cl	632.437(5)	0.1110(16)	0.00949(14)
³¹ P	4008.59(5)	0.00122(5)	1.19(5)E-4	³⁵ Cl	786.3020(10)	3.420(7)	0.2923(6)
³¹ P	4199.87(4)	0.0055(3)	0.00054(3)	³⁵ Cl	788.4280(10)	5.42(5)	0.463(4)
³¹ P	4359.57(3)	0.00195(7)	1.91(7)E-4	³⁵ Cl	936.920(8)	0.1720(13)	0.01470(11)
³¹ P	4364.30(4)	0.0073(3)	0.00071(3)	³⁵ Cl	1034.27(22)	0.100(16)	0.0085(14)
³¹ P	4491.00(4)	0.00323(12)	0.000316(12)	³⁵ Cl	1131.250(9)	0.626(3)	0.0535(3)
³¹ P	4628.94(4)	0.00082(10)	8.0(10)E-5	³⁵ Cl	1162.7390(20)	0.76(3)	0.065(3)
³¹ P	4661.07(4)	0.00568(21)	0.000556(21)	³⁵ Cl	1164.8650(10)	8.91(4)	0.762(3)
³¹P	4671.37(3)	0.0194(7)	0.00190(7)	³⁵ Cl	1170.946(4)	0.154(5)	0.0132(4)
³¹ P	4876.87(4)	0.00111(9)	1.09(9)E-4	³⁵ Cl	1327.405(9)	0.4020(23)	0.03436(20)
³¹ P	4912.30(5)	0.00114(5)	1.12(5)E-4	³⁵ Cl	1372.872(12)	0.105(4)	0.0090(3)
³¹ P	5194.91(5)	0.00236(23)	2.31(23)E-4	³⁵ Cl	1601.072(4)	1.210(7)	0.1034(6)
³¹ P	5265.51(4)	0.0058(4)	0.00057(4)	³⁵ Cl	1627.04(8)	0.094(5)	0.0080(4)
³¹ P	5277.66(6)	0.00188(9)	1.84(9)E-4	³⁵ Cl	1640.099(10)	0.158(17)	0.0135(15)
³¹ P	5699.99(4)	0.00102(4)	9.98(4)E-5	³⁵ Cl	1648.306(9)	0.174(5)	0.0149(4)
³¹ P	5705.37(3)	0.00428(16)	0.000419(16)	³⁵ Cl	1729.929(9)	0.107(12)	0.0091(10)
³¹ P	5778.06(4)	0.00152(6)	1.49(6)E-4	³⁵ Cl	1787.82(8)	0.177(6)	0.0151(5)
³¹P	6785.504(24)	0.0267(15)	0.00261(15)	³⁵ Cl	1828.49(4)	0.111(5)	0.0095(4)
³¹P	7422.022(25)	0.0082(3)	0.00080(3)	³⁵ Cl	1936.97(5)	0.153(9)	0.0131(8)
³¹ P	7856.48(3)	0.00150(8)	1.47(8)E-4	³⁵ Cl	1951.1400(20)	6.33(4)	0.541(3)
Sulfur (Z=16), At.Wt.=32.065(5), $\sigma_{\gamma}^Z = 0.534(10)$				³⁵ Cl	1959.346(4)	4.10(3)	0.350(3)
³⁶ S	646.171(14)	4.5(5)E-5	4.3(5)E-6	³⁵ Cl	1975.22(7)	0.214(22)	0.0183(19)
³²S	840.993(13)	0.347(6)	0.0328(6)	³⁷ Cl	1980.94(7)	0.045(4)	0.0038(3)
³² S	1472.401(14)	0.00870(19)	0.000822(18)	³⁵ Cl	2022.091(7)	0.161(6)	0.0138(5)
³⁴ S	1572.333(6)	0.00408(12)	0.000386(11)	³⁵ Cl	2034.63(3)	0.239(5)	0.0204(4)
³² S	1697.24(3)	0.01250(25)	0.001181(24)	³⁵ Cl	2041.40(6)	0.121(5)	0.0103(4)
³² S	1964.86(3)	0.00659(22)	0.000623(21)	³⁵ Cl	2075.440(13)	0.252(7)	0.0215(6)
³² S	1967.11(3)	0.00357(18)	0.000337(17)	³⁵ Cl	2104(5)	0.105(7)	0.0090(6)
³³ S	2127.491(12)	0.00246(10)	2.32(10)E-4	³⁵ Cl	2156.19(4)	0.205(7)	0.0175(6)
³² S	2216.722(17)	0.01210(23)	0.001144(22)	³⁷ Cl	2166.90(20)d	0.0568(15)	0.00486[40%]
³² S	2313.354(17)	0.00366(13)	0.000346(12)	³⁵ Cl	2179.51(4)	0.12(5)	0.010(4)
³⁴ S	2347.695(7)	0.0060(3)	0.00057(3)	³⁵ Cl	2200.10(4)	0.123(5)	0.0105(4)
³²S	2379.661(14)	0.208(5)	0.0197(5)	³⁵ Cl	2289.78(16)	0.102(14)	0.0087(12)
³² S	2490.14(3)	0.0125(3)	0.00118(3)	³⁵ Cl	2311.38(4)	0.35(10)	0.030(9)
³² S	2753.16(3)	0.0277(5)	0.00262(5)	³⁵ Cl	2468.1830(20)	0.097(8)	0.0083(7)
³² S	2867.580(23)	0.00425(15)	0.000402(14)	³⁵ Cl	2469.97(3)	0.24(3)	0.021(3)
³²S	2930.67(3)	0.0832(13)	0.00786(12)	³⁵ Cl	2478(5)	0.101(20)	0.0086(17)
³⁶ S	3103.36d	2.8(14)E-5	2.7E-6[88%]	³⁵ Cl	2489.74(9)	0.141(6)	0.0121(5)
³²S	3220.588(17)	0.117(5)	0.0111(5)	³⁵ Cl	2492.223(9)	0.11(4)	0.009(3)
³² S	3369.70(4)	0.0271(5)	0.00256(5)	³⁵ Cl	2529.2(11)	0.121(13)	0.0103(11)
³² S	3397.37(3)	0.00544(15)	0.000514(14)	³⁵ Cl	2537.25(7)	0.135(14)	0.0115(12)
³² S	3723.54(4)	0.0133(3)	0.00126(3)	³⁵ Cl	2549.74(7)	0.090(15)	0.0077(13)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
³⁵ Cl	2622.86(5)	0.178(6)	0.0152(5)	³⁵ Cl	6086.804(20)	0.295(15)	0.0252(13)
³⁵ Cl	2676.31(3)	0.533(4)	0.0456(3)	³⁵ Cl	6110.842(18)	6.59(6)	0.563(5)
³⁵ Cl	2797.90(4)	0.095(10)	0.0081(9)	³⁵ Cl	6267.63(4)	0.13(4)	0.011(3)
³⁵ Cl	2800.96(12)	0.183(7)	0.0156(6)	³⁵ Cl	6619.615(19)	2.530(23)	0.2163(20)
³⁵ Cl	2808.86(7)	0.10(5)	0.009(4)	³⁵ Cl	6627.821(18)	1.470(16)	0.1257(14)
³⁵ Cl	2810.988(9)	0.144(7)	0.0123(6)	³⁵ Cl	6977.836(19)	0.741(10)	0.0633(9)
³⁵ Cl	2845.50(3)	0.349(3)	0.0298(3)	³⁵ Cl	7413.968(18)	3.29(5)	0.281(4)
³⁵ Cl	2863.819(12)	1.820(10)	0.1556(9)	³⁵ Cl	7790.330(18)	2.66(3)	0.227(3)
³⁵ Cl	2866.9(5)	0.192(12)	0.0164(10)	³⁵ Cl	8578.575(18)	0.883(13)	0.0755(11)
³⁵ Cl	2876.49(5)	0.164(7)	0.0140(6)	Argon (Z=18), At.Wt.=39.948(1), σ_γ^Z = 0.675(10)			
³⁵ Cl	2896.212(8)	0.146(6)	0.0125(5)	⁴⁰ Ar	167.30(20)	0.53(5)	0.040(4)
³⁵ Cl	2975.21(7)	0.377(4)	0.0322(3)	⁴⁰ Ar	348.7(3)	0.044(9)	0.0033(7)
³⁵ Cl	2994.548(15)	0.279(8)	0.0238(7)	⁴⁰ Ar	516.0(3)	0.167(17)	0.0127(13)
³⁵ Cl	3001.07(5)	0.216(7)	0.0185(6)	⁴⁰ Ar	518.7	0.0060(20)	0.00046(15)
³⁵ Cl	3015.97(4)	0.328(3)	0.0280(3)	⁴⁰ Ar	837.7(3)	0.063(7)	0.0048(5)
³⁵ Cl	3061.82(4)	1.130(7)	0.0966(6)	⁴⁰ Ar	867.3(6)	0.0070(20)	0.00053(15)
³⁵ Cl	3116.04(5)	0.297(3)	0.0254(3)	⁴⁰ Ar	1044.3(4)	0.040(8)	0.0030(6)
³⁵ Cl	3332.87(8)	0.241(7)	0.0206(6)	⁴⁰ Ar	1186.8(3)	0.34(3)	0.0258(23)
³⁵ Cl	3374.7(11)	0.179(7)	0.0153(6)	⁴⁰ Ar	1354.0(4)	0.015(4)	0.0011(3)
³⁵ Cl	3428.83(5)	0.271(3)	0.0232(3)	³⁶ Ar	1409.7(10)	0.0060(12)	0.00046(9)
³⁵ Cl	3500.35(9)	0.100(6)	0.0085(5)	⁴⁰ Ar	1828.8(12)	0.0070(20)	0.00053(15)
³⁵ Cl	3561.37(7)	0.21(4)	0.018(3)	⁴⁰ Ar	1881.5(10)	0.009(3)	0.00068(23)
³⁵ Cl	3566.32(4)	0.093(24)	0.0079(21)	⁴⁰ Ar	2130.8(8)	0.029(5)	0.0022(4)
³⁵ Cl	3589.16(13)	0.18(5)	0.015(4)	⁴⁰ Ar	2432.5(8)	0.0055(14)	0.00042(11)
³⁵ Cl	3599.350(9)	0.164(6)	0.0140(5)	³⁶ Ar	2490.8(8)	0.0088(22)	0.00067(17)
³⁵ Cl	3604.14(17)	0.119(6)	0.0102(5)	⁴⁰ Ar	2566.1(8)	0.018(4)	0.0014(3)
³⁵ Cl	3634.75(3)	0.098(6)	0.0084(5)	⁴⁰ Ar	2614.4(8)	0.019(4)	0.0014(3)
³⁵ Cl	3749.91(10)	0.096(5)	0.0082(4)	⁴⁰ Ar	2771.9(8)	0.057(9)	0.0043(7)
³⁵ Cl	3821.33(16)	0.320(10)	0.0274(9)	⁴⁰ Ar	2781.8(15)	0.011(3)	0.00083(23)
³⁵ Cl	3825.22(13)	0.250(9)	0.0214(8)	⁴⁰ Ar	2810.6(8)	0.039(8)	0.0030(6)
³⁵ Cl	3827.06(12)	0.238(17)	0.0203(15)	⁴⁰ Ar	2842.6(10)	0.0058(14)	0.00044(11)
³⁵ Cl	3962.67(4)	0.118(8)	0.0101(7)	⁴⁰ Ar	3089.5(10)	0.0070(20)	0.00053(15)
³⁵ Cl	3980.98(8)	0.331(7)	0.0283(6)	⁴⁰ Ar	3150.3(10)	0.026(5)	0.0020(4)
³⁵ Cl	4054.25(5)	0.194(8)	0.0166(7)	⁴⁰ Ar	3365.6(10)	0.028(6)	0.0021(5)
³⁵ Cl	4082.67(7)	0.263(5)	0.0225(4)	⁴⁰ Ar	3452.0(10)	0.013(3)	0.00099(23)
³⁵ Cl	4138.39(9)	0.113(17)	0.0097(15)	⁴⁰ Ar	3700.6(8)	0.065(7)	0.0049(5)
³⁵ Cl	4138.73(4)	0.095(10)	0.0081(9)	⁴⁰ Ar	4745.3(8)	0.36(4)	0.027(3)
³⁵ Cl	4298.33(4)	0.122(10)	0.0104(9)	⁴⁰ Ar	5582.4(8)	0.077(8)	0.0058(6)
³⁵ Cl	4440.39(4)	0.377(4)	0.0322(3)	³⁶ Ar	6298.9(10)	0.0076(19)	0.00058(14)
³⁵ Cl	4524.87(4)	0.148(7)	0.0127(6)	Potassium (Z=19), At.Wt.=39.0983(1), σ_γ^Z = 2.06(19)			
³⁵ Cl	4547.5(5)	0.146(8)	0.0125(7)	³⁹ K	29.8300(10)	1.380(20)	0.1070(16)
³⁵ Cl	4616.45(9)	0.210(10)	0.0180(9)	⁴¹ K	106.836(7)	0.0320(6)	0.00248(5)
³⁵ Cl	4728.94(4)	0.223(9)	0.0191(8)	³⁹ K	522.319(7)	0.0347(7)	0.00269(5)
³⁵ Cl	4944.36(4)	0.379(8)	0.0324(7)	³⁹ K	646.222(5)	0.0451(8)	0.00350(6)
³⁵ Cl	4945.25(3)	0.194(18)	0.0166(15)	⁴¹ K	681.937(8)	0.0149(5)	0.00115(4)
³⁵ Cl	4979.759(20)	1.230(10)	0.1051(9)	³⁹ K	770.3050(20)	0.903(12)	0.0700(9)
³⁵ Cl	4989.66(12)	0.10(6)	0.009(5)	³⁹ K	843.468(10)	0.0197(5)	0.00153(4)
³⁵ Cl	5017.74(7)	0.161(8)	0.0138(7)	³⁹ K	891.385(13)	0.019(4)	0.0015(3)
³⁵ Cl	5246.958(21)	0.195(10)	0.0167(9)	³⁹ K	1086.707(16)	0.0222(7)	0.00172(5)
³⁵ Cl	5517.25(4)	0.560(5)	0.0479(4)	³⁹ K	1158.887(10)	0.1600(25)	0.01240(19)
³⁵ Cl	5584.525(23)	0.158(11)	0.0135(9)	³⁹ K	1247.193(11)	0.0784(13)	0.00608(10)
³⁵ Cl	5603.76(9)	0.11(3)	0.009(3)	⁴⁰ K	1293.589(5)	0.0041(8)	0.00032(6)
³⁵ Cl	5702.58(6)	0.127(10)	0.0109(9)	³⁹ K	1303.515(19)	0.0550(12)	0.00426(9)
³⁵ Cl	5715.244(21)	1.820(16)	0.1556(14)	³⁹ K	1373.227(18)	0.0251(7)	0.00195(5)
³⁵ Cl	5733.56(3)	0.161(11)	0.0138(9)	⁴⁰ K	1460.822(6)	3.24(5) s⁻¹g⁻¹	Abundant
³⁵ Cl	5902.74(3)	0.372(4)	0.0318(3)	³⁹ K	1480.024(24)	0.0353(9)	0.00274(7)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
³⁹ K	1489.676(10)	0.0277(8)	0.00215(6)	³⁹ K	4384.88(3)	0.0247(11)	0.00191(9)
⁴¹ K	1524.6(3)d	0.02000(4)	0.001550[2.8%]	³⁹ K	4507.03(3)	0.0159(9)	0.00123(7)
³⁹ K	1613.756(10)	0.1190(20)	0.00922(16)	³⁹ K	4670.76(3)	0.0138(9)	0.00107(7)
³⁹ K	1618.973(10)	0.1300(21)	0.01008(16)	³⁹ K	4991.34(3)	0.0432(14)	0.00335(11)
³⁹ K	1704.656(23)	0.0244(8)	0.00189(6)	³⁹ K	5012.48(3)	0.0226(11)	0.00175(9)
³⁹ K	1795.438(24)	0.0292(8)	0.00226(6)	³⁹ K	5042.507(25)	0.0351(15)	0.00272(12)
³⁹ K	1825.815(19)	0.0147(7)	0.00114(5)	³⁹ K	5068.870(21)	0.0224(12)	0.00174(9)
³⁹ K	1929.169(10)	0.0397(9)	0.00308(7)	³⁹ K	5173.196(21)	0.048(3)	0.00372(23)
³⁹ K	1956.515(24)	0.0406(11)	0.00315(9)	³⁹ K	5380.018(16)	0.146(4)	0.0113(3)
³⁹ K	2007.69(3)	0.0513(12)	0.00398(9)	³⁹ K	5508.660(21)	0.066(4)	0.0051(3)
³⁹ K	2017.472(11)	0.0540(12)	0.00419(9)	³⁹ K	5695.442(20)	0.114(3)	0.00884(23)
³⁹ K	2039.924(18)	0.0519(13)	0.00402(10)	³⁹ K	5729.308(22)	0.0437(18)	0.00339(14)
³⁹ K	2047.301(11)	0.0537(13)	0.00416(10)	³⁹ K	5751.758(17)	0.108(3)	0.00837(23)
³⁹ K	2069.752(18)	0.0363(10)	0.00281(8)	³⁹ K	6998.758(14)	0.0447(20)	0.00346(16)
³⁹ K	2073.793(19)	0.1370(24)	0.01062(19)	³⁹ K	7768.919(14)	0.117(7)	0.0091(5)
³⁹ K	2153.86(3)	0.0158(7)	0.00122(5)	Calcium (Z=20), At.Wt.=40.078(4), σ_γ^Z = 0.431(19)			
³⁹ K	2206.22(4)	0.0166(12)	0.00129(9)	⁴⁴ Ca	174.12(7)	0.0168(4)	0.00127(3)
³⁹ K	2206.26(3)	0.0157(17)	0.00122(13)	⁴⁰ Ca	519.66(5)	0.0503(13)	0.00380(10)
³⁹ K	2230.54(3)	0.0202(10)	0.00157(8)	⁴⁰ Ca	660.00(5)	0.00487(18)	0.000368(14)
³⁹ K	2290.420(19)	0.0582(13)	0.00451(10)	⁴⁰ Ca	727.17(5)	0.0117(4)	0.00088(3)
³⁹ K	2346.22(4)	0.0138(7)	0.00107(5)	⁴³ Ca	1126.12(10)	0.00471(23)	0.000356(17)
³⁹ K	2367.30(3)	0.0157(7)	0.00122(5)	⁴⁰ Ca	1150.95(5)	0.0052(3)	0.000393(23)
³⁹ K	2389.245(10)	0.0301(10)	0.00233(8)	⁴³ Ca	1156.94(12)	0.0088(4)	0.00067(3)
³⁹ K	2545.99(3)	0.0536(12)	0.00415(9)	⁴⁴ Ca	1260.62(6)	0.00394(24)	0.000298(18)
³⁹ K	2609.97(3)	0.0213(7)	0.00165(5)	⁴⁰ Ca	1389.82(5)	0.0106(4)	0.00080(3)
³⁹ K	2614.18(3)	0.0165(6)	0.00128(5)	⁴⁰ Ca	1481.67(5)	0.0051(3)	0.000386(23)
³⁹ K	2638.866(24)	0.0144(6)	0.00112(5)	⁴⁰ Ca	1670.60(6)	0.0069(3)	0.000522(23)
³⁹ K	2726.780(24)	0.0225(9)	0.00174(7)	⁴⁴ Ca	1725.71(7)	0.0090(4)	0.00068(3)
³⁹ K	2756.678(17)	0.0404(22)	0.00313(17)	⁴⁰ Ca	1942.67(3)	0.352(7)	0.0266(5)
³⁹ K	2799.04(3)	0.0145(7)	0.00112(5)	⁴⁰ Ca	2001.31(3)	0.0659(15)	0.00498(11)
³⁹ K	2806.42(3)	0.0256(9)	0.00198(7)	⁴⁰ Ca	2009.84(3)	0.0409(10)	0.00309(8)
³⁹ K	2938.17(3)	0.0140(9)	0.00109(7)	⁴⁶ Ca	2013.57(20)	2.90E-05	2.20E-06
³⁹ K	3055.30(3)	0.0464(12)	0.00360(9)	⁴⁰ Ca	2290.43(5)	0.0077(4)	0.00058(3)
³⁹ K	3262.28(4)	0.0376(11)	0.00291(9)	⁴⁰ Ca	2605.34(6)	0.0061(4)	0.00046(3)
³⁹ K	3304.17(4)	0.0146(7)	0.00113(5)	⁴⁰ Ca	2660.37(7)	0.0074(4)	0.00056(3)
³⁹ K	3338.05(6)	0.036(17)	0.0028(13)	⁴⁰ Ca	2767.92(7)	0.0070(15)	0.00053(11)
³⁹ K	3348.72(3)	0.0172(8)	0.00133(6)	⁴⁰ Ca	2810.06(5)	0.0167(5)	0.00126(4)
³⁹ K	3403.58(3)	0.0167(8)	0.00129(6)	⁴⁸ Ca	3084.40(10)d	0.00190(21)	1.44E-4[79%]
³⁹ K	3453.38(3)	0.0247(14)	0.00191(11)	⁴⁰ Ca	3584.77(7)	0.0100(5)	0.00076(4)
³⁹ K	3518.77(6)	0.0186(9)	0.00144(7)	⁴⁰ Ca	3609.80(6)	0.0283(9)	0.00214(7)
³⁹ K	3526.97(3)	0.0170(9)	0.00132(7)	⁴⁰ Ca	3759.48(7)	0.0117(5)	0.00088(4)
³⁹ K	3545.71(3)	0.0746(18)	0.00578(14)	⁴⁰ Ca	4418.52(5)	0.0708(18)	0.00535(14)
³⁹ K	3650.37(3)	0.0355(13)	0.00275(10)	⁴⁰ Ca	4516.54(17)	0.0049(3)	0.000371(23)
³⁹ K	3688.54(3)	0.0276(11)	0.00214(9)	⁴⁰ Ca	4749.21(7)	0.0134(7)	0.00101(5)
³⁹ K	3694.91(4)	0.0231(10)	0.00179(8)	⁴⁰ Ca	4962.79(7)	0.0067(4)	0.00051(3)
³⁹ K	3736.81(3)	0.0193(6)	0.00150(5)	⁴⁸ Ca	5146.19(21)	0.00147(20)	1.11(15)E-4
³⁹ K	3778.97(4)	0.0143(7)	0.00111(5)	⁴⁴ Ca	5514.55(14)	0.0104(8)	0.00079(6)
³⁹ K	3911.43(5)	0.0168(9)	0.00130(7)	⁴⁰ Ca	5692.53(6)	0.0067(5)	0.00051(4)
³⁹ K	3930.63(4)	0.0275(11)	0.00213(9)	⁴² Ca	5885.87(16)	0.0024(4)	1.8(3)E-4
³⁹ K	3943.78(3)	0.0205(11)	0.00159(9)	⁴⁰ Ca	5900.02(6)	0.0258(12)	0.00195(9)
³⁹ K	3959.10(3)	0.0252(10)	0.00195(8)	⁴⁰ Ca	6419.59(5)	0.176(5)	0.0133(4)
³⁹ K	3977.89(3)	0.0219(10)	0.00170(8)	Scandium (Z=21), At.Wt.=44.955910(8), σ _γ ^Z = 27.20(20)			
³⁹ K	4001.80(3)	0.0263(11)	0.00204(9)	⁴⁵ Sc	52.0110(10)	0.87(3)	0.0586(20)
³⁹ K	4060.91(3)	0.0244(10)	0.00189(8)	⁴⁵ Sc	142.528(8)d	4.88(7)	0.329[99%]
³⁹ K	4135.586(23)	0.0563(17)	0.00436(13)	⁴⁵ Sc	147.011(10)	6.08(9)	0.410(6)
³⁹ K	4200.04(3)	0.0398(14)	0.00308(11)	⁴⁵ Sc	216.44(4)	2.49(4)	0.168(3)
³⁹ K	4360.201(25)	0.0776(21)	0.00601(16)				

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁴⁵ Sc	227.773(12)	7.13(11)	0.481(7)	⁴⁵ Sc	1900.85(4)	0.274(11)	0.0185(7)
⁴⁵ Sc	228.716(12)	3.31(5)	0.223(3)	⁴⁵ Sc	1913.59(6)	0.077(7)	0.0052(5)
⁴⁵ Sc	280.726(12)	0.248(7)	0.0167(5)	⁴⁵ Sc	1966.59(8)	0.080(8)	0.0054(5)
⁴⁵ Sc	295.243(10)	3.97(11)	0.268(7)	⁴⁵ Sc	1975.36(6)	0.078(8)	0.0053(5)
⁴⁵ Sc	399.691(19)	0.202(7)	0.0136(5)	⁴⁵ Sc	2005.24(4)	0.351(11)	0.0237(7)
⁴⁵ Sc	402.87(5)	0.107(6)	0.0072(4)	⁴⁵ Sc	2058.84(9)	0.097(10)	0.0065(7)
⁴⁵ Sc	442.254(13)	0.096(6)	0.0065(4)	⁴⁵ Sc	2106.25(8)	0.143(11)	0.0096(7)
⁴⁵ Sc	478.14(13)	0.073(10)	0.0049(7)	⁴⁵ Sc	2110.20(10)	0.117(11)	0.0079(7)
⁴⁵ Sc	486.026(21)	0.593(14)	0.0400(9)	⁴⁵ Sc	2114.14(6)	0.210(13)	0.0142(9)
⁴⁵ Sc	539.437(20)	0.738(19)	0.0497(13)	⁴⁵ Sc	2129.69(4)	0.101(10)	0.0068(7)
⁴⁵ Sc	547.15(4)	0.373(12)	0.0251(8)	⁴⁵ Sc	2203.45(13)	0.102(10)	0.0069(7)
⁴⁵ Sc	554.44(4)	1.82(4)	0.123(3)	⁴⁵ Sc	2243.06(6)	0.110(11)	0.0074(7)
⁴⁵ Sc	584.785(13)	1.77(3)	0.1193(20)	⁴⁵ Sc	2351.59(15)	0.074(9)	0.0050(6)
⁴⁵ Sc	627.462(18)	2.23(5)	0.150(3)	⁴⁵ Sc	2362.36(9)	0.085(9)	0.0057(6)
⁴⁵ Sc	643.037(25)	0.259(9)	0.0175(6)	⁴⁵ Sc	2373.41(17)	0.086(9)	0.0058(6)
⁴⁵ Sc	685.71(3)	0.149(9)	0.0100(6)	⁴⁵ Sc	2404.82(7)	0.127(10)	0.0086(7)
⁴⁵ Sc	711.21(6)	0.104(8)	0.0070(5)	⁴⁵ Sc	2410.40(4)	0.087(9)	0.0059(6)
⁴⁵ Sc	721.841(17)	0.487(15)	0.0328(10)	⁴⁵ Sc	2477.42(6)	0.145(14)	0.0098(9)
⁴⁵ Sc	773.851(17)	0.572(13)	0.0386(9)	⁴⁵ Sc	2502.20(10)	0.082(12)	0.0055(8)
⁴⁵ Sc	807.754(20)	0.523(13)	0.0353(9)	⁴⁵ Sc	2635.55(8)	0.301(15)	0.0203(10)
⁴⁵ Sc	835.16(4)	0.265(8)	0.0179(5)	⁴⁵ Sc	2667.03(11)	0.127(14)	0.0086(9)
⁴⁵ Sc	843.494(23)	0.138(6)	0.0093(4)	⁴⁵ Sc	2693.90(9)	0.107(14)	0.0072(9)
⁴⁵ Sc	860.707(19)	0.396(13)	0.0267(9)	⁴⁵ Sc	2697.12(8)	0.084(14)	0.0057(9)
⁴⁵ Sc	899.27(5)	0.133(9)	0.0090(6)	⁴⁵ Sc	2721.37(16)	0.096(8)	0.0065(5)
⁴⁵ Sc	941.95(5)	0.107(24)	0.0072(16)	⁴⁵ Sc	2797.52(10)	0.105(11)	0.0071(7)
⁴⁵ Sc	1015.22(3)	0.256(12)	0.0173(8)	⁴⁵ Sc	2991.04(11)	0.092(14)	0.0062(9)
⁴⁵ Sc	1057.89(3)	0.322(14)	0.0217(9)	⁴⁵ Sc	2995.96(11)	0.079(13)	0.0053(9)
⁴⁵ Sc	1082.52(4)	0.160(11)	0.0108(7)	⁴⁵ Sc	3011.73(8)	0.278(19)	0.0187(13)
⁴⁵ Sc	1123.17(5)	0.380(14)	0.0256(9)	⁴⁵ Sc	3049.06(7)	0.106(12)	0.0071(8)
⁴⁵ Sc	1134.43(8)	0.132(9)	0.0089(6)	⁴⁵ Sc	3080.8(5)	0.087(12)	0.0059(8)
⁴⁵ Sc	1166.45(6)	0.386(14)	0.0260(9)	⁴⁵ Sc	3265.48(7)	0.146(14)	0.0098(9)
⁴⁵ Sc	1227.77(4)	0.332(13)	0.0224(9)	⁴⁵ Sc	3281.87(8)	0.08(4)	0.005(3)
⁴⁵ Sc	1251.68(6)	0.101(9)	0.0068(6)	⁴⁵ Sc	3309.70(9)	0.08(3)	0.0054(20)
⁴⁵ Sc	1251.69(6)	0.129(23)	0.0087(16)	⁴⁵ Sc	3351.10(12)	0.121(14)	0.0082(9)
⁴⁵ Sc	1268.87(6)	0.10(3)	0.0067(20)	⁴⁵ Sc	3458.45(19)	0.156(15)	0.0105(10)
⁴⁵ Sc	1270.49(3)	0.269(13)	0.0181(9)	⁴⁵ Sc	3596.86(10)	0.077(14)	0.0052(9)
⁴⁵ Sc	1285.34(4)	0.373(19)	0.0251(13)	⁴⁵ Sc	3623.19(10)	0.13(6)	0.009(4)
⁴⁵ Sc	1321.18(4)	0.206(23)	0.0139(16)	⁴⁵ Sc	3799.13(8)	0.125(13)	0.0084(9)
⁴⁵ Sc	1321.96(4)	0.139(9)	0.0094(6)	⁴⁵ Sc	3878.05(12)	0.088(11)	0.0059(7)
⁴⁵ Sc	1335.05(3)	0.640(22)	0.0431(15)	⁴⁵ Sc	3999.48(12)	0.086(17)	0.0058(11)
⁴⁵ Sc	1510.13(6)	0.13(4)	0.009(3)	⁴⁵ Sc	4006.31(10)	0.091(17)	0.0061(11)
⁴⁵ Sc	1575.27(3)	0.317(13)	0.0214(9)	⁴⁵ Sc	4021.46(9)	0.092(17)	0.0062(11)
⁴⁵ Sc	1592.71(17)	0.11(3)	0.0074(20)	⁴⁵ Sc	4059.52(8)	0.18(3)	0.0121(20)
⁴⁵ Sc	1618.36(6)	0.362(19)	0.0244(13)	⁴⁵ Sc	4065.97(9)	0.079(19)	0.0053(13)
⁴⁵ Sc	1658.21(7)	0.107(12)	0.0072(8)	⁴⁵ Sc	4109.60(9)	0.073(10)	0.0049(7)
⁴⁵ Sc	1693.30(4)	0.465(19)	0.0313(13)	⁴⁵ Sc	4173.36(17)	0.11(3)	0.0074(20)
⁴⁵ Sc	1707.94(5)	0.077(10)	0.0052(7)	⁴⁵ Sc	4231.81(16)	0.073(9)	0.0049(6)
⁴⁵ Sc	1753.85(4)	0.170(12)	0.0115(8)	⁴⁵ Sc	4237.72(10)	0.096(17)	0.0065(11)
⁴⁵ Sc	1763.12(10)	0.077(10)	0.0052(7)	⁴⁵ Sc	4293.30(21)	0.073(11)	0.0049(7)
⁴⁵ Sc	1777.43(11)	0.125(12)	0.0084(8)	⁴⁵ Sc	4377.46(8)	0.127(15)	0.0086(10)
⁴⁵ Sc	1803.69(12)	0.075(9)	0.0051(6)	⁴⁵ Sc	4465.89(13)	0.106(13)	0.0071(9)
⁴⁵ Sc	1814.92(4)	0.271(13)	0.0183(9)	⁴⁵ Sc	4498.85(11)	0.149(15)	0.0100(10)
⁴⁵ Sc	1829.68(6)	0.152(10)	0.0102(7)	⁴⁵ Sc	4617.93(9)	0.089(15)	0.0060(10)
⁴⁵ Sc	1857.59(4)	0.393(17)	0.0265(11)	⁴⁵ Sc	4679.04(18)	0.112(14)	0.0075(9)
⁴⁵ Sc	1870.06(5)	0.206(13)	0.0139(9)	⁴⁵ Sc	4720.86(11)	0.171(16)	0.0115(11)
⁴⁵ Sc	1885.97(7)	0.090(11)	0.0061(7)	⁴⁵ Sc	4823.18(9)	0.078(11)	0.0053(7)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁴⁵ Sc	4891.84(10)	0.094(12)	0.0063(8)				
⁴⁵ Sc	4919.38(11)	0.092(13)	0.0062(9)				
⁴⁵ Sc	4974.76(9)	0.498(24)	0.0336(16)				
⁴⁵ Sc	4993.58(10)	0.177(15)	0.0119(10)				
⁴⁵ Sc	5085.09(10)	0.103(14)	0.0069(9)				
⁴⁵ Sc	5128.48(12)	0.093(15)	0.0063(10)				
⁴⁵ Sc	5163.42(10)	0.149(20)	0.0100(13)				
⁴⁵ Sc	5210.11(12)	0.085(15)	0.0057(10)				
⁴⁵ Sc	5267.04(7)	0.38(3)	0.0256(20)				
⁴⁵ Sc	5286.20(8)	0.123(15)	0.0083(10)				
⁴⁵ Sc	5335.89(8)	0.20(3)	0.0135(20)				
⁴⁵ Sc	5346.19(10)	0.094(19)	0.0063(13)				
⁴⁵ Sc	5445.75(8)	0.170(19)	0.0115(13)				
⁴⁵ Sc	5481.62(9)	0.142(19)	0.0096(13)				
⁴⁵ Sc	5555.57(10)	0.079(14)	0.0053(9)				
⁴⁵ Sc	5583.82(10)	0.118(16)	0.0080(11)				
⁴⁵ Sc	5624.09(8)	0.198(20)	0.0133(13)				
⁴⁵ Sc	5665.71(9)	0.145(19)	0.0098(13)				
⁴⁵ Sc	5678.79(13)	0.077(16)	0.0052(11)				
⁴⁵ Sc	5743.38(7)	0.184(17)	0.0124(11)				
⁴⁵ Sc	5781.24(15)	0.072(15)	0.0049(10)				
⁴⁵ Sc	5896.94(8)	0.42(3)	0.0283(20)				
⁴⁵ Sc	5904.31(12)	0.084(17)	0.0057(11)				
⁴⁵ Sc	5977.32(10)	0.075(12)	0.0051(8)				
⁴⁵ Sc	6046.15(9)	0.144(19)	0.0097(13)				
⁴⁵ Sc	6055.05(5)	0.265(24)	0.0179(16)				
⁴⁵ Sc	6097.64(10)	0.082(12)	0.0055(8)				
⁴⁵ Sc	6170.22(4)	0.47(5)	0.032(3)				
⁴⁵ Sc	6201.40(13)	0.073(8)	0.0049(5)				
⁴⁵ Sc	6300.79(8)	0.183(25)	0.0123(17)				
⁴⁵ Sc	6309.27(11)	0.075(8)	0.0051(5)				
⁴⁵ Sc	6317.86(4)	0.58(4)	0.039(3)				
⁴⁵ Sc	6329.00(13)	0.185(22)	0.0125(15)				
⁴⁵ Sc	6349.80(4)	0.53(4)	0.036(3)				
⁴⁵ Sc	6364.43(9)	0.119(20)	0.0080(13)				
⁴⁵ Sc	6457.68(7)	0.099(14)	0.0067(9)				
⁴⁵ Sc	6468.55(13)	0.122(21)	0.0082(14)				
⁴⁵ Sc	6507.47(10)	0.107(12)	0.0072(8)				
⁴⁵ Sc	6557.06(6)	0.384(24)	0.0259(16)				
⁴⁵ Sc	6640.96(6)	0.150(23)	0.0101(16)				
⁴⁵ Sc	6646.04(6)	0.113(12)	0.0076(8)				
⁴⁵ Sc	6716.79(4)	0.312(22)	0.0210(15)				
⁴⁵ Sc	6839.09(4)	0.95(4)	0.064(3)				
⁴⁵ Sc	6840.34(4)	0.76(11)	0.051(7)				
⁴⁵ Sc	6874.18(7)	0.125(14)	0.0084(9)				
⁴⁵ Sc	7117.46(3)	0.39(3)	0.0263(20)				
⁴⁵ Sc	7233.39(5)	0.110(14)	0.0074(9)				
⁴⁵ Sc	7489.58(3)	0.077(12)	0.0052(8)				
⁴⁵ Sc	7635.84(3)	0.40(3)	0.0270(20)				
⁴⁵ Sc	7924.84(4)	0.095(18)	0.0064(12)				
⁴⁵ Sc	8132.507(25)	0.48(3)	0.0324(20)				
⁴⁵ Sc	8175.176(21)	1.80(6)	0.121(4)				
⁴⁵ Sc	8315.73(4)	0.41(3)	0.0276(20)				
⁴⁵ Sc	8470.363(20)	0.120(14)	0.0081(9)				
⁴⁵ Sc	8532.122(20)	0.89(4)	0.060(3)				
⁴⁵ Sc	8759.850(20)	0.168(16)	0.0113(11)				

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
⁵¹ V	1358.498(19)	0.151(5)	0.0090(3)	⁵³ Cr	1241.33(7)	0.0140(5)	0.00082(3)
⁵¹ V	1401.641(16)	0.070(4)	0.00416(24)	⁵⁴ Cr	1528.00(20)d	3.800(12)E-6	2.215E-7[92%]
⁵¹ V	1418.793(15)	0.068(4)	0.00405(24)	⁵³Cr	1784.70(4)	0.1760(20)	0.01026(12)
⁵¹V	1434.10(3)d	4.81(10)	0.286[91%]	⁵⁰ Cr	1898.90(3)	0.0852(21)	0.00497(12)
⁵¹V	1558.843(18)	0.323(8)	0.0192(5)	⁵³ Cr	1994.52(6)	0.0545(14)	0.00318(8)
⁵⁰ V	1609.220(20)	0.0359(17)	0.00214(10)	⁵⁰ Cr	2001.05(5)	0.0199(10)	0.00116(6)
⁵¹ V	1611.758(25)	0.0236(15)	0.00140(9)	⁵² Cr	2105.8(5)	0.021(4)	0.00122(23)
⁵¹ V	1622.296(25)	0.0206(7)	0.00123(4)	⁵³Cr	2239.04(8)	0.186(3)	0.01084(17)
⁵¹ V	1634.068(22)	0.0359(19)	0.00214(11)	⁵² Cr	2320.8(3)	0.136(3)	0.00793(17)
⁵¹ V	1635.382(24)	0.020(4)	0.00119(24)	⁵⁰ Cr	2348.52(7)	0.0164(10)	0.00096(6)
⁵¹ V	1664.192(17)	0.0519(24)	0.00309(14)	⁵⁰ Cr	2376.49(5)	0.0362(9)	0.00211(5)
⁵¹ V	1732.563(20)	0.0161(16)	0.00096(10)	⁵³ Cr	2558.19(11)	0.0197(7)	0.00115(4)
⁵¹ V	1775.431(21)	0.027(6)	0.0016(4)	⁵³ Cr	2601.79(8)	0.0404(12)	0.00235(7)
⁵¹V	1777.961(19)	0.169(13)	0.0101(8)	⁵² Cr	2669.8(5)	0.0263(12)	0.00153(7)
⁵¹ V	1952.964(14)	0.0677(25)	0.00403(15)	⁵⁰ Cr	3021.27(12)	0.0139(8)	0.00081(5)
⁵¹ V	2020.749(18)	0.0214(17)	0.00127(10)	⁵³ Cr	3177.78(15)	0.0234(8)	0.00136(5)
⁵¹ V	2083.652(14)	0.0339(19)	0.00202(11)	⁵² Cr	3616.7(4)	0.0260(12)	0.00152(7)
⁵¹ V	2100.804(14)	0.0239(15)	0.00142(9)	⁵³ Cr	3719.70(6)	0.0675(24)	0.00393(14)
⁵¹ V	2145.826(18)	0.140(4)	0.00833(24)	⁵² Cr	4322.1(3)	0.0269(15)	0.00157(9)
⁵¹ V	2168.589(18)	0.0166(12)	0.00099(7)	⁵³ Cr	4847.56(8)	0.0346(15)	0.00202(9)
⁵¹ V	2410.436(21)	0.0253(17)	0.00151(10)	⁵³ Cr	4871.96(8)	0.0180(10)	0.00105(6)
⁵¹ V	2422.18(3)	0.112(24)	0.0067(14)	⁵⁰ Cr	5220.72(12)	0.0184(17)	0.00107(10)
⁵¹ V	2841.64(3)	0.0333(19)	0.00198(11)	⁵³ Cr	5268.15(11)	0.0465(25)	0.00271(15)
⁵¹ V	3032.60(9)	0.0249(20)	0.00148(12)	⁵² Cr	5268.9(5)	0.050(6)	0.0029(4)
⁵¹ V	3502.64(4)	0.0306(18)	0.00182(11)	⁵⁰ Cr	5489.85(14)	0.024(4)	0.00140(23)
⁵¹ V	3534.07(3)	0.0243(21)	0.00145(12)	⁵⁰ Cr	5493.99(12)	0.016(3)	0.00093(17)
⁵¹ V	3577.98(3)	0.0271(20)	0.00161(12)	⁵² Cr	5617.9(3)	0.132(5)	0.0077(3)
⁵¹ V	3715.86(3)	0.0256(21)	0.00152(12)	⁵³ Cr	5706.94(16)	0.024(4)	0.00140(23)
⁵¹ V	4116.821(23)	0.094(4)	0.00559(24)	⁵³ Cr	5858.72(9)	0.0266(21)	0.00155(12)
⁵¹ V	4452.20(3)	0.050(10)	0.0030(6)	⁵³ Cr	5999.80(7)	0.085(7)	0.0050(4)
⁵¹ V	4486.46(3)	0.0187(20)	0.00111(12)	⁵⁰ Cr	6134.58(9)	0.078(4)	0.00455(23)
⁵¹ V	4772.17(3)	0.018(6)	0.0011(4)	⁵⁴ Cr	6245.89(17)	0.0056(9)	0.00033(5)
⁵¹ V	4883.379(24)	0.073(4)	0.00434(24)	⁵³ Cr	6282.90(9)	0.036(3)	0.00210(17)
⁵¹ V	4992.94(4)	0.036(3)	0.00214(18)	⁵³ Cr	6326.49(12)	0.0212(23)	0.00124(13)
⁵¹V	5142.363(23)	0.200(6)	0.0119(4)	⁵⁰ Cr	6370.15(10)	0.028(17)	0.0016(10)
⁵¹V	5210.143(19)	0.244(20)	0.0145(12)	⁵³Cr	6645.61(8)	0.183(13)	0.0107(8)
⁵¹V	5515.813(23)	0.39(4)	0.0232(24)	⁵³Cr	6890.11(7)	0.042(3)	0.00245(17)
⁵¹ V	5551.32(3)	0.027(3)	0.00161(18)	⁵³Cr	7099.91(6)	0.146(9)	0.0085(5)
⁵¹ V	5578.358(24)	0.019(3)	0.00113(18)	⁵⁰ Cr	7361.12(8)	0.092(4)	0.00536(23)
⁵¹V	5752.064(22)	0.366(24)	0.0218(14)	⁵² Cr	7374.49(22)	0.080(4)	0.00466(23)
⁵¹ V	5892.101(20)	0.126(7)	0.0075(4)	⁵²Cr	7938.46(23)	0.424(11)	0.0247(6)
⁵¹V	6464.887(18)	0.43(4)	0.0256(24)	⁵⁰ Cr	8482.80(9)	0.169(7)	0.0098(4)
⁵¹ V	6517.282(19)	0.78(4)	0.0464(24)	⁵⁰ Cr	8510.77(8)	0.233(8)	0.0136(5)
⁵¹ V	6874.157(19)	0.49(6)	0.029(4)	⁵³ Cr	8884.36(5)	0.78(5)	0.045(3)
⁵¹ V	7162.898(15)	0.59(4)	0.0351(24)	⁵³ Cr	9719.06(5)	0.260(18)	0.0152(10)
⁵¹ V	7287.961(15)	0.056(4)	0.00333(24)	Manganese (Z=25), At.Wt.=54.938049(9), σ _γ ^Z =13.36(5)			
⁵¹ V	7293.572(16)	0.089(5)	0.0053(3)	⁵⁵ Mn	26.560(20)	3.42(4)	0.1887(22)
⁵¹V	7310.721(15)	0.227(9)	0.0135(5)	⁵⁵ Mn	83.884(23)	3.11(5)	0.172(3)
Chromium (Z=24), At.Wt.=51.9961(6), σ _γ ^Z =3.07(15)				⁵⁵ Mn	104.611(23)	1.74(3)	0.0960(17)
⁵⁰ Cr	27.97(7)	0.124(4)	0.00723(23)	⁵⁵ Mn	118.77(4)	0.0526(22)	0.00290(12)
⁵² Cr	564.05(12)	0.1130(20)	0.00659(12)	⁵⁵ Mn	123.46(4)	0.0612(23)	0.00338(13)
⁵⁰Cr	749.09(3)	0.569(9)	0.0332(5)	⁵⁵ Mn	188.521(22)	0.330(6)	0.0182(3)
⁵³Cr	834.849(22)	1.38(3)	0.0804(17)	⁵⁵ Mn	212.039(21)	2.13(3)	0.1175(17)
⁵⁰ Cr	888.95(7)	0.015(5)	0.0009(3)	⁵⁵ Mn	215.150(22)	0.168(3)	0.00927(17)
⁵³ Cr	989.074(23)	0.0139(5)	0.00081(3)	⁵⁵ Mn	230.096(24)	0.193(4)	0.01065(22)
⁵⁰ Cr	1149.83(3)	0.0214(4)	0.001247(23)	⁵⁵Mn	271.198(22)	0.94(6)	0.052(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁵⁵ Mn	274.32(5)	0.075(6)	0.0041(3)	⁵⁵ Mn	5198.52(13)	0.095(7)	0.0052(4)
⁵⁵Mn	314.398(20)	1.460(20)	0.0805(11)	⁵⁵ Mn	5253.98(12)	0.132(13)	0.0073(7)
⁵⁵ Mn	335.502(24)	0.147(3)	0.00811(17)	⁵⁵ Mn	5403.7(3)	0.050(6)	0.0028(3)
⁵⁵ Mn	341.01(3)	0.0912(25)	0.00503(14)	⁵⁵ Mn	5437.71(15)	0.087(7)	0.0048(4)
⁵⁵ Mn	354.12(4)	0.093(4)	0.00513(22)	⁵⁵Mn	5527.08(8)	0.788(22)	0.0435(12)
⁵⁵ Mn	375.192(22)	0.124(3)	0.00684(17)	⁵⁵ Mn	5761.23(11)	0.200(12)	0.0110(7)
⁵⁵Mn	454.378(21)	0.388(7)	0.0214(4)	⁵⁵Mn	5920.39(8)	1.06(3)	0.0585(17)
⁵⁵ Mn	459.754(23)	0.210(5)	0.0116(3)	⁵⁵ Mn	6031.03(18)	0.067(7)	0.0037(4)
⁵⁵ Mn	499.57(4)	0.0402(20)	0.00222(11)	⁵⁵ Mn	6104.29(12)	0.213(10)	0.0117(6)
⁵⁵ Mn	504.74(4)	0.096(4)	0.00530(22)	⁵⁵ Mn	6430.04(19)	0.088(7)	0.0049(4)
⁵⁵ Mn	716.20(5)	0.055(3)	0.00303(17)	⁵⁵Mn	6783.74(12)	0.378(17)	0.0209(9)
⁵⁵Mn	846.754(20)d	13.10(4)	0.7226[12%]	⁵⁵ Mn	6929.22(13)	0.248(12)	0.0137(7)
⁵⁵Mn	1810.72(4)d	3.62(11)	0.200[12%]	⁵⁵Mn	7057.89(9)	1.22(3)	0.0673(17)
⁵⁵ Mn	2016.47(5)	0.0527(25)	0.00291(14)	⁵⁵ Mn	7159.63(10)	0.643(24)	0.0355(13)
⁵⁵ Mn	2043.99(5)	0.243(5)	0.0134(3)	⁵⁵ Mn	7243.52(9)	1.36(3)	0.0750(17)
⁵⁵ Mn	2045.76(15)	0.0384(23)	0.00212(13)	⁵⁵ Mn	7270.14(12)	0.362(15)	0.0200(8)
⁵⁵ Mn	2062.81(4)	0.179(5)	0.0099(3)	Iron (Z=26), At.Wt.=55.845(2), $\sigma_{\gamma}^z = 2.56(13)$			
⁵⁵Mn	2113.05(4)d	1.91(5)	0.105[12%]	⁵⁶ Fe	14.411(14)	0.149(3)	0.00809(16)
⁵⁵ Mn	2175.91(5)	0.111(4)	0.00612(22)	⁵⁶ Fe	122.077(14)	0.096(3)	0.00521(16)
⁵⁵ Mn	2210.29(9)	0.080(5)	0.0044(3)	⁵⁶ Fe	136.488(14)	0.0118(3)	0.000640(16)
⁵⁵ Mn	2294.42(7)	0.112(6)	0.0062(3)	⁵⁶ Fe	230.270(13)	0.0274(5)	0.00149(3)
⁵⁵ Mn	2330.55(7)	0.191(8)	0.0105(4)	⁵⁸ Fe	287.025(19)	0.00218(15)	1.18(8)E-4
⁵⁵ Mn	2469.99(12)	0.083(6)	0.0046(3)	⁵⁶ Fe	352.347(12)	0.273(3)	0.01481(16)
⁵⁵ Mn	2677.20(19)	0.068(10)	0.0038(6)	⁵⁶ Fe	366.758(10)	0.0497(7)	0.00270(4)
⁵⁵ Mn	2873.23(11)	0.070(4)	0.00386(22)	⁵⁴ Fe	411.57(21)	0.022(5)	0.0012(3)
⁵⁵ Mn	2953.77(11)	0.069(5)	0.0038(3)	⁵⁶ Fe	569.885(19)	0.0139(3)	0.000754(16)
⁵⁵ Mn	3002.85(15)	0.055(5)	0.0030(3)	⁵⁶ Fe	657.46(11)	0.0067(18)	0.00036(10)
⁵⁵ Mn	3267.17(7)	0.188(6)	0.0104(3)	⁵⁶ Fe	691.960(19)	0.1370(18)	0.00743(10)
⁵⁵ Mn	3408.61(5)	0.303(10)	0.0167(6)	⁵⁷ Fe	810.71(3)	0.0274(9)	0.00149(5)
⁵⁵ Mn	3641.21(13)	0.061(5)	0.0034(3)	⁵⁷ Fe	863.80(5)	0.0072(4)	0.000391(22)
⁵⁵ Mn	3751.50(15)	0.054(5)	0.0030(3)	⁵⁷ Fe	867.4(4)	~0.007	~0.0004
⁵⁵ Mn	3813.99(9)	0.088(8)	0.0049(4)	⁵⁶ Fe	898.27(3)	0.0540(10)	0.00293(5)
⁵⁵ Mn	3820.48(16)	0.042(5)	0.0023(3)	⁵⁶ Fe	920.839(19)	0.0199(6)	0.00108(3)
⁵⁵ Mn	3927.8(3)	0.044(6)	0.0024(3)	⁵⁶ Fe	1018.93(3)	0.0507(11)	0.00275(6)
⁵⁵ Mn	3979.0(3)	0.039(5)	0.0022(3)	⁵⁶ Fe	1260.448(19)	0.0684(11)	0.00371(6)
⁵⁵ Mn	4222.85(17)	0.066(5)	0.0036(3)	⁵⁶ Fe	1358.540(22)	0.0211(6)	0.00115(3)
⁵⁵ Mn	4267.69(12)	0.078(6)	0.0043(3)	⁵⁶Fe	1612.786(18)	0.1530(22)	0.00830(12)
⁵⁵ Mn	4379.90(16)	0.073(6)	0.0040(3)	⁵⁶ Fe	1627.197(20)	0.0100(5)	0.00054(3)
⁵⁵ Mn	4445.06(20)	0.077(8)	0.0042(4)	⁵⁷ Fe	1674.31(21)	~0.007	~0.0004
⁵⁵ Mn	4549.70(23)	0.056(6)	0.0031(3)	⁵⁷ Fe	1674.49(6)	~0.007	~0.0004
⁵⁵ Mn	4566.56(10)	0.197(9)	0.0109(5)	⁵⁶ Fe	1722.38(10)	0.0074(6)	0.00040(3)
⁵⁵ Mn	4588.23(18)	0.053(5)	0.0029(3)	⁵⁶Fe	1725.288(21)	0.181(3)	0.00982(16)
⁵⁵ Mn	4643.40(13)	0.073(10)	0.0040(6)	⁵⁶ Fe	1810.54(16)	0.0067(7)	0.00036(4)
⁵⁵ Mn	4689.14(11)	0.120(9)	0.0066(5)	⁵⁶ Fe	1965.39(15)	0.0078(14)	0.00042(8)
⁵⁵ Mn	4724.84(8)	0.281(10)	0.0155(6)	⁵⁶ Fe	2066.08(6)	0.0146(7)	0.00079(4)
⁵⁵ Mn	4840.72(16)	0.064(6)	0.0035(3)	⁵⁶ Fe	2129.47(7)	0.0206(7)	0.00112(4)
⁵⁵ Mn	4874.52(13)	0.069(5)	0.0038(3)	⁵⁴ Fe	2469.24(13)	0.0116(7)	0.00063(4)
⁵⁵ Mn	4907.36(19)	0.070(7)	0.0039(4)	⁵⁶ Fe	2526.34(7)	0.0112(5)	0.00061(3)
⁵⁵ Mn	4934.09(18)	0.055(6)	0.0030(3)	⁵⁶ Fe	2682.69(11)	0.0114(9)	0.00062(5)
⁵⁵ Mn	4949.21(8)	0.274(10)	0.0151(6)	⁵⁶ Fe	2697.10(11)	0.0090(9)	0.00049(5)
⁵⁵ Mn	4969.28(21)	0.043(5)	0.0024(3)	⁵⁶ Fe	2721.21(4)	0.0384(13)	0.00208(7)
⁵⁵Mn	5014.37(7)	0.737(20)	0.0407(11)	⁵⁶ Fe	2755.93(19)	0.015(5)	0.0008(3)
⁵⁵ Mn	5034.60(15)	0.108(8)	0.0060(4)	⁵⁶ Fe	2832.84(10)	0.0142(22)	0.00077(12)
⁵⁵ Mn	5067.87(9)	0.265(12)	0.0146(7)	⁵⁶ Fe	2835.82(7)	0.0067(14)	0.00036(8)
⁵⁵ Mn	5110.97(22)	0.050(5)	0.0028(3)	⁵⁶ Fe	2873.00(7)	0.0099(14)	0.00054(8)
⁵⁵Mn	5180.89(8)	0.412(13)	0.0227(7)	⁵⁶ Fe	2954.12(10)	0.0110(7)	0.00060(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
⁵⁶ Fe	3103.26(7)	0.0172(7)	0.00093(4)	⁵⁹ Co	781.79(4)	0.146(6)	0.0075(3)
⁵⁶ Fe	3168.40(10)	0.0092(7)	0.00050(4)	⁵⁹Co	785.628(21)	2.41(7)	0.124(4)
⁵⁶ Fe	3185.86(9)	0.0183(8)	0.00099(4)	⁵⁹ Co	798.97(7)	0.120(10)	0.0062(5)
⁵⁶ Fe	3225.33(7)	0.0105(7)	0.00057(4)	⁵⁹ Co	854.06(4)	0.187(6)	0.0096(3)
⁵⁶ Fe	3239.74(7)	0.0094(13)	0.00051(7)	⁵⁹ Co	862.30(6)	0.079(8)	0.0041(4)
⁵⁶ Fe	3267.25(8)	0.0367(13)	0.00199(7)	⁵⁹ Co	883.11(4)	0.075(5)	0.0039(3)
⁵⁶ Fe	3291.06(5)	0.0072(6)	0.00039(3)	⁵⁹ Co	884.98(4)	0.156(6)	0.0080(3)
⁵⁶ Fe	3356.67(12)	0.0098(6)	0.00053(3)	⁵⁹ Co	901.28(3)	0.418(9)	0.0215(5)
⁵⁶ Fe	3413.13(5)	0.0449(14)	0.00244(8)	⁵⁹ Co	908.37(3)	0.100(4)	0.00514(21)
⁵⁶ Fe	3436.66(9)	0.045(4)	0.00244(22)	⁵⁹ Co	928.48(3)	0.145(9)	0.0075(5)
⁵⁷ Fe	3486.74(11)	0.0114(6)	0.00062(3)	⁵⁹ Co	930.612(23)	0.408(22)	0.0210(11)
⁵⁶ Fe	3776.90(6)	0.0075(7)	0.00041(4)	⁵⁹ Co	944.07(6)	0.18(7)	0.009(4)
⁵⁴ Fe	3790.80(25)	0.0075(7)	0.00041(4)	⁵⁹Co	945.314(17)	0.98(4)	0.0504(21)
⁵⁶ Fe	3842.43(9)	0.0086(7)	0.00047(4)	⁵⁹ Co	947.41(6)	0.121(7)	0.0062(4)
⁵⁶ Fe	3854.51(6)	0.0333(12)	0.00181(7)	⁵⁹ Co	963.58(3)	0.191(11)	0.0098(6)
⁵⁶ Fe	3921.5(8)	0.036(4)	0.00195(22)	⁵⁹ Co	972.82(16)	0.082(8)	0.0042(4)
⁵⁶Fe	4218.27(5)	0.099(3)	0.00537(16)	⁵⁹ Co	1005.668(22)	0.127(6)	0.0065(3)
⁵⁶ Fe	4274.74(12)	0.0141(8)	0.00077(4)	⁵⁹ Co	1023.64(3)	0.22(3)	0.0113(15)
⁵⁶ Fe	4378.56(8)	0.0067(6)	0.00036(3)	⁵⁹ Co	1075.66(10)	0.099(7)	0.0051(4)
⁵⁶ Fe	4406.07(7)	0.0453(13)	0.00246(7)	⁵⁹ Co	1103.73(6)	0.277(12)	0.0142(6)
⁵⁶ Fe	4463.01(10)	0.0162(11)	0.00088(6)	⁵⁹ Co	1117.76(8)	0.106(5)	0.0055(3)
⁵⁶ Fe	4674.99(11)	0.0125(11)	0.00068(6)	⁵⁹ Co	1206.47(3)	0.072(11)	0.0037(6)
⁵⁶ Fe	4724.54(10)	0.0075(11)	0.00041(6)	⁵⁹ Co	1207.77(3)	0.202(12)	0.0104(6)
⁵⁶ Fe	4809.99(7)	0.0416(13)	0.00226(7)	⁵⁹ Co	1215.96(3)	0.520(9)	0.0267(5)
⁵⁶ Fe	4948.70(11)	0.0173(10)	0.00094(5)	⁵⁹ Co	1216.44(18)	0.24(22)	0.012(11)
⁵⁴ Fe	5507.29(19)	0.0247(15)	0.00134(8)	⁵⁹ Co	1226.78(5)	0.100(4)	0.00514(21)
⁵⁶Fe	5920.449(21)	0.225(5)	0.0122(3)	⁵⁹ Co	1238.566(24)	0.290(7)	0.0149(4)
⁵⁶Fe	6018.532(20)	0.227(5)	0.0123(3)	⁵⁹ Co	1274.32(4)	0.205(6)	0.0105(3)
⁵⁶ Fe	6380.67(3)	0.0187(20)	0.00101(11)	⁵⁹ Co	1277.46(3)	0.175(6)	0.0090(3)
⁵⁶Fe	7278.838(10)	0.137(4)	0.00743(22)	⁵⁹ Co	1283.22(7)	0.194(6)	0.0100(3)
⁵⁶Fe	7631.136(14)	0.653(13)	0.0354(7)	⁵⁹ Co	1334.74(6)	0.155(9)	0.0080(5)
⁵⁶Fe	7645.5450(10)	0.549(11)	0.0298(6)	⁵⁹ Co	1362.53(4)	0.092(6)	0.0047(3)
⁵⁴ Fe	8886.18(23)	0.0162(12)	0.00088(7)	⁵⁹ Co	1419.30(8)	0.077(6)	0.0040(3)
⁵⁴Fe	9297.68(19)	0.0747(25)	0.00405(14)	⁵⁹ Co	1472.04(3)	0.195(8)	0.0100(4)
Cobalt (Z=27), At.Wt.=58.933200(9), σ _γ ^Z =37.18(6)							
⁵⁹ Co	58.603(7)d	0.411(4)	0.02113[75%]	⁵⁹Co	1515.720(25)	1.740(25)	0.0895(13)
⁵⁹Co	158.517(17)	1.200(15)	0.0617(8)	⁵⁹ Co	1553.65(3)	0.120(6)	0.0062(3)
⁵⁹ Co	195.90(3)	0.190(4)	0.00977(21)	⁵⁹ Co	1556.08(9)	0.099(6)	0.0051(3)
⁵⁹ Co	224.12(7)	0.106(23)	0.0055(12)	⁵⁹ Co	1690.72(3)	0.215(14)	0.0111(7)
⁵⁹Co	229.879(17)	7.18(8)	0.369(4)	⁵⁹ Co	1692.83(5)	0.214(14)	0.0110(7)
⁵⁹Co	254.379(17)	1.290(16)	0.0663(8)	⁵⁹ Co	1703.91(10)	0.074(5)	0.0038(3)
⁵⁹Co	277.161(17)	6.77(8)	0.348(4)	⁵⁹ Co	1774.65(4)	0.30(8)	0.015(4)
⁵⁹ Co	337.296(18)	0.226(4)	0.01162(21)	⁵⁹ Co	1786.01(17)	0.157(9)	0.0081(5)
⁵⁹ Co	349.954(24)	0.124(4)	0.00638(21)	⁵⁹ Co	1787.45(4)	0.08(5)	0.004(3)
⁵⁹Co	391.218(15)	1.080(14)	0.0555(7)	⁵⁹ Co	1799.92(4)	0.269(7)	0.0138(4)
⁵⁹Co	435.677(17)	0.789(10)	0.0406(5)	⁵⁹ Co	1808.82(7)	0.211(7)	0.0109(4)
⁵⁹Co	447.711(19)	3.41(4)	0.1754(21)	⁵⁹ Co	1808.98(10)	0.15(8)	0.008(4)
⁵⁹ Co	461.061(18)	0.519(9)	0.0267(5)	⁵⁹ Co	1818.58(5)	0.179(7)	0.0092(4)
⁵⁹Co	484.257(16)	0.804(11)	0.0413(6)	⁵⁹Co	1830.800(25)	1.700(23)	0.0874(12)
⁵⁹Co	497.269(16)	2.16(4)	0.1111(21)	⁵⁹ Co	1844.96(8)	0.092(5)	0.0047(3)
⁵⁹Co	555.972(13)	5.76(6)	0.296(3)	⁵⁹ Co	1852.70(3)	0.456(10)	0.0234(5)
⁵⁹ Co	602.71(4)	0.132(7)	0.0068(4)	⁵⁹ Co	1888.77(4)	0.089(6)	0.0046(3)
⁵⁹ Co	665.48(3)	0.0769(24)	0.00395(12)	⁵⁹ Co	1933.82(8)	0.094(6)	0.0048(3)
⁵⁹ Co	680.15(3)	0.273(5)	0.0140(3)	⁵⁹ Co	2022.51(16)	0.082(6)	0.0042(3)
⁵⁹Co	717.310(18)	0.845(14)	0.0435(7)	⁵⁹ Co	2032.83(7)	0.393(11)	0.0202(6)
⁵⁹ Co	726.640(21)	0.448(10)	0.0230(5)	⁵⁹ Co	2074.83(8)	0.102(9)	0.0052(5)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	
⁵⁹ Co	2099.19(7)	0.089(8)	0.0046(4)	⁵⁹ Co	4921.85(9)	0.285(13)	0.0147(7)	
⁵⁹ Co	2221.61(4)	0.261(8)	0.0134(4)	⁵⁹ Co	5003.24(8)	0.264(11)	0.0136(6)	
⁵⁹ Co	2279.78(6)	0.079(11)	0.0041(6)	⁵⁹ Co	5040.76(16)	0.086(8)	0.0044(4)	
⁵⁹ Co	2281.57(9)	0.123(11)	0.0063(6)	⁵⁹ Co	5068.69(9)	0.109(10)	0.0056(5)	
⁵⁹ Co	2309.66(10)	0.087(6)	0.0045(3)	⁵⁹ Co	5127.84(9)	0.205(12)	0.0105(6)	
⁵⁹ Co	2319.46(10)	0.122(7)	0.0063(4)	⁵⁹ Co	5150.08(9)	0.302(13)	0.0155(7)	
⁵⁹ Co	2453.82(20)	0.072(5)	0.0037(3)	⁵⁹Co	5181.77(7)	0.912(23)	0.0469(12)	
⁵⁹ Co	2527.12(7)	0.146(8)	0.0075(4)	⁵⁹ Co	5211.98(6)	0.072(11)	0.0037(6)	
⁵⁹ Co	2557.46(21)	0.086(6)	0.0044(3)	⁵⁹ Co	5217.09(20)	0.081(10)	0.0042(5)	
⁵⁹ Co	2569.92(9)	0.154(7)	0.0079(4)	⁵⁹ Co	5270.15(4)	0.404(11)	0.0208(6)	
⁵⁹ Co	2607.47(10)	0.165(8)	0.0085(4)	⁵⁹ Co	5358.44(8)	0.160(8)	0.0082(4)	
⁵⁹ Co	2680.64(24)	0.11(3)	0.0057(15)	⁵⁹ Co	5370.21(8)	0.188(9)	0.0097(5)	
⁵⁹ Co	2692.02(15)	0.076(7)	0.0039(4)	⁵⁹ Co	5510.56(6)	0.163(11)	0.0084(6)	
⁵⁹ Co	2727.19(13)	0.100(7)	0.0051(4)	⁵⁹ Co	5602.97(4)	0.434(16)	0.0223(8)	
⁵⁹ Co	2740.06(18)	0.103(7)	0.0053(4)	⁵⁹ Co	5614.67(5)	0.399(15)	0.0205(8)	
⁵⁹ Co	2790.22(20)	0.080(19)	0.0041(10)	⁵⁹ Co	5639.03(4)	0.379(15)	0.0195(8)	
⁵⁹ Co	2900.50(24)	0.076(20)	0.0039(10)	⁵⁹Co	5660.93(4)	1.89(6)	0.097(3)	
⁵⁹ Co	2926.19(18)	0.116(8)	0.0060(4)	⁵⁹ Co	5704.28(5)	0.177(9)	0.0091(5)	
⁵⁹ Co	2978.11(17)	0.075(7)	0.0039(4)	⁵⁹Co	5742.53(4)	0.766(23)	0.0394(12)	
⁵⁹ Co	2995.43(13)	0.097(7)	0.0050(4)	⁵⁹ Co	5852.04(5)	0.110(10)	0.0057(5)	
⁵⁹ Co	3193.65(16)	0.089(6)	0.0046(3)	⁵⁹ Co	5925.89(4)	0.643(18)	0.0331(9)	
⁵⁹ Co	3216.43(19)	0.105(13)	0.0054(7)	⁵⁹Co	5975.98(4)	2.9(4)	0.149(21)	
⁵⁹ Co	3238.16(19)	0.089(8)	0.0046(4)	⁵⁹ Co	6040.60(4)	0.166(13)	0.0085(7)	
⁵⁹ Co	3283.78(13)	0.101(8)	0.0052(4)	⁵⁹ Co	6110.81(6)	0.213(11)	0.0110(6)	
⁵⁹ Co	3335.29(14)	0.104(7)	0.0053(4)	⁵⁹ Co	6149.99(7)	0.186(9)	0.0096(5)	
⁵⁹ Co	3380.22(14)	0.210(10)	0.0108(5)	⁵⁹ Co	6274.84(3)	0.222(11)	0.0114(6)	
⁵⁹ Co	3664.13(21)	0.080(9)	0.0041(5)	⁵⁹ Co	6283.91(4)	0.204(11)	0.0105(6)	
⁵⁹ Co	3677.05(13)	0.109(8)	0.0056(4)	⁵⁹Co	6485.99(3)	2.32(5)	0.119(3)	
⁵⁹ Co	3749.21(7)	0.415(13)	0.0213(7)	⁵⁹Co	6706.01(3)	3.02(6)	0.155(3)	
⁵⁹ Co	3815.20(19)	0.081(7)	0.0042(4)	⁵⁹Co	6877.16(3)	3.02(6)	0.155(3)	
⁵⁹ Co	3823.54(19)	0.073(7)	0.0038(4)	⁵⁹ Co	6948.87(3)	0.249(11)	0.0128(6)	
⁵⁹ Co	3840.83(15)	0.129(8)	0.0066(4)	⁵⁹Co	6985.41(3)	1.05(13)	0.054(7)	
⁵⁹ Co	3897.02(17)	0.092(7)	0.0047(4)	⁵⁹ Co	7055.92(3)	0.666(19)	0.0342(10)	
⁵⁹ Co	3929.84(12)	0.272(11)	0.0140(6)	⁵⁹ Co	7203.22(3)	0.369(16)	0.0190(8)	
⁵⁹ Co	3966.15(18)	0.239(11)	0.0123(6)	⁵⁹Co	7214.42(3)	1.38(3)	0.0710(15)	
⁵⁹ Co	3994.92(24)	0.095(17)	0.0049(9)	⁵⁹ Co	7433.07(3)	0.083(7)	0.0043(4)	
⁵⁹ Co	4026.26(12)	0.272(10)	0.0140(5)	⁵⁹Co	7491.54(3)	1.16(3)	0.0596(15)	
⁵⁹ Co	4032.03(18)	0.208(9)	0.0107(5)	Nickel (Z=28), At. Wt.=58.6934(2), $\sigma_{\gamma}^Z = 4.39(15)$				
⁵⁹ Co	4148.74(21)	0.086(21)	0.0044(11)	⁶² Ni	155.500(16)	0.0666(12)	0.00344(6)	
⁵⁹ Co	4155.64(24)	0.128(8)	0.0066(4)	⁶⁰Ni	282.917(18)	0.211(3)	0.01089(15)	
⁵⁹ Co	4208.01(12)	0.255(13)	0.0131(7)	⁵⁸Ni	339.420(11)	0.1670(21)	0.00862(11)	
⁵⁹ Co	4212.56(14)	0.082(9)	0.0042(5)	⁶² Ni	362.385(18)	0.0342(5)	0.00177(3)	
⁵⁹ Co	4329.00(18)	0.105(8)	0.0054(4)	⁵⁸Ni	464.978(12)	0.843(10)	0.0435(5)	
⁵⁹ Co	4350.40(12)	0.091(13)	0.0047(7)	⁶² Ni	483.351(20)	0.0156(3)	0.000805(15)	
⁵⁹ Co	4370.46(19)	0.078(12)	0.0040(6)	⁶² Ni	845.733(18)	0.0184(3)	0.000950(15)	
⁵⁹ Co	4377.29(19)	0.119(10)	0.0061(5)	⁵⁸Ni	877.977(11)	0.236(3)	0.01219(15)	
⁵⁹ Co	4395.62(11)	0.128(11)	0.0066(6)	⁶¹ Ni	1172.84(5)	0.0122(4)	0.000630(21)	
⁵⁹ Co	4547.05(11)	0.115(9)	0.0059(5)	⁵⁸ Ni	1188.781(13)	0.0559(9)	0.00289(5)	
⁵⁹ Co	4607.00(7)	0.311(13)	0.0160(7)	⁵⁸ Ni	1301.434(13)	0.052(3)	0.00268(15)	
⁵⁹ Co	4624.29(16)	0.104(8)	0.0053(4)	⁵⁸ Ni	1340.230(20)	0.0200(5)	0.00103(3)	
⁵⁹ Co	4646.83(15)	0.081(10)	0.0042(5)	⁶⁴ Ni	1481.84(5)d	0.003300(7)	1.704E-4[13%]	
⁵⁹ Co	4666.15(10)	0.085(8)	0.0044(4)	⁶⁰ Ni	1502.04(6)	0.0154(4)	0.000795(21)	
⁵⁹ Co	4706.11(13)	0.137(9)	0.0070(5)	⁵⁸ Ni	1536.920(16)	0.0194(5)	0.00100(3)	
⁵⁹ Co	4731.06(17)	0.089(8)	0.0046(4)	⁵⁸ Ni	1734.687(16)	0.0172(4)	0.000888(21)	
⁵⁹ Co	4884.30(10)	0.237(10)	0.0122(5)	⁵⁸ Ni	1949.911(17)	0.0476(10)	0.00246(5)	
⁵⁹ Co	4893.76(10)	0.217(11)	0.0112(6)	⁶⁰ Ni	2123.93(3)	0.0379(10)	0.00196(5)	
⁵⁹ Co	4906.17(7)	0.43(3)	0.0221(15)	-----				

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁵⁸ Ni	2554.116(19)	0.0431(9)	0.00223(5)	⁶³ Cu	648.80(3)	0.102(3)	0.00486(14)
⁵⁸ Ni	2842.130(17)	0.0463(10)	0.00239(5)	⁶³ Cu	662.69(4)	0.072(3)	0.00343(14)
⁵⁸ Ni	3221.146(23)	0.0157(11)	0.00081(6)	⁶³ Cu	739.03(3)	0.0096(3)	0.000458(14)
⁵⁸ Ni	3675.24(3)	0.0281(7)	0.00145(4)	⁶³ Cu	767.77(3)	0.0254(17)	0.00121(8)
⁵⁸ Ni	4858.59(3)	0.0442(10)	0.00228(5)	⁶⁵ Cu	822.673(24)	0.0238(17)	0.00114(8)
⁵⁸ Ni	5312.674(24)	0.0536(13)	0.00277(7)	⁶⁵ Cu	831.14(4)	0.0160(10)	0.00076(5)
⁵⁸ Ni	5435.77(4)	0.0188(6)	0.00097(3)	⁶³ Cu	878.17(5)	0.0421(20)	0.00201(10)
⁶⁰ Ni	5695.80(3)	0.0416(12)	0.00215(6)	⁶³ Cu	897.07(17)	0.0102(4)	0.000486(19)
⁵⁸ Ni	5817.219(20)	0.1090(22)	0.00563(11)	⁶³ Cu	927.05(3)	0.0119(3)	0.000568(14)
⁶² Ni	5836.37(3)	0.0348(10)	0.00180(5)	⁶³ Cu	946.65(7)	0.0091(8)	0.00043(4)
⁵⁸ Ni	5973.06(3)	0.0258(8)	0.00133(4)	⁶³ Cu	962.76(4)	0.0152(9)	0.00072(4)
⁶⁴ Ni	6034.60(11)	0.013(3)	0.00067(15)	⁶⁵ Cu	972.11(3)	0.0115(7)	0.00055(3)
⁵⁸ Ni	6105.215(22)	0.0706(17)	0.00365(9)	⁶⁵ Cu	997.63(3)	0.0093(11)	0.00044(5)
⁶² Ni	6319.67(3)	0.0236(9)	0.00122(5)	⁶³ Cu	1019.59(4)	0.0141(12)	0.00067(6)
⁵⁸ Ni	6583.831(19)	0.0830(20)	0.00429(10)	⁶⁵ Cu	1038.97(3)d	0.0598(13)	0.00285[88%]
⁶² Ni	6837.50(3)	0.458(8)	0.0236(4)	⁶⁵ Cu	1052.01(5)	0.0117(8)	0.00056(4)
⁶⁰ Ni	7536.637(25)	0.190(4)	0.00981(21)	⁶³ Cu	1076.44(4)	0.0097(5)	0.000463(24)
⁵⁸ Ni	7697.163(18)	0.0374(14)	0.00193(7)	⁶³ Cu	1081.72(3)	0.0117(3)	0.000558(14)
⁶⁰ Ni	7819.517(21)	0.336(6)	0.0173(3)	⁶³ Cu	1138.82(3)	0.0296(10)	0.00141(5)
⁵⁸ Ni	8120.567(16)	0.133(3)	0.00687(15)	⁶³ Cu	1158.833(15)	0.0267(6)	0.00127(3)
⁵⁸ Ni	8533.509(17)	0.721(13)	0.0372(7)	⁶³ Cu	1194.92(4)	0.0106(3)	0.000506(14)
⁵⁸ Ni	8998.414(15)	1.49(3)	0.0769(15)	⁶⁵ Cu	1212.53(4)	0.0105(5)	0.000501(24)
Copper (Z=29), At.Wt.=63.546(3), $\sigma_{\gamma}^Z = 3.795(17)$							
⁶⁵ Cu	89.08(4)	0.0970(17)	0.00463(8)	⁶³ Cu	1231.98(4)	0.0110(3)	0.000525(14)
⁶³ Cu	159.281(5)	0.648(10)	0.0309(5)	⁶³ Cu	1241.52(9)	0.0345(16)	0.00165(8)
⁶³ Cu	184.618(13)	0.0106(9)	0.00051(4)	⁶³ Cu	1242.61(9)	0.0181(22)	0.00086(10)
⁶⁵ Cu	185.96(4)	0.244(3)	0.01164(14)	⁶³ Cu	1298.10(3)	0.0147(7)	0.00070(3)
⁶³ Cu	202.950(8)	0.193(3)	0.00920(14)	⁶³ Cu	1320.25(8)	0.0263(10)	0.00125(5)
⁶³ Cu	212.389(15)	0.0362(9)	0.00173(4)	⁶⁵ Cu	1355.16(3)	0.0133(16)	0.00063(8)
⁶³ Cu	214.99(7)	0.0112(14)	0.00053(7)	⁶³ Cu	1361.75(4)	0.0167(5)	0.000796(24)
⁶⁵ Cu	237.80(4)	0.0230(4)	0.001097(19)	⁶³ Cu	1417.27(6)	0.0097(4)	0.000463(19)
⁶³ Cu	247.58(6)	0.0119(15)	0.00057(7)	⁶³ Cu	1438.66(4)	0.013(6)	0.0006(3)
⁶³ Cu	261.33(8)	0.0095(14)	0.00045(7)	⁶⁵ Cu	1439.37(5)	0.0111(16)	0.00053(8)
⁶³ Cu	264.869(22)	0.0289(7)	0.00138(3)	⁶³ Cu	1521.03(4)	0.0143(5)	0.000682(24)
⁶³ Cu	278.250(14)	0.893(15)	0.0426(7)	⁶⁵ Cu	1559.84(7)	0.0305(10)	0.00145(5)
⁶⁵ Cu	315.69(4)	0.0250(4)	0.001192(19)	⁶³ Cu	1582.50(4)	0.0094(7)	0.00045(3)
⁶³ Cu	318.80(4)	0.0120(4)	0.000572(19)	⁶⁵ Cu	1637.46(5)	0.0135(15)	0.00064(7)
⁶³ Cu	330.52(3)	0.0107(8)	0.00051(4)	⁶³ Cu	1682.98(7)	0.0167(8)	0.00080(4)
⁶³ Cu	343.898(14)	0.215(4)	0.01025(19)	⁶⁵ Cu	1743.30(7)	0.014(4)	0.00067(19)
⁶³ Cu	376.80(3)	0.0250(6)	0.00119(3)	⁶³ Cu	1852.57(8)	0.0141(10)	0.00067(5)
⁶³ Cu	384.45(5)	0.0700(14)	0.00334(7)	⁶³ Cu	2141.61(12)	0.0091(5)	0.000434(24)
⁶⁵ Cu	385.77(3)	0.1310(18)	0.00625(9)	⁶³ Cu	2153.51(5)	0.0105(11)	0.00050(5)
⁶⁵ Cu	436.909(20)	0.0112(4)	0.000534(19)	⁶³ Cu	2291.40(10)	0.0115(8)	0.00055(4)
⁶³ Cu	449.486(22)	0.0382(10)	0.00182(5)	⁶³ Cu	2497.85(7)	0.0252(13)	0.00120(6)
⁶³ Cu	460.78(3)	0.0143(5)	0.000682(24)	⁶³ Cu	2932.30(13)	0.0101(7)	0.00048(3)
⁶⁵ Cu	465.14(3)	0.1350(21)	0.00644(10)	⁶³ Cu	3152.95(16)	0.0099(9)	0.00047(4)
⁶³ Cu	467.95(5)	0.0668(14)	0.00319(7)	⁶³ Cu	3315.5(3)	0.0097(7)	0.00046(3)
⁶³ Cu	494.81(5)	0.0242(6)	0.00115(3)	⁶³ Cu	3464.49(14)	0.0094(15)	0.00045(7)
⁶³ Cu	503.41(4)	0.0596(13)	0.00284(6)	⁶³ Cu	3588.50(9)	0.0122(14)	0.00058(7)
⁶³ Cu	533.25(11)	0.0148(8)	0.00071(4)	⁶³ Cu	3844.49(15)	0.0176(11)	0.00084(5)
⁶³ Cu	534.28(5)	0.021(6)	0.0010(3)	⁶³ Cu	4089.19(14)	0.0090(5)	0.000429(24)
⁶⁵ Cu	543.86(3)	0.0256(5)	0.001221(24)	⁶³ Cu	4133.04(12)	0.0138(10)	0.00066(5)
⁶³ Cu	579.75(3)	0.0898(15)	0.00428(7)	⁶³ Cu	4204.26(19)	0.0091(5)	0.000434(24)
⁶³ Cu	608.766(23)	0.270(6)	0.0129(3)	⁶³ Cu	4286.55(15)	0.0121(6)	0.00058(3)
⁶³ Cu	617.47(6)	0.0270(4)	0.001288(19)	⁶³ Cu	4312.76(24)	0.0104(8)	0.00050(4)
⁶³ Cu	632.24(4)	0.0092(4)	0.000439(19)	⁶³ Cu	4319.92(9)	0.047(5)	0.00224(24)
				⁶⁵ Cu	4384.92(9)	0.0206(12)	0.00098(6)
				⁶³ Cu	4404.91(18)	0.0111(5)	0.000529(24)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁶³ Cu	4443.9(3)	0.0110(11)	0.00052(5)	⁶⁴ Zn	207.067(22)	0.0101(3)	0.000468(14)
⁶³ Cu	4475.88(13)	0.0171(6)	0.00082(3)	⁶⁶ Zn	300.219(7)	0.0201(6)	0.00093(3)
⁶³ Cu	4503.94(12)	0.0174(7)	0.00083(3)	⁶⁶ Zn	393.530(7)	0.00486(22)	2.25(10)E-4
⁶³ Cu	4563.20(7)	0.0112(5)	0.000534(24)	⁶⁸ Zn	417.30(4)	0.0043(5)	1.99(23)E-4
⁶³ Cu	4603.01(20)	0.0196(6)	0.00093(3)	⁶⁸ Zn	434.03(3)	0.0128(16)	0.00059(7)
⁶³ Cu	4658.55(9)	0.0278(7)	0.00133(3)	⁶⁸ Zn	438.634(18)d	0.0128(5)	0.000593[2.5%]
⁶³ Cu	5019.16(12)	0.0100(15)	0.00048(7)	⁶⁸ Zn	531.44(3)	0.0163(20)	0.00076(9)
⁶⁵ Cu	5042.68(6)	0.0346(14)	0.00165(7)	⁶⁷ Zn	578.48(5)	0.0121(5)	0.000561(23)
⁶⁵ Cu	5047.56(7)	0.0206(14)	0.00098(7)	⁶⁴ Zn	653.51(7)	0.0050(14)	2.3(7)E-4
⁶³ Cu	5085.54(11)	0.0118(5)	0.000563(24)	⁶⁶ Zn	749.29(7)	0.0058(13)	0.00027(6)
⁶³ Cu	5151.98(15)	0.0096(4)	0.000458(19)	⁶⁴ Zn	751.69(3)	0.0307(10)	0.00142(5)
⁶³ Cu	5183.55(17)	0.0132(6)	0.00063(3)	⁶⁸ Zn	759.29(9)	0.0039(5)	1.81(23)E-4
⁶³ Cu	5189.81(11)	0.0241(7)	0.00115(3)	⁶⁴ Zn	768.74(7)	0.0040(4)	1.85(19)E-4
⁶⁵ Cu	5245.59(4)	0.043(3)	0.00205(14)	⁶⁴ Zn	794.44(3)	0.0089(5)	0.000412(23)
⁶³ Cu	5258.73(7)	0.0372(9)	0.00177(4)	⁶⁷ Zn	805.79(3)	0.045(3)	0.00209(14)
⁶⁵ Cu	5320.08(8)	0.0362(21)	0.00173(10)	⁶⁸ Zn	834.77(3)	0.037(5)	0.00171(23)
⁶³ Cu	5408.64(17)	0.0144(6)	0.00069(3)	⁶⁴ Zn	855.69(3)	0.066(6)	0.0031(3)
⁶³ Cu	5418.45(5)	0.0668(12)	0.00319(6)	⁶⁴ Zn	864.43(6)	0.0094(6)	0.00044(3)
⁶³ Cu	5555.38(19)	0.0098(5)	0.000467(24)	⁶⁴ Zn	909.66(3)	0.0187(8)	0.00087(4)
⁶³ Cu	5614.96(12)	0.0178(6)	0.00085(3)	⁶⁴ Zn	932.10(6)	0.0047(4)	2.18(19)E-4
⁶³ Cu	5636.11(7)	0.0147(5)	0.000701(24)	⁶⁶ Zn	958.24(7)	0.0058(5)	0.000269(23)
⁶³ Cu	5771.47(9)	0.0183(8)	0.00087(4)	⁶⁴ Zn	993.35(6)	0.0059(6)	0.00027(3)
⁶³ Cu	5823.60(20)	0.0108(22)	0.00052(10)	⁶⁸ Zn	1007.809(25)	0.056(7)	0.0026(3)
⁶³ Cu	6010.80(5)	0.0574(12)	0.00274(6)	⁶⁴ Zn	1047.32(7)	0.0036(5)	1.67(23)E-4
⁶⁵ Cu	6048.73(5)	0.0101(6)	0.00048(3)	⁶⁷ Zn	1077.335(16)	0.356(5)	0.01650(23)
⁶³ Cu	6063.24(9)	0.0218(6)	0.00104(3)	⁶⁷ Zn	1126.100(25)	0.0229(6)	0.00106(3)
⁶³ Cu	6166.7(3)	0.0133(21)	0.00063(10)	⁶⁸ Zn	1178.55(9)	0.0102(13)	0.00047(6)
⁶⁵ Cu	6243.14(4)	0.0144(9)	0.00069(4)	⁶⁸ Zn	1252.07(5)	0.0073(9)	0.00034(4)
⁶³ Cu	6321.58(6)	0.0130(5)	0.000620(24)	⁶⁷ Zn	1261.15(3)	0.0431(10)	0.00200(5)
⁶³ Cu	6394.76(5)	0.0503(10)	0.00240(5)	⁶⁴ Zn	1262.58(6)	0.0053(15)	2.5(7)E-4
⁶³ Cu	6595.52(8)	0.0227(8)	0.00108(4)	⁶⁴ Zn	1293.02(8)	0.0061(6)	0.00028(3)
⁶⁵ Cu	6600.63(4)	0.085(5)	0.00405(24)	⁶⁷ Zn	1300.96(6)	0.010(4)	0.00046(19)
⁶³ Cu	6617.66(5)	0.0407(11)	0.00194(5)	⁶⁷ Zn	1340.14(3)	0.0457(16)	0.00212(7)
⁶³ Cu	6673.15(9)	0.053(3)	0.00253(14)	⁶⁴ Zn	1354.42(5)	0.0103(9)	0.00048(4)
⁶³ Cu	6674.76(5)	0.0719(21)	0.00343(10)	⁶⁴ Zn	1415.67(5)	0.0043(7)	2.0(3)E-4
⁶⁵ Cu	6680.00(4)	0.081(6)	0.0039(3)	⁶⁷ Zn	1546.33(8)	0.0082(7)	0.00038(3)
⁶⁵ Cu	6790.72(4)	0.0155(10)	0.00074(5)	⁶⁴ Zn	1593.0(3)	0.0053(13)	2.5(6)E-4
⁶³ Cu	6988.68(5)	0.126(6)	0.0060(3)	⁶⁸ Zn	1594.05(9)	0.0051(6)	2.4(3)E-4
⁶³ Cu	7037.55(5)	0.0140(7)	0.00067(3)	⁶⁷ Zn	1673.46(4)	0.0260(10)	0.00120(5)
⁶⁵ Cu	7065.72(4)	0.0132(8)	0.00063(4)	⁶⁷ Zn	1744.47(5)	0.0147(7)	0.00068(3)
⁶³ Cu	7169.51(5)	0.0109(7)	0.00052(3)	⁶⁸ Zn	1813.18(8)	0.0051(6)	2.4(3)E-4
⁶³ Cu	7176.68(5)	0.0925(17)	0.00441(8)	⁶⁴ Zn	1826.45(6)	0.0161(10)	0.00075(5)
⁶³ Cu	7253.01(5)	0.1500(23)	0.00715(11)	⁶⁷ Zn	1882.09(10)	0.0056(15)	0.00026(7)
⁶³ Cu	7306.93(4)	0.321(17)	0.0153(8)	⁶⁷ Zn	1883.12(3)	0.0718(18)	0.00333(8)
⁶³ Cu	7571.77(4)	0.0629(12)	0.00300(6)	⁶⁴ Zn	2087.44(9)	0.0047(6)	2.2(3)E-4
⁶³ Cu	7637.40(4)	0.54(7)	0.026(3)	⁶⁷ Zn	2106.74(6)	0.0071(7)	0.00033(3)
⁶³ Cu	7756.36(4)	0.0571(12)	0.00272(6)	⁶⁷ Zn	2209.73(9)	0.0269(13)	0.00125(6)
⁶³ Cu	7915.62(4)	0.869(20)	0.0414(10)	⁶⁴ Zn	2212.10(16)	0.0071(17)	0.00033(8)
Zinc (Z=30), At.Wt.=65.39(2), $\sigma_{\gamma}^Z = 1.30(8)$				⁶⁸ Zn	2344.60(8)	0.0100(12)	0.00046(6)
⁶⁴ Zn	53.972(17)	0.0109(6)	0.00051(3)	⁶⁷ Zn	2347.58(14)	0.0048(7)	2.2(3)E-4
⁶⁴ Zn	61.2530(20)	0.0290(9)	0.00134(4)	⁶⁷ Zn	2352.10(8)	0.0059(9)	0.00027(4)
⁶⁶ Zn	91.267(5)	0.0046(3)	2.13(14)E-4	⁶⁸ Zn	2378.6(3)	0.0039(5)	1.81(23)E-4
⁶⁶ Zn	93.311(5)	0.0344(8)	0.00159(4)	⁶⁷ Zn	2418.53(10)	0.0095(7)	0.00044(3)
⁶⁴ Zn	115.225(18)	0.167(3)	0.00774(14)	⁶⁴ Zn	2432.3(5)	0.0037(8)	1.7(4)E-4
⁶⁴ Zn	153.095(21)	0.0322(6)	0.00149(3)	⁶⁷ Zn	2648.75(21)	0.0056(10)	0.00026(5)
⁶⁶ Zn	184.578(6)	0.0321(4)	0.001488(19)	⁶⁷ Zn	2698.91(17)	0.0061(9)	0.00028(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁶⁷ Zn	2857.91(10)	0.0070(8)	0.00032(4)	⁷¹ Ga	46.97(4)	0.013(3)	0.00057(13)
⁶⁴ Zn	3109.05(25)	0.0073(10)	0.00034(5)	⁷¹ Ga	79.75(4)	0.0224(10)	0.00097(4)
⁶⁷ Zn	3287.02(9)	0.0088(9)	0.00041(4)	⁷¹ Ga	88.86(4)	0.0305(9)	0.00133(4)
⁶⁷ Zn	3331.21(20)	0.0049(5)	2.27(23)E-4	⁷¹Ga	103.25(3)d	0.0526(11)	0.00229[100%]
⁶⁷ Zn	3458.14(17)	0.0048(4)	2.22(19)E-4	⁷¹ Ga	110.06(4)	0.0118(8)	0.00051(4)
⁶⁷ Zn	3832.94(25)	0.0048(5)	2.22(23)E-4	⁷¹Ga	112.36(3)	0.155(3)	0.00674(13)
⁶⁸ Zn	4071.4(4)	0.0036(5)	1.67(23)E-4	⁷¹ Ga	121.01(3)	0.0142(6)	0.00062(3)
⁶⁸ Zn	4103.3(3)	0.0089(21)	0.00041(10)	⁷¹ Ga	128.76(4)	0.0063(9)	0.00027(4)
⁶⁸ Zn	4137.29(10)	0.0205(25)	0.00095(12)	⁷¹ Ga	132.07(11)	0.013(3)	0.00057(13)
⁶⁸ Zn	4430.69(14)	0.0055(13)	0.00025(6)	⁷¹Ga	145.14(3)	0.466(7)	0.0203(3)
⁶⁷ Zn	4504.5(4)	0.0042(13)	1.9(6)E-4	⁷¹ Ga	153.78(3)	0.0319(8)	0.00139(4)
⁶⁴ Zn	4582.9(4)	0.00507(10)	2.35(5)E-4	⁷¹ Ga	162.90(4)	0.021(5)	0.00091(22)
⁶⁸ Zn	4652.3(4)	0.0059(7)	0.00027(3)	⁷¹ Ga	181.54(4)	0.040(3)	0.00174(13)
⁶⁷ Zn	4782.8(3)	0.0045(4)	2.09(19)E-4	⁷¹Ga	184.09(3)	0.1040(21)	0.00452(9)
⁶⁷ Zn	4795.0(11)	0.0037(9)	1.7(4)E-4	⁶⁹Ga	187.84(3)	0.1080(21)	0.00469(9)
⁶⁴ Zn	4828.4(3)	0.00676(11)	0.000313(5)	⁷¹Ga	192.11(3)	0.194(3)	0.00843(13)
⁶⁴ Zn	4870.0(3)	0.00380(10)	1.76(5)E-4	⁷¹Ga	194.66(4)	0.1070(21)	0.00465(9)
⁶⁸ Zn	4887.82(13)	0.0080(10)	0.00037(5)	⁷¹Ga	197.94(5)	0.1330(24)	0.00578(10)
⁶⁷ Zn	4899.63(19)	0.0053(5)	2.46(23)E-4	⁷¹ Ga	210.37(11)	0.019(7)	0.0008(3)
⁶⁷ Zn	4914.15(20)	0.0044(4)	2.04(19)E-4	⁷¹ Ga	210.50(20)	0.0343(8)	0.00149(4)
⁶⁸ Zn	5229.78(11)	0.0044(5)	2.04(23)E-4	⁷¹Ga	212.58(4)	0.0583(12)	0.00253(5)
⁶⁷ Zn	5245.84(15)	0.0058(6)	0.00027(3)	⁷¹ Ga	228.97(4)	0.0379(10)	0.00165(4)
⁶⁷ Zn	5287.4(3)	0.0048(6)	2.2(3)E-4	⁷¹ Ga	231.06(4)	0.0111(6)	0.00048(3)
⁶⁷ Zn	5346.37(21)	0.0039(6)	1.8(3)E-4	⁷¹ Ga	246.91(20)	0.0118(19)	0.00051(8)
⁶⁷ Zn	5402.8(5)	0.0043(24)	2.0(11)E-4	⁷¹Ga	248.89(4)	0.136(8)	0.0059(4)
⁶⁸Zn	5474.02(10)	0.042(5)	0.00195(23)	⁷¹ Ga	264.03(4)	0.0238(9)	0.00103(4)
⁶⁴ Zn	5521.5(3)	0.0076(11)	0.00035(5)	⁷¹ Ga	266.14(3)	0.0361(11)	0.00157(5)
⁶⁴ Zn	5541.0(5)	0.0047(7)	2.2(3)E-4	⁷¹ Ga	306.11(14)	0.015(4)	0.00065(17)
⁶⁴ Zn	5559.82(15)	0.01110(15)	0.000514(7)	⁷¹ Ga	306.62(12)	0.0097(8)	0.00042(4)
⁶⁸ Zn	5647.05(10)	0.0082(10)	0.00038(5)	⁷¹ Ga	313.62(11)	0.0209(8)	0.00091(4)
⁶⁷ Zn	5662.23(18)	0.0066(8)	0.00031(4)	⁷¹ Ga	315.40(6)	0.0275(9)	0.00120(4)
⁶⁷ Zn	5677.3(3)	0.0053(7)	2.5(3)E-4	⁶⁹Ga	318.87(3)	0.0592(14)	0.00257(6)
⁶⁷ Zn	5685.90(19)	0.0051(4)	2.36(19)E-4	⁶⁹Ga	344.79(7)	0.0070(6)	0.00030(3)
⁶⁴ Zn	5776.31(10)	0.01360(17)	0.000630(8)	⁶⁹Ga	363.93(13)	0.0048(6)	2.1(3)E-4
⁶⁷ Zn	5789.15(21)	0.0045(6)	2.1(3)E-4	⁶⁹ Ga	374.37(4)	0.0303(10)	0.00132(4)
⁶⁶ Zn	5909.4(3)	0.0110(11)	0.00051(5)	⁷¹ Ga	384.17(5)	0.0058(6)	0.00025(3)
⁶⁴ Zn	6037.28(8)	0.01490(20)	0.000691(9)	⁷¹Ga	390.66(4)	0.0476(12)	0.00207(5)
⁶⁷ Zn	6262.43(12)	0.0085(6)	0.00039(3)	⁶⁹ Ga	393.26(3)	0.021(3)	0.00091(13)
⁶⁸ Zn	6481.75(10)	0.0100(12)	0.00046(6)	⁷¹Ga	393.28(3)	0.1340(23)	0.00582(10)
⁶⁴ Zn	6509.27(8)	0.01190(16)	0.000552(7)	⁷¹ Ga	402.86(4)	0.0172(8)	0.00075(4)
⁶⁶ Zn	6658.6(3)	0.019(4)	0.00088(19)	⁷¹ Ga	408.44(20)	0.0179(9)	0.00078(4)
⁶⁷ Zn	6701.79(12)	0.0066(4)	0.000306(19)	⁷¹ Ga	411.07(14)	0.019(5)	0.00083(22)
⁶⁷ Zn	6768.21(10)	0.0112(9)	0.00052(4)	⁷¹ Ga	411.13(4)	0.0384(11)	0.00167(5)
⁶⁶ Zn	6867.5(3)	0.0254(17)	0.00118(8)	⁷¹ Ga	439.26(6)	0.0154(7)	0.00067(3)
⁶⁷ Zn	6910.58(11)	0.0194(14)	0.00090(7)	⁷¹ Ga	444.65(6)	0.021(5)	0.00091(22)
⁶⁶Zn	6958.8(3)	0.043(3)	0.00199(14)	⁷¹ Ga	458.54(12)	0.0092(7)	0.00040(3)
⁶⁴ Zn	7069.20(7)	0.0204(3)	0.000945(14)	⁷¹ Ga	488.81(4)	0.0227(8)	0.00099(4)
⁶⁴ Zn	7111.95(7)	0.0198(3)	0.000918(14)	⁷¹ Ga	488.81(4)	0.017(4)	0.00074(17)
⁶⁷ Zn	7188.40(8)	0.0131(7)	0.00061(3)	⁶⁹Ga	508.19(3)	0.349(6)	0.0152(3)
⁶⁷ Zn	7859.07(8)	0.0084(7)	0.00039(3)	⁶⁹ Ga	516.564(25)	0.012(4)	0.00052(17)
⁶⁴Zn	7863.55(7)	0.1410(19)	0.00653(9)	⁷¹ Ga	547.90(5)	0.0090(8)	0.00039(4)
⁶⁷ Zn	8314.37(8)	0.0105(5)	0.000487(23)	⁶⁹ Ga	561.97(5)	0.0078(3)	0.000339(13)
⁶⁷ Zn	9120.06(7)	0.0136(6)	0.00063(3)	⁷¹ Ga	564.29(5)	0.0097(3)	0.000422(13)
Gallium (Z=31), At. Wt.=69.723(1), $\sigma_{\gamma}^Z = 2.90(7)$				⁷¹ Ga	579.55(12)	0.0068(9)	0.00030(4)
⁷¹Ga	16.43(3)	0.078(5)	0.00339(22)	⁷¹Ga	601.21(6)d	0.471(22)	0.0205[2.4%]
⁷¹ Ga	41.89(4)	0.0050(4)	2.17(17)E-4	⁷¹ Ga	603.24(4)	0.0155(7)	0.00067(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷¹ Ga	619.63(5)	0.0053(12)	2.3(5)E-4	⁷¹ Ga	4761.5(4)	0.0078(9)	0.00034(4)
⁷¹ Ga	620.23(14)	0.0052(11)	2.3(5)E-4	⁷¹ Ga	4792.6(3)	0.0207(17)	0.00090(7)
⁷¹Ga	629.96(5)d	0.490(22)	0.0213[2.4%]	⁷¹ Ga	4839.89(23)	0.040(3)	0.00174(13)
⁶⁹ Ga	632.34(4)	0.0183(7)	0.00080(3)	⁷¹ Ga	4868.2(3)	0.0189(14)	0.00082(6)
⁶⁹Ga	651.09(3)	0.1030(22)	0.00448(10)	⁷¹ Ga	4890.5(3)	0.0191(14)	0.00083(6)
⁶⁹Ga	690.943(24)	0.305(4)	0.01326(17)	⁶⁹ Ga	4955.2(4)	0.0095(13)	0.00041(6)
⁷¹Ga	786.17(16)d	0.160(22)	0.0070[2.4%]	⁷¹ Ga	5054.0(4)	0.0094(11)	0.00041(5)
⁷¹Ga	834.08(3)d	1.65(5)	0.0717[2.4%]	⁷¹ Ga	5091.8(9)	0.0070(9)	0.00030(4)
⁶⁹ Ga	851.34(7)	0.0127(9)	0.00055(4)	⁶⁹ Ga	5133.6(6)	0.0051(11)	2.2(5)E-4
⁶⁹ Ga	868.3(3)	0.0071(15)	0.00031(7)	⁷¹ Ga	5160.69(21)	0.0154(13)	0.00067(6)
⁷¹ Ga	894.84(20)	0.0111(9)	0.00048(4)	⁶⁹ Ga	5189.2(9)	0.0074(20)	0.00032(9)
⁷¹Ga	894.91(11)d	0.35(3)	0.0152[2.4%]	⁷¹ Ga	5195.1(5)	0.034(3)	0.00148(13)
⁶⁹ Ga	904.91(7)	0.0149(10)	0.00065(4)	⁷¹ Ga	5223.3(7)	0.0157(13)	0.00068(6)
⁷¹ Ga	976.37(13)	0.0101(8)	0.00044(4)	⁷¹ Ga	5233.57(25)	0.0344(19)	0.00150(8)
⁶⁹ Ga	995.68(5)	0.0173(9)	0.00075(4)	⁷¹ Ga	5272.7(6)	0.0057(15)	2.5(7)E-4
⁷¹ Ga	1002.71(25)	0.0073(8)	0.00032(4)	⁷¹ Ga	5313.3(8)	0.0049(10)	2.1(4)E-4
⁶⁹ Ga	1010.34(6)	0.0146(8)	0.00063(4)	⁶⁹ Ga	5334.13(18)	0.0271(18)	0.00118(8)
⁶⁹ Ga	1014.99(8)	0.0077(7)	0.00033(3)	⁷¹ Ga	5334.9(5)	0.020(7)	0.0009(3)
⁶⁹ Ga	1044.90(15)	0.0107(11)	0.00047(5)	⁷¹ Ga	5340.45(25)	0.0406(21)	0.00176(9)
⁷¹Ga	1050.69(5)d	0.119(13)	0.0052[2.4%]	⁷¹ Ga	5390.2(5)	0.0049(10)	2.1(4)E-4
⁷¹ Ga	1051.25(17)	0.0114(10)	0.00050(4)	⁷¹ Ga	5487.2(13)	0.0090(25)	0.00039(11)
⁷¹ Ga	1075.6(5)	0.0053(8)	2.3(4)E-4	⁶⁹ Ga	5488.31(17)	0.0296(19)	0.00129(8)
⁶⁹ Ga	1140.37(4)	0.0422(16)	0.00183(7)	⁷¹ Ga	5497.6(5)	0.0091(13)	0.00040(6)
⁷¹ Ga	1200.3(3)	0.0078(9)	0.00034(4)	⁶⁹ Ga	5510.0(4)	0.0047(9)	2.0(4)E-4
⁶⁹ Ga	1203.40(6)	0.0286(14)	0.00124(6)	⁷¹ Ga	5543.83(19)	0.0142(17)	0.00062(7)
⁷¹ Ga	1217.5(9)	0.0075(21)	0.00033(9)	⁷¹ Ga	5577.0(6)	0.0058(18)	0.00025(8)
⁷¹ Ga	1296.9(7)	0.0065(9)	0.00028(4)	⁷¹Ga	5601.75(25)	0.063(4)	0.00274(17)
⁶⁹ Ga	1306.73(12)	0.0140(20)	0.00061(9)	⁷¹ Ga	5625.35(24)	0.0077(16)	0.00033(7)
⁶⁹ Ga	1311.89(6)	0.0259(12)	0.00113(5)	⁷¹ Ga	5644.8(7)	0.0065(21)	0.00028(9)
⁶⁹ Ga	1359.50(9)	0.0148(11)	0.00064(5)	⁷¹ Ga	5651.3(4)	0.0134(20)	0.00058(9)
⁷¹ Ga	1359.53(17)	0.0148(11)	0.00064(5)	⁷¹ Ga	5664.0(5)	0.0099(11)	0.00043(5)
⁶⁹ Ga	1456.39(7)	0.0168(11)	0.00073(5)	⁷¹ Ga	5692.2(3)	0.0211(13)	0.00092(6)
⁷¹Ga	1464.00(7)d	0.0609(19)	0.00265[2.4%]	⁷¹ Ga	5721.1(13)	0.020(4)	0.00087(17)
⁶⁹ Ga	1518.21(8)	0.0219(13)	0.00095(6)	⁶⁹ Ga	5722.9(3)	0.0067(25)	0.00029(11)
⁷¹ Ga	1532.91(17)	0.0172(12)	0.00075(5)	⁷¹ Ga	5779.11(18)	0.022(4)	0.00096(17)
⁷¹Ga	1596.68(8)d	0.0732(16)	0.00318[2.4%]	⁶⁹ Ga	5783.8(4)	0.0114(13)	0.00050(6)
⁶⁹ Ga	1621.55(12)	0.0096(10)	0.00042(4)	⁶⁹ Ga	5806.4(3)	0.0152(15)	0.00066(7)
⁶⁹ Ga	1725.48(8)	0.0108(7)	0.00047(3)	⁷¹ Ga	5883.55(19)	0.0096(4)	0.000417(17)
⁶⁹ Ga	1794.15(13)	0.0088(9)	0.00038(4)	⁷¹ Ga	5900.55(14)	0.0173(14)	0.00075(6)
⁶⁹ Ga	1846.5(3)	0.0053(10)	2.3(4)E-4	⁷¹ Ga	5919.38(15)	0.0131(12)	0.00057(5)
⁷¹Ga	1861.09(6)d	0.0904(19)	0.00393[2.4%]	⁷¹Ga	6007.25(14)	0.069(5)	0.00300(22)
⁶⁹ Ga	1866.6(5)	0.0060(17)	0.00026(7)	⁷¹Ga	6111.72(24)	0.055(4)	0.00239(17)
⁶⁹ Ga	1907.63(13)	0.0089(11)	0.00039(5)	⁷¹ Ga	6127.57(14)	0.0227(23)	0.00099(10)
⁶⁹ Ga	1930.5(3)	0.0058(11)	0.00025(5)	⁶⁹ Ga	6134.5(5)	0.0058(14)	0.00025(6)
⁶⁹ Ga	2115.98(17)	0.0066(8)	0.00029(4)	⁷¹ Ga	6190.14(17)	0.0218(19)	0.00095(8)
⁶⁹ Ga	2142.88(14)	0.0085(9)	0.00037(4)	⁶⁹ Ga	6238.6(4)	0.0067(10)	0.00029(4)
⁶⁹ Ga	2164.1(7)	0.0056(13)	2.4(6)E-4	⁷¹ Ga	6311.64(14)	0.0194(16)	0.00084(7)
⁷¹Ga	2201.91(13)d	0.52(4)	0.0226[2.4%]	⁷¹ Ga	6322.20(14)	0.0186(16)	0.00081(7)
⁷¹Ga	2491.6(3)d	0.17(4)	0.0074[2.4%]	⁶⁹ Ga	6346.4(3)	0.0140(15)	0.00061(7)
⁷¹Ga	2507.40(12)d	0.28(4)	0.0122[2.4%]	⁷¹Ga	6358.61(14)	0.138(5)	0.00600(22)
⁷¹Ga	3034.6(4)d	0.15(3)	0.0065[2.4%]	⁶⁹ Ga	6513.06(18)	0.0325(20)	0.00141(9)
⁷¹ Ga	4543.3(5)	0.0104(11)	0.00045(5)	⁷¹ Ga	6520.12(14)	0.017(3)	0.00074(13)
⁷¹ Ga	4578.2(7)	0.0058(12)	0.00025(5)	⁶⁹ Ga	7002.30(16)	0.0203(12)	0.00088(5)
⁷¹ Ga	4595.4(5)	0.0093(13)	0.00040(6)	Germanium (Z=32), At.Wt.=72.64(1), $\sigma_{\gamma}^Z=2.30(6)$			
⁷¹ Ga	4686.8(5)	0.0066(9)	0.00029(4)	⁷² Ge	68.750(17)	0.0201(7)	0.00084(3)
⁷¹ Ga	4719.2(9)	0.0052(8)	2.3(4)E-4	⁷⁰Ge	175.05(3)	0.164(4)	0.00684(17)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷⁰ Ge	175.05(3)d	0.078(5)	0.00325[100%]	⁷³ Ge	1509.719(11)	0.0422(17)	0.00176(7)
⁷⁴ Ge	177.49(4)	0.0118(5)	0.000492(21)	⁷³ Ge	1513.41(8)	~0.01	~0.0005
⁷⁰ Ge	247.27(5)	0.0123(6)	0.000513(25)	⁷³ Ge	1513.74(9)	~0.01	~0.0005
⁷⁴ Ge	253.21(5)	0.0609(16)	0.00254(7)	⁷³ Ge	1573.87(3)	0.0115(9)	0.00048(4)
⁷² Ge	284.98(5)	0.0164(7)	0.00068(3)	⁷³ Ge	1617.539(14)	0.0197(12)	0.00082(5)
⁷² Ge	297.41(3)	0.0414(12)	0.00173(5)	⁷⁰ Ge	1631.1(3)	0.0189(13)	0.00079(5)
⁷⁰ Ge	306.18(4)	0.0136(8)	0.00057(3)	⁷³ Ge	1631.83(7)	0.0175(12)	0.00073(5)
⁷² Ge	325.74(3)	0.0649(18)	0.00271(8)	⁷³ Ge	1635.84(7)	0.0138(11)	0.00058(5)
⁷⁰ Ge	326.83(3)	0.058(5)	0.00242(21)	⁷³ Ge	1640.749(12)	0.0128(10)	0.00053(4)
⁷⁰ Ge	391.43(4)	0.0253(10)	0.00106(4)	⁷³ Ge	1712.780(20)	0.0129(9)	0.00054(4)
⁷² Ge	430.34(5)	0.0161(7)	0.00067(3)	⁷³ Ge	1755.86(3)	0.014(4)	0.00058(17)
⁷² Ge	432.86(5)	0.0125(6)	0.000521(25)	⁷³ Ge	1940.422(12)	0.0382(16)	0.00159(7)
⁷³ Ge	492.933(5)	0.133(3)	0.00555(13)	⁷⁰ Ge	1964.98(5)	0.0112(11)	0.00047(5)
⁷⁰ Ge	499.87(3)	0.162(6)	0.00676(25)	⁷³ Ge	2014.478(24)	0.0127(12)	0.00053(5)
⁷³ Ge	516.19(4)	~0.02	~0.0008	⁷³ Ge	2073.746(14)	0.0205(14)	0.00086(6)
⁷⁰ Ge	517.78(8)	0.0114(10)	0.00048(4)	⁷³ Ge	4423.23(6)	0.014(3)	0.00058(13)
⁷³ Ge	531.654(7)	0.0133(7)	0.00055(3)	⁷³ Ge	4423.81(8)	0.014(4)	0.00058(17)
⁷² Ge	541.77(4)	0.0154(6)	0.000642(25)	⁷⁴ Ge	4706.98(23)	0.0151(13)	0.00063(5)
⁷⁰ Ge	572.27(5)	0.018(4)	0.00075(17)	⁷⁰ Ge	4881.79(4)	0.017(3)	0.00071(13)
⁷⁴ Ge	574.91(3)	0.0306(12)	0.00128(5)	⁷³ Ge	5165.56(5)	0.013(9)	0.0005(4)
⁷³ Ge	595.851(5)	1.100(24)	0.0459(10)	⁷³ Ge	5361.77(6)	0.0111(12)	0.00046(5)
⁷³ Ge	606.80(4)	0.015(12)	0.0006(5)	⁷⁰ Ge	5383.85(7)	0.0131(15)	0.00055(6)
⁷³ Ge	608.353(4)	0.250(6)	0.01043(25)	⁷⁰ Ge	5450.69(5)	0.028(4)	0.00117(17)
⁷³ Ge	701.509(8)	0.0642(19)	0.00268(8)	⁷² Ge	5518.30(4)	0.0290(17)	0.00121(7)
⁷⁰ Ge	708.15(3)	0.0825(24)	0.00344(10)	⁷² Ge	5650.80(6)	0.0115(12)	0.00048(5)
⁷³ Ge	770.211(8)	0.0135(8)	0.00056(3)	⁷² Ge	5740.07(10)	0.0151(15)	0.00063(6)
⁷⁰ Ge	788.60(7)	0.014(3)	0.00058(13)	⁷⁰ Ge	5817.17(4)	0.028(3)	0.00117(13)
⁷⁰ Ge	808.14(4)	0.030(5)	0.00125(21)	⁷⁰ Ge	6036.90(6)	0.045(3)	0.00188(13)
⁷³ Ge	808.218(10)	0.0197(18)	0.00082(8)	⁷⁰ Ge	6117.02(7)	0.043(6)	0.00179(25)
⁷⁰ Ge	831.30(3)	0.0445(16)	0.00186(7)	⁷³ Ge	6199.96(5)	0.0120(13)	0.00050(5)
⁷⁰ Ge	851.70(13)	0.012(7)	0.0005(3)	⁷⁴ Ge	6251.97(6)	0.0188(18)	0.00078(8)
⁷³ Ge	867.899(5)	0.553(12)	0.0231(5)	⁷³ Ge	6265.84(6)	0.015(4)	0.00063(17)
⁷³ Ge	878.130(19)	0.0112(8)	0.00047(3)	⁷⁰ Ge	6276.35(6)	0.0214(21)	0.00089(9)
⁷³ Ge	939.249(11)	0.0315(13)	0.00131(5)	⁷⁰ Ge	6320.19(5)	0.0153(14)	0.00064(6)
⁷³ Ge	961.055(7)	0.129(4)	0.00538(17)	⁷² Ge	6390.29(5)	0.0299(19)	0.00125(8)
⁷³ Ge	999.775(8)	0.0581(19)	0.00242(8)	⁷² Ge	6418.62(4)	0.0178(15)	0.00074(6)
⁷⁰ Ge	1095.42(5)	0.053(5)	0.00221(21)	⁷⁰ Ge	6707.43(3)	0.0388(25)	0.00162(10)
⁷⁰ Ge	1098.62(5)	0.0165(10)	0.00069(4)	⁷² Ge	6716.00(4)	0.0160(15)	0.00067(6)
⁷³ Ge	1101.282(6)	0.134(3)	0.00559(13)	⁷³ Ge	6717.462(23)	0.020(5)	0.00083(21)
⁷³ Ge	1105.557(10)	0.0708(20)	0.00295(8)	⁷⁰ Ge	6915.69(3)	0.031(5)	0.00129(21)
⁷³ Ge	1131.360(8)	0.0487(15)	0.00203(6)	⁷³ Ge	7091.164(15)	0.0170(11)	0.00071(5)
⁷⁰ Ge	1139.27(6)	0.0441(23)	0.00184(10)	⁷³ Ge	7260.187(14)	0.0270(15)	0.00113(6)
⁷³ Ge	1150.441(22)	0.0127(8)	0.00053(3)	⁷⁰ Ge	7415.510(23)	0.016(5)	0.00067(21)
⁷³ Ge	1200.75(10)	~0.01	~0.0005	⁷³ Ge	8030.317(13)	0.0117(9)	0.00049(4)
⁷³ Ge	1200.89(18)	~0.01	~0.0005	⁷³ Ge	8498.388(13)	0.0120(9)	0.00050(4)
⁷³ Ge	1200.94(3)	~0.01	~0.0005	⁷³ Ge	8731.744(13)	0.0128(8)	0.00053(3)
⁷³ Ge	1204.199(6)	0.141(4)	0.00588(17)	Arsenic (Z=33), At. Wt.=74.92160(2), $\sigma_{\gamma}^Z = 4.23(8)$			
⁷³ Ge	1205.862(13)	0.0114(21)	0.00048(9)	⁷⁵ As	44.4250(10)	0.560(20)	0.0227(8)
⁷³ Ge	1228.20(9)	0.0116(9)	0.00048(4)	⁷⁵ As	46.0980(10)	0.337(15)	0.0136(6)
⁷⁶ Ge	1250.55(10)	0.0110(21)	0.00046(9)	⁷⁵ As	74.8720(10)	0.12(3)	0.0049(12)
⁷² Ge	1251.30(7)	0.032(9)	0.0013(4)	⁷⁵ As	81.4110(20)	0.0107(15)	0.00043(6)
⁷⁰ Ge	1298.61(6)	0.049(4)	0.00204(17)	⁷⁵ As	83.2840(10)	0.0142(16)	0.00057(7)
⁷³ Ge	1332.081(11)	0.0122(10)	0.00051(4)	⁷⁵ As	86.7880(10)	0.579(11)	0.0234(4)
⁷⁰ Ge	1378.73(6)	0.017(4)	0.00071(17)	⁷⁵ As	91.3670(10)	0.0218(17)	0.00088(7)
⁷³ Ge	1471.712(10)	0.083(3)	0.00346(13)	⁷⁵ As	116.7550(10)	0.107(18)	0.0043(7)
⁷³ Ge	1489.491(24)	0.0234(12)	0.00098(5)	⁷⁵ As	117.3320(10)	0.071(18)	0.0029(7)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷⁵ As	118.680(3)	0.0140(10)	0.00057(4)	⁷⁵ As	360.3830(20)	0.0228(14)	0.00092(6)
⁷⁵ As	120.2580(10)	0.402(8)	0.0163(3)	⁷⁵ As	363.9040(10)	0.059(3)	0.00239(12)
⁷⁵ As	122.2470(10)	0.227(5)	0.00918(20)	⁷⁵ As	378.976(3)	0.030(3)	0.00121(12)
⁷⁵ As	127.5090(20)	0.096(3)	0.00388(12)	⁷⁵ As	379.3230(20)	0.0231(20)	0.00093(8)
⁷⁵ As	135.4110(10)	0.156(4)	0.00631(16)	⁷⁵ As	384.002(5)	0.0186(18)	0.00075(7)
⁷⁵ As	136.3430(10)	0.031(3)	0.00125(12)	⁷⁵ As	394.231(8)	0.0131(20)	0.00053(8)
⁷⁵ As	137.0270(10)	0.0391(19)	0.00158(8)	⁷⁵ As	399.3490(20)	0.0465(23)	0.00188(9)
⁷⁵ As	141.2150(20)	0.0625(21)	0.00253(9)	⁷⁵ As	402.7440(20)	0.061(3)	0.00247(12)
⁷⁵ As	142.4590(10)	0.0211(16)	0.00085(7)	⁷⁵ As	412.7930(20)	0.0117(12)	0.00047(5)
⁷⁵ As	144.5480(10)	0.1000(22)	0.00404(9)	⁷⁵ As	426.5750(10)	0.100(3)	0.00404(12)
⁷⁵ As	152.8430(20)	0.0114(13)	0.00046(5)	⁷⁵ As	428.187(3)	0.0130(14)	0.00053(6)
⁷⁵ As	155.0830(10)	0.0423(19)	0.00171(8)	⁷⁵ As	430.7920(20)	0.0134(12)	0.00054(5)
⁷⁵ As	156.8900(20)	0.0136(18)	0.00055(7)	⁷⁵ As	436.8030(10)	0.0113(12)	0.00046(5)
⁷⁵ As	157.7450(10)	0.117(24)	0.0047(10)	⁷⁵ As	460.7790(20)	0.0111(10)	0.00045(4)
⁷⁵ As	162.6820(10)	0.0257(19)	0.00104(8)	⁷⁵ As	463.647(3)	0.0333(23)	0.00135(9)
⁷⁵ As	165.0490(10)	0.996(16)	0.0403(7)	⁷⁵ As	467.965(13)	0.0165(19)	0.00067(8)
⁷⁵ As	178.0190(10)	0.0979(23)	0.00396(9)	⁷⁵ As	471.0000(10)	0.203(5)	0.00821(20)
⁷⁵ As	178.831(3)	0.0169(11)	0.00068(4)	⁷⁵ As	473.1540(10)	0.176(5)	0.00712(20)
⁷⁵ As	180.121(3)	0.0136(7)	0.00055(3)	⁷⁵ As	477.584(9)	0.0124(18)	0.00050(7)
⁷⁵ As	180.2100(10)	0.0157(8)	0.00064(3)	⁷⁵ As	479.102(5)	0.0115(17)	0.00047(7)
⁷⁵ As	186.0720(10)	0.0285(17)	0.00115(7)	⁷⁵ As	480.137(6)	0.0126(18)	0.00051(7)
⁷⁵ As	186.734(3)	0.0103(6)	0.000417(24)	⁷⁵ As	487.393(4)	0.0139(20)	0.00056(8)
⁷⁵ As	187.3130(20)	0.0152(8)	0.00061(3)	⁷⁵ As	494.105(7)	0.0100(17)	0.00040(7)
⁷⁵ As	188.0620(10)	0.090(3)	0.00364(12)	⁷⁵ As	506.4970(20)	0.0283(23)	0.00114(9)
⁷⁵ As	191.2620(20)	0.0117(17)	0.00047(7)	⁷⁵ As	517.873(10)	0.024(3)	0.00097(12)
⁷⁵ As	193.273(3)	0.0119(15)	0.00048(6)	⁷⁵ As	529.907(8)	0.0111(18)	0.00045(7)
⁷⁵ As	198.8550(10)	0.089(3)	0.00360(12)	⁷⁵ As	550.460(3)	0.071(3)	0.00287(12)
⁷⁵ As	200.446(3)	0.011(3)	0.00044(12)	⁷⁵ As	554.937(24)	0.0230(24)	0.00093(10)
⁷⁵ As	201.1800(20)	0.0140(18)	0.00057(7)	⁷⁵ As	559.10(5)d	2.00(10)	0.081[1.3%]
⁷⁵ As	211.1470(10)	0.113(3)	0.00457(12)	⁷⁵ As	565.547(7)	0.0463(25)	0.00187(10)
⁷⁵ As	220.3810(10)	0.0373(23)	0.00151(9)	⁷⁵ As	582.291(5)	0.0115(15)	0.00047(6)
⁷⁵ As	221.5320(10)	0.0534(25)	0.00216(10)	⁷⁵ As	585.492(8)	0.0161(17)	0.00065(7)
⁷⁵ As	224.004(4)	0.0126(12)	0.00051(5)	⁷⁵ As	624.685(6)	0.0225(20)	0.00091(8)
⁷⁵ As	225.7020(10)	0.0803(24)	0.00325(10)	⁷⁵ As	628.7440(10)	0.0116(17)	0.00047(7)
⁷⁵ As	235.8770(10)	0.181(4)	0.00732(16)	⁷⁵ As	632.396(24)	0.0219(20)	0.00089(8)
⁷⁵ As	238.9960(10)	0.023(10)	0.0009(4)	⁷⁵ As	640.119(10)	0.0141(20)	0.00057(8)
⁷⁵ As	241.6580(10)	0.0262(13)	0.00106(5)	⁷⁵ As	644.329(23)	0.015(3)	0.00061(12)
⁷⁵ As	246.2030(20)	0.0223(14)	0.00090(6)	⁷⁵ As	657.05(5)d	0.279(14)	0.0113[1.3%]
⁷⁵ As	256.0350(10)	0.045(11)	0.0018(4)	⁷⁵ As	669.113(4)	0.0278(13)	0.00112(5)
⁷⁵ As	263.8940(10)	0.18(4)	0.0073(16)	⁷⁵ As	687.103(8)	0.010(5)	0.00040(20)
⁷⁵ As	271.7540(10)	0.013(4)	0.00053(16)	⁷⁵ As	687.618(7)	0.0126(15)	0.00051(6)
⁷⁵ As	281.5750(10)	0.085(20)	0.0034(8)	⁷⁵ As	706.783(4)	0.0339(22)	0.00137(9)
⁷⁵ As	297.248(10)	0.010(4)	0.00040(16)	⁷⁵ As	725.909(24)	0.0118(18)	0.00048(7)
⁷⁵ As	297.5420(10)	0.055(3)	0.00222(12)	⁷⁵ As	731.840(9)	0.0102(17)	0.00041(7)
⁷⁵ As	300.4610(10)	0.051(3)	0.00206(12)	⁷⁵ As	822.346(23)	0.0303(22)	0.00123(9)
⁷⁵ As	301.654(7)	0.0109(24)	0.00044(10)	⁷⁵ As	848.593(9)	0.0282(21)	0.00114(9)
⁷⁵ As	306.639(9)	0.011(3)	0.00044(12)	⁷⁵ As	859.76(22)	0.0210(21)	0.00085(9)
⁷⁵ As	308.3190(10)	0.018(3)	0.00073(12)	⁷⁵ As	880.326(9)	0.0234(21)	0.00095(9)
⁷⁵ As	311.004(5)	0.0161(25)	0.00065(10)	⁷⁵ As	941.116(13)	0.0194(19)	0.00078(8)
⁷⁵ As	314.243(3)	0.031(3)	0.00125(12)	⁷⁵ As	942.240(8)	0.0161(8)	0.00065(3)
⁷⁵ As	322.572(4)	0.016(3)	0.00065(12)	⁷⁵ As	944.229(8)	0.0146(19)	0.00059(8)
⁷⁵ As	326.9120(20)	0.015(3)	0.00061(12)	⁷⁵ As	1216.08(5)d	0.155(8)	0.0063[1.3%]
⁷⁵ As	330.100(7)	0.023(3)	0.00093(12)	⁷⁵ As	5527.02(12)	0.0112(7)	0.00045(3)
⁷⁵ As	340.1560(20)	0.0413(21)	0.00167(9)	⁷⁵ As	5533.94(3)	0.151(7)	0.0061(3)
⁷⁵ As	352.3620(20)	0.071(3)	0.00287(12)	⁷⁵ As	5540.51(15)	0.0131(9)	0.00053(4)
⁷⁵ As	357.4070(10)	0.074(3)	0.00299(12)	⁷⁵ As	5546.04(8)	0.0181(11)	0.00073(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	
⁷⁵ As	5568.99(5)	0.0354(18)	0.00143(7)	⁷⁵ As	6659.378(9)	0.0227(11)	0.00092(4)	
⁷⁵ As	5580.21(3)	0.019(3)	0.00077(12)	⁷⁵ As	6691.241(9)	0.0246(12)	0.00100(5)	
⁷⁵ As	5601.37(7)	0.0138(8)	0.00056(3)	⁷⁵ As	6699.744(8)	0.0109(7)	0.00044(3)	
⁷⁵ As	5612.9(4)	0.0103(21)	0.00042(9)	⁷⁵ As	6718.514(11)	0.0101(6)	0.000409(24)	
⁷⁵ As	5614.99(13)	0.015(3)	0.00061(12)	⁷⁵ As	6778.047(9)	0.0143(9)	0.00058(4)	
⁷⁵ As	5629.53(7)	0.0181(11)	0.00073(4)	⁷⁵ As	6784.456(9)	0.0133(25)	0.00054(10)	
⁷⁵ As	5645.75(8)	0.0119(7)	0.00048(3)	⁷⁵ As	6808.872(8)	0.160(8)	0.0065(3)	
⁷⁵ As	5655.22(6)	0.0172(9)	0.00070(4)	⁷⁵ As	6810.898(8)	0.56(3)	0.0227(12)	
⁷⁵ As	5663.81(3)	0.019(4)	0.00077(16)	⁷⁵ As	6823.272(8)	0.0133(8)	0.00054(3)	
⁷⁵ As	5675.89(3)	0.026(4)	0.00105(16)	⁷⁵ As	6828.896(9)	0.0161(9)	0.00065(4)	
⁷⁵ As	5684.20(4)	0.0414(19)	0.00167(8)	⁷⁵ As	6857.474(8)	0.0168(10)	0.00068(4)	
⁷⁵ As	5690.54(3)	0.023(4)	0.00093(16)	⁷⁵ As	6881.302(8)	0.0162(9)	0.00066(4)	
⁷⁵ As	5698.05(3)	0.0479(22)	0.00194(9)	⁷⁵ As	6926.635(8)	0.061(4)	0.00247(16)	
⁷⁵ As	5723.39(7)	0.0160(9)	0.00065(4)	⁷⁵ As	6976.101(9)	0.0130(21)	0.00053(9)	
⁷⁵ As	5757.22(3)	0.015(3)	0.00061(12)	⁷⁵ As	7020.139(8)	0.104(7)	0.0042(3)	
⁷⁵ As	5778.12(3)	0.0482(23)	0.00195(9)	⁷⁵ As	7027.998(8)	0.0534(25)	0.00216(10)	
⁷⁵ As	5786.82(3)	0.026(4)	0.00105(16)	⁷⁵ As	7048.154(8)	0.0103(21)	0.00042(9)	
⁷⁵ As	5816.39(5)	0.0247(12)	0.00100(5)	⁷⁵ As	7063.648(8)	0.045(3)	0.00182(12)	
⁷⁵ As	5834.21(7)	0.0210(11)	0.00085(4)	⁷⁵ As	7163.396(8)	0.0181(9)	0.00073(4)	
⁷⁵ As	5854.92(13)	0.0218(16)	0.00088(7)	⁷⁵ As	7208.183(8)	0.0127(7)	0.00051(3)	
⁷⁵ As	5869.65(7)	0.015(4)	0.00061(16)	⁷⁵ As	7241.649(8)	0.0167(20)	0.00068(8)	
⁷⁵ As	5877.68(6)	0.0276(14)	0.00112(6)	⁷⁵ As	7284.007(8)	0.036(3)	0.00146(12)	
⁷⁵ As	5884.72(3)	0.0504(24)	0.00204(10)	Selenium (Z=34), At.Wt.=78.96(3), $\sigma_{\gamma}^Z=12.0(7)$				
⁷⁵ As	5906.24(8)	0.0128(8)	0.00052(3)	⁷⁶ Se	51.3610(10)	~0.03	~0.001	
⁷⁵ As	5931.22(9)	0.0143(9)	0.00058(4)	⁷⁶ Se	87.8660(10)	0.210(4)	0.00806(15)	
⁷⁵ As	5942.97(9)	0.0119(7)	0.00048(3)	⁷⁴ Se	112.3880(10)	0.0317(15)	0.00122(6)	
⁷⁵ As	5970.12(5)	0.0210(10)	0.00085(4)	⁷⁶ Se	125.8440(10)	0.074(17)	0.0028(7)	
⁷⁵ As	5976.18(5)	0.0199(10)	0.00080(4)	⁷⁶ Se	139.2270(10)	0.543(9)	0.0208(4)	
⁷⁵ As	6006.34(5)	0.0297(15)	0.00120(6)	⁷⁴ Se	141.3140(20)	0.0246(21)	0.00094(8)	
⁷⁵ As	6014.00(8)	0.0224(12)	0.00091(5)	⁷⁶ Se	161.9220(10)d	0.855(23)	0.0328[99%]	
⁷⁵ As	6019.17(11)	0.0161(10)	0.00065(4)	⁷⁶ Se	180.751(3)	0.0291(12)	0.00112(5)	
⁷⁵ As	6027.524(22)	0.020(3)	0.00081(12)	⁷⁶ Se	200.4530(20)	0.233(9)	0.0089(4)	
⁷⁵ As	6059.483(22)	0.026(3)	0.00105(12)	⁷⁶ Se	231.4270(20)	0.105(3)	0.00403(12)	
⁷⁵ As	6142.79(3)	0.014(3)	0.00057(12)	⁷⁶ Se	238.9980(10)	2.06(3)	0.0791(12)	
⁷⁵ As	6171.99(9)	0.0105(6)	0.000425(24)	⁷⁷ Se	248.43(8)	0.023(5)	0.00088(19)	
⁷⁵ As	6180.14(5)	0.0264(13)	0.00107(5)	⁷⁶ Se	249.7880(10)	0.538(9)	0.0206(4)	
⁷⁵ As	6203.57(4)	0.016(3)	0.00065(12)	⁷⁶ Se	281.6400(20)	0.124(5)	0.00476(19)	
⁷⁵ As	6223.06(3)	0.012(3)	0.00049(12)	⁷⁴ Se	286.5710(20)	0.280(6)	0.01075(23)	
⁷⁵ As	6231.24(4)	0.0413(19)	0.00167(8)	⁷⁴ Se	292.8430(20)	0.0297(21)	0.00114(8)	
⁷⁵ As	6294.295(25)	0.064(6)	0.00259(24)	⁷⁶ Se	297.2160(20)	0.337(7)	0.0129(3)	
⁷⁵ As	6303.71(22)	0.024(4)	0.00097(16)	⁷⁶ Se	303.7930(20)	0.052(3)	0.00200(12)	
⁷⁵ As	6305.37(3)	0.085(4)	0.00344(16)	⁷⁶ Se	331.2210(20)	0.0526(25)	0.00202(10)	
⁷⁵ As	6342.976(15)	0.010(3)	0.00040(12)	⁷⁶ Se	368.733(4)	0.026(3)	0.00100(12)	
⁷⁵ As	6357.58(7)	0.0204(10)	0.00083(4)	⁷⁶ Se	378.9540(20)	0.022(3)	0.00084(12)	
⁷⁵ As	6370.124(9)	0.0274(13)	0.00111(5)	⁷⁶ Se	384.9800(20)	0.032(5)	0.00123(19)	
⁷⁵ As	6388.768(10)	0.0329(18)	0.00133(7)	⁷⁶ Se	390.8920(20)	0.029(4)	0.00111(15)	
⁷⁵ As	6393.133(12)	0.032(4)	0.00129(16)	⁷⁸ Se	432.12(14)	0.0227(15)	0.00087(6)	
⁷⁵ As	6403.761(12)	0.022(3)	0.00089(12)	⁷⁶ Se	439.4510(20)	0.319(8)	0.0122(3)	
⁷⁵ As	6419.378(23)	0.031(4)	0.00125(16)	⁸⁰ Se	467.81(10)	0.128(4)	0.00491(15)	
⁷⁵ As	6465.17(12)	0.0111(24)	0.00045(10)	⁷⁶ Se	484.5440(20)	0.125(4)	0.00480(15)	
⁷⁵ As	6526.051(13)	0.0123(7)	0.00050(3)	⁸⁰ Se	491.46(22)	0.022(3)	0.00084(12)	
⁷⁵ As	6534.932(9)	0.0316(15)	0.00128(6)	⁷⁶ Se	504.7970(20)	0.024(5)	0.00092(19)	
⁷⁵ As	6542.669(10)	0.0408(19)	0.00165(8)	⁷⁶ Se	518.1810(20)	0.273(7)	0.0105(3)	
⁷⁵ As	6583.556(10)	0.027(3)	0.00109(12)	⁷⁶ Se	520.6370(20)	1.260(18)	0.0484(7)	
⁷⁵ As	6587.038(13)	0.045(3)	0.00182(12)	⁷⁷ Se	545.297(12)	0.0635(25)	0.00244(10)	
⁷⁵ As	6600.71(3)	0.0372(17)	0.00150(7)	⁷⁶ Se	565.7300(20)	0.0398(23)	0.00153(9)	
⁷⁵ As	6620.59(5)	0.0304(15)	0.00123(6)					

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷⁶ Se	568.0660(20)	0.103(8)	0.0040(3)	⁷⁷ Se	1713.544(22)	0.163(8)	0.0063(3)
⁷⁶ Se	569.185(4)	0.024(8)	0.0009(3)	⁷⁶ Se	1714.739(10)	0.033(3)	0.00127(12)
⁷⁶ Se	574.6420(20)	0.054(3)	0.00207(12)	⁷⁷ Se	1721.43(8)	0.078(4)	0.00299(15)
⁷⁶Se	578.8550(20)	0.243(5)	0.00933(19)	⁸⁰ Se	1724.88(18)	0.044(5)	0.00169(19)
⁷⁶ Se	585.4320(20)	0.077(4)	0.00296(15)	⁷⁶ Se	1790.24(7)	0.036(4)	0.00138(15)
⁷⁶ Se	607.471(4)	0.027(5)	0.00104(19)	⁷⁶ Se	1847.93(5)	0.046(4)	0.00177(15)
⁷⁶ Se	610.3800(20)	0.0345(21)	0.00132(8)	⁷⁶ Se	1872.21(5)	0.048(4)	0.00184(15)
⁷⁴ Se	610.7130(20)	0.0316(22)	0.00121(8)	⁷⁷ Se	1923.32(10)	0.068(5)	0.00261(19)
⁷⁷Se	613.724(3)	2.14(5)	0.0821(19)	⁷⁶ Se	1963.15(7)	0.034(4)	0.00130(15)
⁷⁶ Se	645.8300(20)	0.099(3)	0.00380(12)	⁷⁶ Se	1980.40(5)	0.022(16)	0.0008(6)
⁷⁷ Se	687.251(5)	0.063(5)	0.00242(19)	⁷⁷ Se	1995.871(6)	0.119(5)	0.00457(19)
⁷⁷Se	694.914(4)	0.443(10)	0.0170(4)	⁷⁶ Se	2035.26(5)	0.043(5)	0.00165(19)
⁷⁶ Se	707.9800(20)	0.0281(20)	0.00108(8)	⁷⁶ Se	2074.08(5)	0.033(20)	0.0013(8)
⁷⁶ Se	749.6060(20)	0.042(3)	0.00161(12)	⁷⁶ Se	2142.65(8)	0.040(4)	0.00154(15)
⁷⁶ Se	755.3920(20)	0.186(4)	0.00714(15)	⁷⁶ Se	2212.02(9)	0.033(3)	0.00127(12)
⁷⁶ Se	817.8520(20)	0.174(5)	0.00668(19)	⁷⁶ Se	2249.88(12)	0.0221(21)	0.00085(8)
⁷⁷ Se	828.188(12)	0.0300(17)	0.00115(7)	⁷⁷ Se	2257.48(13)	0.022(3)	0.00084(12)
⁷⁶ Se	881.840(4)	0.040(3)	0.00154(12)	⁷⁶ Se	2264.68(17)	0.031(4)	0.00119(15)
⁷⁷ Se	884.867(7)	0.100(6)	0.00384(23)	⁷⁷ Se	2284.36(6)	0.054(5)	0.00207(19)
⁷⁶Se	885.8270(20)	0.262(7)	0.0101(3)	⁷⁷ Se	2319.4(4)	0.025(10)	0.0010(4)
⁷⁷ Se	889.095(9)	0.096(6)	0.00368(23)	⁷⁷ Se	2391.87(10)	0.043(4)	0.00165(15)
⁷⁶ Se	889.108(4)	0.180(5)	0.00691(19)	⁷⁷ Se	2391.89(9)	0.038(7)	0.0015(3)
⁷⁶ Se	890.981(5)	0.083(4)	0.00319(15)	⁷⁶ Se	2417.59(12)	0.024(17)	0.0009(7)
⁷⁶ Se	946.9760(20)	0.089(4)	0.00342(15)	⁷⁷ Se	2572.70(8)	0.025(4)	0.00096(15)
⁷⁶ Se	951.809(6)	0.047(3)	0.00180(12)	⁷⁶ Se	2590.77(5)	0.039(13)	0.0015(5)
⁷⁶ Se	990.377(4)	0.028(3)	0.00107(12)	⁷⁶ Se	2600.85(8)	0.0221(21)	0.00085(8)
⁷⁶ Se	991.629(6)	0.057(5)	0.00219(19)	⁷⁶ Se	2614.09(5)	0.047(5)	0.00180(19)
⁷⁶ Se	1005.1770(20)	0.117(5)	0.00449(19)	⁷⁷ Se	2674.47(6)	0.060(5)	0.00230(19)
⁷⁶ Se	1091.64(3)	0.026(5)	0.00100(19)	⁷⁶ Se	2749.78(15)	0.023(5)	0.00088(19)
⁷⁶ Se	1128.104(4)	0.023(4)	0.00088(15)	⁷⁷ Se	2769.87(8)	0.035(3)	0.00134(12)
⁷⁷ Se	1144.952(16)	0.076(3)	0.00292(12)	⁷⁶ Se	2809.08(7)	0.034(24)	0.0013(9)
⁷⁶ Se	1161.828(5)	0.079(4)	0.00303(15)	⁷⁶ Se	2872.93(9)	0.046(3)	0.00177(12)
⁷⁶ Se	1163.476(4)	0.087(4)	0.00334(15)	⁷⁷ Se	2873.47(9)	0.061(8)	0.0023(3)
⁷⁶ Se	1172.617(5)	0.058(3)	0.00223(12)	⁷⁶ Se	2922.68(11)	0.0214(21)	0.00082(8)
⁷⁶ Se	1186.973(3)	0.033(3)	0.00127(12)	⁷⁶ Se	2982.82(11)	0.030(9)	0.0012(4)
⁷⁶ Se	1194.111(10)	0.022(3)	0.00084(12)	⁷⁶ Se	3039.95(11)	0.038(16)	0.0015(6)
⁷⁷ Se	1198.72(10)	0.0379(23)	0.00145(9)	⁷⁷ Se	3072.64(13)	0.0257(17)	0.00099(7)
⁸⁰ Se	1202.0(3)	0.037(3)	0.00142(12)	⁷⁶ Se	3206.54(17)	0.027(14)	0.0010(5)
⁷⁷ Se	1240.206(12)	0.106(4)	0.00407(15)	⁷⁷ Se	3242.39(12)	0.033(7)	0.0013(3)
⁷⁶Se	1296.986(7)	0.240(7)	0.0092(3)	⁷⁶ Se	3279.09(12)	0.023(4)	0.00088(15)
⁷⁶ Se	1306.540(10)	0.061(6)	0.00234(23)	⁷⁶ Se	3296.55(13)	0.028(4)	0.00107(15)
⁷⁷Se	1308.632(5)	0.317(8)	0.0122(3)	⁷⁷ Se	3385.13(12)	0.038(11)	0.0015(4)
⁷⁷ Se	1338.817(12)	0.0354(19)	0.00136(7)	⁷⁷ Se	3439.40(13)	0.028(3)	0.00107(12)
⁷⁶ Se	1378.172(7)	0.048(4)	0.00184(15)	⁷⁶ Se	3466.82(17)	0.022(4)	0.00084(15)
⁷⁷ Se	1382.159(6)	0.069(3)	0.00265(12)	⁷⁶ Se	3517.60(17)	0.032(5)	0.00123(19)
⁷⁶ Se	1384.131(6)	0.080(4)	0.00307(15)	⁷⁶ Se	3550.31(20)	0.042(17)	0.0016(7)
⁷⁶ Se	1395.42(3)	0.024(6)	0.00092(23)	⁷⁶ Se	3620.46(17)	0.028(4)	0.00107(15)
⁷⁶ Se	1402.471(4)	0.032(4)	0.00123(15)	⁷⁶ Se	3636.29(17)	0.030(4)	0.00115(15)
⁷⁶ Se	1411.612(5)	0.115(6)	0.00441(23)	⁷⁶ Se	3693.06(20)	0.024(9)	0.0009(4)
⁷⁶ Se	1475.746(10)	0.030(20)	0.0012(8)	⁷⁶ Se	3700.14(12)	0.034(24)	0.0013(9)
⁷⁶ Se	1529.27(15)	0.034(6)	0.00130(23)	⁷⁶ Se	3858.09(11)	0.037(6)	0.00142(23)
⁷⁷ Se	1529.71(5)	0.061(13)	0.0023(5)	⁷⁶ Se	3866.33(10)	0.024(5)	0.00092(19)
⁷⁶ Se	1578.621(7)	0.042(4)	0.00161(15)	⁷⁶ Se	3873.00(12)	0.025(4)	0.00096(15)
⁷⁶ Se	1623.124(6)	0.063(5)	0.00242(19)	⁷⁶ Se	3901.06(17)	0.073(8)	0.0028(3)
⁷⁶ Se	1677.06(3)	0.023(4)	0.00088(15)	⁷⁶ Se	3945.94(17)	0.033(5)	0.00127(19)
⁷⁶ Se	1712.75(5)	0.023(3)	0.00088(12)	⁷⁶ Se	3968.30(13)	0.040(4)	0.00154(15)
				⁷⁶ Se	4003.78(5)	0.025(4)	0.00096(15)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷⁶ Se	4020.78(7)	0.0225(16)	0.00086(6)	⁷⁹ Br	37.054(3)	0.160(10)	0.0061(4)
⁷⁶ Se	4056.54(11)	0.031(5)	0.00119(19)	⁷⁹ Br	50.112(3)	0.0081(6)	0.000307(23)
⁷⁶ Se	4064.52(11)	0.0229(14)	0.00088(5)	⁷⁹ Br	59.471(4)	0.202(5)	0.00766(19)
⁷⁶ Se	4174.76(12)	0.037(7)	0.0014(3)	⁸¹ Br	72.0210(20)	0.0121(4)	0.000459(15)
⁷⁶ Se	4185.94(13)	0.042(10)	0.0016(4)	⁷⁹ Br	74.972(3)	0.0323(7)	0.00123(3)
⁷⁶ Se	4243.49(13)	0.0220(13)	0.00084(5)	⁸¹ Br	85.267(7)	0.0096(4)	0.000364(15)
⁷⁶ Se	4354.79(9)	0.040(5)	0.00154(19)	⁷⁹ Br	124.028(3)	0.0268(5)	0.001016(19)
⁷⁶ Se	4367.73(15)	0.024(3)	0.00092(12)	⁷⁹ Br	126.280(3)	0.0174(4)	0.000660(15)
⁷⁶ Se	4378.36(8)	0.085(16)	0.0033(6)	⁷⁹ Br	146.904(3)	0.0184(7)	0.00070(3)
⁷⁶ Se	4435.83(11)	0.032(7)	0.0012(3)	⁷⁹ Br	159.044(4)	0.0171(7)	0.00065(3)
⁷⁶ Se	4526.75(5)	0.115(8)	0.0044(3)	⁷⁹ Br	159.800(4)	0.0232(7)	0.00088(3)
⁷⁶ Se	4545.72(9)	0.049(5)	0.00188(19)	⁷⁹ Br	175.084(3)	0.0173(12)	0.00066(5)
⁷⁶ Se	4565.56(5)	0.156(11)	0.0060(4)	⁸¹ Br	184.6440(10)	0.0258(12)	0.00098(5)
⁷⁶ Se	4609.57(7)	0.058(9)	0.0022(4)	⁷⁹ Br	195.602(4)	0.434(14)	0.0165(5)
⁷⁶ Se	4641.97(5)	0.027(6)	0.00104(23)	⁷⁹ Br	197.607(3)	0.0175(11)	0.00066(4)
⁷⁶ Se	4702.43(15)	0.023(4)	0.00088(15)	⁷⁹ Br	211.594(3)	0.0454(21)	0.00172(8)
⁷⁶ Se	4926.78(7)	0.048(8)	0.0018(3)	⁷⁹ Br	213.816(5)	0.0104(11)	0.00039(4)
⁷⁶ Se	4963.217(24)	0.039(5)	0.00150(19)	⁷⁹ Br	218.785(4)	0.019(8)	0.0007(3)
⁷⁶ Se	5025.80(5)	0.150(12)	0.0058(5)	⁷⁹ Br	219.377(3)	0.399(14)	0.0151(5)
⁷⁶ Se	5078.75(5)	0.033(11)	0.0013(4)	⁸¹ Br	221.0950(20)	0.0123(14)	0.00047(5)
⁷⁶ Se	5098.56(10)	0.031(8)	0.0012(3)	⁷⁹ Br	223.627(3)	0.153(5)	0.00580(19)
⁷⁶ Se	5154.33(7)	0.053(5)	0.00203(19)	⁷⁹ Br	226.53(5)	0.0080(20)	0.00030(8)
⁷⁶ Se	5169.734(22)	0.031(4)	0.00119(15)	⁷⁹ Br	234.320(3)	0.205(10)	0.0078(4)
⁷⁶ Se	5206.60(9)	0.045(5)	0.00173(19)	⁷⁹ Br	236.454(3)	0.0372(23)	0.00141(9)
⁷⁶ Se	5275.98(9)	0.024(9)	0.0009(4)	⁷⁹ Br	244.237(3)	0.45(3)	0.0171(11)
⁷⁶ Se	5600.995(21)	0.301(14)	0.0116(5)	⁸¹ Br	244.8310(10)	0.15(5)	0.0057(19)
⁷⁶ Se	5703.864(23)	0.029(5)	0.00111(19)	⁷⁹ Br	245.203(4)	0.80(3)	0.0303(11)
⁷⁶ Se	5795.473(21)	0.127(16)	0.0049(6)	⁸¹ Br	245.54(3)	0.018(4)	0.00068(15)
⁷⁷ Se	5813.24(10)	0.0269(13)	0.00103(5)	⁸¹ Br	250.2080(20)	0.0145(19)	0.00055(7)
⁷⁶ Se	6006.973(21)	0.289(20)	0.0111(8)	⁷⁹ Br	263.460(8)	0.0105(25)	0.00040(10)
⁷⁶ Se	6016.113(21)	0.101(10)	0.0039(4)	⁸¹ Br	264.4350(10)	0.035(3)	0.00133(11)
⁷⁷ Se	6049.20(13)	0.0291(13)	0.00112(5)	⁷⁹ Br	271.374(3)	0.462(7)	0.0175(3)
⁷⁶ Se	6231.597(21)	0.10(4)	0.0038(15)	⁷⁹ Br	274.532(5)	0.158(3)	0.00599(11)
⁸⁰ Se	6232.9(5)	0.10(3)	0.0038(12)	⁷⁹ Br	278.186(3)	0.0238(14)	0.00090(5)
⁷⁷ Se	6244.07(13)	0.043(3)	0.00165(12)	⁸¹ Br	278.3620(20)	0.014(5)	0.00053(19)
⁷⁷ Se	6315.30(9)	0.044(3)	0.00169(12)	⁸¹ Br	287.7390(20)	0.253(4)	0.00960(15)
⁷⁶ Se	6413.379(21)	0.192(15)	0.0074(6)	⁷⁹ Br	294.349(3)	0.1160(22)	0.00440(8)
⁷⁷ Se	6498.52(12)	0.047(4)	0.00180(15)	⁷⁹ Br	296.908(4)	0.0307(15)	0.00116(6)
⁷⁶ Se	6600.690(21)	0.623(20)	0.0239(8)	⁷⁹ Br	299.886(4)	8.00E-02	3.00E-03
⁷⁷ Se	6811.00(13)	0.0257(22)	0.00099(8)	⁷⁹ Br	303.02(5)	0.008(3)	0.00030(11)
⁷⁷ Se	6905.75(8)	0.0234(22)	0.00090(8)	⁷⁹ Br	311.090(6)	0.0080(12)	0.00030(5)
⁷⁷ Se	7113.76(8)	0.037(3)	0.00142(12)	⁷⁹ Br	314.982(3)	0.460(9)	0.0174(3)
⁷⁶ Se	7179.492(21)	0.261(25)	0.0100(10)	⁷⁹ Br	315.524(17)	0.030(8)	0.0011(3)
⁷⁷ Se	7209.15(6)	0.056(3)	0.00215(12)	⁸¹ Br	315.770(5)	0.022(8)	0.0008(3)
⁷⁶ Se	7418.467(21)	0.350(13)	0.0134(5)	⁸¹ Br	316.8510(20)	0.017(5)	0.00064(19)
⁷⁷ Se	7491.71(9)	0.0295(15)	0.00113(6)	⁷⁹ Br	321.937(8)	0.0262(18)	0.00099(7)
⁷⁴ Se	7734.052(18)	0.13(6)	0.0050(23)	⁷⁹ Br	329.551(4)	0.0213(16)	0.00081(6)
⁷⁷ Se	8162.11(9)	0.058(3)	0.00223(12)	⁸¹ Br	339.881(3)	0.0134(14)	0.00051(5)
⁷⁷ Se	8170.00(4)	0.054(4)	0.00207(15)	⁷⁹ Br	343.405(3)	0.118(4)	0.00448(15)
⁷⁷ Se	8501.35(3)	0.048(3)	0.00184(12)	⁸¹ Br	345.0060(10)	0.154(4)	0.00584(15)
⁷⁷ Se	9188.52(3)	0.150(8)	0.0058(3)	⁷⁹ Br	345.580(4)	0.023(4)	0.00087(15)
⁷⁷ Se	9883.35(3)	0.220(22)	0.0084(8)	⁸¹ Br	346.986(4)	0.0122(18)	0.00046(7)
⁷⁷ Se	10496.99(3)	0.0221(25)	0.00085(10)	⁸¹ Br	350.3830(20)	0.0188(15)	0.00071(6)
Bromine (Z=35), At. Wt.=79.904(1), $\sigma_{\gamma}^Z = 6.39(7)$				⁷⁹ Br	366.604(4)	0.233(6)	0.00884(23)
⁸¹ Br	29.1130(10)	0.1680(20)	0.00637(8)	⁷⁹ Br	370.530(5)	0.0171(19)	0.00065(7)
⁷⁹ Br	37.0520(20)d	0.428(12)	0.0162[7.5%]	⁷⁹ Br	370.531(3)	0.0171(9)	0.00065(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁷⁹ Br	373.44(5)	0.0140(19)	0.00053(7)	⁷⁹ Br	689.994(16)	0.083(4)	0.00315(15)
⁸¹ Br	374.1180(10)	0.011(3)	0.00042(11)	⁸¹ Br	698.374(5)d	0.337(3)	0.01278(12)
⁷⁹ Br	377.397(14)	0.0100(19)	0.00038(7)	⁷⁹ Br	702.025(9)	0.0648(14)	0.00246(5)
⁸¹ Br	379.988(12)	0.0190(11)	0.00072(4)	⁸¹ Br	716.14(8)	0.0420(23)	0.00159(9)
⁷⁹ Br	385.598(11)	0.0232(9)	0.00088(3)	⁸¹ Br	717.756(20)	0.0373(8)	0.00141(3)
⁷⁹ Br	389.189(4)	0.0486(13)	0.00184(5)	⁷⁹ Br	721.417(12)	0.026(6)	0.00099(23)
⁸¹ Br	397.147(3)	0.0125(18)	0.00047(7)	⁷⁹ Br	723.983(5)	0.019(3)	0.00072(11)
⁸¹ Br	400.906(20)	0.0234(16)	0.00089(6)	⁷⁹ Br	731.147(4)	0.0139(6)	0.000527(23)
⁸¹ Br	402.743(3)	0.0170(16)	0.00064(6)	⁸¹ Br	746.970(23)	0.0091(14)	0.00035(5)
⁷⁹ Br	408.55(8)	0.0116(20)	0.00044(8)	⁷⁹ Br	751.014(10)	0.029(3)	0.00110(11)
⁷⁹ Br	409.002(6)	0.0150(20)	0.00057(8)	⁷⁹ Br	755.728(11)	0.0126(17)	0.00048(6)
⁷⁹ Br	414.04(7)	0.0332(17)	0.00126(6)	⁷⁹ Br	765.957(10)	0.0537(16)	0.00204(6)
⁷⁹ Br	432.216(4)	0.0783(14)	0.00297(5)	⁸¹ Br	776.517(3)d	0.990(10)	0.0375(4)
⁷⁹ Br	450.906(5)	0.0170(13)	0.00064(5)	⁷⁹ Br	809.28(3)	0.0084(22)	0.00032(8)
⁷⁹ Br	452.611(5)	0.0679(24)	0.00258(9)	⁸¹ Br	816.578(20)	0.0191(15)	0.00072(6)
⁷⁹ Br	455.830(3)	0.0230(13)	0.00087(5)	⁷⁹ Br	827.31(4)	0.015(3)	0.00057(11)
⁷⁹ Br	459.775(4)	0.0455(19)	0.00173(7)	⁸¹ Br	827.828(6)d	0.285(3)	0.01081(11)
⁸¹ Br	465.89(3)	0.026(4)	0.00099(15)	⁷⁹ Br	830.856(14)	0.0413(12)	0.00157(5)
⁸¹ Br	466.63(3)	0.008(4)	0.00030(15)	⁷⁹ Br	845.70(3)	0.0257(21)	0.00097(8)
⁷⁹ Br	468.980(3)	0.29(3)	0.0110(11)	⁷⁹ Br	850.93(4)	0.0082(14)	0.00031(5)
⁷⁹ Br	470.619(16)	0.018(3)	0.00068(11)	⁸¹ Br	856.13(3)	0.0081(11)	0.00031(4)
⁷⁹ Br	479.082(10)	0.018(9)	0.0007(3)	⁷⁹ Br	860.488(18)	0.0450(19)	0.00171(7)
⁷⁹ Br	482.813(21)	0.0120(20)	0.00046(8)	⁷⁹ Br	876.59(4)	0.0111(7)	0.00042(3)
⁸¹ Br	483.886(3)	0.042(18)	0.0016(7)	⁷⁹ Br	883.60(6)	0.0278(10)	0.00105(4)
⁷⁹ Br	492.884(4)	0.0292(10)	0.00111(4)	⁸¹ Br	888.599(20)	0.0224(15)	0.00085(6)
⁷⁹ Br	494.045(7)	0.009(5)	0.00034(19)	⁷⁹ Br	889.949(11)	0.0128(17)	0.00049(6)
⁸¹ Br	495.0380(20)	0.0342(14)	0.00130(5)	⁸¹ Br	895.87(5)	0.0213(10)	0.00081(4)
⁷⁹ Br	498.19(3)	0.0336(13)	0.00127(5)	⁷⁹ Br	908.97(9)	0.0144(9)	0.00055(3)
⁸¹ Br	512.488(20)	0.21(3)	0.0080(11)	⁸¹ Br	910.73(3)	0.0400(12)	0.00152(5)
⁷⁹ Br	529.247(7)	0.0321(9)	0.00122(3)	⁷⁹ Br	914.574(7)	0.0508(14)	0.00193(5)
⁸¹ Br	538.219(20)	0.0109(10)	0.00041(4)	⁷⁹ Br	919.36(5)	0.016(3)	0.00061(11)
⁸¹ Br	541.856(9)	0.0151(23)	0.00057(9)	⁸¹ Br	932.794(25)	0.0216(10)	0.00082(4)
⁷⁹ Br	542.515(6)	0.114(5)	0.00432(19)	⁷⁹ Br	933.823(12)	0.010(3)	0.00038(11)
⁷⁹ Br	545.667(7)	0.0094(14)	0.00036(5)	⁷⁹ Br	952.58(9)	0.0182(8)	0.00069(3)
⁷⁹ Br	549.559(3)	0.0593(14)	0.00225(5)	⁸¹ Br	976.508(24)	0.0459(13)	0.00174(5)
⁸¹ Br	552.1730(20)	0.0161(11)	0.00061(4)	⁷⁹ Br	977.431(12)	0.013(3)	0.00049(11)
⁸¹ Br	554.3480(20)d	0.838(8)	0.0318(3)	⁸¹ Br	1013.03(3)	0.023(3)	0.00087(11)
⁷⁹ Br	557.257(21)	0.0315(23)	0.00119(9)	⁷⁹ Br	1022.385(10)	0.0167(14)	0.00063(5)
⁸¹ Br	566.0990(20)	0.0551(12)	0.00209(5)	⁸¹ Br	1034.706(23)	0.0231(9)	0.00088(3)
⁸¹ Br	581.2860(20)	0.0231(11)	0.00088(4)	⁸¹ Br	1036.890(9)	0.0081(7)	0.00031(3)
⁸¹ Br	595.2120(20)	0.0177(11)	0.00067(4)	⁸¹ Br	1044.002(5)d	0.323(3)	0.01225(12)
⁸¹ Br	599.27(3)	0.0124(9)	0.00047(3)	⁸¹ Br	1079.99(5)	0.0350(19)	0.00133(7)
⁷⁹ Br	604.61(5)	0.013(5)	0.00049(19)	⁷⁹ Br	1087.46(3)	0.0092(10)	0.00035(4)
⁸¹ Br	608.115(19)	0.0438(13)	0.00166(5)	⁸¹ Br	1133.427(20)	0.0110(15)	0.00042(6)
⁷⁹ Br	616.3(5)d	0.39(4)	0.0148[62%]	⁷⁹ Br	1143.370(21)	0.0225(18)	0.00085(7)
⁸¹ Br	619.106(4)d	0.515(5)	0.01953(19)	⁷⁹ Br	1147.96(4)	0.0205(17)	0.00078(6)
⁷⁹ Br	619.17(3)	0.0308(12)	0.00117(5)	⁸¹ Br	1157.506(25)	0.0210(17)	0.00080(6)
⁷⁹ Br	630.710(12)	0.0224(13)	0.00085(5)	⁷⁹ Br	1175.25(3)	0.0116(11)	0.00044(4)
⁷⁹ Br	636.681(8)	0.018(4)	0.00068(15)	⁷⁹ Br	1190.73(5)	0.0216(10)	0.00082(4)
⁸¹ Br	643.291(6)	0.0373(20)	0.00141(8)	⁸¹ Br	1201.13(3)	0.0185(8)	0.00070(3)
⁷⁹ Br	660.561(4)	0.082(3)	0.00311(11)	⁷⁹ Br	1248.801(12)	0.0527(22)	0.00200(8)
⁷⁹ Br	678.69(4)	0.0089(19)	0.00034(7)	⁸¹ Br	1317.473(10)d	0.314(3)	0.01191(12)
⁸¹ Br	684.885(3)	0.050(3)	0.00190(11)	⁷⁹ Br	1320.19(4)	0.012(5)	0.00046(19)
⁷⁹ Br	684.94(5)	0.0120(20)	0.00046(8)	⁷⁹ Br	1321.96(11)	0.0152(14)	0.00058(5)
⁷⁹ Br	686.930(5)	0.014(3)	0.00053(11)	⁸¹ Br	1474.880(10)d	0.1930(20)	0.00732(8)
⁸¹ Br	687.02(8)	0.0157(20)	0.00060(8)	⁸¹ Br	6349.19(4)	0.0168(12)	0.00064(5)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁸¹ Br	6360.18(3)	0.015(5)	0.00057(19)	⁸³ Kr	1543.27(19)	0.486(17)	0.0176(6)
⁸¹ Br	6413.36(3)	0.0136(11)	0.00052(4)	⁸³ Kr	1623.20(20)	0.327(15)	0.0118(5)
⁸¹ Br	6437.69(5)	0.0328(17)	0.00124(6)	⁸³ Kr	1656.15(18)	0.28(5)	0.0101(18)
⁷⁹ Br	6533.28(8)	0.0196(14)	0.00074(5)	⁸³ Kr	1682.0(3)	0.212(17)	0.0077(6)
⁷⁹ Br	6570.15(13)	0.0285(13)	0.00108(5)	⁸³ Kr	1741.7(3)	0.437(19)	0.0158(7)
⁸¹ Br	6570.27(3)	0.008(3)	0.00030(11)	⁸³Kr	1897.79(8)	2.24(3)	0.0810(11)
⁸¹ Br	6621.81(3)	0.0104(22)	0.00039(8)	⁸³ Kr	1979.34(11)	1.070(22)	0.0387(8)
⁷⁹ Br	6643.30(8)	0.0318(18)	0.00121(7)	⁸³ Kr	2160.48(7)	0.577(15)	0.0209(5)
⁷⁹ Br	6668.16(11)	0.0306(18)	0.00116(7)	⁸³ Kr	2200.86(11)	0.241(10)	0.0087(4)
⁷⁹ Br	6689.13(9)	0.0321(14)	0.00122(5)	⁸³ Kr	2544.72(19)	0.27(3)	0.0098(11)
⁷⁹ Br	6701.38(9)	0.0168(10)	0.00064(4)	⁸³ Kr	6281.4(7)	2.70E-01	9.80E-03
⁸¹ Br	6746.030(22)	0.0386(16)	0.00146(6)	⁸³ Kr	6306.8(7)	4.80E-01	1.70E-02
⁷⁹ Br	6894.78(8)	0.0101(7)	0.00038(3)	⁸³ Kr	6519.1(7)	8.80E-01	3.20E-02
⁷⁹ Br	6977.51(8)	0.0110(8)	0.00042(3)	⁸³ Kr	6803.5(8)	6.40E-01	2.30E-02
⁷⁹ Br	7031.43(8)	0.0447(22)	0.00170(8)	⁸³ Kr	6880.7(7)	1.30E+00	4.70E-02
⁷⁹ Br	7078.18(8)	0.0566(24)	0.00215(9)	⁸³ Kr	6931.7(8)	5.40E-01	2.00E-02
⁷⁹ Br	7126.18(8)	0.0154(15)	0.00058(6)	⁸³ Kr	7207.5(9)	2.50E-01	9.00E-03
⁷⁹ Br	7168.08(8)	0.0103(8)	0.00039(3)	Rubidium (Z=37), At.Wt.=85.4678(3), $\sigma_{\gamma}^Z=0.38(7)$			
⁸¹ Br	7172.612(22)	0.0238(12)	0.00090(5)	⁸⁵ Rb	54.01(6)	0.006(3)	2.1(11)E-4
⁸¹ Br	7229.873(22)	0.0250(14)	0.00095(5)	⁸⁵Rb	59.75(6)	0.010(4)	0.00035(14)
⁸¹ Br	7301.888(22)	0.0101(8)	0.00038(3)	⁸⁵ Rb	84.85(8)	0.0052(22)	1.8(8)E-4
⁷⁹ Br	7422.77(8)	0.0495(18)	0.00188(7)	⁸⁵ Rb	96.87(10)	0.0026(9)	9(3)E-5
⁷⁹ Br	7511.57(8)	0.0108(9)	0.00041(3)	⁸⁵ Rb	113.76(4)	0.00535(14)	1.90(5)E-4
⁷⁹Br	7577.04(8)	0.108(3)	0.00410(11)	⁸⁵ Rb	119.94(4)	0.00267(9)	9.5(3)E-5
⁷⁹ Br	7610.73(8)	0.0093(8)	0.00035(3)	⁸⁷ Rb	166.01(3)	0.00215(8)	7.6(3)E-5
Krypton (Z=36), At.Wt.=83.80(1), $\sigma_{\gamma}^Z=25.8(12)$				⁸⁵ Rb	176.2(9)	0.0031(13)	1.1(5)E-4
⁸² Kr	9.4050(10)d	0.122(24)	0.0044[17%]	⁸⁷Rb	196.34(3)	0.00964(19)	0.000342(7)
⁸³ Kr	367.7(5)	0.532(10)	0.0192(4)	⁸⁵ Rb	198.96(10)	0.00266(9)	9.4(3)E-5
⁸³ Kr	419.4(5)	0.630(10)	0.0228(4)	⁸⁵ Rb	224.31(6)	0.00132(7)	4.68(25)E-5
⁸³Kr	425.30(11)	2.960(19)	0.1070(7)	⁸⁷ Rb	240.76(3)	0.00224(8)	7.9(3)E-5
⁸³ Kr	448.11(11)	0.590(19)	0.0213(7)	⁸⁵ Rb	283.80(8)	0.00092(6)	3.26(21)E-5
⁸³ Kr	541.50(12)	0.295(12)	0.0107(4)	⁸⁵ Rb	316.13(4)	0.00138(8)	4.9(3)E-5
⁸³ Kr	546.98(12)	0.328(12)	0.0119(4)	⁸⁵ Rb	322.80(4)	0.00254(10)	9.0(4)E-5
⁸³ Kr	605.5(4)	0.398(25)	0.0144(9)	⁸⁷ Rb	362.62(5)	0.00314(12)	1.11(4)E-4
⁸³ Kr	612.0(3)	0.42(3)	0.0152(11)	⁸⁵ Rb	362.78(9)	0.0061(22)	2.2(8)E-4
⁸³ Kr	637.13(18)	0.251(22)	0.0091(8)	⁸⁷ Rb	390.60(4)	0.00179(8)	6.3(3)E-5
⁸³ Kr	708.24(21)	0.220(21)	0.0080(8)	⁸⁵Rb	421.50(3)	0.0259(5)	0.000918(18)
⁸³ Kr	737.0(9)	0.31(6)	0.0112(22)	⁸⁵Rb	487.89(4)	0.0494(12)	0.00175(4)
⁸³ Kr	802.62(8)	1.520(22)	0.0550(8)	⁸⁵ Rb	514.57(4)	0.00653(20)	2.32(7)E-4
⁸³Kr	881.74(11)	20.8(3)	0.752(11)	⁸⁵ Rb	529.9(9)	0.0031(13)	1.1(5)E-4
⁸³ Kr	919.79(19)	0.222(17)	0.0080(6)	⁸⁵Rb	536.48(4)	0.0167(5)	0.000592(18)
⁸³ Kr	938.12(13)	0.449(21)	0.0162(8)	⁸⁵Rb	538.66(4)	0.0169(5)	0.000599(18)
⁸³ Kr	943.36(14)	0.713(8)	0.0258(3)	⁸⁵Rb	555.61(3)d	0.0407(10)	0.00144[98%]
⁸³ Kr	946.5(5)	0.447(19)	0.0162(7)	⁸⁵Rb	556.82(3)	0.0913(24)	0.00324(9)
⁸³ Kr	963.44(13)	0.660(22)	0.0239(8)	⁸⁵ Rb	565.37(4)	0.00383(10)	1.36(4)E-4
⁸³ Kr	987.69(19)	0.256(25)	0.0093(9)	⁸⁵Rb	638.93(5)	0.0101(13)	0.00036(5)
⁸³ Kr	1016.2(3)	1.08(7)	0.0391(25)	⁸⁵ Rb	640.20(10)	0.0032(7)	1.13(25)E-4
⁸³ Kr	1077.55(25)	0.47(3)	0.0170(11)	⁸⁵ Rb	668.76(7)	0.00211(10)	7.5(4)E-5
⁸³ Kr	1124.44(6)	1.420(21)	0.0514(8)	⁸⁵ Rb	691.57(5)	0.00725(18)	0.000257(6)
⁸³Kr	1213.42(12)	8.28(17)	0.299(6)	⁸⁵ Rb	726.98(5)	0.00421(15)	1.49(5)E-4
⁸³ Kr	1230.82(11)	0.310(12)	0.0112(4)	⁸⁵ Rb	747.67(4)	0.00268(12)	9.5(4)E-5
⁸³ Kr	1293.20(13)	0.383(25)	0.0139(9)	⁸⁵ Rb	816.59(6)	0.0031(9)	1.1(3)E-4
⁸³ Kr	1331.89(13)	0.39(6)	0.0141(22)	⁸⁷ Rb	834.79(6)	0.00197(13)	7.0(5)E-5
⁸³ Kr	1443.43(11)	0.237(10)	0.0086(4)	⁸⁵Rb	872.94(4)	0.0321(5)	0.001138(18)
⁸³Kr	1463.86(6)	7.10(8)	0.257(3)	⁸⁵ Rb	881.50(4)	0.00480(17)	1.70(6)E-4
⁸⁶ Kr	1475.94(17)	2.4(4)E-4	8.7(14)E-6	⁸⁵ Rb	913.12(6)	0.00497(15)	1.76(5)E-4

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁸⁵ Rb	944.49(9)	0.0035(13)	1.2(5)E-4	⁸⁷ Sr	1218.523(16)	0.0599(13)	0.00207(5)
⁸⁵ Rb	945.72(7)	0.00390(15)	1.38(5)E-4	⁸⁷ Sr	1323.92(6)	0.013(3)	0.00045(10)
⁸⁵Rb	1026.55(6)	0.0218(4)	0.000773(14)	⁸⁷ Sr	1368.677(25)	0.038(8)	0.0013(3)
⁸⁵Rb	1032.32(5)	0.0227(4)	0.000805(14)	⁸⁷ Sr	1382.44(4)	0.0239(8)	0.00083(3)
⁸⁵Rb	1076.64(20)d	0.0301(5)	0.001067[<0.1%]	⁸⁷ Sr	1407.89(5)	0.0104(20)	0.00036(7)
⁸⁵Rb	1105.52(10)	0.0151(3)	0.000535(11)	⁸⁷ Sr	1436.264(17)	0.0124(6)	0.000429(21)
⁸⁷ Rb	1141.49(15)	0.00113(11)	4.0(4)E-5	⁸⁷ Sr	1493.06(3)	0.0130(8)	0.00045(3)
⁸⁵ Rb	1178.86(10)	0.0044(13)	1.6(5)E-4	⁸⁷ Sr	1534.561(22)	0.0317(9)	0.00110(3)
⁸⁵ Rb	1219.80(9)	0.00446(21)	1.58(7)E-4	⁸⁷ Sr	1565.48(5)	0.0136(12)	0.00047(4)
⁸⁷ Rb	1245.20(6)	0.00253(12)	9.0(4)E-5	⁸⁷ Sr	1565.54(5)	0.027(4)	0.00093(14)
⁸⁵Rb	1304.48(4)	0.0204(5)	0.000723(18)	⁸⁷ Sr	1706.62(4)	0.0231(8)	0.00080(3)
⁸⁵ Rb	1389.32(7)	0.00809(21)	0.000287(7)	⁸⁷ Sr	1717.804(23)	0.0674(15)	0.00233(5)
⁸⁵ Rb	1438.31(4)	0.00200(15)	7.1(5)E-5	⁸⁷ Sr	1736.33(7)	0.0140(14)	0.00048(5)
⁸⁵ Rb	1666.74(9)	0.00774(23)	0.000274(8)	⁸⁷ Sr	1736.54(3)	0.018(3)	0.00062(10)
⁸⁵Rb	1890.7(4)	0.017(4)	0.00060(14)	⁸⁷ Sr	1799.06(3)	0.0356(11)	0.00123(4)
⁸⁵ Rb	2130.59(17)	0.0031(5)	1.10(18)E-4	⁸⁷Sr	1836.067(21)	1.030(18)	0.0356(6)
⁸⁵ Rb	2149.4(7)	0.00153(19)	5.4(7)E-5	⁸⁷ Sr	2111.36(3)	0.0279(10)	0.00096(4)
⁸⁵ Rb	2179.33(16)	0.00168(17)	6.0(6)E-5	⁸⁷ Sr	2202.92(3)	0.0341(10)	0.00118(4)
⁸⁵ Rb	2353.43(17)	0.00122(9)	4.3(3)E-5	⁸⁷ Sr	2276.52(3)	0.0431(13)	0.00149(5)
⁸⁷ Rb	2391.86(21)	0.00094(12)	3.3(4)E-5	⁸⁷ Sr	2391.09(3)	0.0471(15)	0.00163(5)
⁸⁵ Rb	2461.41(17)	0.00251(17)	8.9(6)E-5	⁸⁷ Sr	2463.52(4)	0.0131(6)	0.000453(21)
⁸⁵ Rb	2476.2(7)	0.0013(4)	4.6(14)E-5	⁸⁷ Sr	2577.85(4)	0.0246(9)	0.00085(3)
⁸⁵ Rb	2568.8(5)	0.0017(4)	6.0(14)E-5	⁸⁷ Sr	3009.39(3)	0.0575(15)	0.00199(5)
⁸⁵ Rb	2585.58(16)	0.00240(18)	8.5(6)E-5	⁸⁸ Sr	4078.39(5)	0.0055(9)	1.9(3)E-4
⁸⁷ Rb	3690.17(20)	0.00184(18)	6.5(6)E-5	⁸⁷ Sr	4604.81(6)	0.0169(7)	0.000585(24)
⁸⁷ Rb	4640.79(25)	0.00292(19)	1.04(7)E-4	⁸⁷ Sr	5161.37(5)	0.0138(6)	0.000477(21)
⁸⁷ Rb	5220.8(3)	0.00176(18)	6.2(6)E-5	⁸⁶ Sr	5361.652(25)	0.0104(6)	0.000360(21)
⁸⁷ Rb	5886.30(24)	0.00217(17)	7.7(6)E-5	⁸⁷ Sr	5423.43(8)	0.0146(7)	0.000505(24)
⁸⁵ Rb	6065.13(17)	0.0047(3)	1.67(11)E-4	⁸⁷ Sr	5684.81(4)	0.0131(9)	0.00045(3)
⁸⁵ Rb	6081.9(5)	0.00097(16)	3.4(6)E-5	⁸⁷ Sr	5791.07(4)	0.0196(9)	0.00068(3)
⁸⁷ Rb	6082.4(4)	0.00097(16)	3.4(6)E-5	⁸⁷ Sr	5999.31(5)	0.0109(6)	0.000377(21)
⁸⁵ Rb	6143.2(4)	0.00132(19)	4.7(7)E-5	⁸⁷ Sr	6101.72(4)	0.0477(17)	0.00165(6)
⁸⁵ Rb	6189.29(18)	0.0036(3)	1.28(11)E-4	⁸⁷ Sr	6266.87(4)	0.077(3)	0.00266(10)
⁸⁵ Rb	6319.4(8)	0.00107(18)	3.8(6)E-5	⁸⁷ Sr	6660.40(3)	0.0644(23)	0.00223(8)
⁸⁵ Rb	6351.44(17)	0.00173(16)	6.1(6)E-5	⁸⁷ Sr	6671.58(4)	0.0132(7)	0.000457(24)
⁸⁵ Rb	6385.11(25)	0.00148(19)	5.2(7)E-5	⁸⁷ Sr	6698.39(5)	0.0127(6)	0.000439(21)
⁸⁵ Rb	6471.37(17)	0.0049(3)	1.74(11)E-4	⁸⁷ Sr	6885.14(3)	0.0478(20)	0.00165(7)
⁸⁵ Rb	6501.3(7)	0.00165(19)	5.9(7)E-5	⁸⁷ Sr	6941.93(3)	0.0502(20)	0.00174(7)
⁸⁵ Rb	6520.11(18)	0.0064(4)	2.27(14)E-4	⁸⁷ Sr	7527.490(25)	0.0687(24)	0.00238(8)
⁸⁵ Rb	6831.64(10)	0.0064(4)	2.27(14)E-4	⁸⁶ Sr	8039.250(19)	0.0260(14)	0.00090(5)
⁸⁵ Rb	6942.98(13)	0.00161(15)	5.7(5)E-5	⁸⁷ Sr	8378.069(23)	0.0197(7)	0.000681(24)
⁸⁵ Rb	7212.34(10)	0.00129(17)	4.6(6)E-5	Yttrium (Z=39), At.Wt.=88.90585(2), $\sigma_{\gamma}^z = 1.280(20)$			
⁸⁵ Rb	7346.16(10)	0.0059(3)	2.09(11)E-4	⁸⁹ Y	176.923(22)	0.0129(7)	0.000440(24)
⁸⁵ Rb	7545.10(13)	0.00099(14)	3.5(5)E-5	⁸⁹ Y	202.53(3)	0.289(7)	0.00985(24)
⁸⁵Rb	7624.07(11)	0.0114(5)	0.000404(18)	⁸⁹ Y	202.53(3)d	0.0018(5)	6.1E-5[10%]
⁸⁵ Rb	8093.76(10)	0.00211(20)	7.5(7)E-5	⁸⁹ Y	574.106(20)	0.174(7)	0.00593(24)
⁸⁵ Rb	8650.52(10)	0.0022(4)	7.8(14)E-5	⁸⁹ Y	604.99(3)	0.0084(7)	0.000286(24)
Strontium (Z=38), At.Wt.=87.62(1), $\sigma_{\gamma}^z = 1.30(21)$				⁸⁹ Y	776.613(18)	0.659(9)	0.0225(3)
⁸⁴ Sr	231.68(4)	0.0017(3)	5.9(10)E-5	⁸⁹ Y	953.534(21)	0.0135(11)	0.00046(4)
⁸⁶ Sr	388.526(22)d	0.0785(23)	0.00272[11%]	⁸⁹ Y	1211.573(22)	0.0453(22)	0.00154(8)
⁸⁷ Sr	434.925(20)	0.0346(8)	0.00120(3)	⁸⁹ Y	1214.060(23)	0.0096(12)	0.00033(4)
⁸⁶ Sr	484.822(14)	0.0315(12)	0.00109(4)	⁸⁹ Y	1369.099(23)	0.0087(12)	0.00030(4)
⁸⁷ Sr	585.613(14)	0.0703(14)	0.00243(5)	⁸⁹ Y	1371.124(20)	0.0404(22)	0.00138(8)
⁸⁷ Sr	850.657(12)	0.275(4)	0.00951(14)	⁸⁹ Y	1416.566(22)	0.0173(13)	0.00059(4)
⁸⁷ Sr	898.055(11)	0.702(10)	0.0243(4)	⁸⁹ Y	1558.459(23)	0.0163(11)	0.00056(4)
⁸⁷ Sr	934.49(3)	0.024(4)	0.00083(14)	⁸⁹ Y	1571.604(22)	0.0148(11)	0.00050(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁸⁹ Y	1640.913(22)	0.0146(15)	0.00050(5)	⁹⁴ Zr	953.77(15)	0.0030(5)	9.97(17)E-5
⁸⁹ Y	1760.964(23)	0.0086(10)	0.00029(3)	⁹¹ Zr	972.332(10)	0.0025(17)	8(6)E-5
⁸⁹ Y	1780.70(6)	0.0082(18)	0.00028(6)	⁹¹ Zr	990.540(7)	0.0029(5)	9.6(17)E-5
⁸⁹ Y	1815.15(3)	0.0223(15)	0.00076(5)	⁹⁴ Zr	1030.83(24)	0.0013(4)	4.3(13)E-5
⁸⁹ Y	2139.11(4)	0.0101(12)	0.00034(4)	⁹⁴ Zr	1054.75(16)	0.0037(5)	1.23(17)E-4
⁸⁹ Y	2196.10(3)	0.0107(10)	0.00036(3)	⁹⁰ Zr	1067.5(7)	0.0017(8)	6(3)E-5
⁸⁹ Y	2273.38(4)	0.0121(24)	0.00041(8)	⁹⁶Zr	1102.67(6)	0.0235(8)	0.00078(3)
⁸⁹ Y	2327.31(5)	0.0108(18)	0.00037(6)	⁹¹ Zr	1132.126(4)	0.0100(7)	0.000332(23)
⁸⁹ Y	2405.36(4)	0.0095(18)	0.00032(6)	⁹⁴ Zr	1198.25(19)	0.0042(5)	1.40(17)E-4
⁸⁹ Y	2504.60(4)	0.0139(17)	0.00047(6)	⁹⁰Zr	1205.6(7)	0.042(5)	0.00140(17)
⁸⁹ Y	2546.68(3)	0.0219(17)	0.00075(6)	⁹¹ Zr	1222.44(4)	0.0018(4)	6.0(13)E-5
⁸⁹ Y	2589.56(5)	0.0137(15)	0.00047(5)	⁹¹ Zr	1248.100(12)	0.0038(4)	1.26(13)E-4
⁸⁹ Y	2749.181(24)	0.0246(19)	0.00084(7)	⁹⁴ Zr	1300.1(5)	0.0015(5)	5.0(17)E-5
⁸⁹ Y	2756.47(5)	0.0103(12)	0.00035(4)	⁹⁴ Zr	1323.20(25)	0.0025(5)	8.3(17)E-5
⁸⁹ Y	2819.38(5)	0.0096(9)	0.00033(3)	⁹¹Zr	1405.159(3)	0.0301(10)	0.00100(3)
⁸⁹ Y	2847.23(7)	0.0096(9)	0.00033(3)	⁹² Zr	1425.2(4)	0.00287(20)	9.5(7)E-5
⁸⁹ Y	2922.48(3)	0.0090(9)	0.00031(3)	⁹¹ Zr	1463.814(8)	0.0017(7)	5.6(23)E-5
⁸⁹ Y	3160.17(4)	0.0109(6)	0.000372(20)	⁹⁰Zr	1465.7(7)	0.063(15)	0.0021(5)
⁸⁹ Y	3164.64(5)	0.0120(6)	0.000409(20)	⁹² Zr	1650.1(5)	0.0029(12)	1.0(4)E-4
⁸⁹ Y	3229.29(3)	0.0116(6)	0.000395(20)	⁹¹ Zr	1847.220(7)	0.0084(8)	0.00028(3)
⁸⁹ Y	3254.87(4)	0.0119(6)	0.000406(20)	⁹⁰Zr	1880.4(4)	0.016(4)	0.00053(13)
⁸⁹ Y	3282.41(4)	0.0192(10)	0.00065(3)	⁹⁴ Zr	1892.9(4)	0.0034(7)	1.13(23)E-4
⁸⁹ Y	3301.23(3)	0.0276(18)	0.00094(6)	⁹² Zr	1917.2(9)	0.0017(8)	6(3)E-5
⁸⁹ Y	3380.87(4)	0.0159(8)	0.00054(3)	⁹¹ Zr	1956.66(4)	0.0035(5)	1.16(17)E-4
⁸⁹ Y	3544.52(4)	0.0163(10)	0.00056(3)	⁹¹ Zr	1974.91(4)	0.0024(5)	8.0(17)E-5
⁸⁹ Y	3696.70(4)	0.0138(8)	0.00047(3)	⁹¹ Zr	1988.71(3)	0.0049(5)	1.63(17)E-4
⁸⁹ Y	3713.08(4)	0.0078(4)	0.000266(14)	⁹⁰Zr	2042.2(4)	0.032(8)	0.0011(3)
⁸⁹ Y	3870.79(5)	0.0089(5)	0.000303(17)	⁹¹ Zr	2105.16(5)	0.0025(5)	8.3(17)E-5
⁸⁹ Y	4009.64(7)	0.0089(6)	0.000303(20)	⁹¹ Zr	2132.84(3)	0.0014(3)	4.7(10)E-5
⁸⁹ Y	4098.82(3)	0.0108(6)	0.000368(20)	⁹² Zr	2190.2(5)	0.0044(5)	1.46(17)E-4
⁸⁹ Y	4107.68(3)	0.067(12)	0.0023(4)	⁹¹ Zr	2328.10(4)	0.0019(8)	6(3)E-5
⁸⁹ Y	4352.26(4)	0.0207(16)	0.00071(6)	⁹¹ Zr	2436.92(3)	0.0015(7)	5.0(23)E-5
⁸⁹ Y	4380.97(4)	0.0085(5)	0.000290(17)	⁹⁰Zr	2533.2(5)	0.0037(14)	1.2(5)E-4
⁸⁹ Y	4490.91(3)	0.0093(6)	0.000317(20)	⁹¹ Zr	2537.17(19)	0.0014(5)	4.7(17)E-5
⁸⁹ Y	4660.75(3)	0.0088(5)	0.000300(17)	⁹⁰Zr	2557.8(8)	0.016(4)	0.00053(13)
⁸⁹ Y	5645.236(25)	0.029(3)	0.00099(10)	⁹⁰Zr	2577.3(14)	0.016(4)	0.00053(13)
⁸⁹Y	6080.171(22)	0.76(4)	0.0259(14)	⁹⁰ Zr	2640.1(8)	0.0105(25)	0.00035(8)
Zirconium (Z=40, At.Wt.=91.224(2), $\sigma_{\gamma}^Z = 0.19(3)$)				⁹¹ Zr	2693.79(3)	0.006(3)	2.0(10)E-4
⁹⁴ Zr	101.17(9)	0.0026(3)	8.6(10)E-5	⁹¹ Zr	2705.74(9)	0.0019(8)	6(3)E-5
⁹⁶ Zr	160.94(10)	0.0111(7)	0.000369(23)	⁹⁰ Zr	3082.6(12)	0.0096(25)	0.00032(8)
⁹² Zr	266.78(16)	0.0091(5)	0.000302(17)	⁹¹ Zr	3371.36(3)	0.0020(5)	6.6(17)E-5
⁹¹ Zr	273.036(5)	0.0029(4)	9.6(13)E-5	⁹² Zr	3459.4(15)	0.00137(17)	4.6(6)E-5
⁹¹ Zr	403.898(13)	0.00137(25)	4.6(8)E-5	⁹⁰Zr	3475.8(15)	0.019(5)	0.00063(17)
⁹¹ Zr	448.217(5)	0.0067(3)	2.23(10)E-4	⁹¹ Zr	3830.13(8)	0.0017(5)	5.6(17)E-5
⁹¹ Zr	492.398(8)	0.0027(3)	9.0(10)E-5	⁹⁰Zr	3982.3(15)	0.015(4)	0.00050(13)
⁹¹Zr	560.958(3)	0.0285(5)	0.000947(17)	⁹⁴ Zr	4104.3(3)	0.0029(5)	9.6(17)E-5
⁹⁴ Zr	569.5(3)	0.0013(3)	4.3(10)E-5	⁹² Zr	4278.1(7)	0.00147(10)	4.9(3)E-5
⁹¹ Zr	571.171(5)	0.0022(3)	7.3(10)E-5	⁹¹ Zr	4994.61(18)	0.0027(5)	9.0(17)E-5
⁹⁰ Zr	652.8(4)	0.0029(14)	1.0(5)E-4	⁹¹ Zr	5006.56(16)	0.0049(7)	1.63(23)E-4
⁹⁶ Zr	743.36(3)d	0.00101(6)	3.36E-5[2.0%]	⁹⁰ Zr	5150.3(9)	0.0017(12)	6(4)E-5
⁹¹ Zr	844.206(4)	0.0095(4)	0.000316(13)	⁹¹ Zr	5182.73(17)	0.0019(4)	6.3(13)E-5
⁹¹ Zr	902.861(8)	0.0047(5)	1.56(17)E-4	⁹¹ Zr	5263.42(17)	0.0064(8)	2.1(3)E-4
⁹¹ Zr	912.766(7)	0.0117(5)	0.000389(17)	⁹² Zr	5309.9(7)	0.0024(4)	8.0(13)E-5
⁹¹Zr	934.4640(10)	0.125(5)	0.00415(17)	⁹¹ Zr	5372.23(17)	0.0016(4)	5.3(13)E-5
⁹⁴ Zr	939.11(10)	0.0017(5)	5.6(17)E-5	⁹⁶ Zr	5574.9(4)	0.0023(4)	7.6(13)E-5
⁹² Zr	946.6(5)	0.0020(5)	6.6(17)E-5	⁹¹Zr	6295.13(16)	0.0279(20)	0.00093(7)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁹⁴ Zr	6357.8(4)	0.0026(4)	8.6(13)E-5	⁹³ Nb	801.91(18)	0.0020(4)	6.5(13)E-5
Niobium (Z=41), At. Wt.=92.90638(2), $\sigma_{\gamma}^Z = 1.15(5)$							
⁹³ Nb	17.810(7)	0.0579(14)	0.00189(5)	⁹³ Nb	812.64(7)	0.0084(5)	0.000274(16)
⁹³ Nb	54.704(7)	0.0058(7)	1.89(23)E-4	⁹³ Nb	835.72(3)	0.0376(8)	0.00123(3)
⁹³ Nb	78.6680(10)	0.0169(3)	0.000551(10)	⁹³ Nb	850.93(5)	0.0025(5)	8.2(16)E-5
⁹³ Nb	99.4070(10)	0.196(9)	0.0064(3)	⁹³ Nb	853.98(3)	0.0028(5)	9.1(16)E-5
⁹³ Nb	113.4010(10)	0.117(3)	0.00382(10)	⁹³ Nb	871.06d	0.00390(8)	1.27E-4[85%]
⁹³ Nb	135.47(6)	0.0029(9)	9(3)E-5	⁹³ Nb	876.64(11)	0.0077(5)	0.000251(16)
⁹³ Nb	136.21(12)	0.0027(7)	8.8(23)E-5	⁹³ Nb	878.61(5)	0.0191(17)	0.00062(6)
⁹³ Nb	138.614(8)	0.0089(19)	0.00029(6)	⁹³ Nb	883.42(5)	0.0192(7)	0.000626(23)
⁹³ Nb	140.10(3)	0.00226(21)	7.4(7)E-5	⁹³ Nb	894.45(11)	0.0185(7)	0.000603(23)
⁹³ Nb	150.711(22)	0.00201(21)	6.6(7)E-5	⁹³ Nb	898.58(5)	0.0144(7)	0.000470(23)
⁹³ Nb	161.2610(20)	0.0190(5)	0.000620(16)	⁹³ Nb	911.476(15)	0.0176(7)	0.000574(23)
⁹³ Nb	193.96(13)	0.0022(4)	7.2(13)E-5	⁹³ Nb	932.65(3)	0.0020(4)	6.5(13)E-5
⁹³ Nb	253.115(5)	0.1320(19)	0.00431(6)	⁹³ Nb	944.61(4)	0.0056(4)	1.83(13)E-4
⁹³ Nb	255.9290(20)	0.176(3)	0.00574(10)	⁹³ Nb	957.28(5)	0.0248(7)	0.000809(23)
⁹³ Nb	270.45(4)	0.0046(3)	1.50(10)E-4	⁹³ Nb	976.71(4)	0.0021(5)	6.8(16)E-5
⁹³ Nb	293.206(4)	0.0651(16)	0.00212(5)	⁹³ Nb	1001.82(11)	0.0037(5)	1.21(16)E-4
⁹³ Nb	309.915(8)	0.0690(17)	0.00225(6)	⁹³ Nb	1100.05(5)	0.0067(6)	2.19(20)E-4
⁹³ Nb	319.703(14)	0.00320(23)	1.04(8)E-4	⁹³ Nb	1106.86(5)	0.0076(7)	2.48(23)E-4
⁹³ Nb	329.178(12)	0.0108(4)	0.000352(13)	⁹³ Nb	1117.85(5)	0.0080(11)	0.00026(4)
⁹³ Nb	329.185(10)	0.0080(9)	0.00026(3)	⁹³ Nb	1118.54(3)	0.022(7)	0.00072(23)
⁹³ Nb	337.527(7)	0.054(6)	0.00176(20)	⁹³ Nb	1120.54(7)	0.0062(8)	2.0(3)E-4
⁹³ Nb	338.661(19)	0.0080(19)	0.00026(6)	⁹³ Nb	1122.55(7)	0.0106(13)	0.00035(4)
⁹³ Nb	355.3360(20)	0.0056(3)	1.83(10)E-4	⁹³ Nb	1128.97(6)	0.0175(15)	0.00057(5)
⁹³ Nb	450.98(9)	0.00238(20)	7.8(7)E-5	⁹³ Nb	1151.47(7)	0.0071(6)	2.32(20)E-4
⁹³ Nb	454.60(5)	0.00328(22)	1.07(7)E-4	⁹³ Nb	1159.61(10)	0.0066(6)	2.15(20)E-4
⁹³ Nb	456.20(10)	0.0058(7)	1.89(23)E-4	⁹³ Nb	1188.45(5)	0.0074(6)	2.41(20)E-4
⁹³ Nb	458.467(10)	0.0240(5)	0.000783(16)	⁹³ Nb	1191.06(3)	0.0137(7)	0.000447(23)
⁹³ Nb	482.72(3)	0.0032(5)	1.04(16)E-4	⁹³ Nb	1206.26(5)	0.0284(10)	0.00093(3)
⁹³ Nb	484.14(5)	0.0073(6)	2.38(20)E-4	⁹³ Nb	1214.31(10)	0.0073(7)	2.38(23)E-4
⁹³ Nb	499.426(8)	0.0648(18)	0.00211(6)	⁹³ Nb	1216.09(9)	0.0021(5)	6.8(16)E-5
⁹³ Nb	518.113(12)	0.0579(13)	0.00189(4)	⁹³ Nb	1219.01(7)	0.0050(6)	1.63(20)E-4
⁹³ Nb	525.81(3)	0.0074(6)	2.41(20)E-4	⁹³ Nb	1222.41(9)	0.0121(7)	0.000395(23)
⁹³ Nb	527.595(9)	0.0127(7)	0.000414(23)	⁹³ Nb	1227.8(4)	0.0114(7)	0.000372(23)
⁹³ Nb	547.73(7)	0.0045(4)	1.47(13)E-4	⁹³ Nb	1230.13(7)	0.0051(7)	1.66(23)E-4
⁹³ Nb	562.328(9)	0.0293(11)	0.00096(4)	⁹³ Nb	1240.22(9)	0.0096(7)	0.000313(23)
⁹³ Nb	573.07(4)	0.0020(3)	6.5(10)E-5	⁹³ Nb	1256.97(9)	0.0059(8)	1.9(3)E-4
⁹³ Nb	583.837(11)	0.0022(3)	7.2(10)E-5	⁹³ Nb	1258.90(8)	0.0039(8)	1.3(3)E-4
⁹³ Nb	590.627(14)	0.0086(5)	0.000281(16)	⁹³ Nb	1264.5(7)	0.0021(5)	6.8(16)E-5
⁹³ Nb	600.43(3)	0.0035(5)	1.14(16)E-4	⁹³ Nb	1273.72(7)	0.0052(12)	1.7(4)E-4
⁹³ Nb	635.80(5)	0.0059(5)	1.92(16)E-4	⁹³ Nb	1291.52(7)	0.0097(7)	0.000316(23)
⁹³ Nb	636.081(16)	0.0043(5)	1.40(16)E-4	⁹³ Nb	1308.1(4)	0.0068(13)	2.2(4)E-4
⁹³ Nb	640.995(9)	0.0048(5)	1.57(16)E-4	⁹³ Nb	1361.66(19)	0.0043(5)	1.40(16)E-4
⁹³ Nb	642.62(4)	0.0069(5)	2.25(16)E-4	⁹³ Nb	1392.73(7)	0.0105(8)	0.00034(3)
⁹³ Nb	645.40(5)	0.0022(7)	7.2(23)E-5	⁹³ Nb	1394.0(4)	0.0058(13)	1.9(4)E-4
⁹³ Nb	672.30(5)	0.0023(4)	7.5(13)E-5	⁹³ Nb	1419.39(11)	0.0048(6)	1.57(20)E-4
⁹³ Nb	689.79(5)	0.0164(6)	0.000535(20)	⁹³ Nb	1440.05(9)	0.0068(15)	2.2(5)E-4
⁹³ Nb	693.74(4)	0.0085(4)	0.000277(13)	⁹³ Nb	1442.0(4)	0.0061(6)	1.99(20)E-4
⁹³ Nb	711.47(4)	0.0024(3)	7.8(10)E-5	⁹³ Nb	1459.6(7)	0.0095(6)	0.000310(20)
⁹³ Nb	748.71(11)	0.0028(4)	9.1(13)E-5	⁹³ Nb	1460.02(9)	0.0097(22)	0.00032(7)
⁹³ Nb	751.671(11)	0.0143(6)	0.000466(20)	⁹³ Nb	1478.58(14)	0.0029(6)	9.5(20)E-5
⁹³ Nb	755.354(8)	0.0123(6)	0.000401(20)	⁹³ Nb	1481.19(13)	0.0039(8)	1.3(3)E-4
⁹³ Nb	775.93(3)	0.0158(6)	0.000515(20)	⁹³ Nb	1487.9(4)	0.0039(8)	1.3(3)E-4
⁹³ Nb	782.247(11)	0.0042(6)	1.37(20)E-4	⁹³ Nb	1492.55(24)	0.0022(5)	7.2(16)E-5
⁹³ Nb	783.02(7)	0.0065(5)	2.12(16)E-4	⁹³ Nb	1614.72(8)	0.0028(5)	9.1(16)E-5

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁹³ Nb	1678.05(17)	0.0033(5)	1.08(16)E-4	⁹³ Nb	3931.73(12)	0.0024(3)	7.8(10)E-5
⁹³ Nb	1716.16(8)	0.0034(5)	1.11(16)E-4	⁹³ Nb	3936.72(12)	0.0033(7)	1.08(23)E-4
⁹³ Nb	1763.20(10)	0.0034(5)	1.11(16)E-4	⁹³ Nb	3972.03(12)	0.0030(4)	9.8(13)E-5
⁹³ Nb	1863.63(8)	0.0028(6)	9.1(20)E-5	⁹³ Nb	3978.62(12)	0.0024(3)	7.8(10)E-5
⁹³ Nb	1878.88(8)	0.0081(7)	0.000264(23)	⁹³ Nb	4000.22(12)	0.0033(4)	1.08(13)E-4
⁹³ Nb	1881.96(10)	0.0036(7)	1.17(23)E-4	⁹³ Nb	4010.72(12)	0.0033(4)	1.08(13)E-4
⁹³ Nb	1919.51(8)	0.0024(4)	7.8(13)E-5	⁹³ Nb	4015.91(12)	0.0055(7)	1.79(23)E-4
⁹³ Nb	1974.93(9)	0.0052(6)	1.70(20)E-4	⁹³ Nb	4090.53(12)	0.0021(4)	6.8(13)E-5
⁹³ Nb	2001.4(3)	0.0025(6)	8.2(20)E-5	⁹³ Nb	4109.13(12)	0.0027(3)	8.8(10)E-5
⁹³ Nb	2019.49(9)	0.0021(5)	6.8(16)E-5	⁹³ Nb	4115.32(12)	0.0026(3)	8.5(10)E-5
⁹³ Nb	2285.80(21)	0.0026(5)	8.5(16)E-5	⁹³ Nb	4130.33(12)	0.0063(7)	2.05(23)E-4
⁹³ Nb	2313.81(9)	0.0046(8)	1.5(3)E-4	⁹³ Nb	4143.52(12)	0.0021(3)	6.8(10)E-5
⁹³ Nb	2319.95(12)	0.0022(9)	7(3)E-5	⁹³ Nb	4153.82(12)	0.0028(6)	9.1(20)E-5
⁹³ Nb	2896.68(12)	0.0025(5)	8.2(16)E-5	⁹³ Nb	4191.06(12)	0.00196(21)	6.4(7)E-5
⁹³ Nb	2922.70(12)	0.0021(6)	6.8(20)E-5	⁹³ Nb	4196.68(11)	0.0027(6)	8.8(20)E-5
⁹³ Nb	3194.65(19)	0.0021(5)	6.8(16)E-5	⁹³ Nb	4208.36(11)	0.0029(6)	9.5(20)E-5
⁹³ Nb	3241.04(12)	0.0026(3)	8.5(10)E-5	⁹³ Nb	4237.17(13)	0.0020(5)	6.5(16)E-5
⁹³ Nb	3260.34(12)	0.0041(5)	1.34(16)E-4	⁹³ Nb	4260.84(12)	0.0036(6)	1.17(20)E-4
⁹³ Nb	3266.45(12)	0.0042(5)	1.37(16)E-4	⁹³ Nb	4304.78(12)	0.0049(8)	1.6(3)E-4
⁹³ Nb	3267.12(20)	0.0021(6)	6.8(20)E-5	⁹³ Nb	4314.26(12)	0.0022(6)	7.2(20)E-5
⁹³ Nb	3319.93(12)	0.0028(6)	9.1(20)E-5	⁹³ Nb	4327.32(11)	0.0027(3)	8.8(10)E-5
⁹³ Nb	3343.94(12)	0.0023(6)	7.5(20)E-5	⁹³ Nb	4330.80(12)	0.0043(7)	1.40(23)E-4
⁹³ Nb	3353.64(12)	0.0028(6)	9.1(20)E-5	⁹³ Nb	4347.62(11)	0.0027(7)	8.8(23)E-5
⁹³ Nb	3361.64(12)	0.0027(3)	8.8(10)E-5	⁹³ Nb	4384.27(11)	0.0029(3)	9.5(10)E-5
⁹³ Nb	3367.05(12)	0.0020(6)	6.5(20)E-5	⁹³ Nb	4389.04(11)	0.00196(21)	6.4(7)E-5
⁹³ Nb	3383.54(12)	0.0022(6)	7.2(20)E-5	⁹³ Nb	4395.07(9)	0.0044(12)	1.4(4)E-4
⁹³ Nb	3388.53(12)	0.0034(6)	1.11(20)E-4	⁹³ Nb	4431.97(9)	0.0043(9)	1.4(3)E-4
⁹³ Nb	3428.34(12)	0.0020(3)	6.5(10)E-5	⁹³ Nb	4455.30(10)	0.0027(3)	8.8(10)E-5
⁹³ Nb	3430.66(20)	0.0031(6)	1.01(20)E-4	⁹³ Nb	4459.03(11)	0.0030(6)	9.8(20)E-5
⁹³ Nb	3431.74(12)	0.0030(4)	9.8(13)E-5	⁹³ Nb	4466.50(10)	0.0028(3)	9.1(10)E-5
⁹³ Nb	3458.34(12)	0.0030(6)	9.8(20)E-5	⁹³ Nb	4470.69(11)	0.0033(7)	1.08(23)E-4
⁹³ Nb	3465.55(14)	0.0025(3)	8.2(10)E-5	⁹³ Nb	4501.43(10)	0.0056(7)	1.83(23)E-4
⁹³ Nb	3502.64(12)	0.0022(3)	7.2(10)E-5	⁹³ Nb	4505.78(10)	0.0029(3)	9.5(10)E-5
⁹³ Nb	3508.04(12)	0.0041(5)	1.34(16)E-4	⁹³ Nb	4524.10(9)	0.0038(6)	1.24(20)E-4
⁹³ Nb	3538.94(12)	0.00198(22)	6.5(7)E-5	⁹³ Nb	4538.64(9)	0.0058(7)	1.89(23)E-4
⁹³ Nb	3543.43(12)	0.0021(6)	6.8(20)E-5	⁹³ Nb	4553.99(10)	0.0033(4)	1.08(13)E-4
⁹³ Nb	3561.54(12)	0.0027(3)	8.8(10)E-5	⁹³ Nb	4558.53(11)	0.0049(7)	1.60(23)E-4
⁹³ Nb	3634.02(12)	0.0027(5)	8.8(16)E-5	⁹³ Nb	4594.44(9)	0.0047(7)	1.53(23)E-4
⁹³ Nb	3646.03(12)	0.0022(3)	7.2(10)E-5	⁹³ Nb	4606.89(13)	0.0046(6)	1.50(20)E-4
⁹³ Nb	3651.22(12)	0.0023(5)	7.5(16)E-5	⁹³ Nb	4629.91(9)	0.0049(7)	1.60(23)E-4
⁹³ Nb	3658.53(12)	0.0023(3)	7.5(10)E-5	⁹³ Nb	4635.44(9)	0.0047(6)	1.53(20)E-4
⁹³ Nb	3676.62(12)	0.0028(6)	9.1(20)E-5	⁹³ Nb	4662.32(9)	0.0028(6)	9.1(20)E-5
⁹³ Nb	3680.54(12)	0.0028(3)	9.1(10)E-5	⁹³ Nb	4672.16(9)	0.0065(7)	2.12(23)E-4
⁹³ Nb	3720.63(12)	0.0033(6)	1.08(20)E-4	⁹³ Nb	4681.99(9)	0.0059(7)	1.92(23)E-4
⁹³ Nb	3740.94(12)	0.0021(3)	6.8(10)E-5	⁹³ Nb	4711.67(10)	0.0052(7)	1.70(23)E-4
⁹³ Nb	3745.55(14)	0.0033(4)	1.08(13)E-4	⁹³ Nb	4739.00(8)	0.0153(9)	0.00050(3)
⁹³ Nb	3760.94(12)	0.00200(22)	6.5(7)E-5	⁹³ Nb	4749.12(9)	0.0038(6)	1.24(20)E-4
⁹³ Nb	3773.94(12)	0.0045(5)	1.47(16)E-4	⁹³ Nb	4756.28(9)	0.0039(6)	1.27(20)E-4
⁹³ Nb	3837.12(12)	0.0020(5)	6.5(16)E-5	⁹³ Nb	4772.35(8)	0.0045(7)	1.47(23)E-4
⁹³ Nb	3867.53(12)	0.0026(3)	8.5(10)E-5	⁹³ Nb	4791.62(13)	0.0071(7)	2.32(23)E-4
⁹³ Nb	3879.13(12)	0.0048(6)	1.57(20)E-4	⁹³ Nb	4828.2(4)	0.0057(6)	1.86(20)E-4
⁹³ Nb	3888.74(12)	0.0051(6)	1.66(20)E-4	⁹³ Nb	4913.65(9)	0.0078(7)	0.000254(23)
⁹³ Nb	3892.83(12)	0.0039(5)	1.27(16)E-4	⁹³ Nb	4927.94(8)	0.0027(6)	8.8(20)E-5
⁹³ Nb	3907.03(12)	0.00207(23)	6.8(8)E-5	⁹³ Nb	4942.7(4)	0.0029(3)	9.5(10)E-5
⁹³ Nb	3912.73(12)	0.0022(3)	7.2(10)E-5	⁹³ Nb	4949.70(10)	0.0051(7)	1.66(23)E-4
⁹³ Nb	3919.65(12)	0.0038(7)	1.24(23)E-4	⁹³ Nb	4982.53(9)	0.0078(7)	0.000254(23)
⁹³ Nb	3927.83(12)	0.0026(3)	8.5(10)E-5	⁹³ Nb	4997.97(8)	0.0033(6)	1.08(20)E-4

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
⁹³ Nb	5032.08(8)	0.0058(7)	1.89(23)E-4	⁹⁶ Mo	480.97(13)	0.0604(23)	0.00191(7)
⁹³ Nb	5052.89(9)	0.0022(5)	7.2(16)E-5	⁹⁵ Mo	568.88(3)	0.0280(11)	0.00088(4)
⁹³ Nb	5065.65(8)	0.0034(6)	1.11(20)E-4	⁹⁵ Mo	591.21(3)	0.0315(14)	0.00100(4)
⁹³ Nb	5070.27(7)	0.0102(8)	0.00033(3)	⁹⁵ Mo	608.744(14)	0.121(4)	0.00382(13)
⁹³ Nb	5087.36(8)	0.0030(5)	9.8(16)E-5	⁹⁵Mo	719.528(14)	0.310(10)	0.0098(3)
⁹³Nb	5103.34(7)	0.0232(12)	0.00076(4)	⁹⁵ Mo	721.54(4)	0.025(3)	0.00079(10)
⁹³ Nb	5129.16(8)	0.0034(5)	1.11(16)E-4	⁹⁷ Mo	723.338(19)	0.051(11)	0.0016(4)
⁹³ Nb	5179.99(7)	0.0072(7)	2.35(23)E-4	⁹⁵ Mo	736.820(14)	0.119(4)	0.00376(13)
⁹³ Nb	5193.62(18)	0.0114(8)	0.00037(3)	⁹⁵Mo	778.221(10)	2.02(6)	0.0638(19)
⁹³ Nb	5207.96(9)	0.0072(7)	2.35(23)E-4	⁹⁷ Mo	787.39(3)	0.168(6)	0.00531(19)
⁹³ Nb	5213.75(9)	0.00196(21)	6.4(7)E-5	⁹⁵ Mo	812.26(5)	0.0264(15)	0.00083(5)
⁹³ Nb	5252.52(9)	0.0080(8)	0.00026(3)	⁹⁵Mo	847.603(11)	0.324(9)	0.0102(3)
⁹³ Nb	5257.70(9)	0.00214(23)	7.0(8)E-5	⁹⁵Mo	849.85(3)	0.43(3)	0.0136(10)
⁹³ Nb	5284.14(8)	0.0050(7)	1.63(23)E-4	⁹⁵ Mo	852.93(3)	0.0444(17)	0.00140(5)
⁹³ Nb	5290.46(8)	0.0022(3)	7.2(10)E-5	⁹² Mo	943.6(3)	0.0075(9)	2.4(3)E-4
⁹³ Nb	5301.22(8)	0.0031(6)	1.01(20)E-4	⁹⁵ Mo	968.46(5)	0.0323(19)	0.00102(6)
⁹³ Nb	5307.94(8)	0.0063(7)	2.05(23)E-4	⁹⁵ Mo	1091.289(20)	0.201(6)	0.00635(19)
⁹³ Nb	5348.57(8)	0.0082(7)	0.000267(23)	⁹⁵ Mo	1106.36(4)	0.0309(18)	0.00098(6)
⁹³ Nb	5363.82(8)	0.0073(7)	2.38(23)E-4	⁹⁵ Mo	1190.28(6)	0.0240(14)	0.00076(4)
⁹³ Nb	5368.1(4)	0.0039(6)	1.27(20)E-4	⁹⁵ Mo	1200.10(3)	0.124(4)	0.00392(13)
⁹³ Nb	5399.86(7)	0.0050(7)	1.63(23)E-4	⁹⁷ Mo	1230.13(5)	0.0253(15)	0.00080(5)
⁹³ Nb	5447.70(7)	0.0026(3)	8.5(10)E-5	⁹⁵ Mo	1317.35(8)	0.091(6)	0.00287(19)
⁹³ Nb	5450.96(7)	0.0053(7)	1.73(23)E-4	⁹⁵ Mo	1497.742(17)	0.122(4)	0.00385(13)
⁹³Nb	5496.24(10)	0.0205(14)	0.00067(5)	⁹⁵ Mo	1625.817(15)	0.0264(15)	0.00083(5)
⁹³ Nb	5507.79(7)	0.0041(5)	1.34(16)E-4	⁹⁵ Mo	1702.78(4)	0.0220(15)	0.00069(5)
⁹³ Nb	5511.28(8)	0.0053(7)	1.73(23)E-4	⁹⁵ Mo	1846.26(15)	0.022(3)	0.00069(10)
⁹³ Nb	5532.16(8)	0.0027(5)	8.8(16)E-5	⁹⁵ Mo	1923.47(13)	0.0250(18)	0.00079(6)
⁹³ Nb	5572.33(8)	0.0037(5)	1.21(16)E-4	⁹⁵ Mo	2011.87(5)	0.0226(16)	0.00071(5)
⁹³ Nb	5591.31(6)	0.0080(7)	0.000261(23)	⁹⁵ Mo	2663.47(9)	0.0455(21)	0.00144(7)
⁹³ Nb	5607.32(8)	0.0041(5)	1.34(16)E-4	⁹⁵ Mo	5602.15(15)	0.0242(17)	0.00076(5)
⁹³ Nb	5612.72(8)	0.0037(5)	1.21(16)E-4	⁹⁵ Mo	5711.98(12)	0.048(4)	0.00152(13)
⁹³ Nb	5645.93(7)	0.0026(4)	8.5(13)E-5	⁹⁵ Mo	6363.55(10)	0.0235(17)	0.00074(5)
⁹³ Nb	5769.77(7)	0.0054(6)	1.76(20)E-4	⁹⁷ Mo	6624.801(20)	0.027(10)	0.0009(3)
⁹³ Nb	5880.80(9)	0.0035(4)	1.14(13)E-4	⁹⁵ Mo	6919.05(9)	0.106(6)	0.00335(19)
⁹³ Nb	5895.01(7)	0.0183(8)	0.00060(3)	⁹⁵ Mo	7527.75(9)	0.0264(20)	0.00083(6)
⁹³ Nb	5946.31(9)	0.0045(6)	1.47(20)E-4	Ruthenium (Z=44), At.Wt.=101.07(2), $\sigma_{\gamma}^Z = 2.75(21)$			
⁹³ Nb	5954.41(10)	0.0025(3)	8.2(10)E-5	¹⁰⁴ Ru	75.251(25)	0.0233(22)	0.00070(7)
⁹³ Nb	5964.58(7)	0.0055(6)	1.79(20)E-4	⁹⁸ Ru	89.69(10)	0.0036(7)	1.08(21)E-4
⁹³ Nb	5980.27(5)	0.0029(5)	9.5(16)E-5	¹⁰⁴ Ru	107.917(14)	0.0153(14)	0.00046(4)
⁹³ Nb	5995.47(3)	0.0033(5)	1.08(16)E-4	¹⁰⁰ Ru	127.18(8)	0.049(4)	0.00147(12)
⁹³ Nb	6068.67(5)	0.0026(4)	8.5(13)E-5	¹⁰² Ru	136.05(4)	0.066(6)	0.00198(18)
⁹³ Nb	6292.06(11)	0.0033(4)	1.08(13)E-4	¹⁰⁴ Ru	143.206(9)	0.0206(20)	0.00062(6)
⁹³ Nb	6331.751(16)	0.0029(4)	9.5(13)E-5	¹⁰⁴ Ru	159.303(16)	0.0179(20)	0.00054(6)
⁹³ Nb	6434.833(18)	0.0047(4)	1.53(13)E-4	¹⁰² Ru	174.27(3)	0.076(7)	0.00228(21)
⁹³ Nb	6595.867(18)	0.0020(3)	6.5(10)E-5	⁹⁶ Ru	189.24(4)	0.0099(11)	0.00030(3)
⁹³ Nb	6831.141(14)	0.0175(8)	0.00057(3)	¹⁰² Ru	250.78(6)	0.0238(23)	0.00071(7)
⁹³ Nb	6915.546(15)	0.0024(3)	7.8(10)E-5	¹⁰² Ru	270.58(8)	0.034(3)	0.00102(9)
⁹³ Nb	7186.449(14)	0.0089(6)	0.000290(20)	¹⁰² Ru	294.66(4)	0.071(6)	0.00213(18)
Molybdenum (Z=42), At.Wt.=95.94(1), $\sigma_{\gamma}^Z = 2.51(6)$				¹⁰⁴ Ru	301.75(5)	0.0192(19)	0.00058(6)
⁹⁸ Mo	140.5110(10)d	0.0276(7)	0.000872[<0.1%]	¹⁰⁴ Ru	321.526(24)	0.0175(18)	0.00052(5)
¹⁰⁰ Mo	180.711(15)	0.0017(4)	5.4(13)E-5	¹⁰² Ru	346.23(6)	0.030(3)	0.00090(9)
⁹⁸ Mo	198.38(11)	0.0108(9)	0.00034(3)	¹⁰⁴ Ru	358.57(7)	0.0173(24)	0.00052(7)
⁹⁴ Mo	204.20(5)	0.0117(6)	0.000370(19)	¹⁰² Ru	403.10(5)	0.062(6)	0.00186(18)
⁹⁵ Mo	349.77(4)	0.0327(13)	0.00103(4)	⁹⁹ Ru	403.18(8)	0.050(10)	0.0015(3)
⁹⁵ Mo	369.68(9)	0.0319(19)	0.00101(6)	¹⁰¹ Ru	418.531(22)	0.033(4)	0.00099(12)
⁹⁵ Mo	480.57(3)	0.028(5)	0.00088(16)	⁹⁹ Ru	424.87(5)	0.0170(21)	0.00051(6)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁰² Ru	432.00(6)	0.0267(25)	0.00080(8)	¹⁰² Ru	2074.98(20)	0.022(3)	0.00066(9)
¹⁰⁴ Ru	462.93(7)	0.025(3)	0.00075(9)	⁹⁹ Ru	3016.61(9)	0.0175(21)	0.00052(6)
¹⁰¹ Ru	468.69(4)	0.049(5)	0.00147(15)	⁹⁹ Ru	3981.1(3)	0.0186(24)	0.00056(7)
¹⁰¹Ru	475.0950(20)	0.98(9)	0.029(3)	¹⁰² Ru	4627.38(14)	0.0187(24)	0.00056(7)
¹⁰² Ru	500.96(10)	0.0175(19)	0.00052(6)	¹⁰⁴ Ru	4943.1(3)	0.020(3)	0.00060(9)
⁹⁹ Ru	518.92(4)	0.026(3)	0.00078(9)	¹⁰⁰ Ru	6266.6(3)	0.0180(13)	0.00054(4)
⁹⁹Ru	539.538(15)	1.53(13)	0.046(4)	¹⁰¹ Ru	6274.68(4)	0.017(3)	0.00051(9)
¹⁰² Ru	545.44(5)	0.0253(25)	0.00076(8)	⁹⁹ Ru	6340.59(6)	0.024(4)	0.00072(12)
¹⁰² Ru	554.54(7)	0.027(3)	0.00081(9)	¹⁰¹ Ru	6627.200(20)	0.093(9)	0.0028(3)
¹⁰⁴ Ru	562.70(6)	0.028(3)	0.00084(9)	¹⁰¹ Ru	6978.81(16)	0.041(5)	0.00123(15)
¹⁰² Ru	562.86(12)	0.017(4)	0.00051(12)	⁹⁹ Ru	7103.08(8)	0.018(3)	0.00054(9)
⁹⁹ Ru	590.91(6)	0.053(5)	0.00159(15)	⁹⁹ Ru	7792.04(3)	0.132(13)	0.0040(4)
¹⁰¹Ru	627.970(22)	0.176(16)	0.0053(5)	Rhodium (Z=45), At.Wt.=102.90550(2), σ_γ^Z=145.0(20)			
¹⁰¹Ru	631.22(4)	0.30(3)	0.0090(9)	¹⁰³ Rh	32.18(4)	0.25(5)	0.0074(15)
⁹⁹ Ru	631.48(6)	0.017(5)	0.00051(15)	¹⁰³ Rh	35.56(13)	0.65(7)	0.0191(21)
¹⁰¹ Ru	636.86(6)	0.033(3)	0.00099(9)	¹⁰³ Rh	46.20(5)	0.37(5)	0.0109(15)
¹⁰⁴ Ru	640.16(7)	0.0171(22)	0.00051(7)	¹⁰³Rh	51.50(3)d	5.2(3)	0.153[90%]
¹⁰¹ Ru	680.57(6)	0.0162(22)	0.00049(7)	¹⁰³Rh	51.50(3)	16.0(4)	0.471(12)
⁹⁹Ru	686.907(17)	0.52(5)	0.0156(15)	¹⁰³ Rh	55.46(4)	0.76(15)	0.022(4)
¹⁰¹ Ru	692.28(9)	0.025(3)	0.00075(9)	¹⁰³ Rh	80.80(3)	0.73(16)	0.021(5)
¹⁰¹ Ru	695.53(9)	0.039(5)	0.00117(15)	¹⁰³ Rh	83.74(3)	0.63(14)	0.019(4)
¹⁰¹ Ru	697.31(15)	0.020(3)	0.00060(9)	¹⁰³Rh	85.19(3)	3.2(3)	0.094(9)
⁹⁹ Ru	700.53(3)	0.018(3)	0.00054(9)	¹⁰³ Rh	85.97(4)	0.30(6)	0.0088(18)
⁹⁹ Ru	710.70(4)	0.034(3)	0.00102(9)	¹⁰³Rh	97.14(3)	19.5(4)	0.574(12)
¹⁰⁴ Ru	724.30(3)d	0.0760(11)	0.00228[7.4%]	¹⁰³Rh	100.74(4)	4.96(10)	0.146(3)
⁹⁹ Ru	734.60(6)	0.0254(25)	0.00076(8)	¹⁰³ Rh	105.40(6)	0.47(4)	0.0138(12)
¹⁰¹ Ru	739.614(21)	0.0196(20)	0.00059(6)	¹⁰³ Rh	118.10(3)	0.570(15)	0.0168(4)
¹⁰¹ Ru	766.82(10)	0.019(3)	0.00057(9)	¹⁰³ Rh	119.50(3)	1.5(3)	0.044(9)
⁹⁹ Ru	822.579(22)	0.137(12)	0.0041(4)	¹⁰³Rh	127.20(3)	5.27(21)	0.155(6)
⁹⁹ Ru	836.20(3)	0.029(5)	0.00087(15)	¹⁰³ Rh	129.37(3)	0.465(20)	0.0137(6)
⁹⁹ Ru	849.23(4)	0.030(3)	0.00090(9)	¹⁰³ Rh	131.86(6)	0.437(24)	0.0129(7)
¹⁰¹ Ru	940.42(3)	0.038(4)	0.00114(12)	¹⁰³Rh	134.54(3)	6.8(4)	0.200(12)
¹⁰¹ Ru	1046.498(3)	0.103(9)	0.0031(3)	¹⁰³ Rh	135.16(4)	0.66(16)	0.019(5)
¹⁰² Ru	1075.37(14)	0.0188(21)	0.00056(6)	¹⁰³ Rh	137.65(3)	0.45(4)	0.0133(12)
¹⁰¹ Ru	1103.062(22)	0.100(9)	0.0030(3)	¹⁰³ Rh	138.74(4)	0.54(4)	0.0159(12)
¹⁰¹ Ru	1105.54(6)	0.055(5)	0.00165(15)	¹⁰³ Rh	146.72(3)	1.5(3)	0.044(9)
⁹⁹ Ru	1107.20(5)	0.0236(24)	0.00071(7)	¹⁰³ Rh	157.00(3)	1.05(3)	0.0309(9)
⁹⁹ Ru	1207.93(8)	0.022(6)	0.00066(18)	¹⁰³ Rh	159.49(3)	0.380(16)	0.0112(5)
⁹⁹ Ru	1266.58(4)	0.0178(20)	0.00053(6)	¹⁰³ Rh	161.55(4)	1.00(3)	0.0294(9)
⁹⁹ Ru	1325.51(4)	0.034(4)	0.00102(12)	¹⁰³ Rh	165.20(4)	0.89(4)	0.0262(12)
⁹⁹ Ru	1341.50(3)	0.137(12)	0.0041(4)	¹⁰³ Rh	168.21(5)	0.45(10)	0.013(3)
⁹⁹ Ru	1362.111(24)	0.111(13)	0.0033(4)	¹⁰³Rh	169.16(5)	2.88(19)	0.085(6)
⁹⁹ Ru	1365.29(4)	0.023(3)	0.00069(9)	¹⁰³ Rh	170.08(6)	0.64(19)	0.019(6)
⁹⁹ Ru	1520.71(8)	0.022(3)	0.00066(9)	¹⁰³ Rh	177.64(4)	1.85(12)	0.054(4)
⁹⁹ Ru	1523.10(3)	0.034(4)	0.00102(12)	¹⁰³Rh	178.66(4)	3.27(14)	0.096(4)
⁹⁹ Ru	1535.75(19)	0.0155(21)	0.00046(6)	¹⁰³Rh	180.87(3)	22.6(15)	0.67(4)
⁹⁹ Ru	1559.51(6)	0.027(3)	0.00081(9)	¹⁰³ Rh	186.04(3)	1.50(5)	0.0442(15)
¹⁰¹ Ru	1568.383(20)	0.044(4)	0.00132(12)	¹⁰³ Rh	196.55(5)	0.80(16)	0.024(5)
⁹⁹ Ru	1627.32(3)	0.129(12)	0.0039(4)	¹⁰³ Rh	198.89(4)	0.52(10)	0.015(3)
⁹⁹ Ru	1701.11(7)	0.032(3)	0.00096(9)	¹⁰³ Rh	202.85(6)	1.6(3)	0.047(9)
¹⁰² Ru	1730.6(3)	0.0176(23)	0.00053(7)	¹⁰³ Rh	213.05(3)	1.27(3)	0.0374(9)
⁹⁹ Ru	1827.09(5)	0.045(4)	0.00135(12)	¹⁰³Rh	215.340(22)	5.20(12)	0.153(4)
⁹⁹ Ru	1865.04(4)	0.028(3)	0.00084(9)	¹⁰³ Rh	215.36(3)	1.54(12)	0.045(4)
⁹⁹ Ru	1929.77(4)	0.025(3)	0.00075(9)	¹⁰³Rh	216.54(8)	5.0(10)	0.15(3)
¹⁰²Ru	1959.30(7)	0.210(19)	0.0063(6)	¹⁰³Rh	217.82(3)	7.38(13)	0.217(4)
⁹⁹ Ru	1996.62(6)	0.0223(25)	0.00067(8)	¹⁰³ Rh	218.44(4)	0.30(6)	0.0088(18)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁰³ Rh	219.85(4)	0.480(19)	0.0141(6)	¹⁰³ Rh	718.26(6)	0.267(10)	0.0079(3)
¹⁰³ Rh	222.74(5)	0.26(3)	0.0077(9)	¹⁰³ Rh	720.58(9)	0.297(9)	0.0087(3)
¹⁰³ Rh	235.93(6)	0.345(10)	0.0102(3)	¹⁰³ Rh	722.81(4)	0.255(11)	0.0075(3)
¹⁰³ Rh	245.07(5)	0.29(4)	0.0085(12)	¹⁰³ Rh	734.90(7)	0.68(5)	0.0200(15)
¹⁰³ Rh	245.45(4)	0.387(17)	0.0114(5)	¹⁰³ Rh	762.83(6)	0.339(21)	0.0100(6)
¹⁰³ Rh	246.61(5)	0.27(5)	0.0080(15)	¹⁰³ Rh	787.12(4)	1.16(3)	0.0342(9)
¹⁰³ Rh	247.55(5)	0.387(17)	0.0114(5)	¹⁰³ Rh	790.43(12)	0.7(4)	0.021(12)
¹⁰³ Rh	261.38(5)	1.09(3)	0.0321(9)	¹⁰³ Rh	791.41(7)	0.84(5)	0.0247(15)
¹⁰³Rh	266.84(3)	2.66(17)	0.078(5)	¹⁰³ Rh	817.71(8)	0.5(3)	0.015(9)
¹⁰³ Rh	269.18(3)	1.42(11)	0.042(3)	¹⁰³ Rh	834.94(7)	0.277(13)	0.0082(4)
¹⁰³ Rh	273.62(3)	0.814(18)	0.0240(5)	¹⁰³ Rh	868.28(6)	0.56(3)	0.0165(9)
¹⁰³ Rh	284.36(4)	0.26(3)	0.0077(9)	¹⁰³ Rh	872.24(4)	0.440(16)	0.0130(5)
¹⁰³ Rh	286.18(8)	0.42(4)	0.0124(12)	¹⁰³ Rh	907.66(7)	0.28(6)	0.0082(18)
¹⁰³ Rh	303.59(5)	0.794(17)	0.0234(5)	¹⁰³ Rh	951.96(6)	1.090(24)	0.0321(7)
¹⁰³ Rh	305.7(3)	1.070(21)	0.0315(6)	¹⁰³ Rh	5798.18(14)	0.59(3)	0.0174(9)
¹⁰³ Rh	317.07(4)	0.74(3)	0.0218(9)	¹⁰³ Rh	5917.43(5)	1.31(4)	0.0386(12)
¹⁰³ Rh	323.48(4)	1.54(19)	0.045(6)	¹⁰³ Rh	6046.79(6)	0.88(4)	0.0259(12)
¹⁰³ Rh	324.64(4)	0.57(9)	0.017(3)	¹⁰³ Rh	6082.98(7)	0.58(4)	0.0171(12)
¹⁰³Rh	333.44(3)	3.27(8)	0.0963(24)	¹⁰³ Rh	6110.21(6)	0.278(19)	0.0082(6)
¹⁰³ Rh	352.99(3)	0.668(19)	0.0197(6)	¹⁰³ Rh	6172.33(5)	0.75(3)	0.0221(9)
¹⁰³ Rh	352.99(3)	0.668(19)	0.0197(6)	¹⁰³ Rh	6211.62(4)	0.89(3)	0.0262(9)
¹⁰³ Rh	356.82(3)	0.668(19)	0.0197(6)	¹⁰³ Rh	6354.87(7)	0.46(3)	0.0135(9)
¹⁰³ Rh	370.48(7)	0.429(18)	0.0126(5)	¹⁰³ Rh	6785.66(4)	0.470(20)	0.0138(6)
¹⁰³ Rh	374.826(23)	1.300(25)	0.0383(7)	Palladium (Z=46, At.Wt.=106.42(1), σ_γ^Z = 6.9(4))			
¹⁰³ Rh	379.823(5)	0.301(21)	0.0089(6)	¹⁰⁸ Pd	113.4010(10)	0.335(5)	0.00954(14)
¹⁰³ Rh	382.24(3)	0.374(25)	0.0110(7)	¹⁰⁶ Pd	115.86(7)	0.0141(13)	0.00040(4)
¹⁰³ Rh	385.10(3)	0.819(19)	0.0241(6)	¹⁰² Pd	118.68(3)	0.0042(11)	1.2(3)E-4
¹⁰³ Rh	391.18(5)	0.358(17)	0.0105(5)	¹⁰⁸ Pd	152.9420(10)	0.1450(22)	0.00413(6)
¹⁰³ Rh	403.96(11)	0.350(15)	0.0103(4)	¹⁰⁸ Pd	178.0340(10)	0.1090(22)	0.00310(6)
¹⁰³ Rh	408.16(4)	0.293(18)	0.0086(5)	¹⁰⁸ Pd	188.9900(10)d	0.0273(15)	0.00078[89%]
¹⁰³ Rh	420.62(3)	2.06(4)	0.0607(12)	¹⁰⁸ Pd	197.3446(5)	0.0650(20)	0.00185(6)
¹⁰³ Rh	427.44(3)	1.12(3)	0.0330(9)	¹⁰⁸ Pd	211.8840(20)	0.0540(18)	0.00154(5)
¹⁰³ Rh	431.91(12)	0.461(23)	0.0136(7)	¹⁰⁸ Pd	245.0790(20)	0.250(4)	0.00712(11)
¹⁰³ Rh	440.55(3)	2.23(10)	0.066(3)	¹⁰⁸ Pd	266.3430(20)	0.0515(12)	0.00147(3)
¹⁰³ Rh	459.69(6)	0.555(17)	0.0163(5)	¹⁰⁸ Pd	276.289(6)	0.0562(18)	0.00160(5)
¹⁰³Rh	470.40(3)	2.61(7)	0.0769(21)	¹⁰⁴ Pd	280.65(6)	0.0158(14)	0.00045(4)
¹⁰³ Rh	482.230(25)	1.78(6)	0.0524(18)	¹⁰⁸ Pd	291.4350(20)	0.1040(20)	0.00296(6)
¹⁰³ Rh	497.80(4)	0.88(4)	0.0259(12)	¹⁰⁸ Pd	325.2840(20)	0.208(3)	0.00592(9)
¹⁰³ Rh	503.00(13)	0.23(6)	0.0068(18)	¹⁰⁸ Pd	326.8690(20)	0.0793(20)	0.00226(6)
¹⁰³ Rh	529.98(5)	0.885(21)	0.0261(6)	¹⁰⁸ Pd	333.960(4)	0.1110(25)	0.00316(7)
¹⁰³Rh	538.04(3)	2.43(7)	0.0716(21)	¹⁰⁸ Pd	339.5290(20)	0.195(3)	0.00555(9)
¹⁰³ Rh	542.31(8)	0.48(3)	0.0141(9)	¹⁰⁸ Pd	359.4290(20)	0.120(3)	0.00342(9)
¹⁰³ Rh	550.87(8)	0.31(3)	0.0091(9)	¹⁰⁸ Pd	378.1890(20)	0.0411(20)	0.00117(6)
¹⁰³Rh	555.81(4)d	3.14(9)	0.092[98%]	¹⁰⁸ Pd	428.409(4)	0.0504(21)	0.00144(6)
¹⁰³ Rh	562.78(4)	0.299(22)	0.0088(7)	¹⁰⁵ Pd	429.63(4)	0.145(3)	0.00413(9)
¹⁰³ Rh	574.07(5)	0.539(20)	0.0159(6)	¹⁰⁸ Pd	433.5640(20)	0.097(3)	0.00276(9)
¹⁰³ Rh	577.92(5)	0.342(19)	0.0101(6)	¹⁰⁵Pd	511.843(20)	4.00(4)	0.1139(11)
¹⁰³ Rh	597.65(3)	0.997(23)	0.0294(7)	¹⁰⁵Pd	616.192(20)	0.629(9)	0.0179(3)
¹⁰³ Rh	609.55(12)	0.58(3)	0.0171(9)	¹⁰⁵ Pd	621.95(6)	0.126(7)	0.00359(20)
¹⁰³ Rh	633.45(6)	0.239(17)	0.0070(5)	¹⁰⁸ Pd	685.914(8)	0.042(7)	0.00120(20)
¹⁰³ Rh	680.61(6)	0.25(5)	0.0074(15)	¹⁰⁵Pd	717.356(22)	0.777(9)	0.0221(3)
¹⁰³ Rh	689.47(5)	0.35(8)	0.0103(24)	¹⁰⁵ Pd	748.34(5)	0.0802(23)	0.00228(7)
¹⁰³ Rh	695.38(7)	1.07(3)	0.0315(9)	¹⁰⁸ Pd	754.894(9)	0.0474(18)	0.00135(5)
¹⁰³ Rh	702.72(7)	0.869(25)	0.0256(7)	¹⁰⁵ Pd	804.33(4)	0.091(3)	0.00259(9)
¹⁰³ Rh	707.67(6)	0.843(25)	0.0248(7)	¹⁰⁵ Pd	846.29(10)	0.0452(18)	0.00129(5)
¹⁰³ Rh	710.69(5)	0.46(4)	0.0135(12)	¹⁰⁵ Pd	848.16(6)	0.1000(25)	0.00285(7)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁰⁸ Pd	1019.872(9)	0.0467(25)	0.00133(7)	¹⁰⁷ Ag	259.17(3)	1.560(25)	0.0438(7)
¹⁰⁵ Pd	1045.82(3)	0.321(7)	0.00914(20)	¹⁰⁷ Ag	262.31(6)	0.161(11)	0.0045(3)
¹⁰⁵ Pd	1050.31(4)	0.360(8)	0.01025(23)	¹⁰⁹ Ag	267.08(3)	2.73(6)	0.0767(17)
¹⁰⁵ Pd	1053.68(9)	0.057(3)	0.00162(9)	¹⁰⁹ Ag	269.05(4)	0.6(5)	0.017(14)
¹⁰⁵ Pd	1128.03(3)	0.323(6)	0.00920(17)	¹⁰⁹ Ag	269.97(4)	0.565(25)	0.0159(7)
¹⁰⁵ Pd	1168.16(8)	0.0588(22)	0.00167(6)	¹⁰⁹ Ag	282.66(6)	0.079(10)	0.0022(3)
¹⁰⁵ Pd	1397.54(7)	0.089(3)	0.00253(9)	¹⁰⁷ Ag	286.91(4)	0.400(25)	0.0112(7)
¹⁰⁵ Pd	1572.54(7)	0.207(25)	0.0059(7)	¹⁰⁷ Ag	294.39(3)	2.05(12)	0.058(3)
¹⁰⁵ Pd	1909.40(11)	0.0423(20)	0.00120(6)	¹⁰⁷ Ag	295.22(18)	0.10(4)	0.0028(11)
¹⁰⁵ Pd	1927.25(10)	0.041(3)	0.00117(9)	¹⁰⁷ Ag	299.95(3)	1.15(5)	0.0323(14)
¹⁰⁵ Pd	1988.14(12)	0.060(4)	0.00171(11)	¹⁰⁷ Ag	301.75(7)	0.187(15)	0.0053(4)
¹⁰⁵ Pd	2484.73(25)	0.052(4)	0.00148(11)	¹⁰⁹ Ag	302.83(13)	0.129(14)	0.0036(4)
¹⁰⁸ Pd	4794.02(12)	0.112(10)	0.0032(3)	¹⁰⁹ Ag	304.43(15)	0.135(9)	0.00379(25)
¹⁰⁸ Pd	5212.31(12)	0.061(5)	0.00174(14)	¹⁰⁹ Ag	316.88(3)	0.206(7)	0.00579(20)
¹¹⁰ Pd	5531.9(4)	0.0120(20)	0.00034(6)	¹⁰⁷ Ag	320.36(6)	0.091(7)	0.00256(20)
Silver (Z=47), At.Wt.=107.8682(2), $\sigma_{\gamma}^Z = 63.3(8)$							
¹⁰⁹ Ag	68.36(4)	0.113(8)	0.00317(22)	¹⁰⁹ Ag	338.74(3)	0.595(10)	0.0167(3)
¹⁰⁹ Ag	72.67(5)	-0.9	-0.03	¹⁰⁷ Ag	349.95(3)	0.70(4)	0.0197(11)
¹⁰⁷ Ag	78.91(4)	3.90(12)	0.110(3)	¹⁰⁷ Ag	350.99(9)	0.145(12)	0.0041(3)
¹⁰⁹ Ag	79.91(6)	-1.0	-0.03	¹⁰⁹ Ag	357.82(5)	0.561(22)	0.0158(6)
¹⁰⁹ Ag	93.34(5)	0.5(3)	0.014(8)	¹⁰⁹ Ag	360.41(3)	1.55(3)	0.0435(8)
¹⁰⁷ Ag	101.55(8)	0.189(20)	0.0053(6)	¹⁰⁷ Ag	365.41(23)	0.16(4)	0.0045(11)
¹⁰⁹ Ag	105.95(6)	0.87(13)	0.024(4)	¹⁰⁹ Ag	366.97(10)	0.21(4)	0.0059(11)
¹⁰⁷ Ag	110.24(7)	0.273(22)	0.0077(6)	¹⁰⁷ Ag	372.1(3)	0.09(3)	0.0025(8)
¹⁰⁷ Ag	113.51(6)	0.52(3)	0.0146(8)	¹⁰⁷ Ag	376.71(9)	0.294(13)	0.0083(4)
¹⁰⁹ Ag	117.45(8)	3.85(7)	0.1082(20)	¹⁰⁹ Ag	378.11(6)	0.744(20)	0.0209(6)
¹⁰⁹ Ag	124.86(5)	0.158(12)	0.0044(3)	¹⁰⁷ Ag	380.90(3)	1.59(3)	0.0447(8)
¹⁰⁷ Ag	143.94(4)	0.121(5)	0.00340(14)	¹⁰⁹ Ag	380.97(15)	0.7(5)	0.020(14)
¹⁰⁷ Ag	147.11(4)	0.114(5)	0.00320(14)	¹⁰⁷ Ag	384.31(13)	0.128(22)	0.0036(6)
¹⁰⁷ Ag	148.79(3)	0.214(6)	0.00601(17)	¹⁰⁷ Ag	386.18(13)	0.192(24)	0.0054(7)
¹⁰⁹ Ag	152.58(4)	0.326(6)	0.00916(17)	¹⁰⁹ Ag	387.99(7)	0.121(21)	0.0034(6)
¹⁰⁷ Ag	155.22(11)	0.081(13)	0.0023(4)	¹⁰⁷ Ag	396.25(4)	0.138(6)	0.00388(17)
¹⁰⁹ Ag	161.69(5)	0.217(8)	0.00610(22)	¹⁰⁷ Ag	399.87(7)	0.093(6)	0.00261(17)
¹⁰⁹ Ag	166.62(4)	0.295(10)	0.0083(3)	¹⁰⁹ Ag	408.61(4)	0.459(9)	0.01290(25)
¹⁰⁷ Ag	178.32(4)	0.208(8)	0.00584(22)	¹⁰⁷ Ag	410.31(6)	0.142(6)	0.00399(17)
¹⁰⁷ Ag	191.39(3)	1.81(5)	0.0509(14)	¹⁰⁹ Ag	416.93(5)	0.243(13)	0.0068(4)
¹⁰⁷ Ag	192.90(3)	2.20(6)	0.0618(17)	¹⁰⁹ Ag	427.96(16)	0.273(11)	0.0077(3)
¹⁰⁹ Ag	194.56(14)	~0.2	~0.006	¹⁰⁷ Ag	429.09(7)	0.253(11)	0.0071(3)
¹⁰⁹ Ag	195.33(6)	0.50(3)	0.0140(8)	¹⁰⁹ Ag	431.36(7)	0.248(13)	0.0070(4)
¹⁰⁹ Ag	195.74(8)	~0.2	~0.006	¹⁰⁷ Ag	437.713(15)	0.079(10)	0.0022(3)
¹⁰⁹ Ag	198.72(4)	7.75(13)	0.218(4)	¹⁰⁷ Ag	438.26(12)	0.191(11)	0.0054(3)
¹⁰⁷ Ag	201.31(6)	0.45(3)	0.0126(8)	¹⁰⁷ Ag	439.69(12)	0.216(11)	0.0061(3)
¹⁰⁷ Ag	204.02(9)	0.088(22)	0.0025(6)	¹⁰⁷ Ag	441.79(8)	0.181(21)	0.0051(6)
¹⁰⁷ Ag	206.46(3)	3.58(7)	0.1006(20)	¹⁰⁹ Ag	446.10(7)	0.183(10)	0.0051(3)
¹⁰⁷ Ag	212.30(4)	0.26(4)	0.0073(11)	¹⁰⁹ Ag	450.80(7)	0.098(16)	0.0028(5)
¹⁰⁷ Ag	215.15(4)	1.55(3)	0.0435(8)	¹⁰⁹ Ag	461.56(6)	0.265(16)	0.0074(5)
¹⁰⁹ Ag	220.77(10)	~0.08	~0.002	¹⁰⁷ Ag	464.04(12)	0.236(20)	0.0066(6)
¹⁰⁹ Ag	231.46(5)	0.224(12)	0.0063(3)	¹⁰⁷ Ag	465.37(6)	0.46(3)	0.0129(8)
¹⁰⁹ Ag	235.62(4)	4.62(7)	0.1298(20)	¹⁰⁹ Ag	468.65(7)	0.166(9)	0.00466(25)
¹⁰⁷ Ag	236.85(4)	1.95(3)	0.0548(8)	¹⁰⁷ Ag	479.36(7)	0.095(12)	0.0027(3)
¹⁰⁹ Ag	236.89(7)	1.3(9)	0.037(25)	¹⁰⁹ Ag	484.18(8)	0.253(18)	0.0071(5)
¹⁰⁷ Ag	237.63(3)	0.26(5)	0.0073(14)	¹⁰⁷ Ag	485.68(13)	0.098(7)	0.00275(20)
¹⁰⁷ Ag	239.10(4)	0.327(11)	0.0092(3)	¹⁰⁹ Ag	488.66(6)	0.149(12)	0.0042(3)
¹⁰⁷ Ag	244.56(6)	0.146(20)	0.0041(6)	¹⁰⁹ Ag	495.71(3)	1.080(18)	0.0303(5)
¹⁰⁷ Ag	249.15(6)	0.087(7)	0.00244(20)	¹⁰⁷ Ag	497.57(8)	0.157(9)	0.00441(25)
¹⁰⁹ Ag	252.17(5)	0.096(6)	0.00270(17)	¹⁰⁷ Ag	499.97(4)	0.265(13)	0.0074(4)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁰⁷ Ag	522.43(9)	0.125(7)	0.00351(20)	¹⁰⁹ Ag	5539.17(21)	0.106(9)	0.00298(25)
¹⁰⁹ Ag	524.47(3)	0.804(11)	0.0226(3)	¹⁰⁹ Ag	5545.6(3)	0.106(12)	0.0030(3)
¹⁰⁹ Ag	526.07(8)	0.364(7)	0.01023(20)	¹⁰⁹ Ag	5554.8(3)	0.111(10)	0.0031(3)
¹⁰⁷ Ag	527.23(5)	0.371(10)	0.0104(3)	¹⁰⁹ Ag	5580.62(19)	0.302(14)	0.0085(4)
¹⁰⁹ Ag	536.13(3)	1.090(16)	0.0306(5)	¹⁰⁹ Ag	5615.11(20)	0.208(11)	0.0058(3)
¹⁰⁹ Ag	544.14(5)	0.34(3)	0.0096(8)	¹⁰⁹ Ag	5642.24(22)	0.199(12)	0.0056(3)
¹⁰⁹ Ag	549.56(3)	1.540(24)	0.0433(7)	¹⁰⁹ Ag	5701.49(19)	0.716(18)	0.0201(5)
¹⁰⁷ Ag	563.91(5)	0.191(6)	0.00537(17)	¹⁰⁹ Ag	5710.22(20)	0.229(10)	0.0064(3)
¹⁰⁷ Ag	572.10(6)	0.080(6)	0.00225(17)	¹⁰⁹ Ag	5773.12(21)	0.225(9)	0.00632(25)
¹⁰⁷ Ag	574.77(3)	0.299(7)	0.00840(20)	¹⁰⁹ Ag	5795.0(3)	0.513(14)	0.0144(4)
¹⁰⁹ Ag	586.85(3)	0.459(8)	0.01290(22)	¹⁰⁹ Ag	5913.3(5)	0.084(7)	0.00236(20)
¹⁰⁹ Ag	593.86(4)	0.484(11)	0.0136(3)	¹⁰⁹ Ag	5996.81(10)	0.154(7)	0.00433(20)
¹⁰⁷ Ag	599.87(4)	0.37(3)	0.0104(8)	¹⁰⁹ Ag	6022.46(10)	0.250(10)	0.0070(3)
¹⁰⁹ Ag	610.33(15)	0.105(25)	0.0029(7)	¹⁰⁹ Ag	6034.70(11)	0.080(6)	0.00225(17)
¹⁰⁷ Ag	611.98(18)	0.09(3)	0.0025(8)	¹⁰⁹ Ag	6057.25(9)	0.663(19)	0.0186(5)
¹⁰⁹ Ag	614.15(8)	0.20(5)	0.0056(14)	¹⁰⁹ Ag	6101.98(11)	0.080(5)	0.00225(14)
¹⁰⁷ Ag	616.89(4)	0.20(4)	0.0056(11)	¹⁰⁷ Ag	6268.80(24)	0.146(7)	0.00410(20)
¹⁰⁹ Ag	620.07(5)	0.40(5)	0.0112(14)	¹⁰⁷ Ag	6372.7(9)	0.11(4)	0.0031(11)
¹⁰⁷ Ag	626.41(4)	0.39(6)	0.0110(17)	¹⁰⁹ Ag	6540.92(9)	0.259(11)	0.0073(3)
¹⁰⁷ Ag	629.499(20)	0.12(3)	0.0034(8)	¹⁰⁷ Ag	6707.6(3)	0.083(7)	0.00233(20)
¹⁰⁹ Ag	632.47(10)	0.42(12)	0.012(3)	¹⁰⁹ Ag	6807.13(11)	0.083(3)	0.00233(8)
¹⁰⁷ Ag	636.53(4)	0.31(11)	0.009(3)	¹⁰⁷ Ag	6892.1(3)	0.079(6)	0.00222(17)
¹⁰⁷ Ag	640.18(4)	0.24(6)	0.0067(17)	¹⁰⁷ Ag	6977.2(3)	0.121(8)	0.00340(22)
¹⁰⁷ Ag	652.041(20)	0.117(19)	0.0033(5)	¹⁰⁷ Ag	7065.3(3)	0.103(8)	0.00289(22)
¹⁰⁹ Ag	652.96(5)	0.255(12)	0.0072(3)	¹⁰⁷ Ag	7078.5(3)	0.291(13)	0.0082(4)
¹⁰⁹ Ag	655.02(11)	0.107(14)	0.0030(4)	¹⁰⁷ Ag	7271.8(3)	0.284(14)	0.0080(4)
¹⁰⁹ Ag	657.50(10)d	1.86(5)	0.0523[99%]	Cadmium (Z=48), At.Wt.=112.411(8), $\sigma_{\gamma}^z = 2522(50)$			
¹⁰⁷ Ag	662.55(11)	0.088(12)	0.0025(3)	¹¹³ Cd	95.88(4)	21.2(6)	0.572(16)
¹⁰⁷ Ag	664.91(3)	0.329(22)	0.0092(6)	¹¹⁰ Cd	171.3(3)	57(6)	1.54(16)
¹⁰⁷ Ag	670.53(7)	0.104(17)	0.0029(5)	¹¹⁰ Cd	245.3(3)	274(25)	7.4(7)
¹⁰⁷ Ag	674.07(6)	0.094(16)	0.0026(5)	¹¹⁰ Cd	284.3(3)	29(3)	0.78(8)
¹⁰⁷ Ag	685.8(3)	0.081(20)	0.0023(6)	¹¹⁰ Cd	342.2(3)	1.00E+02	2.70E+00
¹⁰⁷ Ag	687.48(8)	0.35(5)	0.0098(14)	¹¹³ Cd	558.32(3)	1860(30)	50.1(8)
¹⁰⁹ Ag	698.44(6)	0.158(6)	0.00444(17)	¹¹³ Cd	576.04(3)	107.0(17)	2.88(5)
¹⁰⁷ Ag	718.17(6)	0.199(12)	0.0056(3)	¹¹¹ Cd	617.54(15)	2.9(4)	0.078(11)
¹⁰⁹ Ag	724.75(5)	0.393(14)	0.0110(4)	¹¹⁰ Cd	620.3(3)	38(4)	1.02(11)
¹⁰⁷ Ag	746.21(19)	0.088(10)	0.0025(3)	¹¹³ Cd	648.79(10)	34.1(9)	0.919(24)
¹⁰⁹ Ag	748.40(6)	0.328(9)	0.00921(25)	¹¹³ Cd	651.19(3)	358(5)	9.65(13)
¹⁰⁹ Ag	750.77(4)	0.529(11)	0.0149(3)	¹¹³ Cd	654.47(4)	34.1(9)	0.919(24)
¹⁰⁹ Ag	767.01(5)	0.31(4)	0.0087(11)	¹¹³ Cd	707.39(3)	29.3(5)	0.790(13)
¹⁰⁹ Ag	773.32(8)	0.22(3)	0.0062(8)	¹¹³ Cd	725.19(3)	107.0(13)	2.88(4)
¹⁰⁷ Ag	781.21(11)	0.094(22)	0.0026(6)	¹¹³ Cd	748.04(6)	37(3)	1.00(8)
¹⁰⁹ Ag	785.57(5)	0.34(4)	0.0096(11)	¹¹³ Cd	805.85(3)	134.0(18)	3.61(5)
¹⁰⁷ Ag	796.15(8)	0.38(4)	0.0107(11)	¹¹³ Cd	1209.65(4)	122.0(19)	3.29(5)
¹⁰⁷ Ag	812.10(6)	0.131(5)	0.00368(14)	¹¹³ Cd	1283.45(4)	47.5(9)	1.281(24)
¹⁰⁷ Ag	819.26(8)	0.291(6)	0.00818(17)	¹¹³ Cd	1300.98(5)	31.1(11)	0.84(3)
¹⁰⁷ Ag	845.19(14)	0.085(19)	0.0024(5)	¹¹³ Cd	1364.30(4)	123.0(21)	3.32(6)
¹⁰⁷ Ag	881.01(7)	0.178(7)	0.00500(20)	¹¹³ Cd	1370.55(5)	30.2(9)	0.814(24)
¹⁰⁷ Ag	895.48(3)	0.376(8)	0.01056(22)	¹¹³ Cd	1399.54(4)	97.7(15)	2.63(4)
¹⁰⁷ Ag	918.97(11)	0.124(22)	0.0035(6)	¹¹³ Cd	1489.53(4)	68.5(11)	1.85(3)
¹⁰⁷ Ag	938.04(5)	0.186(6)	0.00523(17)	¹¹³ Cd	1660.36(5)	66.7(13)	1.80(4)
¹⁰⁷ Ag	960.13(4)	0.199(10)	0.0056(3)	¹¹³ Cd	1826.19(7)	25.2(7)	0.679(19)
¹⁰⁷ Ag	972.69(7)	0.078(9)	0.00219(25)	¹¹³ Cd	2102.39(8)	24.0(9)	0.647(24)
¹⁰⁷ Ag	1013.11(3)	0.698(13)	0.0196(4)	¹¹³ Cd	2398.27(12)	22.4(8)	0.604(22)
¹⁰⁷ Ag	1051.36(5)	0.225(8)	0.00632(22)	¹¹³ Cd	2455.93(7)	87.3(18)	2.35(5)
¹⁰⁷ Ag	1079.68(13)	0.165(15)	0.0046(4)	¹¹³ Cd	2550.30(8)	38.7(11)	1.04(3)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹¹³ Cd	2659.96(7)	64.0(15)	1.73(4)	¹¹⁵ In	364.995(20)	0.53(4)	0.0140(11)
¹¹³ Cd	2767.67(13)	22.4(13)	0.60(4)	¹¹⁵ In	373.149(24)	0.38(3)	0.0100(8)
¹¹³ Cd	2799.98(9)	27.6(9)	0.744(24)	¹¹⁵ In	375.969(12)	2.66(20)	0.070(5)
¹¹³ Cd	2999.69(12)	29.1(14)	0.78(4)	¹¹⁵ In	384.421(11)	2.9(7)	0.077(18)
¹¹³ Cd	3109.08(12)	28.6(12)	0.77(3)	¹¹⁵In	385.111(8)	12.1(9)	0.319(24)
¹¹³ Cd	3218.96(12)	19.0(9)	0.512(24)	¹¹⁵ In	387.636(13)	0.344(25)	0.0091(7)
¹¹³ Cd	5824.31(16)	69.1(18)	1.86(5)	¹¹⁵ In	393.09(11)	0.39(3)	0.0103(8)
¹¹³ Cd	5934.39(20)	19.3(10)	0.52(3)	¹¹⁵ In	396.496(12)	0.51(4)	0.0135(11)
Indium ($Z=49$), At. Wt.=114.818(3), $\sigma_{\gamma}^Z = 272(8)$				¹¹⁵ In	410.433(11)	0.69(5)	0.0182(13)
¹¹⁵In	22.796(7)	7(3)	0.18(8)	¹¹⁵In	416.86(3)d	43.0(18)	1.13[30%]
¹¹⁵In	60.9160(10)	15.8(11)	0.42(3)	¹¹⁵ In	422.213(11)	1.70(13)	0.045(3)
¹¹⁵ In	76.7580(20)	0.41(3)	0.0108(8)	¹¹⁵In	433.723(8)	6.0(4)	0.158(11)
¹¹⁵ In	84.3080(20)	1.32(9)	0.0348(24)	¹¹⁵ In	443.229(13)	0.58(4)	0.0153(11)
¹¹⁵In	85.5690(20)	22.1(16)	0.58(4)	¹¹⁵ In	447.531(11)	0.39(3)	0.0103(8)
¹¹⁵ In	95.380(4)	1.0(4)	0.026(11)	¹¹⁵In	471.349(11)	4.3(3)	0.113(8)
¹¹⁵In	96.036(5)	11.4(14)	0.30(4)	¹¹⁵ In	475.906(10)	1.88(13)	0.050(3)
¹¹⁵In	96.062(3)	24.6(18)	0.65(5)	¹¹⁵ In	489.314(10)	0.63(5)	0.0166(13)
¹¹⁵ In	112.4540(20)	1.38(9)	0.0364(24)	¹¹⁵ In	490.374(12)	0.80(11)	0.021(3)
¹¹⁵ In	114.997(3)	0.47(3)	0.0124(8)	¹¹⁵In	492.532(11)	3.31(24)	0.087(6)
¹¹⁵In	126.3720(20)	4.0(3)	0.106(8)	¹¹⁵ In	497.670(19)	0.67(5)	0.0177(13)
¹¹⁵In	138.326(8)d	5.11(18)	0.135[30%]	¹¹⁵ In	499.875(8)	0.37(3)	0.0098(8)
¹¹⁵ In	140.4560(20)	1.58(11)	0.042(3)	¹¹⁵ In	515.661(8)	0.60(4)	0.0158(11)
¹¹⁵ In	141.1700(20)	2.63(18)	0.069(5)	¹¹⁵ In	517.957(20)	2.8(4)	0.074(11)
¹¹⁵ In	149.6700(20)	0.69(5)	0.0182(13)	¹¹⁵ In	518.119(12)	3.15(22)	0.083(6)
¹¹⁵ In	155.272(3)	2.48(18)	0.065(5)	¹¹⁵ In	521.501(9)	1.97(14)	0.052(4)
¹¹⁵ In	159.932(4)	1.07(7)	0.0282(18)	¹¹⁵ In	540.382(8)	0.60(4)	0.0158(11)
¹¹⁵In	162.393(3)d	15.8(8)	0.417[100%]	¹¹⁵ In	548.720(9)	2.01(14)	0.053(4)
¹¹⁵ In	163.802(8)	0.67(5)	0.0177(13)	¹¹⁵ In	555.47(11)	0.7(5)	0.018(13)
¹¹⁵In	171.059(5)	3.44(25)	0.091(7)	¹¹⁵ In	556.169(8)	1.6(9)	0.042(24)
¹¹⁵In	173.886(6)	4.1(3)	0.108(8)	¹¹⁵In	556.845(21)	4.7(3)	0.124(8)
¹¹⁵ In	175.066(4)	1.12(7)	0.0296(18)	¹¹⁵ In	560.095(9)	0.85(5)	0.0224(13)
¹¹⁵In	186.2100(20)	26.6(18)	0.70(5)	¹¹⁵ In	567.596(20)	0.94(7)	0.0248(18)
¹¹⁵ In	196.738(5)	0.89(7)	0.0235(18)	¹¹⁵ In	577.523(18)	1.92(14)	0.051(4)
¹¹⁵ In	202.602(3)	2.70(20)	0.071(5)	¹¹⁵ In	602.36(4)	2.86(20)	0.075(5)
¹¹⁵ In	213.625(12)	0.64(5)	0.0169(13)	¹¹⁵In	608.422(11)	3.51(25)	0.093(7)
¹¹⁵ In	234.618(11)	0.71(25)	0.019(7)	¹¹⁵ In	622.57(11)	0.83(5)	0.0219(13)
¹¹⁵In	235.275(4)	4.9(3)	0.129(8)	¹¹⁵ In	633.740(11)	1.54(11)	0.041(3)
¹¹⁵ In	240.30(3)	0.44(3)	0.0116(8)	¹¹⁵ In	634.288(9)	1.68(13)	0.044(3)
¹¹⁵ In	267.960(20)	0.52(4)	0.0137(11)	¹¹⁵ In	647.72(8)	1.18(9)	0.0311(24)
¹¹⁵In	272.9660(20)	33.1(24)	0.87(6)	¹¹⁵ In	654.95(7)	0.47(3)	0.0124(8)
¹¹⁵In	284.914(4)	4.5(3)	0.119(8)	¹¹⁵ In	657.084(11)	1.52(11)	0.040(3)
¹¹³ In	287.726(19)	0.20(5)	0.0053(13)	¹¹⁵ In	662.115(10)	0.44(3)	0.0116(8)
¹¹⁵ In	290.952(15)	2.55(18)	0.067(5)	¹¹⁵ In	693.29(9)	1.83(13)	0.048(3)
¹¹⁵ In	293.393(15)	0.40(16)	0.011(4)	¹¹⁵ In	706.21(10)	0.40(9)	0.0106(24)
¹¹⁵ In	293.644(14)	1.38(11)	0.036(3)	¹¹⁵ In	746.978(9)	0.71(5)	0.0187(13)
¹¹⁵ In	295.515(17)	2.86(20)	0.075(5)	¹¹⁵ In	771.01(8)	1.52(11)	0.040(3)
¹¹⁵In	298.664(3)	9.4(7)	0.248(18)	¹¹⁵ In	792.16(6)	1.34(9)	0.0354(24)
¹¹⁵ In	300.388(4)	0.45(3)	0.0119(8)	¹¹⁵ In	807.897(25)	0.44(3)	0.0116(8)
¹¹⁵ In	305.108(8)	1.30(9)	0.0343(24)	¹¹⁵In	818.70(20)d	17.8(7)	0.470[30%]
¹¹⁵ In	315.053(12)	0.69(5)	0.0182(13)	¹¹⁵ In	819.04(11)	2.59(18)	0.068(5)
¹¹⁵ In	318.48(4)	0.60(4)	0.0158(11)	¹¹⁵ In	847.54(8)	2.15(16)	0.057(4)
¹¹⁵ In	320.895(8)	2.30(16)	0.061(4)	¹¹⁵ In	992.10(10)	0.91(7)	0.0240(18)
¹¹⁵ In	321.653(18)	0.7(3)	0.018(8)	¹¹⁵In	1097.30(20)d	87.3(17)	2.30[30%]
¹¹⁵In	335.450(10)	9.1(7)	0.240(18)	¹¹⁵ In	1293.54(15)d	131(3)	3.46[30%]
¹¹⁵ In	337.687(8)	2.52(18)	0.067(5)	¹¹⁵ In	1507.40(20)d	15.5(5)	0.409[30%]
¹¹⁵ In	339.15(4)	0.47(11)	0.012(3)	¹¹⁵ In	1753.8(6)d	3.82(12)	0.101[30%]

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹¹⁵ In	2112.1(4)d	24.1(7)	0.636[30%]	¹¹⁵ Sn	1546.40(6)	0.00140(15)	3.6(4)E-5
¹¹⁵ In	5333.54(18)	0.89(7)	0.0235(18)	¹¹⁵ Sn	1550.71(18)	0.00170(16)	4.3(4)E-5
¹¹⁵ In	5347.4(6)	0.362(25)	0.0096(7)	¹¹⁵ Sn	1650.72(6)	0.0021(3)	5.4(8)E-5
¹¹⁵ In	5358.9(5)	0.51(4)	0.0135(11)	¹¹⁸ Sn	1695.0(3)	0.00138(22)	3.5(6)E-5
¹¹⁵ In	5410.56(19)	0.53(4)	0.0140(11)	¹¹⁵ Sn	1702.67(3)	0.00169(17)	4.3(4)E-5
¹¹⁵ In	5891.89(17)	2.10(14)	0.055(4)	¹¹⁵ Sn	1711.17(7)	0.00151(19)	3.9(5)E-5
Tin (Z=50), At.Wt.=118.710(7), $\sigma_{\gamma}^Z = 0.54(5)$				¹¹⁵ Sn	1886.09(7)	0.0026(3)	6.6(8)E-5
¹²⁰ Sn	60.66(15)	0.0052(7)	1.33(18)E-4	¹¹⁵ Sn	1900.72(5)	0.0025(3)	6.4(8)E-5
¹²² Sn	125.80(7)	0.00178(9)	4.54(23)E-5	¹¹⁵ Sn	1926.02(19)	0.0014(6)	3.6(15)E-5
¹¹⁶ Sn	158.65(6)	0.0145(3)	0.000370(8)	¹¹⁵ Sn	1934.93(18)	0.0027(4)	6.9(10)E-5
¹²⁴ Sn	187.67(7)	0.00363(12)	9.3(3)E-5	¹¹⁵ Sn	1975.73(18)	0.0016(3)	4.1(8)E-5
¹²⁴ Sn	331.90(20)d	0.00830(20)	2.12E-4[77%]	¹¹⁵ Sn	2042.74(10)	0.0067(4)	1.71(10)E-4
¹¹⁵ Sn	416.99(4)	0.00251(11)	6.4(3)E-5	¹¹⁵ Sn	2050.76(5)	0.0025(4)	6.4(10)E-5
¹¹⁵ Sn	463.242(17)	0.0128(3)	0.000327(8)	¹¹⁵ Sn	2077.80(8)	0.0016(6)	4.1(15)E-5
¹¹⁷ Sn	528.85(6)	0.00425(14)	1.08(4)E-4	¹¹⁹ Sn	2097.01(9)	0.0048(3)	1.23(8)E-4
¹¹⁶ Sn	552.90(9)	0.00137(13)	3.5(3)E-5	¹¹⁵ Sn	2112.302(16)	0.0152(5)	0.000388(13)
¹¹⁹ Sn	703.87(7)	0.0078(3)	1.99(8)E-4	¹¹⁵ Sn	2148.03(5)	0.0021(4)	5.4(10)E-5
¹¹⁵ Sn	733.89(3)	0.00925(21)	2.36(5)E-4	¹¹⁵ Sn	2211.69(8)	0.0018(6)	4.6(15)E-5
¹¹⁷ Sn	813.26(7)	0.0071(3)	1.81(8)E-4	¹¹⁵ Sn	2220.00(23)	0.0019(5)	4.9(13)E-5
¹¹⁵ Sn	818.721(14)	0.0128(4)	0.000327(10)	¹¹⁵ Sn	2225.40(3)	0.0082(5)	2.09(13)E-4
¹¹⁷ Sn	827.37(8)	0.00361(23)	9.2(6)E-5	¹¹⁵ Sn	2244.19(6)	0.0029(10)	7(3)E-5
¹¹⁶ Sn	861.39(10)	0.00191(19)	4.9(5)E-5	¹¹⁹ Sn	2355.3	1.80E-03	4.60E-05
¹²⁰ Sn	869.38(8)	0.00320(22)	8.2(6)E-5	¹¹⁹ Sn	2420.83(15)	0.0029(3)	7.4(8)E-5
¹¹⁸ Sn	897.28(8)	0.00368(21)	9.4(5)E-5	¹¹⁵ Sn	2585.57(3)	0.0047(4)	1.20(10)E-4
¹²⁰ Sn	908.89(8)	0.00307(19)	7.8(5)E-5	¹¹⁷ Sn	2677.47(20)	0.0022(3)	5.6(8)E-5
¹²² Sn	920.87(7)	0.00404(21)	1.03(5)E-4	¹¹⁵ Sn	2707.43(6)	0.0024(6)	6.1(15)E-5
¹¹⁸ Sn	920.87(7)	0.00404(21)	1.03(5)E-4	¹¹⁷ Sn	2738.1	2.00E-03	5.10E-05
¹¹⁹ Sn	925.90(6)	0.0097(3)	2.48(8)E-4	¹¹⁵ Sn	2843.82(5)	0.0032(4)	8.2(10)E-5
¹²⁰ Sn	925.90(6)	0.0097(3)	2.48(8)E-4	¹¹⁵ Sn	2907.53(18)	0.0027(5)	6.9(13)E-5
¹¹⁵ Sn	931.819(23)	0.0111(3)	0.000283(8)	¹¹⁵ Sn	2960.03(4)	0.0023(3)	5.9(8)E-5
¹²⁰ Sn	943.20(12)	0.00150(17)	3.8(4)E-5	¹¹⁵ Sn	2985.00(25)	0.0025(8)	6.4(20)E-5
¹¹⁵ Sn	972.619(17)	0.0158(5)	0.000403(13)	¹¹⁵ Sn	3088.55(5)	0.00184(19)	4.7(5)E-5
¹¹⁹ Sn	988.67(7)	0.00668(22)	1.71(6)E-4	¹¹⁵ Sn	3330.6(4)	0.0016(5)	4.1(13)E-5
¹¹⁶ Sn	1004.49(8)	0.00388(18)	9.9(5)E-5	¹¹⁵ Sn	3333.75(5)	0.0061(5)	1.56(13)E-4
¹²⁰ Sn	1041.60(14)	0.00189(20)	4.8(5)E-5	¹¹⁵ Sn	3658.30(17)	0.0022(4)	5.6(10)E-5
¹¹⁷ Sn	1050.66(9)	0.00293(22)	7.5(6)E-5	¹¹⁵ Sn	4013.00(11)	0.00169(16)	4.3(4)E-5
¹¹⁸ Sn	1065.17(13)	0.00214(21)	5.5(5)E-5	¹¹⁵ Sn	4392.56(8)	0.00148(16)	3.8(4)E-5
¹¹⁷ Sn	1095.18(10)	0.0067(3)	1.71(8)E-4	¹¹⁵ Sn	4695.80(8)	0.0031(3)	7.9(8)E-5
¹¹⁵ Sn	1097.323(18)	0.0039(5)	9.96(13)E-5	¹¹⁵ Sn	4780.1(4)	0.0048(5)	1.23(13)E-4
¹²⁰ Sn	1101.25(16)	0.00322(25)	8.2(6)E-5	¹¹⁵ Sn	4809.43(9)	0.00165(16)	4.2(4)E-5
¹¹⁵ Sn	1115.15(4)	0.00150(16)	3.8(4)E-5	¹¹⁵ Sn	5173.5(7)	0.0016(4)	4.1(10)E-5
¹¹⁵ Sn	1118.95(5)	0.00155(22)	4.0(6)E-5	¹¹⁵ Sn	5361.91(6)	0.0043(4)	1.10(10)E-4
¹¹⁹ Sn	1171.28(6)	0.0879(13)	0.00224(3)	¹¹⁵ Sn	5423.57(11)	0.00188(21)	4.8(5)E-5
¹¹⁷ Sn	1173.66(8)	0.0050(3)	1.28(8)E-4	¹¹⁵ Sn	5449.51(5)	0.00191(19)	4.9(5)E-5
¹¹⁹ Sn	1184.19(8)	0.0051(3)	1.30(8)E-4	¹¹⁵ Sn	5562.35(6)	0.0021(5)	5.4(13)E-5
¹¹⁵ Sn	1200.56(12)	0.00163(22)	4.2(6)E-5	¹¹⁵ Sn	5904.65(6)	0.00223(17)	5.7(4)E-5
¹¹⁵ Sn	1202.70(12)	0.0022(3)	5.6(8)E-5	¹¹⁵ Sn	6229.57(6)	0.00159(16)	4.1(4)E-5
¹¹⁷ Sn	1229.64(6)	0.0673(13)	0.00172(3)	¹¹⁵ Sn	6335.30(12)	0.0023(3)	5.9(8)E-5
¹¹⁸ Sn	1249.62(7)	0.0052(3)	1.33(8)E-4	¹¹⁵ Sn	6335.89(5)	0.0014(3)	3.6(8)E-5
¹¹⁵ Sn	1252.119(23)	0.00348(19)	8.9(5)E-5	¹¹⁵ Sn	6603.27(4)	0.00168(19)	4.3(5)E-5
¹¹⁵ Sn	1291.99(3)	0.0050(10)	1.3(3)E-4	¹¹⁵ Sn	7450.97(3)	0.00137(14)	3.5(4)E-5
¹¹⁵ Sn	1293.591(15)	0.1340(21)	0.00342(5)	¹¹⁷ Sn	9327.5(11)	0.00204(20)	5.2(5)E-5
¹¹⁵ Sn	1356.846(20)	0.0075(3)	1.91(8)E-4	Antimony (Z=51), At.Wt.=121.760(1), $\sigma_{\gamma}^Z = 5.13(12)$			
¹¹⁹ Sn	1415.76(10)	0.00291(19)	7.4(5)E-5	¹²³ Sb	39.96	0.028(6)	0.00070(15)
¹¹⁷ Sn	1447.09(14)	0.00212(21)	5.4(5)E-5	¹²³ Sb	40.8040(10)	0.10(3)	0.0025(8)
¹¹⁷ Sn	1508.43(11)	0.0058(3)	1.48(8)E-4	¹²³ Sb	44.0910(10)	0.016(3)	0.00040(8)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹²¹ Sb	45.7330(10)	0.027(7)	0.00067(17)	¹²¹ Sb	286.5180(20)	0.034(3)	0.00085(8)
¹²¹ Sb	45.8480(10)	0.0076(21)	1.9(5)E-4	¹²³ Sb	288.0170(20)	0.018(6)	0.00045(15)
¹²¹ Sb	46.8350(10)	0.0082(25)	2.0(6)E-4	¹²³ Sb	313.938(3)	0.015(4)	0.00037(10)
¹²¹Sb	61.4130(10)	0.75(18)	0.019(5)	¹²³ Sb	313.990(6)	0.0317(24)	0.00079(6)
¹²¹ Sb	67.5940(10)	0.0082(22)	2.0(6)E-4	¹²³ Sb	322.1140(20)	0.036(3)	0.00090(8)
¹²¹Sb	71.4670(10)	0.095(22)	0.0024(6)	¹²¹ Sb	330.555(3)	0.058(3)	0.00144(8)
¹²¹ Sb	76.0590(10)	0.039(9)	0.00097(22)	¹²¹ Sb	331.3030(20)	0.011(3)	0.00027(8)
¹²¹Sb	78.0910(10)	0.48(11)	0.012(3)	¹²³ Sb	331.4600(20)	0.048(3)	0.00119(8)
¹²¹ Sb	86.7140(10)	0.0080(19)	2.0(5)E-4	¹²¹Sb	332.2860(10)	0.101(3)	0.00251(8)
¹²³Sb	87.601	0.212(8)	0.00528(20)	¹²³ Sb	334.980(3)	0.028(3)	0.00070(8)
¹²¹Sb	88.2690(10)	0.083(19)	0.0021(5)	¹²³ Sb	338.2980(20)	0.0142(16)	0.00035(4)
¹²³ Sb	88.3850(10)	0.0196(11)	0.00049(3)	¹²³ Sb	351.567(3)	0.0344(20)	0.00086(5)
¹²¹ Sb	101.5520(10)	0.028(6)	0.00070(15)	¹²¹ Sb	378.1380(20)	0.0500(18)	0.00124(5)
¹²³ Sb	103.6510(10)	0.063(5)	0.00157(12)	¹²³ Sb	384.533(3)	0.069(3)	0.00172(8)
¹²¹Sb	105.8160(10)	0.21(5)	0.0052(12)	¹²³ Sb	390.4960(20)	0.008(3)	2.0(8)E-4
¹²¹ Sb	113.8870(10)	0.014(3)	0.00035(8)	¹²¹ Sb	392.3340(20)	0.0121(25)	0.00030(6)
¹²¹Sb	114.8680(10)	0.31(7)	0.0077(17)	¹²³ Sb	410.285(7)	0.0127(20)	0.00032(5)
¹²¹ Sb	115.4210(10)	0.0110(25)	0.00027(6)	¹²¹ Sb	418.8240(20)	0.013(3)	0.00032(8)
¹²¹Sb	121.4970(10)	0.40(9)	0.0100(22)	¹²¹ Sb	419.925(5)	0.064(7)	0.00159(17)
¹²¹ Sb	124.0290(10)	0.037(9)	0.00092(22)	¹²¹ Sb	422.231(3)	0.022(5)	0.00055(12)
¹²³ Sb	133.8390(10)	0.056(4)	0.00139(10)	¹²¹ Sb	437.601(18)	0.0175(18)	0.00044(5)
¹²³ Sb	137.9190(10)	0.0207(10)	0.000515(25)	¹²³ Sb	441.9270(20)	0.0101(7)	0.000251(17)
¹²¹ Sb	141.4390(10)	0.060(14)	0.0015(4)	¹²¹ Sb	453.7470(20)	0.011(3)	0.00027(8)
¹²³ Sb	143.2080(10)	0.028(4)	0.00070(10)	¹²³ Sb	455.240(13)	0.0095(7)	2.36(17)E-4
¹²¹Sb	148.238	0.26(6)	0.0065(15)	¹²³ Sb	462.001(4)	0.0097(23)	2.4(6)E-4
¹²¹ Sb	148.6540(10)	0.016(4)	0.00040(10)	¹²³ Sb	466.964(3)	0.0115(23)	0.00029(6)
¹²¹ Sb	149.9720(10)	0.013(3)	0.00032(8)	¹²³ Sb	473.1350(20)	0.013(4)	0.00032(10)
¹²¹ Sb	153.3850(10)	0.0085(11)	2.1(3)E-4	¹²¹ Sb	485.35(4)	0.0212(21)	0.00053(5)
¹²³Sb	155.1780(10)	0.081(9)	0.00202(22)	¹²¹ Sb	491.215(5)	0.0344(16)	0.00086(4)
¹²¹ Sb	166.4510(10)	0.074(4)	0.00184(10)	¹²¹ Sb	501.034(3)	0.0076(21)	1.9(5)E-4
¹²³ Sb	167.6050(10)	0.046(4)	0.00114(10)	¹²³ Sb	501.151(4)	0.0129(10)	0.000321(25)
¹²¹ Sb	173.7880(20)	0.0192(11)	0.00048(3)	¹²¹ Sb	513.96(4)	0.0356(21)	0.00089(5)
¹²³ Sb	173.7990(10)	0.0171(9)	0.000426(22)	¹²¹ Sb	542.304(17)	0.0267(20)	0.00066(5)
¹²¹ Sb	177.4070(10)	0.0085(20)	2.1(5)E-4	¹²¹ Sb	546.056(10)	0.0313(20)	0.00078(5)
¹²¹ Sb	184.0480(10)	0.031(7)	0.00077(17)	¹²³ Sb	555.057(5)	0.021(5)	0.00052(12)
¹²³ Sb	185.1190(10)	0.0116(17)	0.00029(4)	¹²¹Sb	564.24(4)d	2.700(5)	0.06720[<0.1%]
¹²¹ Sb	194.0850(10)	0.0534(18)	0.00133(5)	¹²¹ Sb	564.4720(20)	0.0532(25)	0.00132(6)
¹²¹Sb	201.5950(10)	0.091(3)	0.00226(8)	¹²³ Sb	571.051(4)	0.0080(20)	2.0(5)E-4
¹²¹ Sb	204.5580(10)	0.0354(15)	0.00088(4)	¹²³ Sb	598.656(3)	0.055(4)	0.00137(10)
¹²¹ Sb	217.4170(20)	0.0118(8)	0.000294(20)	¹²¹ Sb	603.65(4)	0.019(3)	0.00047(8)
¹²¹ Sb	229.7080(10)	0.021(5)	0.00052(12)	¹²¹ Sb	631.82(3)	0.0586(16)	0.00146(4)
¹²¹ Sb	232.1880(10)	0.039(3)	0.00097(8)	¹²³ Sb	634.003(15)	0.0101(14)	0.00025(4)
¹²¹Sb	233.1690(10)	0.0996(24)	0.00248(6)	¹²³ Sb	647.012(13)	0.0113(24)	0.00028(6)
¹²³ Sb	246.3260(20)	0.0586(21)	0.00146(5)	¹²¹Sb	692.65(4)d	0.146(5)	0.00363[<0.1%]
¹²³ Sb	252.841(3)	0.0468(24)	0.00116(6)	¹²³ Sb	695.372(13)	0.008(3)	2.0(8)E-4
¹²¹ Sb	255.4980(10)	0.030(4)	0.00075(10)	¹²³ Sb	704.145(6)	0.009(3)	2.2(8)E-4
¹²¹ Sb	256.2270(10)	0.019(6)	0.00047(15)	¹²¹ Sb	718.52(4)	0.015(6)	0.00037(15)
¹²¹ Sb	261.6790(10)	0.0087(16)	2.2(4)E-4	¹²³ Sb	723.49(3)	0.016(3)	0.00040(8)
¹²³ Sb	265.629(6)	0.024(4)	0.00060(10)	¹²³ Sb	737.717(7)	0.012(3)	0.00030(8)
¹²³ Sb	269.3960(20)	0.0093(25)	2.3(6)E-4	¹²¹ Sb	746.861(17)	0.030(3)	0.00075(8)
¹²¹ Sb	272.2670(10)	0.019(3)	0.00047(8)	¹²³ Sb	763.44(3)	0.0169(24)	0.00042(6)
¹²¹ Sb	274.0010(10)	0.031(6)	0.00077(15)	¹²³ Sb	768.364(6)	0.0114(24)	0.00028(6)
¹²³ Sb	275.2780(20)	0.0135(8)	0.000336(20)	¹²³ Sb	775.395(7)	0.015(6)	0.00037(15)
¹²¹ Sb	275.4400(10)	0.0306(16)	0.00076(4)	¹²¹ Sb	796.61(4)	0.015(4)	0.00037(10)
¹²³ Sb	276.2670(20)	0.0095(5)	2.36(12)E-4	¹²¹ Sb	824.952(17)	0.040(3)	0.00100(8)
¹²¹Sb	282.6500(10)	0.274(7)	0.00682(17)	¹²¹ Sb	842.91(7)	0.017(10)	0.00042(25)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹²³ Sb	862.996(7)	0.009(4)	2.2(10)E-4	¹²³ Te	1720.15(5)	0.083(8)	0.00197(19)
121Sb	921.00(7)	0.075(4)	0.00187(10)	¹²⁴ Te	1851.37(10)	0.030(3)	0.00071(7)
¹²³ Sb	972.024(17)	0.015(3)	0.00037(8)	¹²³ Te	1918.71(7)	0.047(4)	0.00112(10)
¹²³ Sb	1020.942(10)	0.015(5)	0.00037(12)	¹²³ Te	1998.24(7)	0.035(4)	0.00083(10)
¹²³ Sb	5224.99(24)	0.0083(23)	2.1(6)E-4	¹²³ Te	2038.91(6)	0.064(7)	0.00152(17)
¹²³ Sb	5338.31(23)	0.0078(25)	1.9(6)E-4	¹²³ Te	2078.76(9)	0.031(3)	0.00074(7)
¹²³ Sb	5407.83(6)	0.014(5)	0.00035(12)	¹²³ Te	2091.21(8)	0.031(3)	0.00074(7)
¹²³ Sb	5446.51(5)	0.008(3)	2.0(8)E-4	¹²³ Te	2144.20(5)	0.034(4)	0.00081(10)
¹²¹ Sb	5558.3(4)	0.0149(21)	0.00037(5)	¹²³ Te	2214.56(10)	0.027(3)	0.00064(7)
¹²¹ Sb	5563.43(24)	0.0210(25)	0.00052(6)	¹²³ Te	2385.57(5)	0.034(4)	0.00081(10)
¹²¹ Sb	5600.4(3)	0.016(3)	0.00040(8)	¹²³ Te	2609.36(10)	0.039(4)	0.00093(10)
¹²³ Sb	5604.45(5)	0.012(3)	0.00030(8)	¹²³ Te	2746.92(5)	0.138(11)	0.0033(3)
¹²¹ Sb	5619.2(4)	0.015(3)	0.00037(8)	¹²³ Te	2783.15(10)	0.035(3)	0.00083(7)
¹²¹ Sb	5685.1(3)	0.0141(21)	0.00035(5)	¹²³ Te	2974.83(14)	0.025(3)	0.00059(7)
¹²¹ Sb	5775.50(25)	0.011(7)	0.00027(17)	¹²³ Te	3152.85(12)	0.026(3)	0.00062(7)
¹²¹ Sb	5787.62(25)	0.0093(17)	2.3(4)E-4	¹³⁰ Te	3347.35(10)	0.027(3)	0.00064(7)
¹²¹ Sb	5800.65(24)	0.0107(19)	0.00027(5)	¹²³ Te	3543.10(10)	0.039(4)	0.00093(10)
¹²³ Sb	5868.78(5)	0.034(4)	0.00085(10)	¹²⁸ Te	3721.75(12)	0.0209(21)	0.00050(5)
¹²¹ Sb	5885.19(9)	0.054(4)	0.00134(10)	¹²³ Te	5668.13(13)	0.037(3)	0.00088(7)
¹²¹ Sb	6009.58(8)	0.020(3)	0.00050(8)	¹²³ Te	5880.59(11)	0.034(4)	0.00081(10)
¹²³ Sb	6048.36(5)	0.018(3)	0.00045(8)	¹²³ Te	6211.61(12)	0.0262(25)	0.00062(6)
¹²³ Sb	6082.89(5)	0.018(3)	0.00045(8)	¹²⁶ Te	6287.6(4)	0.0023(7)	5.5(17)E-5
¹²¹ Sb	6163.62(7)	0.0121(18)	0.00030(5)	¹²³ Te	6322.95(8)	0.099(8)	0.00235(19)
¹²³ Sb	6335.72(5)	0.017(3)	0.00042(8)	¹²³ Te	7332.04(8)	0.027(4)	0.00064(10)
¹²³ Sb	6363.76(5)	0.025(4)	0.00062(10)	Iodine (Z=53), At.Wt.=126.90447(3), σ_γ^Z=6.20(20)			
¹²³ Sb	6379.80(5)	0.044(6)	0.00110(15)	127I	27.3620(10)	0.43(4)	0.0103(10)
¹²³ Sb	6456.54(5)	0.0077(20)	1.9(5)E-4	¹²⁷ I	42.767(4)	0.038(5)	0.00091(12)
¹²³ Sb	6467.40(5)	0.021(4)	0.00052(10)	127I	52.385(3)	0.167(19)	0.0040(5)
¹²¹ Sb	6494.91(7)	0.0076(24)	1.9(6)E-4	127I	58.1100(20)	0.28(4)	0.0067(10)
121Sb	6523.52(7)	0.075(3)	0.00187(8)	¹²⁷ I	58.734(4)	0.028(3)	0.00067(7)
¹²¹ Sb	6728.06(7)	0.044(4)	0.00110(10)	¹²⁷ I	67.120(3)	~0.1	~0.002
¹²¹ Sb	6744.74(7)	0.0090(16)	2.2(4)E-4	¹²⁷ I	68.256(4)	0.023(13)	0.0005(3)
¹²¹ Sb	6806.15(7)	0.0102(11)	0.00025(3)	¹²⁷ I	96.637(3)	0.0156(22)	0.00037(5)
Tellurium (Z=52), At.Wt.=127.60(3), σ_γ^Z=4.6(4)				¹²⁷ I	102.344(5)	0.0165(21)	0.00039(5)
¹³⁰ Te	149.716(5)d	0.0630(11)	0.00150[51%]	¹²⁷ I	106.2490(10)	0.066(5)	0.00158(12)
¹³⁰ Te	296.017(16)	0.029(3)	0.00069(7)	127I	124.2810(20)	0.180(13)	0.0043(3)
¹²³ Te	353.820(23)	0.100(8)	0.00237(19)	¹²⁷ I	126.989(3)	0.031(3)	0.00074(7)
¹²² Te	440.04(4)	0.0100(14)	2.4(3)E-4	¹²⁷ I	131.8640(20)	0.016(3)	0.00038(7)
¹²⁴ Te	443.53(4)	0.030(3)	0.00071(7)	¹²⁷ I	133.3940(10)	0.049(6)	0.00117(14)
¹²³ Te	557.46(4)	0.038(4)	0.00090(10)	127I	133.6110(10)	1.42(10)	0.0339(24)
¹²³ Te	602.729(17)	2.46(16)	0.058(4)	^{127I}	134.911(3)	0.015(11)	0.0004(3)
¹²³ Te	645.819(20)	0.263(22)	0.0062(5)	^{127I}	142.1370(20)	0.140(14)	0.0033(3)
¹²⁵ Te	666.3100(20)	0.045(5)	0.00107(12)	^{127I}	144.025(3)	0.0157(24)	0.00037(6)
¹²³ Te	709.18(6)	0.026(3)	0.00062(7)	^{127I}	147.105(3)	0.101(8)	0.00241(19)
¹²³ Te	713.79(3)	0.058(5)	0.00138(12)	127I	153.011(3)	0.209(14)	0.0050(3)
123Te	722.772(25)	0.52(4)	0.0123(10)	^{127I}	156.5060(20)	0.116(10)	0.00277(24)
¹²³ Te	790.74(3)	0.025(4)	0.00059(10)	127I	160.7570(10)	0.187(16)	0.0045(4)
¹²³ Te	1054.51(4)	0.063(5)	0.00150(12)	^{127I}	164.1390(20)	0.040(4)	0.00096(10)
¹²³ Te	1325.50(3)	0.074(6)	0.00176(14)	^{127I}	193.5630(20)	0.124(12)	0.0030(3)
¹²³ Te	1355.00(6)	0.025(3)	0.00059(7)	^{127I}	205.412(3)	0.0227(20)	0.00054(5)
¹²³ Te	1376.09(6)	0.039(4)	0.00093(10)	^{127I}	224.098(3)	0.07(3)	0.0017(7)
¹²³ Te	1436.55(3)	0.098(9)	0.00233(21)	^{127I}	231.245(3)	0.017(4)	0.00041(10)
¹²³ Te	1461.82(13)	0.028(7)	0.00066(17)	^{127I}	235.900(4)	0.028(3)	0.00067(7)
¹²³ Te	1488.88(5)	0.120(9)	0.00285(21)	^{127I}	248.7410(20)	0.11(4)	0.0026(10)
¹²³ Te	1579.50(8)	0.072(10)	0.00171(24)	^{127I}	251.534(5)	0.025(3)	0.00060(7)
¹²³ Te	1691.06(6)	0.073(7)	0.00173(17)	^{127I}	255.517(5)	0.028(3)	0.00067(7)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹²⁷ I	259.040(4)	0.0251(24)	0.00060(6)	¹²⁹ Xe	586.17(5)	0.48(7)	0.0111(16)
¹²⁷ I	268.305(3)	0.080(8)	0.00191(19)	¹³¹ Xe	600.19(8)	0.52(4)	0.0120(9)
¹²⁷ I	282.611(12)	0.0193(20)	0.00046(5)	¹³⁶ Xe	600.99(8)	0.010(3)	2.3(7)E-4
¹²⁷ I	283.968(4)	0.028(3)	0.00067(7)	¹³¹ Xe	621.13(10)	0.085(8)	0.00196(18)
¹²⁷ I	291.511(7)	0.0172(21)	0.00041(5)	¹³¹Xe	630.29(4)	1.41(11)	0.0325(25)
¹²⁷ I	297.393(17)	0.0155(25)	0.00037(6)	¹³¹Xe	667.79(6)	6.7(5)	0.155(12)
¹²⁷I	301.906(5)	0.17(6)	0.0041(14)	¹²⁹ Xe	668.59(15)	0.17(9)	0.0039(21)
¹²⁷ I	310.419(6)	0.0166(18)	0.00040(4)	¹³¹ Xe	670.02(10)	0.22(3)	0.0051(7)
¹²⁷ I	314.349(4)	0.060(5)	0.00143(12)	¹³¹Xe	772.72(4)	1.78(14)	0.041(3)
¹²⁷ I	325.35(4)	0.020(3)	0.00048(7)	¹³¹ Xe	812.45(10)	0.082(8)	0.00189(18)
¹²⁷ I	330.801(5)	0.0146(21)	0.00035(5)	¹³¹ Xe	832.43(12)	0.108(15)	0.0025(4)
¹²⁷ I	344.758(7)	0.100(9)	0.00239(21)	¹³¹ Xe	889.54(8)	0.084(8)	0.00194(18)
¹²⁷ I	364.640(3)	0.0211(25)	0.00050(6)	¹³¹ Xe	954.65(12)	0.076(8)	0.00175(18)
¹²⁷ I	369.358(17)	0.0170(21)	0.00041(5)	¹³¹ Xe	984.54(9)	0.093(18)	0.0021(4)
¹²⁷ I	374.218(5)	0.041(7)	0.00098(17)	¹³¹ Xe	1028.86(6)	0.40(3)	0.0092(7)
¹²⁷ I	374.456(7)	0.028(6)	0.00067(14)	¹²⁹ Xe	1096.49(7)	0.087(12)	0.0020(3)
¹²⁷ I	385.447(5)	0.086(7)	0.00205(17)	¹³¹ Xe	1115.34(9)	0.149(20)	0.0034(5)
¹²⁷ I	388.911(5)	0.022(3)	0.00053(7)	¹²⁹ Xe	1122.33(10)	0.119(17)	0.0027(4)
¹²⁷ I	392.002(3)	0.045(14)	0.0011(3)	¹³¹ Xe	1136.13(7)	0.45(4)	0.0104(9)
¹²⁷ I	392.687(6)	0.028(9)	0.00067(21)	¹³¹ Xe	1140.84(11)	0.067(9)	0.00155(21)
¹²⁷ I	398.975(4)	0.018(3)	0.00043(7)	¹³¹ Xe	1171.29(6)	0.217(19)	0.0050(4)
¹²⁷ I	416.579(6)	0.065(5)	0.00155(12)	¹³¹ Xe	1298.09(7)	0.12(3)	0.0028(7)
¹²⁷ I	420.826(7)	0.139(18)	0.0033(4)	¹³¹Xe	1317.93(8)	0.89(7)	0.0205(16)
¹²⁷I	442.901(10)d	0.595(4)	0.0140(1)	¹²⁹ Xe	1482.06(9)	0.112(16)	0.0026(4)
¹²⁷ I	458.056(9)	0.0266(23)	0.00064(6)	¹³¹ Xe	1519.83(8)	0.131(25)	0.0030(6)
¹²⁷ I	502.607(18)	0.061(5)	0.00146(12)	¹³¹ Xe	1801.58(6)	0.272(22)	0.0063(5)
¹²⁷ I	528.91(9)	0.054(5)	0.00129(12)	¹³¹ Xe	1888.05(8)	0.225(23)	0.0052(5)
¹²⁷ I	557.43(4)	0.027(3)	0.00064(7)	¹³¹ Xe	1985.71(10)	0.54(5)	0.0125(12)
¹²⁷ I	4950.10(7)	0.037(10)	0.00088(24)	¹³¹ Xe	2713.93(10)	0.079(9)	0.00182(21)
¹²⁷ I	5018.648(17)	0.024(11)	0.0006(3)	¹³¹ Xe	3699.40(15)	0.082(16)	0.0019(4)
¹²⁷ I	5091.988(12)	0.015(7)	0.00036(17)	¹³¹ Xe	4734.85(17)	0.071(10)	0.00164(23)
¹²⁷ I	5096.357(17)	0.024(8)	0.00057(19)	¹³¹ Xe	4841.70(14)	0.107(15)	0.0025(4)
¹²⁷ I	5197.957(12)	0.032(14)	0.0008(3)	¹³¹ Xe	5078.91(18)	0.106(16)	0.0024(4)
¹²⁷ I	5298.245(12)	0.031(7)	0.00074(17)	¹²⁹ Xe	5956.18(18)	0.16(3)	0.0037(7)
¹²⁷ I	5463.453(12)	0.018(6)	0.00043(14)	¹³¹ Xe	6380.62(13)	0.21(3)	0.0048(7)
¹²⁷ I	5482.853(12)	0.018(13)	0.0004(3)	¹³¹Xe	6467.09(12)	1.33(19)	0.031(4)
¹²⁷ I	5524.28(5)	0.015(5)	0.00036(12)	Cesium (Z=55), At.Wt.=132.90545(2), $\sigma_{\gamma}^z = 30.3(11)$			
¹²⁷ I	5559.662(12)	0.044(22)	0.0011(5)	¹³³ Cs	11.2450(20)	0.142(7)	0.00324(16)
¹²⁷ I	5574.501(12)	0.021(5)	0.00050(12)	¹³³ Cs	17.2130(20)	0.110(18)	0.0025(4)
¹²⁷ I	5725.929(12)	0.020(13)	0.0005(3)	¹³³ Cs	38.6240(20)	0.080(12)	0.0018(3)
¹²⁷ I	6307.586(6)	0.024(8)	0.00057(19)	¹³³Cs	48.790(20)	0.345(10)	0.00787(23)
¹²⁷ I	6692.417(5)	0.037(8)	0.00088(19)	¹³³Cs	60.0300(10)	0.443(14)	0.0101(3)
Xenon (Z=54), At.Wt.=131.293(6), $\sigma_{\gamma}^z = 24(3)$				¹³³ Cs	67.2540(20)	0.088(5)	0.00201(11)
¹³¹ Xe	324.80(16)	0.09(5)	0.0021(12)	¹³³ Cs	73.5660(20)	0.117(19)	0.0027(4)
¹²⁴ Xe	335.46(16)	0.0054(12)	1.2(3)E-4	¹³³ Cs	74.0460(20)	0.14(3)	0.0032(7)
¹²⁸ Xe	403.1(3)	0.0106(23)	2.4(5)E-4	¹³³ Cs	87.2520(20)	0.107(4)	0.00244(9)
¹³⁰ Xe	404.8(3)	0.0096(23)	2.2(5)E-4	¹³³ Cs	93.1850(20)	0.043(3)	0.00098(7)
¹³⁶ Xe	455.490(3)d	0.00350(6)	8.08E-5[91 %]	¹³³Cs	113.7650(20)	0.777(15)	0.0177(3)
¹³¹ Xe	471.72(12)	0.19(3)	0.0044(7)	¹³³ Cs	114.3270(20)	0.05(3)	0.0011(7)
¹³¹ Xe	483.66(10)	0.55(4)	0.0127(9)	¹³³Cs	116.3740(20)	1.39(12)	0.032(3)
¹³¹ Xe	505.84(8)	0.40(3)	0.0092(7)	¹³³Cs	116.612(4)	1.44(12)	0.033(3)
¹²⁹ Xe	510.33(8)	0.33(7)	0.0076(16)	¹³³ Cs	117.1730(20)	0.04(3)	0.0009(7)
¹³¹ Xe	522.78(7)	0.273(22)	0.0063(5)	¹³³ Cs	118.3630(20)	0.230(7)	0.00524(16)
¹²⁹Xe	536.17(9)	1.71(24)	0.039(6)	¹³³Cs	120.588(3)	0.414(10)	0.00944(23)
¹³¹ Xe	546.95(11)	0.094(16)	0.0022(4)	¹³³Cs	127.5000(20)d	0.310(11)	0.0071(3)
¹³¹ Xe	570.13(7)	0.188(15)	0.0043(4)	¹³³Cs	130.2320(20)	1.410(21)	0.0322(5)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹³³ Cs	131.171(3)	0.054(5)	0.00123(11)	¹³³ Cs	384.290(5)	0.034(7)	0.00078(16)
¹³³ Cs	133.5860(20)	0.038(3)	0.00087(7)	¹³³ Cs	386.855(3)	0.163(9)	0.00372(21)
¹³³ Cs	137.7530(20)	0.030(4)	0.00068(9)	¹³³ Cs	391.3960(20)	0.080(7)	0.00182(16)
¹³³ Cs	142.7680(20)	0.073(4)	0.00166(9)	¹³³ Cs	393.535(5)	0.065(8)	0.00148(18)
¹³³ Cs	174.3040(20)	0.420(11)	0.00958(25)	¹³³ Cs	402.491(4)	0.051(10)	0.00116(23)
¹³³ Cs	176.4040(20)	2.47(4)	0.0563(9)	¹³³ Cs	405.484(4)	0.079(12)	0.0018(3)
¹³³ Cs	177.068(3)	0.098(16)	0.0022(4)	¹³³ Cs	408.483(7)	0.032(12)	0.0007(3)
¹³³ Cs	179.0180(20)	0.15(5)	0.0034(11)	¹³³ Cs	412.448(5)	0.051(13)	0.0012(3)
¹³³ Cs	180.0770(20)	0.087(7)	0.00198(16)	¹³³ Cs	417.277(4)	0.095(17)	0.0022(4)
¹³³ Cs	186.8400(20)	0.282(9)	0.00643(21)	¹³³ Cs	421.052(5)	0.086(8)	0.00196(18)
¹³³ Cs	189.8320(20)	0.093(10)	0.00212(23)	¹³³ Cs	422.491(6)	0.029(6)	0.00066(14)
¹³³ Cs	193.7250(20)	0.042(9)	0.00096(21)	¹³³ Cs	426.258(4)	0.041(7)	0.00093(16)
¹³³ Cs	194.724(3)	0.045(9)	0.00103(21)	¹³³ Cs	434.334(3)	0.066(7)	0.00150(16)
¹³³ Cs	198.3010(20)	1.100(19)	0.0251(4)	¹³³ Cs	438.9920(20)	0.140(9)	0.00319(21)
¹³³ Cs	200.847(4)	0.135(10)	0.00308(23)	¹³³ Cs	442.8430(20)	0.316(12)	0.0072(3)
¹³³ Cs	205.615(3)	1.560(25)	0.0356(6)	¹³³ Cs	444.465(7)	0.114(9)	0.00260(21)
¹³³ Cs	207.675(4)	0.093(6)	0.00212(14)	¹³³ Cs	450.2370(20)	0.07(3)	0.0016(7)
¹³³ Cs	209.5460(20)	0.073(6)	0.00166(14)	¹³³ Cs	450.345(3)	0.99(5)	0.0226(11)
¹³³ Cs	211.3190(10)	0.223(10)	0.00508(23)	¹³³ Cs	451.4250(20)	0.058(10)	0.00132(23)
¹³³ Cs	218.341(3)	0.309(9)	0.00705(21)	¹³³ Cs	454.0870(20)	0.056(11)	0.00128(25)
¹³³ Cs	219.7530(20)	0.344(9)	0.00784(21)	¹³³ Cs	458.357(6)	0.072(5)	0.00164(11)
¹³³ Cs	232.165(3)	0.125(9)	0.00285(21)	¹³³ Cs	461.180(5)	0.099(5)	0.00226(11)
¹³³ Cs	234.3340(20)	1.070(23)	0.0244(5)	¹³³ Cs	464.481(4)	0.095(5)	0.00217(11)
¹³³ Cs	245.8620(20)	0.740(15)	0.0169(3)	¹³³ Cs	479.624(6)	0.030(10)	0.00068(23)
¹³³ Cs	254.740(3)	0.069(7)	0.00157(16)	¹³³ Cs	485.038(3)	0.094(10)	0.00214(23)
¹³³ Cs	256.6210(20)	0.235(8)	0.00536(18)	¹³³ Cs	486.200(5)	0.08(3)	0.0018(7)
¹³³ Cs	261.1640(20)	0.401(11)	0.00914(25)	¹³³ Cs	487.388(4)	0.047(6)	0.00107(14)
¹³³ Cs	263.8260(20)	0.079(7)	0.00180(16)	¹³³ Cs	490.843(4)	0.042(10)	0.00096(23)
¹³³ Cs	268.987(3)	0.199(6)	0.00454(14)	¹³³ Cs	495.593(3)	0.077(11)	0.00176(25)
¹³³ Cs	271.3490(20)	0.127(15)	0.0029(3)	¹³³ Cs	502.840(3)	0.256(13)	0.0058(3)
¹³³ Cs	272.212(4)	0.069(12)	0.0016(3)	¹³³ Cs	508.077(3)	0.057(10)	0.00130(23)
¹³³ Cs	277.6310(20)	0.066(5)	0.00150(11)	¹³³ Cs	508.380(3)	0.053(10)	0.00121(23)
¹³³ Cs	279.648(3)	0.065(5)	0.00148(11)	¹³³ Cs	510.795(3)	1.54(3)	0.0351(7)
¹³³ Cs	284.987(3)	0.044(5)	0.00100(11)	¹³³ Cs	517.601(7)	0.028(21)	0.0006(5)
¹³³ Cs	293.295(3)	0.185(9)	0.00422(21)	¹³³ Cs	519.101(4)	0.349(18)	0.0080(4)
¹³³ Cs	295.431(3)	0.231(10)	0.00527(23)	¹³³ Cs	519.321(3)	0.086(14)	0.0020(3)
¹³³ Cs	302.463(3)	0.13(4)	0.0030(9)	¹³³ Cs	524.1500(20)	0.151(23)	0.0034(5)
¹³³ Cs	303.164(3)	0.055(6)	0.00125(14)	¹³³ Cs	525.356(4)	0.39(3)	0.0089(7)
¹³³ Cs	305.058(3)	0.061(7)	0.00139(16)	¹³³ Cs	525.592(3)	0.13(6)	0.0030(14)
¹³³ Cs	307.015(4)	1.45(3)	0.0331(7)	¹³³ Cs	526.072(4)	0.03(3)	0.0007(7)
¹³³ Cs	309.776(3)	0.237(9)	0.00540(21)	¹³³ Cs	528.409(6)	0.08(3)	0.0018(7)
¹³³ Cs	317.0720(20)	0.149(10)	0.00340(23)	¹³³ Cs	529.504(6)	0.519(23)	0.0118(5)
¹³³ Cs	329.060(3)	0.055(6)	0.00125(14)	¹³³ Cs	529.891(4)	~0.03	~0.0007
¹³³ Cs	338.027(6)	0.043(6)	0.00098(14)	¹³³ Cs	539.180(4)	0.360(11)	0.00821(25)
¹³³ Cs	345.358(5)	0.075(7)	0.00171(16)	¹³³ Cs	539.416(4)	0.18(7)	0.0041(16)
¹³³ Cs	347.148(7)	0.073(6)	0.00166(14)	¹³³ Cs	540.679(9)	0.134(8)	0.00306(18)
¹³³ Cs	347.152(4)	0.030(4)	0.00068(9)	¹³³ Cs	554.642(5)	0.206(9)	0.00470(21)
¹³³ Cs	349.846(3)	0.030(6)	0.00068(14)	¹³³ Cs	559.084(3)	0.076(10)	0.00173(23)
¹³³ Cs	356.157(4)	0.445(12)	0.0101(3)	¹³³ Cs	561.964(5)	0.130(10)	0.00296(23)
¹³³ Cs	356.345(3)	0.14(7)	0.0032(16)	¹³³ Cs	564.019(4)	0.040(8)	0.00091(18)
¹³³ Cs	365.8570(20)	0.04(3)	0.0009(7)	¹³³ Cs	567.483(4)	0.052(9)	0.00119(21)
¹³³ Cs	365.859(6)	0.103(6)	0.00235(14)	¹³³ Cs	570.825(3)	0.221(12)	0.0050(3)
¹³³ Cs	367.870(5)	0.173(8)	0.00394(18)	¹³³ Cs	574.574(4)	0.061(12)	0.0014(3)
¹³³ Cs	371.7380(20)	0.131(7)	0.00299(16)	¹³³ Cs	576.060(4)	0.073(14)	0.0017(3)
¹³³ Cs	377.311(5)	0.310(9)	0.00707(21)	¹³³ Cs	576.296(3)	0.038(21)	0.0009(5)
¹³³ Cs	381.628(5)	0.066(7)	0.00150(16)	¹³³ Cs	579.131(4)	0.038(10)	0.00087(23)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹³³ Cs	584.180(3)	0.027(14)	0.0006(3)	¹³³ Cs	1072.547(6)	0.066(19)	0.0015(4)
¹³³ Cs	591.680(5)	0.031(8)	0.00071(18)	¹³³ Cs	1077.557(6)	0.209(12)	0.0048(3)
¹³³ Cs	601.381(5)	0.080(9)	0.00182(21)	¹³³ Cs	1077.794(5)	0.088(12)	0.0020(3)
¹³³ Cs	601.775(5)	0.034(11)	0.00078(25)	¹³³ Cs	1102.473(5)	0.047(8)	0.00107(18)
¹³³ Cs	603.457(5)	0.061(8)	0.00139(18)	¹³³ Cs	1114.65(21)	0.049(10)	0.00112(23)
¹³³ Cs	610.896(4)	0.068(6)	0.00155(14)	¹³³ Cs	1118.04(16)	0.069(9)	0.00157(21)
¹³³ Cs	623.831(9)	0.055(8)	0.00125(18)	¹³³ Cs	1209.54(11)	0.138(11)	0.00315(25)
¹³³ Cs	628.595(4)	0.097(7)	0.00221(16)	¹³³ Cs	5493.52(23)	0.230(19)	0.0052(4)
¹³³ Cs	633.809(6)	0.112(7)	0.00255(16)	¹³³Cs	5505.46(20)	0.333(22)	0.0076(5)
¹³³Cs	645.453(5)	0.248(13)	0.0057(3)	¹³³Cs	5572.00(25)	0.249(20)	0.0057(5)
¹³³ Cs	646.195(3)	0.064(11)	0.00146(25)	¹³³ Cs	5625.091(17)	0.111(13)	0.0025(3)
¹³³ Cs	648.511(4)	0.233(13)	0.0053(3)	¹³³Cs	5637.056(17)	0.277(21)	0.0063(5)
¹³³ Cs	663.171(4)	0.155(9)	0.00353(21)	¹³³ Cs	5728.747(17)	0.087(16)	0.0020(4)
¹³³ Cs	663.407(3)	0.07(3)	0.0016(7)	¹³³ Cs	5748.392(17)	0.146(15)	0.0033(3)
¹³³ Cs	666.017(4)	0.089(8)	0.00203(18)	¹³³ Cs	5790.920(17)	0.137(13)	0.0031(3)
¹³³ Cs	678.271(5)	0.078(13)	0.0018(3)	¹³³ Cs	5802.823(18)	0.120(13)	0.0027(3)
¹³³ Cs	681.247(4)	0.110(24)	0.0025(6)	¹³³ Cs	5899.368(17)	0.116(12)	0.0026(3)
¹³³ Cs	682.562(4)	0.12(3)	0.0027(7)	¹³³ Cs	5914.935(17)	0.047(8)	0.00107(18)
¹³³ Cs	688.625(4)	0.058(10)	0.00132(23)	¹³³ Cs	5949.884(22)	0.045(10)	0.00103(23)
¹³³ Cs	691.434(5)	0.030(10)	0.00068(23)	¹³³ Cs	5975.068(17)	0.027(10)	0.00062(23)
¹³³ Cs	692.670(3)	0.037(6)	0.00084(14)	¹³³ Cs	5978.636(17)	0.099(14)	0.0023(3)
¹³³ Cs	695.340(6)	0.039(10)	0.00089(23)	¹³³ Cs	6051.426(17)	0.240(20)	0.0055(5)
¹³³ Cs	701.38(21)	0.036(10)	0.00082(23)	¹³³ Cs	6138.534(17)	0.061(8)	0.00139(18)
¹³³ Cs	703.290(5)	0.043(10)	0.00098(23)	¹³³ Cs	6149.955(17)	0.038(6)	0.00087(14)
¹³³ Cs	708.417(5)	0.220(11)	0.00502(25)	¹³³Cs	6175.412(17)	0.252(16)	0.0057(4)
¹³³ Cs	708.646(4)	0.105(14)	0.0024(3)	¹³³ Cs	6189.235(17)	0.191(14)	0.0044(3)
¹³³ Cs	712.268(5)	0.113(9)	0.00258(21)	¹³³ Cs	6197.392(17)	0.035(8)	0.00080(18)
¹³³ Cs	722.343(5)	0.116(11)	0.00265(25)	¹³³ Cs	6247.267(17)	0.038(6)	0.00087(14)
¹³³ Cs	730.033(4)	0.045(8)	0.00103(18)	¹³³ Cs	6307.046(17)	0.044(10)	0.00100(23)
¹³³ Cs	741.277(4)	0.071(9)	0.00162(21)	¹³³ Cs	6320.400(17)	0.050(8)	0.00114(18)
¹³³ Cs	770.544(5)	0.104(11)	0.00237(25)	¹³³ Cs	6439.794(16)	0.082(8)	0.00187(18)
¹³³ Cs	799.668(4)	0.075(10)	0.00171(23)	¹³³ Cs	6514.114(16)	0.044(7)	0.00100(16)
¹³³ Cs	799.904(4)	0.029(6)	0.00066(14)	¹³³ Cs	6697.590(16)	0.224(17)	0.0051(4)
¹³³ Cs	814.739(6)	0.056(13)	0.0013(3)	¹³³ Cs	6714.802(16)	0.090(11)	0.00205(25)
¹³³ Cs	820.763(7)	0.059(11)	0.00135(25)	¹³³ Cs	6831.169(16)	0.035(4)	0.00080(9)
¹³³ Cs	852.574(5)	0.034(8)	0.00078(18)	Barium (Z=56), At.Wt.=137.327(7), $\sigma_{\gamma}^Z=1.18(7)$			
¹³³ Cs	861.766(7)	0.070(9)	0.00160(21)	¹³⁵ Ba	66.32(16)	0.0067(6)	1.48(13)E-4
¹³³ Cs	868.99(10)	0.140(11)	0.00319(25)	¹³⁵ Ba	87.08(13)	0.0093(6)	2.05(13)E-4
¹³³ Cs	869.099(4)	0.140(11)	0.00319(25)	¹³⁵ Ba	157.3(4)	0.0057(11)	1.26(24)E-4
¹³³ Cs	880.343(4)	0.114(14)	0.0026(3)	¹³⁵ Ba	158.58(12)	0.0077(4)	1.70(9)E-4
¹³³ Cs	894.509(7)	0.103(12)	0.0023(3)	¹³⁸Ba	165.8570(10)d	0.074(8)	0.00163[21%]
¹³³ Cs	894.808(7)	0.052(16)	0.0012(4)	¹³⁷ Ba	191.65(10)	0.0081(3)	1.79(7)E-4
¹³³ Cs	901.360(5)	0.053(11)	0.00121(25)	¹³⁴ Ba	220.969(17)	0.0067(5)	1.48(11)E-4
¹³³ Cs	904.288(4)	0.040(11)	0.00091(25)	¹³⁵ Ba	273.77(11)	0.0079(5)	1.74(11)E-4
¹³³ Cs	911.784(7)	0.177(14)	0.0040(3)	¹³⁶Ba	283.58(6)	0.0404(12)	0.00089(3)
¹³³ Cs	912.021(7)	0.057(8)	0.00130(18)	¹³⁷ Ba	325.11(7)	0.00368(19)	8.1(4)E-5
¹³³ Cs	930.112(15)	0.126(9)	0.00287(21)	¹³⁷ Ba	364.32(13)	0.00407(20)	9.0(4)E-5
¹³³ Cs	931.72(15)	0.073(8)	0.00166(18)	¹³⁷ Ba	408.88(7)	0.0096(6)	2.12(13)E-4
¹³³ Cs	935.69(11)	0.130(9)	0.00296(21)	¹³⁸Ba	454.73(5)	0.0853(22)	0.00188(5)
¹³³ Cs	966.454(5)	0.168(13)	0.0038(3)	¹³⁷Ba	462.78(4)	0.0660(16)	0.00146(4)
¹³³ Cs	985.863(5)	0.078(12)	0.0018(3)	¹³⁶ Ba	480.41(6)	0.00350(16)	7.7(4)E-5
¹³³ Cs	986.100(5)	0.027(9)	0.00062(21)	¹³⁴ Ba	480.543(24)	0.00320(20)	7.1(4)E-5
¹³³ Cs	998.502(7)	0.103(11)	0.00235(25)	¹³⁷ Ba	516.76(8)	0.0083(6)	1.83(13)E-4
¹³³ Cs	1009.2(5)	0.05(3)	0.0011(7)	¹³⁷ Ba	546.95(5)	0.00604(23)	1.33(5)E-4
¹³³ Cs	1028.394(7)	0.038(15)	0.0009(3)	¹³⁸Ba	627.29(5)	0.294(6)	0.00649(13)
¹³³ Cs	1034.519(4)	0.028(8)	0.00064(18)	¹³⁸ Ba	665.98(9)	0.0053(3)	1.17(7)E-4
¹³³ Cs	1045.251(7)	0.120(11)	0.00274(25)	-----			

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹³⁵ Ba	671.60(9)	0.0045(3)	9.9(7)E-5	¹³⁸ Ba	2566.0(11)	0.009(5)	2.0(11)E-4
¹³⁵ Ba	732.49(7)	0.0238(8)	0.000525(18)	¹³⁷ Ba	2582.87(8)	0.0033(3)	7.3(7)E-5
¹³⁵ Ba	746.6(4)	0.0031(3)	6.8(7)E-5	¹³⁸ Ba	2593.42(11)	0.0187(8)	0.000413(18)
¹³⁷ Ba	754.03(7)	0.0067(3)	1.48(7)E-4	¹³⁷ Ba	2639.20(7)	0.0184(16)	0.00041(4)
¹³⁵ Ba	760.31(11)	0.0073(5)	1.61(11)E-4	¹³⁶ Ba	2662.66(5)	0.00401(16)	8.8(4)E-5
¹³⁵Ba	818.514(12)	0.212(4)	0.00468(9)	¹³⁷ Ba	2806.29(11)	0.0032(4)	7.1(9)E-5
¹³⁷ Ba	871.66(6)	0.0124(4)	0.000274(9)	¹³⁵ Ba	2976.64(17)	0.0181(7)	0.000399(15)
¹³⁵ Ba	880.01(17)	0.0042(5)	9.3(11)E-5	¹³⁵ Ba	3045.19(23)	0.00336(16)	7.4(4)E-5
¹³⁵ Ba	981.61(9)	0.0040(3)	8.8(7)E-5	¹³⁷ Ba	3049.93(12)	0.0037(3)	8.2(7)E-5
¹³⁷ Ba	1009.73(5)	0.0167(5)	0.000369(11)	¹³⁷ Ba	3099.89(14)	0.0032(5)	7.1(11)E-5
¹³⁷ Ba	1041.42(8)	0.00422(22)	9.3(5)E-5	¹³⁵ Ba	3338.60(10)	0.0090(5)	1.99(11)E-4
¹³⁸Ba	1047.73(6)	0.0319(10)	0.000704(22)	¹³⁷ Ba	3435.5(4)	0.0043(5)	9.5(11)E-5
¹³⁵ Ba	1048.0730(20)	0.025(4)	0.00055(9)	¹³⁷ Ba	3503.94(17)	0.0046(4)	1.02(9)E-4
¹³⁸ Ba	1103.43(8)	0.0044(4)	9.7(9)E-5	¹³⁸Ba	3641.12(9)	0.0562(16)	0.00124(4)
¹³⁷ Ba	1147.11(7)	0.0150(5)	0.000331(11)	¹³⁷ Ba	3643.59(3)	0.0033(17)	7(4)E-5
¹³⁵ Ba	1235.29(12)	0.0148(7)	0.000327(15)	¹³⁴ Ba	3676.5(5)	0.0045(3)	9.9(7)E-5
¹³⁵Ba	1261.52(7)	0.095(5)	0.00210(11)	¹³⁷ Ba	3739.50(12)	0.0042(5)	9.3(11)E-5
¹³⁷ Ba	1264.54(10)	0.00352(22)	7.8(5)E-5	¹³⁷ Ba	3965.98(13)	0.00342(22)	7.5(5)E-5
¹³⁵ Ba	1310.21(9)	0.0094(7)	2.07(15)E-4	¹³⁷ Ba	4025.52(14)	0.0038(4)	8.4(9)E-5
¹³⁷ Ba	1343.53(8)	0.0087(4)	1.92(9)E-4	¹³⁷ Ba	4025.70(14)	0.0038(8)	8.4(18)E-5
¹³⁵ Ba	1404.08(9)	0.0051(5)	1.13(11)E-4	¹³⁷ Ba	4083.64(16)	0.0067(6)	1.48(13)E-4
¹³⁴ Ba	1415.30(19)	0.0067(5)	1.48(11)E-4	¹³⁸Ba	4095.84(9)	0.155(4)	0.00342(9)
¹³⁸ Ba	1420.41(9)	0.0090(5)	1.99(11)E-4	¹³⁷ Ba	4103.50(19)	0.0032(5)	7.1(11)E-5
¹³⁷Ba	1435.77(4)	0.308(7)	0.00680(15)	¹³⁷ Ba	4114.45(19)	0.00329(24)	7.3(5)E-5
¹³⁷Ba	1444.91(5)	0.0801(20)	0.00177(4)	¹³⁷ Ba	4166.05(12)	0.0052(3)	1.15(7)E-4
¹³⁷ Ba	1495.58(9)	0.0104(7)	2.30(15)E-4	¹³⁶ Ba	4242.98(8)	0.0087(10)	1.92(22)E-4
¹³⁵ Ba	1537.0(5)	0.0049(13)	1.1(3)E-4	¹³⁷ Ba	4251.82(13)	0.0057(4)	1.26(9)E-4
¹³⁵ Ba	1551.01(6)	0.0231(9)	0.000510(20)	¹³⁷ Ba	4279.55(14)	0.0039(5)	8.6(11)E-5
¹³⁷ Ba	1555.32(11)	0.00433(23)	9.6(5)E-5	¹³⁷ Ba	4280.25(16)	0.0038(3)	8.4(7)E-5
¹³⁸ Ba	1558.16(8)	0.0078(5)	1.72(11)E-4	¹³⁷ Ba	4288.15(14)	0.0059(3)	1.30(7)E-4
¹³⁵ Ba	1572.12(18)	0.0055(10)	1.21(22)E-4	¹³⁷ Ba	4323.34(14)	0.0079(4)	1.74(9)E-4
¹³⁵ Ba	1581.46(6)	0.0096(7)	2.12(15)E-4	¹³⁷ Ba	4331.24(16)	0.0091(12)	2.0(3)E-4
¹³⁷ Ba	1614.18(11)	0.015(7)	0.00033(15)	¹³⁷ Ba	4331.94(14)	0.0090(6)	1.99(13)E-4
¹³⁷ Ba	1614.68(10)	0.0147(10)	0.000324(22)	¹³⁷ Ba	4369.47(10)	0.0069(5)	1.52(11)E-4
¹³⁷ Ba	1619.88(15)	0.00328(24)	7.2(5)E-5	¹³⁷ Ba	4445.44(12)	0.0039(3)	8.6(7)E-5
¹³⁵ Ba	1666.69(9)	0.0047(5)	1.04(11)E-4	¹³⁷ Ba	4597.95(22)	0.0044(4)	9.7(9)E-5
¹³⁵ Ba	1714.09(9)	0.0076(12)	1.7(3)E-4	¹³⁷ Ba	4689.43(9)	0.0140(8)	0.000309(18)
¹³⁷ Ba	1717.16(20)	0.0071(8)	1.57(18)E-4	¹³⁶ Ba	4723.38(8)	0.0264(8)	0.000583(18)
¹³⁷ Ba	1727.32(10)	0.0056(4)	1.24(9)E-4	¹³⁷ Ba	4773.79(15)	0.0063(4)	1.39(9)E-4
¹³⁷ Ba	1745.07(6)	0.0035(4)	7.7(9)E-5	¹³⁷ Ba	4967.90(6)	0.0098(7)	2.16(15)E-4
¹³⁵ Ba	1842.90(11)	0.0054(7)	1.19(15)E-4	¹³⁷ Ba	5107.54(17)	0.0060(4)	1.32(9)E-4
¹³⁸ Ba	1853.30(12)	0.0074(6)	1.63(13)E-4	¹³⁷ Ba	5272.88(10)	0.0088(10)	1.94(22)E-4
¹³⁶ Ba	1898.68(5)	0.0305(10)	0.000673(22)	¹³⁵ Ba	5312.42(17)	0.0082(3)	1.81(7)E-4
¹³⁸ Ba	1951.9(5)	0.009(6)	2.0(13)E-4	¹³⁷ Ba	5448.42(11)	0.0053(6)	1.17(13)E-4
¹³⁵ Ba	1955.19(19)	0.0031(9)	6.8(20)E-5	¹³⁷Ba	5730.81(6)	0.0617(20)	0.00136(4)
¹³⁵ Ba	1993.15(16)	0.0044(11)	9.7(24)E-5	¹³⁷ Ba	5972.26(9)	0.0044(3)	9.7(7)E-5
¹³⁷ Ba	2023.55(8)	0.0091(6)	2.01(13)E-4	¹³⁷ Ba	6028.60(8)	0.0093(6)	2.05(13)E-4
¹³⁵ Ba	2080.04(5)	0.0074(5)	1.63(11)E-4	¹³⁵ Ba	6062.37(23)	0.00516(14)	1.14(3)E-4
¹³⁵ Ba	2128.73(9)	0.0114(6)	0.000252(13)	¹³⁷ Ba	6421.67(8)	0.00337(19)	7.4(4)E-5
¹³⁷ Ba	2207.85(5)	0.0038(6)	8.4(13)E-5	¹³⁶ Ba	6621.99(8)	0.0034(6)	7.5(13)E-5
¹³⁷ Ba	2210.82(16)	0.0038(8)	8.4(18)E-5	¹³⁵ Ba	8288.93(5)	0.00349(11)	7.70(24)E-5
¹³⁷Ba	2217.84(8)	0.044(5)	0.00097(11)	¹³⁵ Ba	9107.41(4)	0.00635(23)	1.40(5)E-4
¹³⁸ Ba	2242.58(13)	0.0116(13)	0.00026(3)	Lanthanum (Z=57), At.Wt.=138.9055(2), σ_γ^Z=9.08(4)			
¹³⁷ Ba	2401.96(15)	0.0031(3)	6.8(7)E-5	¹³⁹ La	14.2380(20)	0.028(6)	0.00061(13)
¹³⁵ Ba	2485.20(8)	0.00349(24)	7.7(5)E-5	¹³⁹ La	28.5330(10)	0.0103(11)	2.25(24)E-4
¹³⁸ Ba	2537.72(10)	0.0102(7)	2.25(15)E-4	¹³⁹La	29.9640(10)	0.169(8)	0.00369(17)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹³⁹ La	34.6460(10)	0.0220(20)	0.00048(4)	¹³⁹ La	787.3(4)	0.008(4)	1.7(9)E-4
¹³⁹ La	45.913(6)	0.0120(7)	0.000262(15)	¹³⁸ La	788.742	0.273(5) s⁻¹g⁻¹	Abundant
¹³⁹ La	54.9440(10)	0.143(7)	0.00312(15)	¹³⁹ La	796.27(5)	0.0162(6)	0.000353(13)
¹³⁹ La	63.1790(10)	0.208(8)	0.00454(17)	¹³⁹ La	815.772(19)d	1.430(12)	0.0312[<0.1%]
¹³⁹ La	69.1830(20)	0.0137(5)	0.000299(11)	¹³⁹ La	848.99(3)	0.0290(11)	0.000633(24)
¹³⁹ La	132.695(3)	0.0146(6)	0.000319(13)	¹³⁹ La	863.28(3)	0.0149(6)	0.000325(13)
¹³⁹ La	155.560(5)	0.192(7)	0.00419(15)	¹³⁹ La	867.846(20)d	0.337(4)	0.00735[<0.1%]
¹³⁹ La	162.659(3)	0.489(18)	0.0107(4)	¹³⁹ La	868.32(5)	0.0558(21)	0.00122(5)
¹³⁸ La	166.04(7)	0.0119(12)	0.00026(3)	¹³⁹ La	882.21(3)	0.0343(13)	0.00075(3)
¹³⁹ La	169.392(10)	0.0382(14)	0.00083(3)	¹³⁹ La	887.70(11)	0.0222(8)	0.000484(17)
¹³⁹ La	209.127(4)	0.0431(16)	0.00094(4)	¹³⁹ La	919.550(23)d	0.1630(18)	0.00356[<0.1%]
¹³⁹ La	215.02(16)	0.025(6)	0.00055(13)	¹³⁹ La	925.189(21)d	0.422(4)	0.00921[<0.1%]
¹³⁹ La	218.225(22)	0.78(3)	0.0170(7)	¹³⁹ La	941.79(17)	0.0236(9)	0.000515(20)
¹³⁹ La	235.771(8)	0.111(4)	0.00242(9)	¹³⁹ La	986.74(3)	0.008(4)	1.7(9)E-4
¹³⁹ La	237.660(4)	0.320(12)	0.0070(3)	¹³⁹ La	991.859(20)	0.0487(18)	0.00106(4)
¹³⁹ La	255.040(5)	0.017(4)	0.00037(9)	¹³⁹ La	1006.153(20)	0.0347(13)	0.00076(3)
¹³⁹ La	258.875(22)	0.0233(9)	0.000508(20)	¹³⁹ La	1020.392(20)	0.0535(20)	0.00117(4)
¹³⁹ La	272.306(4)	0.502(19)	0.0110(4)	¹³⁹ La	1055.038(20)	0.015(5)	0.00033(11)
¹³⁹ La	279.979(22)	0.0640(24)	0.00140(5)	¹³⁸ La	1215.72(22)	0.019(4)	0.00041(9)
¹³⁹ La	283.617(16)	0.0409(15)	0.00089(3)	¹³⁸ La	1219.79(17)	0.026(4)	0.00057(9)
¹³⁹ La	287.408(22)	0.013(4)	0.00028(9)	¹³⁸ La	1435.795(10)	0.539(7) s⁻¹g⁻¹	Abundant
¹³⁹ La	288.255(5)	0.73(3)	0.0159(7)	¹³⁸ La	1537.7(3)	0.009(3)	2.0(7)E-4
¹³⁹ La	290.92(3)	0.0167(6)	0.000364(13)	¹³⁹ La	1596.21(4)d	5.84(9)	0.1274[<0.1%]
¹³⁹ La	305.04(8)	0.0147(6)	0.000321(13)	¹³⁹ La	2345.21(6)	0.0164(6)	0.000358(13)
¹³⁹ La	310.14(3)	0.0184(7)	0.000401(15)	¹³⁹ La	2512.55(17)	0.0194(7)	0.000423(15)
¹³⁹ La	328.762(8)d	1.250(18)	0.0273[<0.1%]	¹³⁹ La	2517.04(8)	0.0353(13)	0.00077(3)
¹³⁹ La	329.727(12)	0.0140(5)	0.000305(11)	¹³⁹ La	2521.40(5)d	0.2120(23)	0.00463[<0.1%]
¹³⁹ La	422.66(4)	0.370(14)	0.0081(3)	¹³⁹ La	2532.39(4)	0.0188(7)	0.000410(15)
¹³⁹ La	426.49(3)	0.0435(16)	0.00095(4)	¹³⁹ La	2538.82(7)	0.0119(5)	0.000260(11)
¹³⁹ La	432.493(12)d	0.1780(18)	0.00388[<0.1%]	¹³⁹ La	2555.76(4)	0.0231(9)	0.000504(20)
¹³⁹ La	478.05(5)	0.0407(15)	0.00089(3)	¹³⁹ La	2561.85(3)	0.0259(10)	0.000565(22)
¹³⁹ La	487.021(12)d	2.79(4)	0.0609[<0.1%]	¹³⁹ La	2564.79(3)	0.0373(14)	0.00081(3)
¹³⁹ La	495.620(13)	0.081(3)	0.00177(7)	¹³⁹ La	2598.16(4)	0.0231(9)	0.000504(20)
¹³⁹ La	528.34(11)	0.0197(7)	0.000430(15)	¹³⁹ La	2607.17(3)	0.0344(13)	0.00075(3)
¹³⁹ La	538.854(12)	0.0455(17)	0.00099(4)	¹³⁹ La	2611.6(3)	0.0086(3)	1.88(7)E-4
¹³⁹ La	549.01(3)	0.098(4)	0.00214(9)	¹³⁹ La	2617.76(4)	0.0149(6)	0.000325(13)
¹³⁹ La	553.148(12)	0.0602(23)	0.00131(5)	¹³⁹ La	2637.97(6)	0.0084(5)	1.83(11)E-4
¹³⁹ La	567.386(12)	0.335(13)	0.0073(3)	¹³⁹ La	2640.00(3)	0.0160(6)	0.000349(13)
¹³⁹ La	592.05(18)	0.0128(5)	0.000279(11)	¹³⁹ La	2661.55(4)	0.0263(10)	0.000574(22)
¹³⁹ La	595.099(12)	0.103(4)	0.00225(9)	¹³⁹ La	2668.00(4)	0.0247(9)	0.000539(20)
¹³⁹ La	602.032(12)	0.0522(20)	0.00114(4)	¹³⁹ La	2677.63(12)	0.0100(4)	2.18(9)E-4
¹³⁹ La	623.632(12)	0.0517(20)	0.00113(4)	¹³⁹ La	2688.09(3)	0.0254(10)	0.000554(22)
¹³⁹ La	628.314(12)	0.0284(11)	0.000620(24)	¹³⁹ La	2692.30(6)	0.0115(7)	0.000251(15)
¹³⁹ La	640.88(3)	0.0534(20)	0.00117(4)	¹³⁹ La	2698.19(4)	0.0185(7)	0.000404(15)
¹³⁹ La	658.278(12)	0.103(4)	0.00225(9)	¹³⁹ La	2702.38(6)	0.0109(4)	2.38(9)E-4
¹³⁹ La	667.594(14)	0.0580(22)	0.00127(5)	¹³⁹ La	2710.62(4)	0.0117(4)	0.000255(9)
¹³⁹ La	708.244(14)	0.134(5)	0.00292(11)	¹³⁹ La	2714.63(3)	0.0141(5)	0.000308(11)
¹³⁹ La	710.07(3)	0.0668(25)	0.00146(6)	¹³⁹ La	2724.26(4)	0.0151(6)	0.000329(13)
¹³⁹ La	711.22(20)	0.0164(6)	0.000358(13)	¹³⁹ La	2735.13(4)	0.0188(7)	0.000410(15)
¹³⁹ La	722.538(14)	0.212(8)	0.00463(17)	¹³⁹ La	2739.00(4)	0.0200(8)	0.000436(17)
¹³⁹ La	725.11(20)	0.0125(5)	0.000273(11)	¹³⁹ La	2747.65(4)	0.0198(8)	0.000432(17)
¹³⁹ La	736.777(14)	0.0388(15)	0.00085(3)	¹³⁹ La	2757.726(24)	0.0515(19)	0.00112(4)
¹³⁹ La	744.71(3)	0.010(4)	2.2(9)E-4	¹³⁹ La	2764.51(4)	0.0289(11)	0.000631(24)
¹³⁹ La	751.637(18)d	0.2650(23)	0.00578[<0.1%]	¹³⁹ La	2767.58(4)	0.0287(11)	0.000626(24)
¹³⁹ La	766.30(5)	0.0127(5)	0.000277(11)	¹³⁹ La	2799.65(6)	0.0109(4)	2.38(9)E-4
¹³⁹ La	782.733(20)	0.0396(15)	0.00086(3)	¹³⁹ La	2804.82(4)	0.0203(8)	0.000443(17)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹³⁹ La	2837.50(4)	0.0195(7)	0.000425(15)	¹³⁹ La	3606.467(24)	0.0556(21)	0.00121(5)
¹³⁹ La	2852.55(4)	0.0139(5)	0.000303(11)	¹³⁹ La	3610.026(24)	0.0548(21)	0.00120(5)
¹³⁹ La	2863.06(3)	0.073(3)	0.00159(7)	¹³⁹La	3665.631(24)	0.135(5)	0.00295(11)
¹³⁹ La	2880.60(6)	0.0101(4)	2.20(9)E-4	¹³⁹La	3679.641(24)	0.139(5)	0.00303(11)
¹³⁹ La	2896.63(6)	0.0081(5)	1.77(11)E-4	¹³⁹ La	3683.89(3)	0.0322(21)	0.00070(5)
¹³⁹ La	2903.65(5)	0.0112(4)	2.44(9)E-4	¹³⁹ La	3691.35(3)	0.0350(13)	0.00076(3)
¹³⁹ La	2913.16(4)	0.0124(5)	0.000271(11)	¹³⁹ La	3718.321(24)	0.0384(15)	0.00084(3)
¹³⁹ La	2916.89(4)	0.0130(8)	0.000284(17)	¹³⁹ La	3727.700(24)	0.073(3)	0.00159(7)
¹³⁹ La	2919.73(6)	0.0086(3)	1.88(7)E-4	¹³⁹ La	3735.30(4)	0.0170(6)	0.000371(13)
¹³⁹ La	2925.00(3)	0.0435(16)	0.00095(4)	¹³⁹ La	3738.56(4)	0.0352(13)	0.00077(3)
¹³⁹ La	2961.34(4)	0.0262(10)	0.000572(22)	¹³⁹ La	3744.87(4)	0.0234(9)	0.000511(20)
¹³⁹ La	2969.27(4)	0.0409(15)	0.00089(3)	¹³⁹ La	3821.40(4)	0.0131(9)	0.000286(20)
¹³⁹ La	2977.35(5)	0.0164(6)	0.000358(13)	¹³⁹ La	3900.979(24)	0.0531(20)	0.00116(4)
¹³⁹ La	2985.02(6)	0.0100(4)	2.18(9)E-4	¹³⁹ La	3951.14(3)	0.0198(8)	0.000432(17)
¹³⁹ La	2988.53(3)	0.0458(17)	0.00100(4)	¹³⁹ La	3973.56(4)	0.0120(5)	0.000262(11)
¹³⁹ La	2998.36(5)	0.0136(5)	0.000297(11)	¹³⁹ La	4044.182(21)	0.0297(11)	0.000648(24)
¹³⁹ La	3017.070(24)	0.0671(25)	0.00146(6)	¹³⁹ La	4060.007(20)	0.0297(11)	0.000648(24)
¹³⁹ La	3031.27(4)	0.0330(12)	0.00072(3)	¹³⁹ La	4105.897(20)	0.0238(9)	0.000519(20)
¹³⁹ La	3035.56(3)	0.0518(20)	0.00113(4)	¹³⁹ La	4125.31(3)	0.0183(7)	0.000399(15)
¹³⁹ La	3040.94(4)	0.0294(11)	0.000641(24)	¹³⁹La	4389.505(14)	0.255(10)	0.00556(22)
¹³⁹ La	3051.49(5)	0.0183(7)	0.000399(15)	¹³⁹ La	4416.22(3)	0.247(9)	0.00539(20)
¹³⁹ La	3057.66(6)	0.0194(7)	0.000423(15)	¹³⁹ La	4502.647(13)	0.164(6)	0.00358(13)
¹³⁹ La	3078.80(6)	0.0130(5)	0.000284(11)	¹³⁹ La	4558.891(13)	0.0488(18)	0.00106(4)
¹³⁹La	3082.979(24)	0.140(5)	0.00305(11)	¹³⁹ La	4842.695(7)	0.661(25)	0.0144(6)
¹³⁹ La	3091.30(6)	0.0114(4)	2.49(9)E-4	¹³⁹ La	4888.606(7)	0.150(6)	0.00327(13)
¹³⁹ La	3095.50(4)	0.0191(7)	0.000417(15)	¹³⁹ La	4998.250(6)	0.0145(8)	0.000316(17)
¹³⁹ La	3112.38(3)	0.0320(12)	0.00070(3)	¹³⁹ La	5097.726(6)	0.68(3)	0.0148(7)
¹³⁹ La	3115.94(3)	0.0176(7)	0.000384(15)	¹³⁹ La	5126.257(6)	0.114(4)	0.00249(9)
¹³⁹ La	3119.05(4)	0.0118(8)	0.000257(17)	¹³⁹ La	5130.939(6)	0.0159(9)	0.000347(20)
¹³⁹ La	3137.21(4)	0.0239(9)	0.000521(20)	¹³⁹ La	5160.902(6)	0.089(5)	0.00194(11)
¹³⁹ La	3142.75(3)	0.0320(12)	0.00070(3)	Cerium (Z=58), At.Wt.=140.116(1), σ_γ^Z=0.635(18)			
¹³⁹ La	3155.06(6)	0.0090(3)	1.96(7)E-4	¹³⁶ Ce	254.29(5)d	2.0(6)E-4	4.3E-6[1.0%]
¹³⁹ La	3163.792(24)	0.0324(12)	0.00071(3)	¹³⁸ Ce	255.65(6)	0.0082(7)	1.77(15)E-4
¹³⁹ La	3174.77(4)	0.0135(5)	0.000295(11)	¹⁴⁰Ce	475.04(4)	0.082(7)	0.00177(15)
¹³⁹ La	3189.09(3)	0.0538(20)	0.00117(4)	¹³⁶ Ce	513.7(4)	0.0021(5)	4.5(11)E-5
¹³⁹ La	3197.52(6)	0.0213(8)	0.000465(17)	¹⁴⁰Ce	661.99(5)	0.241(15)	0.0052(3)
¹³⁹ La	3213.35(4)	0.0144(5)	0.000314(11)	¹⁴⁰ Ce	671.64(5)	0.0057(5)	1.23(11)E-4
¹³⁹ La	3219.80(3)	0.0300(11)	0.000655(24)	¹⁴²Ce	737.43(7)	0.026(3)	0.00056(7)
¹³⁹ La	3265.263(24)	0.0532(20)	0.00116(4)	¹⁴² Ce	765.97(5)	0.0145(12)	0.00031(3)
¹³⁹ La	3281.248(24)	0.0506(19)	0.00110(4)	¹⁴² Ce	789.40(8)	0.0050(6)	1.08(13)E-4
¹³⁹ La	3318.99(4)	0.0319(12)	0.00070(3)	¹⁴² Ce	808.35(6)	0.0102(9)	2.21(19)E-4
¹³⁹ La	3341.48(4)	0.0090(5)	1.96(11)E-4	¹⁴² Ce	820.07(8)	0.0026(3)	5.6(7)E-5
¹³⁹ La	3359.88(3)	0.0120(7)	0.000262(15)	¹⁴² Ce	862.23(7)	0.0044(4)	9.5(9)E-5
¹³⁹ La	3383.39(3)	0.0242(9)	0.000528(20)	¹⁴² Ce	915.03(7)	0.0086(11)	1.86(24)E-4
¹³⁹ La	3395.44(4)	0.0161(6)	0.000351(13)	¹⁴² Ce	987.69(9)	0.0040(5)	8.7(11)E-5
¹³⁹ La	3404.81(4)	0.0171(6)	0.000373(13)	¹⁴⁰ Ce	1052.58(5)	0.0051(5)	1.10(11)E-4
¹³⁹ La	3417.24(4)	0.0181(7)	0.000395(15)	¹⁴²Ce	1107.66(5)	0.040(3)	0.00087(7)
¹³⁹ La	3424.29(3)	0.0232(14)	0.00051(3)	¹⁴⁰ Ce	1146.68(4)	0.0096(9)	2.08(19)E-4
¹³⁹ La	3425.399(24)	0.058(3)	0.00127(7)	¹⁴² Ce	1153.97(5)	0.0146(12)	0.00032(3)
¹³⁹ La	3437.83(4)	0.0247(9)	0.000539(20)	¹⁴² Ce	1165.71(8)	0.0040(4)	8.7(9)E-5
¹³⁹ La	3442.20(3)	0.0410(15)	0.00089(3)	¹⁴⁰ Ce	1288.69(5)	0.0076(6)	1.64(13)E-4
¹³⁹ La	3459.91(3)	0.0199(8)	0.000434(17)	¹⁴⁰ Ce	1331.63(7)	0.0058(5)	1.25(11)E-4
¹³⁹ La	3477.14(3)	0.0444(17)	0.00097(4)	¹³⁸ Ce	1347.24(13)	0.0028(3)	6.1(7)E-5
¹³⁹ La	3488.77(3)	0.0170(6)	0.000371(13)	¹⁴⁰ Ce	1385.74(6)	0.0060(6)	1.30(13)E-4
¹³⁹ La	3564.87(4)	0.0130(5)	0.000284(11)	¹⁴⁰ Ce	1497.03(12)	0.0062(9)	1.34(19)E-4
¹³⁹ La	3580.90(4)	0.0129(5)	0.000281(11)	¹⁴⁰ Ce	1527.61(6)	0.0027(3)	5.8(7)E-5
¹³⁹ La	3596.45(4)	0.0157(6)	0.000343(13)	-----			

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁴² Ce	1587.90(11)	0.0028(3)	6.1(7)E-5	¹⁴¹ Pr	633.34(4)	0.113(4)	0.00243(9)
¹⁴⁰ Ce	1673.95(9)	0.0033(4)	7.1(9)E-5	¹⁴¹ Pr	645.720(24)	0.311(7)	0.00669(15)
¹⁴⁰ Ce	1747.90(7)	0.0078(7)	1.69(15)E-4	¹⁴¹ Pr	684.59(3)	0.098(22)	0.0021(5)
¹⁴⁰ Ce	1808.67(6)	0.0038(4)	8.2(9)E-5	¹⁴¹ Pr	698.65(3)	0.22(6)	0.0047(13)
¹⁴² Ce	2203.36(10)	0.0039(5)	8.4(11)E-5	¹⁴¹ Pr	705.309(24)	0.0399(20)	0.00086(4)
¹⁴⁰ Ce	2905.37(7)	0.0058(5)	1.25(11)E-4	¹⁴¹ Pr	718.014(24)	0.0435(21)	0.00094(5)
¹⁴² Ce	2931.94(14)	0.0029(3)	6.3(7)E-5	¹⁴¹ Pr	729.233(14)	0.0712(23)	0.00153(5)
¹⁴⁰ Ce	3002.41(6)	0.0104(8)	2.25(17)E-4	¹⁴¹ Pr	737.65(7)	0.0396(17)	0.00085(4)
¹⁴⁰ Ce	3018.24(7)	0.0114(10)	2.47(22)E-4	¹⁴¹ Pr	746.973(14)	0.146(4)	0.00314(9)
¹⁴⁰ Ce	3092.19(8)	0.0072(6)	1.56(13)E-4	¹⁴¹ Pr	772.566(24)	0.044(16)	0.0009(3)
¹⁴⁰ Ce	3238.52(6)	0.0066(6)	1.43(13)E-4	¹⁴¹ Pr	790.306(24)	0.051(3)	0.00110(7)
¹⁴⁰ Ce	3434.50(8)	0.0039(4)	8.4(9)E-5	¹⁴¹ Pr	801.29(4)	0.10(3)	0.0022(7)
¹⁴⁰ Ce	3619.46(5)	0.0095(8)	2.05(17)E-4	¹⁴¹ Pr	804.91(7)	0.0455(25)	0.00098(5)
¹⁴² Ce	3990.70(15)	0.0038(4)	8.2(9)E-5	¹⁴¹ Pr	822.65(7)	0.0179(15)	0.00038(3)
¹⁴² Ce	4282.22(12)	0.0037(4)	8.0(9)E-5	¹⁴¹ Pr	864.98(3)	0.14(3)	0.0030(7)
¹⁴⁰ Ce	4291.08(4)	0.053(4)	0.00115(9)	¹⁴¹ Pr	893.16(4)	0.053(3)	0.00114(7)
¹⁴² Ce	4336.46(8)	0.0251(20)	0.00054(4)	¹⁴¹ Pr	956.84(3)	0.091(7)	0.00196(15)
¹⁴⁰ Ce	4766.10(5)	0.113(8)	0.00244(17)	¹⁴¹ Pr	974.47(4)	0.076(22)	0.0016(5)
Praseodymium (Z=59), At. Wt.=140.90765(2), $\sigma_{\gamma}^Z = 11.5(3)$				¹⁴¹ Pr	992.00(4)	0.138(10)	0.00297(22)
¹⁴¹ Pr	32.276(3)	0.055(11)	0.00118(24)	¹⁴¹ Pr	1006.361(22)	0.153(8)	0.00329(17)
¹⁴¹ Pr	54.5530(20)	0.022(4)	0.00047(9)	¹⁴¹ Pr	1024.10(3)	0.048(3)	0.00103(7)
¹⁴¹ Pr	55.957(3)	0.014(3)	0.00030(7)	¹⁴¹ Pr	1102.51(4)	0.056(3)	0.00120(7)
¹⁴¹ Pr	60.0630(20)	0.134(14)	0.0029(3)	¹⁴¹ Pr	1150.946(21)	0.141(5)	0.00303(11)
¹⁴¹ Pr	64.5050(20)	0.137(6)	0.00295(13)	¹⁴¹ Pr	1575.6(5)d	0.426(12)	0.0092[1.8%]
¹⁴¹ Pr	68.6110(20)	0.116(6)	0.00249(13)	¹⁴¹ Pr	3532.83(3)	0.026(3)	0.00056(7)
¹⁴¹ Pr	84.998(3)	0.207(11)	0.00445(24)	¹⁴¹ Pr	3535.33(3)	0.026(3)	0.00056(7)
¹⁴¹ Pr	86.37(7)	0.085(7)	0.00183(15)	¹⁴¹ Pr	3549.71(3)	0.0288(24)	0.00062(5)
¹⁴¹ Pr	104.570(3)	0.0397(13)	0.00085(3)	¹⁴¹ Pr	3556.85(3)	0.0127(17)	0.00027(4)
¹⁴¹ Pr	115.528(4)	0.0419(13)	0.00090(3)	¹⁴¹ Pr	3563.23(3)	0.0110(23)	2.4(5)E-4
¹⁴¹ Pr	124.5680(20)	0.0339(18)	0.00073(4)	¹⁴¹ Pr	3582.48(3)	0.0236(21)	0.00051(5)
¹⁴¹ Pr	126.8460(20)	0.307(15)	0.0066(3)	¹⁴¹ Pr	3587.84(3)	0.0128(17)	0.00028(4)
¹⁴¹ Pr	140.9050(20)	0.479(10)	0.01030(22)	¹⁴¹ Pr	3591.03(3)	0.0139(19)	0.00030(4)
¹⁴¹ Pr	153.28(3)	0.0135(7)	0.000290(15)	¹⁴¹ Pr	3599.14(3)	0.0234(24)	0.00050(5)
¹⁴¹ Pr	159.1230(20)	0.0122(7)	0.000262(15)	¹⁴¹ Pr	3602.51(3)	0.054(3)	0.00116(7)
¹⁴¹ Pr	176.8630(20)	1.06(4)	0.0228(9)	¹⁴¹ Pr	3620.02(3)	0.024(3)	0.00052(7)
¹⁴¹ Pr	182.786(4)	0.377(14)	0.0081(3)	¹⁴¹ Pr	3629.19(3)	0.020(4)	0.00043(9)
¹⁴¹ Pr	185.62(7)	0.017(4)	0.00037(9)	¹⁴¹ Pr	3645.82(3)	0.015(3)	0.00032(7)
¹⁴¹ Pr	187.85(5)	0.048(12)	0.0010(3)	¹⁴¹ Pr	3650.20(3)	0.061(3)	0.00131(7)
¹⁴¹ Pr	200.526(4)	0.0379(12)	0.00082(3)	¹⁴¹ Pr	3651.73(3)	0.0127(8)	0.000273(17)
¹⁴¹ Pr	231.18(4)	0.0127(10)	0.000273(22)	¹⁴¹ Pr	3654.47(3)	0.060(4)	0.00129(9)
¹⁴¹ Pr	251.53(4)	0.0172(19)	0.00037(4)	¹⁴¹ Pr	3664.35(3)	0.0193(25)	0.00042(5)
¹⁴¹ Pr	268.38(4)	0.0166(8)	0.000357(17)	¹⁴¹ Pr	3678.37(3)	0.034(3)	0.00073(7)
¹⁴¹ Pr	294.87(3)	0.0275(18)	0.00059(4)	¹⁴¹ Pr	3690.27(3)	0.0107(19)	2.3(4)E-4
¹⁴¹ Pr	360.64(3)	0.0342(19)	0.00074(4)	¹⁴¹ Pr	3713.73(3)	0.047(3)	0.00101(7)
¹⁴¹ Pr	403.976(24)	0.0322(14)	0.00069(3)	¹⁴¹ Pr	3742.46(3)	0.0191(24)	0.00041(5)
¹⁴¹ Pr	415.17(5)	0.0122(10)	0.000262(22)	¹⁴¹ Pr	3762.26(3)	0.0177(24)	0.00038(5)
¹⁴¹ Pr	460.16(4)	0.057(3)	0.00123(7)	¹⁴¹ Pr	3771.88(3)	0.023(3)	0.00049(7)
¹⁴¹ Pr	508.78(4)	0.104(10)	0.00224(22)	¹⁴¹ Pr	3776.46(3)	0.0117(8)	0.000252(17)
¹⁴¹ Pr	528.219(23)	0.0579(19)	0.00125(4)	¹⁴¹ Pr	3790.37(3)	0.140(6)	0.00301(13)
¹⁴¹ Pr	546.448(15)	0.148(4)	0.00318(9)	¹⁴¹ Pr	3800.04(3)	0.0144(23)	0.00031(5)
¹⁴¹ Pr	557.75(3)	0.15(4)	0.0032(9)	¹⁴¹ Pr	3811.64(3)	0.0231(23)	0.00050(5)
¹⁴¹ Pr	560.495(23)	0.150(7)	0.00323(15)	¹⁴¹ Pr	3862.86(3)	0.0199(25)	0.00043(5)
¹⁴¹ Pr	570.111(14)	0.112(5)	0.00241(11)	¹⁴¹ Pr	3871.70(3)	0.0164(23)	0.00035(5)
¹⁴¹ Pr	573.28(4)	0.12(3)	0.0026(7)	¹⁴¹ Pr	3892.63(3)	0.039(3)	0.00084(7)
¹⁴¹ Pr	619.29(4)	0.152(4)	0.00327(9)	¹⁴¹ Pr	3902.50(3)	0.0117(20)	0.00025(4)
¹⁴¹ Pr	630.04(3)	0.16(6)	0.0034(13)	¹⁴¹ Pr	3911.07(3)	0.042(3)	0.00090(7)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁴¹ Pr	3923.07(3)	0.023(3)	0.00049(7)	¹⁴³ Nd	864.301(10)	4.27(11)	0.0897(23)
¹⁴¹ Pr	3941.19(3)	0.0153(25)	0.00033(5)	¹⁴³ Nd	980.60(4)	1.21(3)	0.0254(6)
¹⁴¹ Pr	3947.09(3)	0.0169(23)	0.00036(5)	¹⁴³ Nd	1136.92(6)	0.669(18)	0.0141(4)
¹⁴¹ Pr	4000.97(3)	0.0187(24)	0.00040(5)	¹⁴³ Nd	1357.04(8)	0.337(9)	0.00708(19)
¹⁴¹ Pr	4012.20(3)	0.027(3)	0.00058(7)	¹⁴³ Nd	1376.19(7)	0.751(20)	0.0158(4)
¹⁴¹ Pr	4058.05(3)	0.0133(16)	0.00029(3)	¹⁴³ Nd	1413.16(4)	1.90(5)	0.0399(11)
¹⁴¹ Pr	4090.15(3)	0.0137(16)	0.00029(3)	¹⁴³ Nd	1418.07(10)	0.353(11)	0.00742(23)
¹⁴¹ Pr	4120.77(3)	0.0130(16)	0.00028(3)	¹⁴³ Nd	1481.95(8)	0.608(21)	0.0128(4)
¹⁴¹ Pr	4134.04(3)	0.0408(25)	0.00088(5)	¹⁴³ Nd	1515.84(9)	0.455(13)	0.0096(3)
¹⁴¹ Pr	4163.89(3)	0.035(3)	0.00075(7)	¹⁴³ Nd	1560.796(14)	0.404(11)	0.00849(23)
¹⁴¹ Pr	4177.00(3)	0.0387(25)	0.00083(5)	¹⁴³ Nd	1671.74(10)	0.97(8)	0.0204(17)
¹⁴¹ Pr	4252.14(3)	0.032(3)	0.00069(7)	¹⁴³ Nd	1895.74(16)	0.387(12)	0.00813(25)
¹⁴¹ Pr	4276.54(3)	0.044(4)	0.00095(9)	¹⁴⁴ Nd	4836.36(25)	0.32(3)	0.0067(6)
¹⁴¹ Pr	4325.50(3)	0.0124(17)	0.00027(4)	¹⁴² Nd	5381.19(7)	0.49(4)	0.0103(8)
¹⁴¹ Pr	4347.62(3)	0.0166(18)	0.00036(4)	¹⁴³ Nd	6255.99(17)	1.50(12)	0.0315(25)
¹⁴¹ Pr	4372.53(3)	0.0269(22)	0.00058(5)	¹⁴³ Nd	6502.22(17)	3.18(17)	0.067(4)
¹⁴¹ Pr	4440.54(3)	0.0252(20)	0.00054(4)	¹⁴⁵ Nd	7110.98(8)	0.368(11)	0.00773(23)
¹⁴¹ Pr	4449.26(3)	0.0228(19)	0.00049(4)	Samarium (Z=62), At.Wt.=150.36(3), $\sigma_{\gamma}^Z = 5621(80)$			
¹⁴¹ Pr	4496.44(3)	0.098(6)	0.00211(13)	¹⁵⁴ Sm	104.320(5)d	1.43(4)	0.0288[55%]
¹⁴¹ Pr	4579.64(3)	0.0126(17)	0.00027(4)	¹⁵² Sm	127.297(3)	4.1(3)	0.083(6)
¹⁴¹ Pr	4592.28(3)	0.0165(19)	0.00035(4)	¹⁵⁰ Sm	167.77(5)	0.73(13)	0.015(3)
¹⁴¹ Pr	4692.120(22)	0.291(10)	0.00626(22)	¹⁴⁹ Sm	333.97(4)	4790(60)	96.5(12)
¹⁴¹ Pr	4722.82(4)	0.083(4)	0.00179(9)	¹⁴⁹ Sm	403.02(3)	85.2(16)	1.72(3)
¹⁴¹ Pr	4731.284(9)	0.0149(18)	0.00032(4)	¹⁴⁹ Sm	439.40(4)	2860(150)	58(3)
¹⁴¹ Pr	4801.22(3)	0.140(8)	0.00301(17)	¹⁴⁹ Sm	485.95(7)	72(3)	1.45(6)
¹⁴¹ Pr	4864.91(4)	0.0112(16)	2.4(3)E-4	¹⁴⁹ Sm	505.51(3)	528(80)	10.6(16)
¹⁴¹ Pr	5020.41(7)	0.0135(17)	0.00029(4)	¹⁴⁷ Sm	550.10(9)	9.6(6)	0.193(12)
¹⁴¹ Pr	5052.750(24)	0.0329(21)	0.00071(5)	¹⁴⁹ Sm	584.27(3)	480(70)	9.7(14)
¹⁴¹ Pr	5096.081(15)	0.208(8)	0.00447(17)	¹⁴⁹ Sm	675.83(3)	172(7)	3.47(14)
¹⁴¹ Pr	5137.972(24)	0.098(4)	0.00211(9)	¹⁴⁹ Sm	712.20(3)	267(4)	5.38(8)
¹⁴¹ Pr	5140.72(3)	0.269(11)	0.00579(24)	¹⁴⁹ Sm	731.20(4)	54(4)	1.09(8)
¹⁴¹ Pr	5206.03(4)	0.033(3)	0.00071(7)	¹⁴⁹ Sm	737.44(4)	597(8)	12.03(16)
¹⁴¹ Pr	5666.170(6)	0.379(15)	0.0082(3)	¹⁴⁹ Sm	748.13(4)	67.9(20)	1.37(4)
¹⁴¹ Pr	5698.445(6)	0.0117(14)	0.00025(3)	¹⁵⁴ Sm	819.880(5)	0.153(10)	0.00308(20)
¹⁴¹ Pr	5770.736(6)	0.0371(23)	0.00080(5)	¹⁴⁹ Sm	831.78(5)	62.7(17)	1.26(3)
¹⁴¹ Pr	5825.286(5)	0.040(3)	0.00086(7)	¹⁴⁹ Sm	859.86(4)	88(4)	1.77(8)
¹⁴¹ Pr	5843.026(5)	0.147(6)	0.00316(13)	¹⁴⁹ Sm	869.29(3)	119(6)	2.40(12)
Neodymium (Z=60), At.Wt.=144.24(3), $\sigma_{\gamma}^Z = 49.5(12)$				¹⁴⁹ Sm	1165.76(5)	61(3)	1.23(6)
¹⁴⁸ Nd	165.0870(10)	0.032(8)	0.00067(17)	¹⁴⁹ Sm	1170.59(4)	230(10)	4.64(20)
¹⁵⁰ Nd	189.0530(10)	0.020(7)	0.00042(15)	¹⁴⁹ Sm	1177.3(4)	57(3)	1.15(6)
¹⁴³ Nd	201.86(7)	0.343(23)	0.0072(5)	¹⁴⁹ Sm	1193.84(4)	106(3)	2.14(6)
¹⁴⁸ Nd	211.309(7)d	0.0370(16)	0.00078[18%]	¹⁴⁹ Sm	1247.04(8)	51(3)	1.03(6)
¹⁴⁶ Nd	314.675(4)	0.0280(24)	0.00059(5)	¹⁴⁹ Sm	1262.07(10)	62(5)	1.25(10)
¹⁴³ Nd	426.73(5)	0.574(15)	0.0121(3)	¹⁴⁹ Sm	1321.95(7)	76(9)	1.53(18)
¹⁴⁵ Nd	453.89(5)	3.03(8)	0.0637(17)	¹⁴⁹ Sm	1350.39(5)	94(12)	1.89(24)
¹⁴³ Nd	476.82(5)	1.93(5)	0.0405(11)	Europium (Z=63), At.Wt.=151.964(1), $\sigma_{\gamma}^Z = 4560(140)$			
¹⁴² Nd	563.87(3)	0.74(3)	0.0155(6)	¹⁵¹ Eu	19.700(10)	59(30)	1.2(6)
¹⁴⁵ Nd	589.46(6)	0.97(4)	0.0204(8)	¹⁵¹ Eu	48.31(17)	181(70)	3.6(14)
¹⁴³ Nd	618.062(19)	13.4(3)	0.282(6)	¹⁵¹ Eu	52.39(9)	55(3)	1.10(6)
¹⁴³ Nd	696.499(10)	33.3(23)	0.70(5)	¹⁵¹ Eu	65.1(3)	16(8)	0.32(16)
¹⁴⁵ Nd	735.85(9)	0.479(13)	0.0101(3)	¹⁵³ Eu	68.23(9)	69(20)	1.4(4)
¹⁴² Nd	742.106(22)	3.8(4)	0.080(8)	¹⁵³ Eu	71.24(12)	45(14)	0.9(3)
¹⁴³ Nd	778.58(4)	0.791(20)	0.0166(4)	¹⁵¹ Eu	73.21(9)	106(22)	2.1(4)
¹⁴³ Nd	814.12(3)	4.98(12)	0.1046(25)	¹⁵³ Eu	74.86(12)	43(12)	0.86(24)
¹⁴³ Nd	834.9(5)	0.333(24)	0.0070(5)	¹⁵¹ Eu	77.23(4)	187(13)	3.7(3)
¹⁴³ Nd	863.89(8)	1.07(4)	0.0225(8)	¹⁵¹ Eu	87.13(11)	29(3)	0.58(6)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁵¹ Eu	88.31(12)	42(5)	0.84(10)	¹⁵⁷ Gd	637.474(12)	114(4)	2.20(8)
¹⁵¹Eu	89.847(6)	1430(30)	28.5(6)	¹⁵⁷ Gd	675.43(3)	76(5)	1.46(10)
¹⁵¹ Eu	89.847(6)d	1.300(3)	0.02592[19%]	¹⁵⁷ Gd	688.892(11)	122(7)	2.35(13)
¹⁵¹ Eu	91.20(10)	20(10)	0.40(20)	¹⁵⁷ Gd	743.066(21)	177(5)	3.41(10)
¹⁵³ Eu	100.86(23)	24(5)	0.48(10)	¹⁵⁷ Gd	750.109(10)	118(11)	2.27(21)
¹⁵¹ Eu	103.34(13)	48(5)	0.96(10)	¹⁵⁷ Gd	768.37(3)	221(11)	4.26(21)
¹⁵³ Eu	106.57(14)	42(6)	0.84(12)	¹⁵⁷Gd	780.174(10)	1010(22)	19.5(4)
¹⁵¹ Eu	111.0(3)	22(6)	0.44(12)	¹⁵⁷ Gd	782.28(3)	134(5)	2.58(10)
¹⁵¹ Eu	113.1(3)	15(5)	0.30(10)	¹⁵⁷ Gd	814.602(10)	89(8)	1.72(15)
¹⁵¹ Eu	117.54(10)	14.7(22)	0.29(4)	¹⁵⁷ Gd	820.107(24)	118(7)	2.27(13)
¹⁵¹ Eu	121.71(11)	17.7(25)	0.35(5)	¹⁵⁷ Gd	824.127(24)	133(8)	2.56(15)
¹⁵¹ Eu	124.01(16)	25(3)	0.50(6)	¹⁵⁵ Gd	841.218(12)	80(24)	1.5(5)
¹⁵³ Eu	125.19(16)	25(3)	0.50(6)	¹⁵⁷ Gd	852.885(25)	194(5)	3.74(10)
¹⁵³ Eu	129.06(12)	14.7(16)	0.29(3)	¹⁵⁷ Gd	852.947(9)	202(30)	3.9(6)
¹⁵¹ Eu	132.71(10)	20.7(13)	0.41(3)	¹⁵⁷ Gd	867.682(11)	83(4)	1.60(8)
¹⁵¹ Eu	135.42(9)	27.8(14)	0.55(3)	¹⁵⁷ Gd	870.690(25)	127(19)	2.4(4)
¹⁵¹ Eu	140.19(9)	21(4)	0.42(8)	¹⁵⁷ Gd	870.815(25)	434(11)	8.36(21)
¹⁵¹ Eu	143.54(8)	43(3)	0.86(6)	¹⁵⁷ Gd	870.877(9)	216(40)	4.2(8)
¹⁵³ Eu	154.14(9)	22(3)	0.44(6)	¹⁵⁷ Gd	874.93(3)	151(5)	2.91(10)
¹⁵¹ Eu	167.01(13)	18.9(19)	0.38(4)	¹⁵⁷ Gd	879.29(3)	139(5)	2.68(10)
¹⁵¹ Eu	169.28(9)	54.8(22)	1.09(4)	¹⁵⁷Gd	897.502(10)	1200(50)	23.1(10)
¹⁵¹ Eu	171.95(9)	40(3)	0.80(6)	¹⁵⁷Gd	897.611(10)	1090(50)	21.0(10)
¹⁵³ Eu	179.83(13)	20(3)	0.40(6)	¹⁵⁷ Gd	915.017(10)	394(10)	7.59(19)
¹⁵¹ Eu	182.38(11)	23(3)	0.46(6)	¹⁵⁷ Gd	917.378(25)	262(16)	5.0(3)
¹⁵³ Eu	187.37(8)	31.2(14)	0.62(3)	¹⁵⁷ Gd	917.54(3)	268(7)	5.16(13)
¹⁵¹ Eu	190.96(11)	19.7(14)	0.39(3)	¹⁵⁷ Gd	922.466(20)	98(8)	1.89(15)
¹⁵¹ Eu	193.11(13)	28.3(20)	0.56(4)	¹⁵⁷ Gd	942.404(11)	120(11)	2.31(21)
¹⁵¹ Eu	199.12(10)	25.5(15)	0.51(3)	¹⁵⁷Gd	944.174(10)	3090(70)	59.5(13)
¹⁵¹ Eu	203.63(10)	18.4(14)	0.37(3)	¹⁵⁷ Gd	953.067(21)	73(6)	1.41(12)
¹⁵¹ Eu	206.53(8)	58.7(20)	1.17(4)	¹⁵⁷ Gd	954.296(10)	89(15)	1.7(3)
¹⁵¹ Eu	208.51(18)	16.1(21)	0.32(4)	¹⁵⁵ Gd	959.774(12)	147(50)	2.8(10)
¹⁵¹ Eu	221.30(8)	73(3)	1.46(6)	¹⁵⁷ Gd	960.082(11)	216(17)	4.2(3)
¹⁵¹ Eu	233.22(14)	15.9(23)	0.32(5)	¹⁵⁵ Gd	960.553(14)	84(40)	1.6(8)
¹⁵¹ Eu	244.88(24)	26.3(22)	0.52(4)	¹⁵⁷Gd	962.104(10)	2050(130)	39.5(25)
¹⁵¹ Eu	246.5(3)	15(3)	0.30(6)	¹⁵⁵ Gd	969.877(18)	172(50)	3.3(10)
¹⁵¹ Eu	260.66(9)	15.9(18)	0.32(4)	¹⁵⁷Gd	977.121(10)	1440(21)	27.8(4)
¹⁵¹ Eu	273.65(8)	17.3(12)	0.345(24)	¹⁵⁵ Gd	987.908(21)	144(40)	2.8(8)
¹⁵³ Eu	281.78(9)	20.4(8)	0.407(16)	¹⁵⁷ Gd	998.398(9)	559(40)	10.8(8)
¹⁵¹ Eu	285.10(9)	23.2(18)	0.46(4)	¹⁵⁷ Gd	1000.859(10)	93(4)	1.79(8)
¹⁵³ Eu	299.83(8)	24.0(6)	0.479(12)	¹⁵⁷ Gd	1004.058(9)	404(22)	7.8(4)
Gadolinium (Z=64), At.Wt.=157.25(3), $\sigma_{\gamma}^Z = 48770(150)$				¹⁵⁷ Gd	1007.340(20)	105(4)	2.02(8)
¹⁵⁷Gd	79.5100(10)	4010(100)	77.3(19)	¹⁵⁷ Gd	1010.19(3)	232(7)	4.47(13)
¹⁵⁴ Gd	86.5470(10)	0.57(9)	0.0110(17)	¹⁵⁷ Gd	1034.45(4)	142(5)	2.74(10)
¹⁵⁵Gd	88.9670(10)	1380(40)	26.6(8)	¹⁵⁵ Gd	1040.430(12)	209(60)	4.0(12)
¹⁵² Gd	109.7600(10)	0.089(4)	0.00172(8)	¹⁵⁵ Gd	1065.136(12)	410(120)	7.9(23)
¹⁵⁷Gd	181.931(4)	7200(300)	139(6)	¹⁵⁵ Gd	1067.185(12)	160(50)	3.1(10)
¹⁵⁵Gd	199.2130(10)	2020(60)	38.9(12)	¹⁵⁵ Gd	1079.25(3)	87(30)	1.7(6)
¹⁵⁷ Gd	255.654(4)	350(19)	6.7(4)	¹⁵⁷ Gd	1097.002(10)	662(15)	12.8(3)
¹⁵⁷ Gd	277.544(7)	493(12)	9.50(23)	¹⁵⁷Gd	1107.612(9)	1830(40)	35.3(8)
¹⁵⁵ Gd	296.526(3)	187(5)	3.60(10)	¹⁵⁷ Gd	1116.624(12)	419(9)	8.07(17)
¹⁶⁰ Gd	360.940(20)d	0.199(5)	0.00384[91%]	¹⁵⁷Gd	1119.163(10)	1180(30)	22.7(6)
¹⁵⁷ Gd	528.024(8)	97(11)	1.87(21)	¹⁵⁷ Gd	1141.458(10)	530(30)	10.2(6)
¹⁵⁷ Gd	539.608(5)	144(5)	2.78(10)	¹⁵⁷ Gd	1145.225(9)	82(9)	1.58(17)
¹⁵⁷ Gd	595.728(7)	75(3)	1.45(6)	¹⁵⁵ Gd	1154.102(12)	290(170)	6(3)
¹⁵⁷ Gd	606.400(8)	271(8)	5.22(15)	¹⁵⁵ Gd	1158.986(12)	490(150)	9(3)
¹⁵⁵ Gd	626.275(8)	73(22)	1.4(4)	¹⁵⁵ Gd	1168.874(13)	140(40)	2.7(8)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	
¹⁵⁵ Gd	1174.058(13)	110(30)	2.1(6)	¹⁵⁷ Gd	3989.3(4)	103(22)	2.0(4)	
¹⁵⁷ Gd	1180.328(9)	223(21)	4.3(4)	¹⁵⁷ Gd	4058.48(18)	74(5)	1.43(10)	
¹⁵⁵ Gd	1180.36(4)	189(60)	3.6(12)	¹⁵⁷ Gd	4310.0(3)	76(5)	1.46(10)	
¹⁵⁷Gd	1183.968(10)	958(60)	18.5(12)	¹⁵⁷ Gd	4925.25(13)	235(8)	4.53(15)	
¹⁵⁷Gd	1185.988(9)	1600(90)	30.8(17)	¹⁵⁷ Gd	5058.37(17)	105(5)	2.02(10)	
¹⁵⁵ Gd	1187.120(21)	340(100)	6.6(19)	¹⁵⁷ Gd	5179.16(16)	110(6)	2.12(12)	
¹⁵⁷Gd	1187.122(9)	1420(90)	27.4(17)	¹⁵⁷ Gd	5239.83(17)	83(10)	1.60(19)	
¹⁵⁷ Gd	1219.947(9)	242(12)	4.66(23)	¹⁵⁷ Gd	5250.2(4)	103(17)	2.0(3)	
¹⁵⁵ Gd	1222.349(12)	139(40)	2.7(8)	¹⁵⁷ Gd	5403.38(20)	120(5)	2.31(10)	
¹⁵⁵ Gd	1230.789(23)	390(120)	7.5(23)	¹⁵⁷ Gd	5542.93(12)	112(5)	2.16(10)	
¹⁵⁷ Gd	1237.625(9)	208(9)	4.01(17)	¹⁵⁷ Gd	5582.26(15)	155(6)	2.99(12)	
¹⁵⁵ Gd	1242.481(17)	204(60)	3.9(12)	¹⁵⁷ Gd	5592.95(21)	91(4)	1.75(8)	
¹⁵⁵ Gd	1250.637(21)	113(30)	2.2(6)	¹⁵⁷ Gd	5609.80(20)	75(4)	1.45(8)	
¹⁵⁷ Gd	1255.980(10)	109(4)	2.10(8)	¹⁵⁷ Gd	5661.19(16)	124(5)	2.39(10)	
¹⁵⁷ Gd	1259.837(9)	417(10)	8.04(19)	¹⁵⁷ Gd	5677.28(5)	138(15)	2.7(3)	
¹⁵⁷ Gd	1263.478(10)	641(15)	12.4(3)	¹⁵⁷ Gd	5784.15(5)	105(5)	2.02(10)	
¹⁵⁵ Gd	1277.508(18)	180(50)	3.5(10)	¹⁵⁷ Gd	5903.39(6)	457(14)	8.8(3)	
¹⁵⁷ Gd	1278.932(9)	228(12)	4.39(23)	¹⁵⁷ Gd	6419.82(5)	131(6)	2.52(12)	
¹⁵⁷ Gd	1301.093(9)	213(6)	4.10(12)	¹⁵⁷ Gd	6671.73(5)	83(4)	1.60(8)	
¹⁵⁷ Gd	1323.387(10)	641(16)	12.4(3)	¹⁵⁷Gd	6750.11(5)	965(30)	18.6(6)	
¹⁵⁷ Gd	1327.154(9)	294(9)	5.67(17)	Terbium (Z=65), At.Wt.=158.92534(2), $\sigma_{\gamma}^Z = 23.3(4)$				
¹⁵⁵ Gd	1366.473(18)	97(30)	1.9(6)	¹⁵⁹ Tb	15.413(6)	0.071(12)	0.00135(23)	
¹⁵⁷ Gd	1372.805(10)	195(15)	3.8(3)	¹⁵⁹Tb	29.0170(20)	0.21(4)	0.0040(8)	
¹⁵⁷ Gd	1377.86(8)	87(5)	1.68(10)	¹⁵⁹ Tb	32.652(3)	0.19(3)	0.0036(6)	
¹⁵⁷ Gd	1405.877(10)	101(4)	1.95(8)	¹⁵⁹ Tb	33.1590(10)	0.22(4)	0.0042(8)	
¹⁵⁷ Gd	1437.910(10)	276(10)	5.32(19)	¹⁵⁹ Tb	41.8900(10)	0.64(10)	0.0122(19)	
¹⁵⁵ Gd	1449.849(21)	106(30)	2.0(6)	¹⁵⁹ Tb	50.8690(10)	0.60(15)	0.011(3)	
¹⁵⁷ Gd	1517.419(10)	219(18)	4.2(4)	¹⁵⁹ Tb	54.1290(10)	0.60(15)	0.011(3)	
¹⁵⁷ Gd	1530.279(12)	107(8)	2.06(15)	¹⁵⁹ Tb	59.6430(10)	0.48(6)	0.0092(11)	
¹⁵⁷ Gd	1587.806(10)	105(4)	2.02(8)	¹⁵⁹ Tb	62.374(6)	0.052(15)	0.0010(3)	
¹⁵⁷ Gd	1663.561(11)	105(8)	2.02(15)	¹⁵⁹ Tb	63.6860(10)	1.46(16)	0.028(3)	
¹⁵⁵ Gd	1682.081(19)	108(30)	2.1(6)	¹⁵⁹ Tb	64.1100(20)	1.2(3)	0.023(6)	
¹⁵⁷ Gd	1692.30(6)	88(13)	1.70(25)	¹⁵⁹ Tb	64.8240(20)	0.13(4)	0.0025(8)	
¹⁵⁷ Gd	1774.37(12)	122(40)	2.4(8)	¹⁵⁹ Tb	68.413(3)	0.035(14)	0.0007(3)	
¹⁵⁷ Gd	1781.711(10)	91(22)	1.8(4)	¹⁵⁹ Tb	75.0500(10)	1.78(18)	0.034(3)	
¹⁵⁷ Gd	1815.045(11)	92(20)	1.8(4)	¹⁵⁹ Tb	75.7880(10)	0.14(4)	0.0027(8)	
¹⁵⁷ Gd	1856.41(3)	147(50)	2.8(10)	¹⁵⁹ Tb	78.137(7)	0.034(18)	0.0006(3)	
¹⁵⁷ Gd	1944.269(20)	181(24)	3.5(5)	¹⁵⁹ Tb	78.8670(10)	0.19(4)	0.0036(8)	
¹⁵⁷ Gd	1956.29(12)	175(21)	3.4(4)	¹⁵⁹ Tb	79.099(6)	0.43(6)	0.0082(11)	
¹⁵⁵ Gd	1965.970(25)	80(25)	1.5(5)	¹⁵⁹ Tb	83.8940(20)	0.050(10)	0.00095(19)	
¹⁵⁷ Gd	2023.778(20)	114(30)	2.2(6)	¹⁵⁹ Tb	87.7150(10)	0.160(19)	0.0031(4)	
¹⁵⁷ Gd	2073.593(11)	84(7)	1.62(13)	¹⁵⁹Tb	89.4080(20)	0.21(3)	0.0040(6)	
¹⁵⁷ Gd	2180.474(22)	159(50)	3.1(10)	¹⁵⁹ Tb	92.7590(10)	0.052(16)	0.0010(3)	
¹⁵⁷ Gd	2196.56(16)	120(12)	2.31(23)	¹⁵⁹Tb	93.3060(20)	0.218(25)	0.0042(5)	
¹⁵⁷ Gd	2203.51(11)	151(10)	2.91(19)	¹⁵⁹ Tb	94.0440(20)	0.052(14)	0.0010(3)	
¹⁵⁷ Gd	2259.983(23)	92(6)	1.77(12)	¹⁵⁹ Tb	94.829(3)	0.071(11)	0.00135(21)	
¹⁵⁷ Gd	2314.82(12)	142(6)	2.74(12)	¹⁵⁹ Tb	97.194(10)	0.024(8)	0.00046(15)	
¹⁵⁷ Gd	2459.07(18)	75(6)	1.45(12)	¹⁵⁹Tb	97.503(3)	0.50(6)	0.0095(11)	
¹⁵⁷ Gd	2515.41(20)	88(6)	1.70(12)	¹⁵⁹ Tb	97.967(3)	0.077(19)	0.0015(4)	
¹⁵⁷ Gd	2577.32(15)	100(6)	1.93(12)	¹⁵⁹ Tb	101.0660(20)	0.023(5)	0.00044(10)	
¹⁵⁷ Gd	2617.93(16)	100(6)	1.93(12)	¹⁵⁹ Tb	104.0670(20)	0.15(3)	0.0029(6)	
¹⁵⁷ Gd	2678.60(16)	101(20)	1.9(4)	¹⁵⁹ Tb	108.943(5)	0.026(5)	0.00050(10)	
¹⁵⁷ Gd	2702.34(14)	116(5)	2.24(10)	¹⁵⁹ Tb	112.3730(20)	0.089(10)	0.00170(19)	
¹⁵⁷ Gd	2799.39(17)	87(7)	1.68(13)	¹⁵⁹ Tb	117.950(4)	0.028(5)	0.00053(10)	
¹⁵⁷ Gd	3520.6(3)	83(9)	1.60(17)	¹⁵⁹ Tb	131.058(5)	0.064(8)	0.00122(15)	
¹⁵⁷ Gd	3700.3(4)	99(17)	1.9(3)	¹⁵⁹Tb	135.5970(20)	0.39(4)	0.0074(8)	

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁵⁹ Tb	138.5840(10)	0.052(6)	0.00099(11)	¹⁵⁹ Tb	339.821(6)	0.040(9)	0.00076(17)
¹⁵⁹ Tb	140.784(6)	0.107(12)	0.00204(23)	¹⁵⁹ Tb	340.780(6)	0.069(9)	0.00132(17)
¹⁵⁹ Tb	150.603(3)	0.144(15)	0.0027(3)	¹⁵⁹ Tb	341.731(6)	0.089(15)	0.0017(3)
¹⁵⁹Tb	153.6870(20)	0.44(5)	0.0084(10)	¹⁵⁹ Tb	345.581(8)	0.041(8)	0.00078(15)
¹⁵⁹ Tb	158.9430(20)	0.111(12)	0.00212(23)	¹⁵⁹ Tb	347.032(6)	0.020(4)	0.00038(8)
¹⁵⁹ Tb	163.2420(20)	0.105(11)	0.00200(21)	¹⁵⁹ Tb	348.924(13)	0.053(10)	0.00101(19)
¹⁵⁹ Tb	176.833(3)	0.070(9)	0.00133(17)	¹⁵⁹ Tb	351.095(9)	0.176(22)	0.0034(4)
¹⁵⁹ Tb	178.674(5)	0.049(8)	0.00093(15)	¹⁵⁹ Tb	352.027(10)	0.020(4)	0.00038(8)
¹⁵⁹Tb	178.881(3)	0.42(8)	0.0080(15)	¹⁵⁹ Tb	352.514(6)	0.160(21)	0.0031(4)
¹⁵⁹ Tb	179.832(7)	0.023(4)	0.00044(8)	¹⁵⁹ Tb	356.224(10)	0.117(17)	0.0022(3)
¹⁵⁹ Tb	181.864(5)	0.072(13)	0.00137(25)	¹⁵⁹Tb	357.748(5)	0.26(3)	0.0050(6)
¹⁵⁹ Tb	184.456(5)	0.11(3)	0.0021(6)	¹⁵⁹ Tb	359.960(10)	0.048(9)	0.00092(17)
¹⁵⁹ Tb	185.187(7)	0.094(17)	0.0018(3)	¹⁵⁹ Tb	361.680(14)	0.095(12)	0.00181(23)
¹⁵⁹Tb	193.431(4)	0.37(4)	0.0071(8)	¹⁵⁹ Tb	363.821(6)	0.120(15)	0.0023(3)
¹⁵⁹ Tb	209.738(6)	0.055(6)	0.00105(11)	¹⁵⁹ Tb	370.320(7)	0.057(7)	0.00109(13)
¹⁵⁹ Tb	215.026(6)	0.036(5)	0.00069(10)	¹⁵⁹ Tb	372.980(6)	0.070(8)	0.00133(15)
¹⁵⁹ Tb	221.029(6)	0.022(4)	0.00042(8)	¹⁵⁹ Tb	373.055(12)	0.074(13)	0.00141(25)
¹⁵⁹ Tb	228.252(11)	0.032(4)	0.00061(8)	¹⁵⁹ Tb	374.678(6)	0.099(11)	0.00189(21)
¹⁵⁹ Tb	234.724(7)	0.026(5)	0.00050(10)	¹⁵⁹ Tb	376.515(9)	0.039(9)	0.00074(17)
¹⁵⁹ Tb	236.094(6)	0.032(6)	0.00061(11)	¹⁵⁹ Tb	378.740(8)	0.024(8)	0.00046(15)
¹⁵⁹ Tb	238.653(7)	0.023(5)	0.00044(10)	¹⁵⁹ Tb	398.252(14)	0.024(5)	0.00046(10)
¹⁵⁹ Tb	241.809(5)	0.035(8)	0.00067(15)	¹⁵⁹ Tb	399.512(9)	0.074(11)	0.00141(21)
¹⁵⁹ Tb	242.548(5)	0.018(4)	0.00034(8)	¹⁵⁹ Tb	403.800(13)	0.028(6)	0.00053(11)
¹⁵⁹Tb	242.973(12)	0.219(24)	0.0042(5)	¹⁵⁹ Tb	406.214(12)	0.027(6)	0.00051(11)
¹⁵⁹ Tb	243.277(6)	0.16(3)	0.0031(6)	¹⁵⁹ Tb	413.492(9)	0.066(12)	0.00126(23)
¹⁵⁹Tb	248.062(5)	0.30(3)	0.0057(6)	¹⁵⁹ Tb	414.870(6)	0.132(24)	0.0025(5)
¹⁵⁹ Tb	255.038(6)	0.112(16)	0.0021(3)	¹⁵⁹ Tb	420.630(8)	0.092(12)	0.00175(23)
¹⁵⁹ Tb	255.927(6)	0.052(9)	0.00099(17)	¹⁵⁹ Tb	427.158(9)	0.147(17)	0.0028(3)
¹⁵⁹ Tb	257.541(4)	0.045(7)	0.00086(13)	¹⁵⁹ Tb	430.905(14)	0.023(4)	0.00044(8)
¹⁵⁹ Tb	258.565(9)	0.033(6)	0.00063(11)	¹⁵⁹ Tb	432.079(13)	0.021(8)	0.00040(15)
¹⁵⁹ Tb	262.964(11)	0.022(6)	0.00042(11)	¹⁵⁹ Tb	437.445(9)	0.077(16)	0.0015(3)
¹⁵⁹ Tb	264.989(5)	0.031(7)	0.00059(13)	¹⁵⁹ Tb	442.212(14)	0.077(12)	0.00147(23)
¹⁵⁹ Tb	270.762(7)	0.102(12)	0.00194(23)	¹⁵⁹ Tb	447.390(9)	0.10(3)	0.0019(6)
¹⁵⁹ Tb	274.385(11)	0.021(4)	0.00040(8)	¹⁵⁹ Tb	448.105(12)	0.054(10)	0.00103(19)
¹⁵⁹ Tb	275.707(5)	0.124(14)	0.0024(3)	¹⁵⁹Tb	451.617(10)	0.21(3)	0.0040(6)
¹⁵⁹ Tb	277.818(6)	0.093(11)	0.00177(21)	¹⁵⁹ Tb	453.266(10)	0.033(12)	0.00063(23)
¹⁵⁹ Tb	278.152(7)	0.025(6)	0.00048(11)	¹⁵⁹ Tb	455.783(10)	0.029(12)	0.00055(23)
¹⁵⁹ Tb	278.803(7)	0.083(11)	0.00158(21)	¹⁵⁹ Tb	459.519(10)	0.085(12)	0.00162(23)
¹⁵⁹ Tb	282.698(5)	0.049(8)	0.00093(15)	¹⁵⁹Tb	464.264(17)	0.192(21)	0.0037(4)
¹⁵⁹ Tb	283.289(7)	0.052(9)	0.00099(17)	¹⁵⁹ Tb	492.460(13)	0.024(6)	0.00046(11)
¹⁵⁹ Tb	284.148(9)	0.087(11)	0.00166(21)	¹⁵⁹ Tb	496.916(17)	0.041(9)	0.00078(17)
¹⁵⁹ Tb	287.738(9)	0.029(5)	0.00055(10)	¹⁵⁹ Tb	519.790(14)	0.059(13)	0.00113(25)
¹⁵⁹ Tb	288.212(5)	0.126(14)	0.0024(3)	¹⁵⁹ Tb	521.308(21)	0.046(12)	0.00088(23)
¹⁵⁹ Tb	290.625(10)	0.052(7)	0.00099(13)	¹⁵⁹ Tb	525.194(17)	0.080(17)	0.0015(3)
¹⁵⁹ Tb	295.757(9)	0.062(8)	0.00118(15)	¹⁵⁹Tb	525.933(17)	0.22(3)	0.0042(6)
¹⁵⁹ Tb	302.735(13)	0.086(10)	0.00164(19)	¹⁵⁹ Tb	529.054(10)	0.022(8)	0.00042(15)
¹⁵⁹ Tb	303.114(10)	0.042(8)	0.00080(15)	¹⁵⁹ Tb	530.981(24)	0.037(10)	0.00071(19)
¹⁵⁹ Tb	308.102(9)	0.056(8)	0.00107(15)	¹⁵⁹ Tb	532.689(21)	0.129(16)	0.0025(3)
¹⁵⁹ Tb	310.470(5)	0.177(21)	0.0034(4)	¹⁵⁹ Tb	532.733(9)	0.15(3)	0.0029(6)
¹⁵⁹ Tb	310.804(6)	0.019(5)	0.00036(10)	¹⁵⁹ Tb	542.840(21)	0.034(8)	0.00065(15)
¹⁵⁹ Tb	315.857(5)	0.118(14)	0.0023(3)	¹⁵⁹ Tb	544.922(10)	0.064(10)	0.00122(19)
¹⁵⁹ Tb	316.564(9)	0.027(5)	0.00051(10)	¹⁵⁹ Tb	545.661(10)	0.056(11)	0.00107(21)
¹⁵⁹ Tb	317.597(5)	0.121(15)	0.0023(3)	¹⁵⁹ Tb	554.509(6)	0.021(7)	0.00040(13)
¹⁵⁹ Tb	319.862(6)	0.132(15)	0.0025(3)	¹⁵⁹ Tb	585.575(17)	0.054(8)	0.00103(15)
¹⁵⁹ Tb	323.809(6)	0.022(4)	0.00042(8)	¹⁵⁹ Tb	598.656(14)	0.020(6)	0.00038(11)
¹⁵⁹Tb	339.487(5)	0.35(4)	0.0067(8)	¹⁵⁹ Tb	600.206(24)	0.155(18)	0.0030(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁵⁹ Tb	611.513(24)	0.034(9)	0.00065(17)	¹⁶² Dy	250.8900(20)	5.2(6)	0.097(11)
¹⁵⁹ Tb	625.994(21)	0.027(7)	0.00051(13)	¹⁶¹ Dy	260.11(7)	8.3(3)	0.155(6)
¹⁵⁹ Tb	634.737(24)	0.037(7)	0.00071(13)	¹⁶⁴ Dy	271.727(9)	2.90(17)	0.054(3)
¹⁵⁹ Tb	5184.2(3)	0.023(9)	0.00044(17)	¹⁶³ Dy	277.500(16)	1.51(16)	0.028(3)
¹⁵⁹ Tb	5199.9(3)	0.033(8)	0.00063(15)	¹⁶¹ Dy	282.89(7)	7.8(3)	0.145(6)
¹⁵⁹ Tb	5204.5(3)	0.040(9)	0.00076(17)	¹⁶³ Dy	294.575(13)	2.78(19)	0.052(4)
¹⁵⁹ Tb	5225.0(3)	0.040(13)	0.00076(25)	¹⁶¹ Dy	311.39(15)	2.1(4)	0.039(8)
¹⁵⁹ Tb	5228.45(25)	0.052(12)	0.00099(23)	¹⁶² Dy	316.3090(10)	3.0(4)	0.056(8)
¹⁵⁹ Tb	5238.1(3)	0.026(10)	0.00050(19)	¹⁶¹ Dy	321.84(12)	1.74(25)	0.032(5)
¹⁵⁹ Tb	5245.6(3)	0.061(13)	0.00116(25)	¹⁶⁴ Dy	331.126(8)	4.5(4)	0.084(8)
¹⁵⁹ Tb	5250.2(3)	0.064(12)	0.00122(23)	¹⁶¹ Dy	334.08(8)	4.9(4)	0.091(8)
¹⁵⁹ Tb	5259.2(3)	0.022(5)	0.00042(10)	¹⁶² Dy	338.5310(20)	1.50(17)	0.028(3)
¹⁵⁹ Tb	5288.99(25)	0.027(7)	0.00051(13)	¹⁶⁴ Dy	343.312(4)	3.2(4)	0.060(8)
¹⁵⁹ Tb	5306.9(3)	0.021(6)	0.00040(11)	¹⁶⁴ Dy	345.860(12)	1.8(3)	0.034(6)
¹⁵⁹ Tb	5373.1(4)	0.024(5)	0.00046(10)	¹⁶² Dy	347.9050(20)	1.84(22)	0.034(4)
¹⁵⁹ Tb	5461.09(25)	0.029(7)	0.00055(13)	¹⁶⁴Dy	349.248(10)	14.7(6)	0.274(11)
¹⁵⁹ Tb	5516.2(5)	0.019(7)	0.00036(13)	¹⁶² Dy	351.1490(10)	10.9(9)	0.203(17)
¹⁵⁹ Tb	5524.2(3)	0.051(13)	0.00097(25)	¹⁶⁴ Dy	352.581(10)	1.7(4)	0.032(8)
¹⁵⁹ Tb	5551.8(3)	0.029(5)	0.00055(10)	¹⁶² Dy	354.2360(10)	3.5(21)	0.07(4)
¹⁵⁹ Tb	5607.07(7)	0.042(9)	0.00080(17)	¹⁶⁴ Dy	354.353(8)	3.3(10)	0.062(19)
¹⁵⁹ Tb	5611.6(3)	0.025(5)	0.00048(10)	¹⁶⁴ Dy	357.686(8)	2.4(4)	0.045(8)
¹⁵⁹ Tb	5661.8(5)	0.037(7)	0.00071(13)	¹⁶¹ Dy	361.70(10)	4.1(4)	0.076(8)
¹⁵⁹ Tb	5682.5(3)	0.027(7)	0.00051(13)	¹⁶⁴ Dy	368.727(8)	1.6(3)	0.030(6)
¹⁵⁹ Tb	5696.8(3)	0.034(6)	0.00065(11)	¹⁶⁴ Dy	380.020(8)	4.1(4)	0.076(8)
¹⁵⁹ Tb	5710.36(7)	0.029(5)	0.00055(10)	¹⁶⁴Dy	385.9840(20)	34.8(10)	0.649(19)
¹⁵⁹ Tb	5754.34(21)	0.031(8)	0.00059(15)	¹⁶² Dy	389.7530(10)	7.7(7)	0.144(13)
¹⁵⁹ Tb	5776.37(7)	0.120(17)	0.0023(3)	¹⁶⁴ Dy	392.651(7)	11.3(5)	0.211(9)
¹⁵⁹ Tb	5782.28(7)	0.041(9)	0.00078(17)	¹⁶⁴ Dy	396.208(4)	2.4(9)	0.045(17)
¹⁵⁹ Tb	5842.29(7)	0.054(10)	0.00103(19)	¹⁶⁴ Dy	399.726(6)	2.0(4)	0.037(8)
¹⁵⁹ Tb	5860.03(23)	0.036(8)	0.00069(15)	¹⁶² Dy	401.9440(10)	1.62(19)	0.030(4)
¹⁵⁹ Tb	5890.70(7)	0.137(19)	0.0026(4)	¹⁶⁴ Dy	403.059(6)	3.5(4)	0.065(8)
¹⁵⁹ Tb	5896.46(7)	0.023(7)	0.00044(13)	¹⁶⁴Dy	411.651(5)	35.1(10)	0.655(19)
¹⁵⁹ Tb	5953.58(7)	0.103(13)	0.00196(25)	¹⁶⁴Dy	414.985(7)	31(5)	0.58(9)
¹⁵⁹ Tb	5993.73(7)	0.114(15)	0.0022(3)	¹⁶² Dy	415.0610(20)	1.57(19)	0.029(4)
¹⁵⁹ Tb	6138.03(7)	0.110(15)	0.0021(3)	¹⁶⁴ Dy	420.833(3)	11.8(11)	0.220(21)
¹⁵⁹Tb	6218.56(7)	0.190(22)	0.0036(4)	¹⁶² Dy	421.8440(10)	7.1(9)	0.132(17)
¹⁵⁹ Tb	6235.53(7)	0.020(6)	0.00038(11)	¹⁶⁴ Dy	425.346(10)	2.4(7)	0.045(13)
¹⁵⁹ Tb	6241.78(7)	0.072(10)	0.00137(19)	¹⁶¹ Dy	427.57(13)	1.66(25)	0.031(5)
¹⁵⁹ Tb	6269.43(7)	0.029(6)	0.00055(11)	¹⁶² Dy	427.6800(10)	1.86(22)	0.035(4)
¹⁵⁹ Tb	6311.32(7)	0.028(6)	0.00053(11)	¹⁶⁴ Dy	430.451(8)	4.2(3)	0.078(6)
Dysprosium (Z=66), At. Wt.=162.50(3), σ_γ^Z=944(21)				¹⁶⁴Dy	447.893(7)	17.4(5)	0.324(9)
¹⁶⁴Dy	50.4310(20)	33.9(15)	0.63(3)	¹⁶⁴Dy	465.416(6)	38.0(10)	0.709(19)
¹⁶⁴ Dy	72.765(3)	7.1(3)	0.132(6)	¹⁶⁴ Dy	470.227(7)	9.3(6)	0.173(11)
¹⁶³ Dy	73.392(8)	1.70(24)	0.032(5)	¹⁶⁴ Dy	474.22(7)	6.4(4)	0.119(8)
¹⁶⁴ Dy	77.520(3)	2.7(5)	0.050(9)	¹⁶⁴ Dy	474.95(4)	3.3(10)	0.062(19)
¹⁶¹Dy	80.64(7)	16.5(5)	0.308(9)	¹⁶² Dy	475.3880(10)	1.71(21)	0.032(4)
¹⁶⁴ Dy	83.395(3)	3.51(20)	0.065(4)	¹⁶⁴Dy	477.061(6)	22(7)	0.41(13)
¹⁶⁴ Dy	108.159(3)d	13.6(5)	0.254[97%]	¹⁶⁴Dy	477.08(4)	15.8(5)	0.295(9)
¹⁶⁴ Dy	116.768(4)	3.28(17)	0.061(3)	¹⁶⁴Dy	496.931(5)	44.9(11)	0.837(21)
¹⁶⁴ Dy	139.102(4)	6.16(19)	0.115(4)	¹⁶⁴ Dy	499.395(6)	13.0(10)	0.242(19)
¹⁶⁴ Dy	156.245(5)	1.82(10)	0.0339(19)	¹⁶⁴ Dy	500.37(8)	10.3(5)	0.192(9)
¹⁶³ Dy	168.838(5)	4.7(6)	0.088(11)	¹⁶⁴ Dy	500.587(6)	10(3)	0.19(6)
¹⁶⁴ Dy	178.382(5)	1.8(3)	0.034(6)	¹⁶⁴ Dy	506.47(4)	6.4(4)	0.119(8)
¹⁶⁴Dy	184.257(4)	146(15)	2.7(3)	¹⁶⁴ Dy	508.96(4)	9.5(6)	0.177(11)
¹⁶¹Dy	185.19(9)	39.1(12)	0.729(22)	¹⁶⁴ Dy	519.05(7)	1.5(3)	0.028(6)
¹⁶³ Dy	215.082(21)	3.07(17)	0.057(3)	¹⁶⁴ Dy	524.41(6)	4.7(5)	0.088(9)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁶⁴ Dy	529.46(7)	3.0(10)	0.056(19)	¹⁶¹ Dy	1308.5(3)	1.7(4)	0.032(8)
¹⁶⁴ Dy	529.54(8)	2.5(4)	0.047(8)	¹⁶¹ Dy	1316.7(5)	1.5(4)	0.028(8)
¹⁶⁴Dy	538.609(8)	69.2(19)	1.29(4)	¹⁶¹ Dy	1371.4(3)	2.4(4)	0.045(8)
¹⁶⁴ Dy	546.54(4)	3.7(4)	0.069(8)	¹⁶⁴ Dy	1410.99(8)	4.6(5)	0.086(9)
¹⁶⁴ Dy	556.932(7)	2.2(4)	0.041(8)	¹⁶⁴ Dy	1433.33(8)	1.9(4)	0.035(8)
¹⁶⁴ Dy	565.567(4)	5.1(5)	0.095(9)	¹⁶⁴ Dy	1483.76(8)	3.6(4)	0.067(8)
¹⁶⁴ Dy	569.53(7)	8.3(25)	0.15(5)	¹⁶¹ Dy	1573.95(23)	1.7(3)	0.032(6)
¹⁶⁴ Dy	569.79(6)	9.7(5)	0.181(9)	¹⁶⁴ Dy	1596.37(15)	2.5(4)	0.047(8)
¹⁶¹ Dy	572.7(4)	2.2(9)	0.041(17)	¹⁶⁴ Dy	1604.4(3)	1.7(4)	0.032(8)
¹⁶¹ Dy	572.88(7)	1.65(12)	0.0308(22)	¹⁶⁴ Dy	1616.1(3)	1.5(4)	0.028(8)
¹⁶⁴Dy	583.982(5)	24(7)	0.45(13)	¹⁶⁴ Dy	1646.80(15)	2.2(3)	0.041(6)
¹⁶⁴ Dy	596.71(4)	5.1(3)	0.095(6)	¹⁶⁴ Dy	1671.84(13)	3.6(5)	0.067(9)
¹⁶⁴ Dy	613.13(9)	2.5(3)	0.047(6)	¹⁶¹ Dy	1717.18(13)	3.0(4)	0.056(8)
¹⁶¹ Dy	647.50(12)	3.11(21)	0.058(4)	¹⁶⁴ Dy	1722.27(13)	3.2(4)	0.060(8)
¹⁶³ Dy	673.71(4)	1.7(4)	0.032(8)	¹⁶⁴ Dy	1737.35(15)	3.8(4)	0.071(8)
¹⁶³ Dy	688.36(4)	4.7(4)	0.088(8)	¹⁶¹ Dy	1781.5(3)	3.5(6)	0.065(11)
¹⁶¹ Dy	697.16(9)	3.3(3)	0.062(6)	¹⁶⁴ Dy	1806.00(25)	2.4(5)	0.045(9)
¹⁶¹ Dy	711.41(12)	2.28(22)	0.043(4)	¹⁶¹ Dy	1823.7(7)	1.9(5)	0.035(9)
¹⁶³ Dy	754.75(4)	6.4(4)	0.119(8)	¹⁶⁴ Dy	1835.40(18)	3.2(6)	0.060(11)
¹⁶³ Dy	761.76(4)	4.1(3)	0.076(6)	¹⁶⁴ Dy	1866.28(13)	2.6(4)	0.048(8)
¹⁶¹ Dy	795.27(8)	6.8(4)	0.127(8)	¹⁶⁴ Dy	2019.4(3)	2.5(5)	0.047(9)
¹⁶¹ Dy	807.46(7)	12.1(5)	0.226(9)	¹⁶⁴ Dy	2091.58(11)	2.6(5)	0.048(9)
¹⁶¹ Dy	842.48(22)	1.6(4)	0.030(8)	¹⁶¹ Dy	2110.01(16)	3.6(4)	0.067(8)
¹⁶¹ Dy	842.5(4)	1.48(25)	0.028(5)	¹⁶⁴ Dy	2113.91(11)	4.0(4)	0.075(8)
¹⁶¹Dy	882.27(6)	18.3(6)	0.341(11)	¹⁶⁴ Dy	2164.34(11)	3.1(4)	0.058(8)
¹⁶¹ Dy	888.13(7)	10.4(5)	0.194(9)	¹⁶⁴ Dy	2226.92(19)	2.7(5)	0.050(9)
¹⁶¹ Dy	917.16(10)	5.4(5)	0.101(9)	¹⁶⁴ Dy	2242.3(3)	3.3(5)	0.062(9)
¹⁶⁴ Dy	922.11(7)	1.6(6)	0.030(11)	¹⁶⁴ Dy	2259.3(3)	2.8(5)	0.052(9)
¹⁶¹ Dy	933.70(23)	3.1(7)	0.058(13)	¹⁶⁴ Dy	2272.0(6)	3.6(7)	0.067(13)
¹⁶⁴ Dy	933.94(8)	4.6(7)	0.086(13)	¹⁶⁴ Dy	2305.5(3)	2.2(5)	0.041(9)
¹⁶¹ Dy	944.40(7)	7.2(3)	0.134(6)	¹⁶⁴ Dy	2313.8(4)	7.2(6)	0.134(11)
¹⁶¹ Dy	976.83(13)	3.4(3)	0.063(6)	¹⁶⁴ Dy	2369.89(24)	4.2(6)	0.078(11)
¹⁶¹ Dy	979.98(9)	8.5(4)	0.159(8)	¹⁶⁴ Dy	2412.2(4)	2.6(6)	0.048(11)
¹⁶¹ Dy	994.64(7)	9.2(4)	0.172(8)	¹⁶⁴ Dy	2552.64(19)	5.3(6)	0.099(11)
¹⁶⁴ Dy	994.87(7)	5.6(17)	0.10(3)	¹⁶⁴ Dy	2593.02(19)	3.0(5)	0.056(9)
¹⁶¹ Dy	1008.42(22)	2.0(3)	0.037(6)	¹⁶⁴ Dy	2606.94(19)	4.1(5)	0.076(9)
¹⁶⁴ Dy	1018.35(8)	3.7(12)	0.069(22)	¹⁶⁴ Dy	2635.0(3)	3.0(5)	0.056(9)
¹⁶¹ Dy	1025.5(3)	1.7(4)	0.032(8)	¹⁶² Dy	2660.1(4)	6.6(11)	0.123(21)
¹⁶¹ Dy	1058.41(9)	5.9(4)	0.110(8)	¹⁶⁴ Dy	2683.54(24)	2.4(5)	0.045(9)
¹⁶⁴ Dy	1059.63(9)	2.2(7)	0.041(13)	¹⁶⁴ Dy	2702.83(21)	6.9(22)	0.13(4)
¹⁶⁴ Dy	1064.18(9)	2.2(6)	0.041(11)	¹⁶⁴ Dy	2823.8(4)	1.7(5)	0.032(9)
¹⁶⁴ Dy	1074.59(9)	4.5(14)	0.08(3)	¹⁶⁴ Dy	2832.15(21)	1.9(5)	0.035(9)
¹⁶¹ Dy	1091.99(13)	2.7(4)	0.050(8)	¹⁶⁴ Dy	2840.1(3)	3.8(5)	0.071(9)
¹⁶¹ Dy	1108.53(10)	5.1(4)	0.095(8)	¹⁶⁴ Dy	2854.48(21)	4.0(5)	0.075(9)
¹⁶⁴ Dy	1110.06(9)	2.6(7)	0.048(13)	¹⁶⁴ Dy	2863.5(4)	5.1(5)	0.095(9)
¹⁶¹ Dy	1124.81(9)	4.0(3)	0.075(6)	¹⁶⁴ Dy	2872.20(21)	4.5(5)	0.084(9)
¹⁶¹ Dy	1129.40(9)	5.7(4)	0.106(8)	¹⁶⁴ Dy	2931.8(3)	2.7(5)	0.050(9)
¹⁶¹ Dy	1158.2(3)	2.1(4)	0.039(8)	¹⁶⁴ Dy	2950.37(19)	4.5(5)	0.084(9)
¹⁶¹ Dy	1185.0(3)	1.5(4)	0.028(8)	¹⁶⁴ Dy	2999.9(4)	1.7(4)	0.032(8)
¹⁶¹ Dy	1187.7(3)	1.6(4)	0.030(8)	¹⁶⁴ Dy	3012.42(17)	7.8(5)	0.145(9)
¹⁶¹ Dy	1195.37(12)	3.6(4)	0.067(8)	¹⁶⁴ Dy	3035.55(15)	10.9(6)	0.203(11)
¹⁶¹ Dy	1219.6(3)	2.7(10)	0.050(19)	¹⁶⁴ Dy	3071.02(24)	3.8(5)	0.071(9)
¹⁶⁴ Dy	1260.19(13)	2.0(6)	0.037(11)	¹⁶⁴ Dy	3098.52(24)	2.1(4)	0.039(8)
¹⁶¹ Dy	1260.66(21)	3.2(5)	0.060(9)	¹⁶⁴ Dy	3105.83(21)	5.8(5)	0.108(9)
¹⁶¹ Dy	1276.3(6)	1.9(4)	0.035(8)	¹⁶⁴ Dy	3114.06(19)	7.4(6)	0.138(11)
¹⁶¹ Dy	1276.78(12)	6.3(6)	0.117(11)	¹⁶⁴ Dy	3169.10(24)	3.3(4)	0.062(8)
				¹⁶⁴ Dy	3198.3(3)	1.6(3)	0.030(6)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁶⁴ Dy	3238.1(3)	4.7(5)	0.088(9)	¹⁶⁵ Ho	181.0870(20)	0.94(5)	0.0173(9)
¹⁶⁴ Dy	3276.05(13)	6.1(5)	0.114(9)	¹⁶⁵ Ho	186.579(4)	0.197(22)	0.0036(4)
¹⁶⁴ Dy	3315.0(3)	3.0(4)	0.056(8)	¹⁶⁵ Ho	197.342(3)	0.34(3)	0.0062(6)
¹⁶⁴ Dy	3443.39(11)	10.6(16)	0.20(3)	¹⁶⁵ Ho	199.700(5)	0.48(3)	0.0088(6)
¹⁶⁴ Dy	3537.9(3)	3.2(5)	0.060(9)	¹⁶⁵ Ho	210.309(4)	0.180(15)	0.0033(3)
¹⁶⁴ Dy	3555.71(20)	4.7(5)	0.088(9)	¹⁶⁵Ho	221.186(4)	2.05(11)	0.0377(20)
¹⁶⁴ Dy	3608.5(4)	3.1(4)	0.058(8)	¹⁶⁵ Ho	231.960(7)	0.23(5)	0.0042(9)
¹⁶⁴ Dy	3628.2(3)	1.9(4)	0.035(8)	¹⁶⁵ Ho	233.116(8)	0.38(4)	0.0070(7)
¹⁶⁴ Dy	3772.33(18)	3.1(4)	0.058(8)	¹⁶⁵Ho	239.132(4)	2.25(12)	0.0413(22)
¹⁶⁴ Dy	3819.95(15)	2.7(5)	0.050(9)	¹⁶⁵ Ho	245.010(5)	0.47(5)	0.0086(9)
¹⁶⁴ Dy	3840.49(24)	4.9(6)	0.091(11)	¹⁶⁵ Ho	257.806(11)	0.18(4)	0.0033(7)
¹⁶⁴ Dy	3885.46(13)	5.2(4)	0.097(8)	¹⁶⁵ Ho	265.983(10)	0.170(14)	0.0031(3)
¹⁶⁴ Dy	3944.8(3)	2.2(3)	0.041(6)	¹⁶⁵ Ho	267.241(6)	0.199(15)	0.0037(3)
¹⁶⁴ Dy	3960.93(15)	4.7(4)	0.088(8)	¹⁶⁵ Ho	289.124(14)	1.16(6)	0.0213(11)
¹⁶⁴ Dy	4067.73(9)	2.5(4)	0.047(8)	¹⁶⁵ Ho	290.617(7)	0.96(5)	0.0176(9)
¹⁶⁴ Dy	4083.81(14)	4.3(4)	0.080(8)	¹⁶⁵ Ho	297.905(4)	0.188(14)	0.0035(3)
¹⁶⁴ Dy	4123.97(8)	13.1(9)	0.244(17)	¹⁶⁵ Ho	304.617(6)	1.34(7)	0.0246(13)
¹⁶⁴ Dy	4155.82(8)	2.1(3)	0.039(6)	¹⁶⁵ Ho	328.239(10)	0.391(23)	0.0072(4)
¹⁶⁴ Dy	4459.45(8)	1.6(3)	0.030(6)	¹⁶⁵ Ho	333.614(5)	1.04(6)	0.0191(11)
¹⁶⁴ Dy	4607.48(6)	1.9(4)	0.035(8)	¹⁶⁵ Ho	335.585(6)	0.33(7)	0.0061(13)
¹⁶⁴ Dy	4612.84(7)	5.7(5)	0.106(9)	¹⁶⁵ Ho	343.540(6)	0.203(13)	0.00373(24)
¹⁶⁴ Dy	4635.84(5)	2.6(4)	0.048(8)	¹⁶⁵ Ho	357.056(5)	0.162(12)	0.00298(22)
¹⁶⁴ Dy	5110.77(3)	6.1(9)	0.114(17)	¹⁶⁵Ho	371.772(5)	1.56(8)	0.0287(15)
¹⁶⁴Dy	5142.29(3)	15.7(10)	0.293(19)	¹⁶⁵ Ho	391.819(7)	0.51(5)	0.0094(9)
¹⁶⁴ Dy	5145.62(3)	8.4(24)	0.16(5)	¹⁶⁵ Ho	401.595(8)	1.07(9)	0.0197(17)
¹⁶⁴ Dy	5177.25(3)	6.6(5)	0.123(9)	¹⁶⁵ Ho	410.265(6)	1.23(7)	0.0226(13)
¹⁶¹ Dy	5450.27(25)	2.1(4)	0.039(8)	¹⁶⁵ Ho	411.087(12)	0.40(12)	0.0073(22)
¹⁶⁴Dy	5557.26(3)	28.7(14)	0.54(3)	¹⁶⁵ Ho	412.030(8)	0.32(7)	0.0059(13)
¹⁶⁴Dy	5607.69(3)	35.9(16)	0.67(3)	¹⁶⁵ Ho	416.550(5)	0.42(4)	0.0077(7)
¹⁶⁰ Dy	6087.25(13)	0.85(5)	0.0159(9)	¹⁶⁵ Ho	425.300(21)	0.69(17)	0.013(3)
Holmium (Z=67, At.Wt.=164.93032(2), $\sigma_{\gamma}^Z = 64.7(12)$)				¹⁶⁵Ho	426.012(5)	2.88(15)	0.053(3)
¹⁶⁵ Ho	19.8290(20)	0.57(8)	0.0105(15)	¹⁶⁵ Ho	427.196(6)	0.21(5)	0.0039(9)
¹⁶⁵ Ho	38.494(5)	0.179(20)	0.0033(4)	¹⁶⁵ Ho	442.231(21)	0.22(3)	0.0040(6)
¹⁶⁵ Ho	54.2400(10)	1.41(4)	0.0259(7)	¹⁶⁵ Ho	443.148(8)	0.164(12)	0.00301(22)
¹⁶⁵ Ho	57.521(6)	0.17(3)	0.0031(6)	¹⁶⁵ Ho	455.567(11)	0.78(4)	0.0143(7)
¹⁶⁵ Ho	69.7610(10)	1.09(6)	0.0200(11)	¹⁶⁵ Ho	457.349(11)	0.213(17)	0.0039(3)
¹⁶⁵ Ho	72.8870(10)	0.17(3)	0.0031(6)	¹⁶⁵ Ho	463.927(6)	0.245(18)	0.0045(3)
¹⁶⁵ Ho	76.4670(10)	0.179(20)	0.0033(4)	¹⁶⁵ Ho	467.227(5)	0.162(17)	0.0030(3)
¹⁶⁵ Ho	76.7270(10)	0.33(3)	0.0061(6)	¹⁶⁵ Ho	481.354(18)	0.45(7)	0.0083(13)
¹⁶⁵Ho	80.574(8)d	3.87(5)	0.0711[1.3%]	¹⁶⁵ Ho	487.538(6)	0.394(24)	0.0072(4)
¹⁶⁵ Ho	82.4710(20)	0.42(3)	0.0077(6)	¹⁶⁵ Ho	489.436(4)	1.15(6)	0.0211(11)
¹⁶⁵ Ho	87.5950(20)	0.71(4)	0.0130(7)	¹⁶⁵ Ho	496.932(6)	0.16(3)	0.0029(6)
¹⁶⁵ Ho	94.628(6)	0.156(23)	0.0029(4)	¹⁶⁵ Ho	509.094(24)	0.332(22)	0.0061(4)
¹⁶⁵ Ho	98.8590(10)	0.270(17)	0.0050(3)	¹⁶⁵ Ho	512.770(6)	0.323(22)	0.0059(4)
¹⁶⁵ Ho	105.516(3)	0.234(16)	0.0043(3)	¹⁶⁵ Ho	524.250(22)	0.260(17)	0.0048(3)
¹⁶⁵ Ho	108.2000(20)	0.40(3)	0.0073(6)	¹⁶⁵ Ho	533.644(21)	0.303(20)	0.0056(4)
¹⁶⁵ Ho	111.3260(20)	0.294(20)	0.0054(4)	¹⁶⁵ Ho	534.572(11)	0.16(3)	0.0029(6)
¹⁶⁵Ho	116.8360(10)	8.1(4)	0.149(7)	¹⁶⁵ Ho	538.259(8)	0.152(21)	0.0028(4)
¹⁶⁵ Ho	126.230(3)	0.55(4)	0.0101(7)	¹⁶⁵Ho	542.780(4)	1.94(13)	0.0356(24)
¹⁶⁵Ho	136.6650(20)	14.5(7)	0.266(13)	¹⁶⁵ Ho	543.676(5)	1.00(5)	0.0184(9)
¹⁶⁵ Ho	140.122(5)	0.27(3)	0.0050(6)	¹⁶⁵ Ho	554.400(11)	0.32(7)	0.0059(13)
¹⁶⁵Ho	149.309(3)	2.25(12)	0.0413(22)	¹⁶⁵ Ho	576.902(16)	0.203(17)	0.0037(3)
¹⁶⁵ Ho	163.353(7)	0.223(15)	0.0041(3)	¹⁶⁵ Ho	577.141(11)	0.37(6)	0.0068(11)
¹⁶⁵ Ho	167.453(5)	0.55(3)	0.0101(6)	¹⁶⁵ Ho	613.768(6)	0.332(22)	0.0061(4)
¹⁶⁵ Ho	169.715(5)	0.150(14)	0.0028(3)	¹⁶⁵ Ho	624.234(8)	0.212(16)	0.0039(3)
¹⁶⁵ Ho	179.036(5)	0.220(16)	0.0040(3)	¹⁶⁵ Ho	633.641(8)	0.36(3)	0.0066(6)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁶⁵ Ho	689.72(3)	0.44(3)	0.0081(6)	¹⁶⁷ Er	821.1680(20)	6.2(3)	0.112(5)
¹⁶⁵ Ho	734.258(16)	0.253(18)	0.0046(3)	¹⁶⁷ Er	823.3810(20)	1.34(10)	0.0243(18)
¹⁶⁵ Ho	4855.89(3)	0.146(18)	0.0027(3)	¹⁶⁷ Er	825.727(3)	0.89(9)	0.0161(16)
¹⁶⁵ Ho	4945.18(5)	0.214(19)	0.0039(4)	¹⁶⁷ Er	829.9480(10)	4.12(19)	0.075(3)
¹⁶⁵ Ho	5108.66(7)	0.33(3)	0.0061(6)	¹⁶⁷ Er	853.4810(10)	7.5(3)	0.136(5)
¹⁶⁵ Ho	5128.946(13)	0.171(17)	0.0031(3)	¹⁶⁷ Er	862.3500(20)	1.16(6)	0.0210(11)
¹⁶⁵ Ho	5181.841(20)	0.253(20)	0.0046(4)	¹⁶⁷ Er	914.9420(10)	6.99(24)	0.127(4)
¹⁶⁵ Ho	5213.240(21)	0.260(24)	0.0048(4)	¹⁶⁷ Er	928.9330(20)	1.55(8)	0.0281(14)
¹⁶⁵ Ho	5428.441(9)	0.223(23)	0.0041(4)	¹⁶⁷ Er	932.2660(20)	0.83(5)	0.0150(9)
¹⁶⁵ Ho	5524.219(11)	0.192(20)	0.0035(4)	¹⁶⁷ Er	965.9330(20)	0.83(5)	0.0150(9)
¹⁶⁵ Ho	5813.531(7)	0.54(4)	0.0099(7)	¹⁶⁷ Er	999.8150(20)	0.99(6)	0.0179(11)
¹⁶⁵ Ho	5870.477(9)	0.224(20)	0.0041(4)	¹⁶⁷ Er	1012.1810(20)	1.42(7)	0.0257(13)
¹⁶⁵ Ho	5871.573(6)	0.196(18)	0.0036(3)	¹⁶⁷ Er	1025.368(4)	0.97(6)	0.0176(11)
¹⁶⁵ Ho	6052.654(6)	0.188(19)	0.0035(4)	¹⁶⁷ Er	1144.133(3)	0.58(5)	0.0105(9)
Erbium (Z=68), At.Wt.=167.259(3), $\sigma_{\gamma}^Z = 156.8(19)$				¹⁶⁷ Er	1147.0040(20)	0.92(6)	0.0167(11)
¹⁶² Er	69.4(6)	0.35(14)	0.0063(25)	¹⁶⁷ Er	1167.373(4)	1.98(8)	0.0359(14)
¹⁶⁷ Er	79.8040(10)	18.2(8)	0.330(14)	¹⁶⁷ Er	1173.577(4)	0.71(5)	0.0129(9)
¹⁶⁷ Er	98.9850(10)	3.73(14)	0.0676(25)	¹⁶⁷ Er	1196.4640(20)	0.82(5)	0.0149(9)
¹⁶⁷ Er	99.2910(10)	2.2(3)	0.040(5)	¹⁶⁷ Er	1229.045(4)	0.63(5)	0.0114(9)
¹⁶⁷ Er	184.2850(10)	56(5)	1.01(9)	¹⁶⁷ Er	1274.530(6)	0.69(10)	0.0125(18)
¹⁷⁰ Er	198.0(6)	0.36(9)	0.0065(16)	¹⁶⁷ Er	1276.2680(20)	0.73(11)	0.0132(20)
¹⁶⁷ Er	198.2440(10)	29.9(16)	0.54(3)	¹⁶⁷ Er	1277.6150(20)	2.82(16)	0.051(3)
¹⁶⁶ Er	207.801(3)d	2.15(8)	0.0390[100%]	¹⁶⁷ Er	1279.088(6)	0.97(13)	0.0176(24)
¹⁶⁷ Er	217.4220(10)	2.66(10)	0.0482(18)	¹⁶⁷ Er	1310.022(3)	1.65(8)	0.0299(14)
¹⁶⁷ Er	255.9310(10)	0.76(3)	0.0138(5)	¹⁶⁷ Er	1323.9270(20)	1.69(8)	0.0306(14)
¹⁶⁷ Er	284.6560(20)	13.7(12)	0.248(22)	¹⁶⁷ Er	1331.2870(20)	1.36(7)	0.0246(13)
¹⁶⁶ Er	346.553(10)	0.83(4)	0.0150(7)	¹⁶⁷ Er	1351.656(4)	1.94(9)	0.0351(16)
¹⁶⁷ Er	396.5320(10)	0.69(4)	0.0125(7)	¹⁶⁷ Er	1353.805(6)	0.56(5)	0.0101(9)
¹⁶⁷ Er	422.3180(10)	1.56(6)	0.0283(11)	¹⁶⁷ Er	1355.1(3)	0.94(12)	0.0170(22)
¹⁶⁷ Er	447.5170(20)	3.07(11)	0.0556(20)	¹⁶⁷ Er	1392.181(4)	1.27(6)	0.0230(11)
¹⁶⁷ Er	457.6660(20)	0.80(4)	0.0145(7)	¹⁶⁷ Er	1515.93(4)	0.57(5)	0.0103(9)
¹⁶⁷ Er	527.8840(10)	0.88(5)	0.0159(9)	¹⁶⁷ Er	1515.948(20)	0.72(12)	0.0130(22)
¹⁶⁶ Er	531.46(3)	0.92(7)	0.0167(13)	¹⁶⁷ Er	1581.18(6)	0.57(6)	0.0103(11)
¹⁶⁷ Er	543.6620(20)	2.01(9)	0.0364(16)	¹⁶⁷ Er	1649.803(7)	0.58(6)	0.0105(11)
¹⁶⁷ Er	546.9600(20)	1.02(5)	0.0185(9)	¹⁶⁷ Er	1767.00(3)	0.91(7)	0.0165(13)
¹⁶⁷ Er	559.5080(20)	2.36(10)	0.0428(18)	¹⁶⁷ Er	1834.085(7)	1.45(9)	0.0263(16)
¹⁶⁷ Er	568.8260(20)	1.20(6)	0.0217(11)	¹⁶⁷ Er	1835.690(4)	0.65(6)	0.0118(11)
¹⁶⁷ Er	601.6060(20)	0.70(4)	0.0127(7)	¹⁶⁷ Er	1942.513(6)	0.88(7)	0.0159(13)
¹⁶⁷ Er	631.7050(20)	7.9(3)	0.143(5)	¹⁶⁷ Er	2046.97(3)	0.56(6)	0.0101(11)
¹⁶⁷ Er	638.711(3)	1.04(6)	0.0188(11)	¹⁶⁷ Er	2522.76(6)	0.59(9)	0.0107(16)
¹⁶⁷ Er	645.7600(20)	0.96(5)	0.0174(9)	¹⁶⁷ Er	4628.7(3)	1.02(21)	0.018(4)
¹⁶⁷ Er	673.655(3)	0.56(3)	0.0101(5)	¹⁶⁷ Er	4643.4(3)	1.7(4)	0.031(7)
¹⁶⁷ Er	713.2440(10)	0.69(5)	0.0125(9)	¹⁶⁷ Er	4647.4(3)	0.87(18)	0.016(3)
¹⁶⁷ Er	715.1610(20)	1.92(8)	0.0348(14)	¹⁶⁷ Er	4653.2(3)	1.18(24)	0.021(4)
¹⁶⁷ Er	719.5460(20)	1.09(20)	0.020(4)	¹⁶⁷ Er	4671.4(3)	0.95(20)	0.017(4)
¹⁶⁷ Er	720.3850(20)	1.54(16)	0.028(3)	¹⁶⁷ Er	4715.4(3)	0.98(20)	0.018(4)
¹⁶⁷ Er	730.6580(10)	11.6(4)	0.210(7)	¹⁶⁷ Er	4745.4(3)	1.3(3)	0.024(5)
¹⁶⁷ Er	737.664(3)	1.20(6)	0.0217(11)	¹⁶⁷ Er	4752.2(3)	0.58(12)	0.0105(22)
¹⁶⁷ Er	741.3650(20)	6.72(24)	0.122(4)	¹⁶⁷ Er	4759.5(3)	0.74(15)	0.013(3)
¹⁶⁷ Er	748.280(3)	1.35(7)	0.0245(13)	¹⁶⁷ Er	4800.76(7)	1.4(4)	0.025(7)
¹⁶⁷ Er	790.0140(20)	0.68(4)	0.0123(7)	¹⁶⁸ Er	4908.73(17)	0.41(14)	0.0074(25)
¹⁶⁷ Er	798.8940(20)	2.18(9)	0.0395(16)	¹⁶⁷ Er	4921.42(22)	0.61(6)	0.0111(11)
¹⁶⁷ Er	808.927(3)	0.81(10)	0.0147(18)	¹⁶⁷ Er	5001.79(6)	0.88(25)	0.016(5)
¹⁶⁷ Er	811.0500(20)	1.72(22)	0.031(4)	¹⁶⁷ Er	5031.73(19)	0.84(24)	0.015(4)
¹⁶⁷ Er	812.289(3)	1.4(3)	0.025(5)	¹⁶⁷ Er	5114.2(3)	1.02(24)	0.018(4)
¹⁶⁷ Er	815.9890(20)	42.5(15)	0.77(3)	¹⁶⁷ Er	5169.82(18)	0.56(5)	0.0101(9)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁶⁷ Er	5200.0(3)	0.67(16)	0.012(3)	¹⁶⁹ Tm	260.3410(10)	0.103(14)	0.00185(25)
¹⁶⁷ Er	5213.15(15)	1.4(3)	0.025(5)	¹⁶⁹ Tm	266.8830(10)	0.134(15)	0.0024(3)
¹⁶⁷ Er	5292.80(6)	0.63(7)	0.0114(13)	¹⁶⁹ Tm	268.5510(10)	0.210(17)	0.0038(3)
¹⁶⁷ Er	5297.19(3)	0.6(3)	0.011(5)	¹⁶⁹ Tm	288.1840(20)	0.172(10)	0.00309(18)
¹⁶⁷ Er	5359.62(5)	0.62(7)	0.0112(13)	¹⁶⁹ Tm	303.6180(20)	0.137(13)	0.00246(23)
¹⁶⁷ Er	5372.79(6)	0.9(4)	0.016(7)	¹⁶⁹Tm	311.0190(10)	2.50(5)	0.0448(9)
¹⁶⁷ Er	5378.65(17)	0.8(4)	0.014(7)	¹⁶⁹ Tm	342.7130(10)	0.14(3)	0.0025(5)
¹⁶⁷ Er	5406.02(9)	0.8(4)	0.014(7)	¹⁶⁹ Tm	343.5520(10)	0.360(16)	0.0065(3)
¹⁶⁷ Er	5468.71(3)	0.73(15)	0.013(3)	¹⁶⁹ Tm	352.9890(20)	0.547(23)	0.0098(4)
¹⁶⁷ Er	5508.66(3)	0.66(14)	0.0120(25)	¹⁶⁹ Tm	359.3570(20)	0.14(3)	0.0025(5)
¹⁶⁷ Er	5866.25(3)	0.77(16)	0.014(3)	¹⁶⁹ Tm	360.8270(20)	0.089(24)	0.0016(4)
¹⁶⁷ Er	5878.24(3)	0.78(7)	0.0141(13)	¹⁶⁹ Tm	367.5560(20)	0.185(18)	0.0033(3)
¹⁶⁷ Er	5943.28(3)	0.95(20)	0.017(4)	¹⁶⁹ Tm	370.5220(20)	0.16(3)	0.0029(5)
¹⁶⁷ Er	5950.86(3)	0.87(18)	0.016(3)	¹⁶⁹ Tm	371.1720(20)	0.153(22)	0.0027(4)
¹⁶⁷ Er	6137.87(3)	0.57(6)	0.0103(11)	¹⁶⁹Tm	384.0790(20)	1.95(5)	0.0350(9)
¹⁶⁷ Er	6155.99(3)	1.5(3)	0.027(5)	¹⁶⁹ Tm	384.2850(20)	0.19(4)	0.0034(7)
¹⁶⁷ Er	6201.88(3)	0.73(15)	0.013(3)	¹⁶⁹ Tm	388.1810(20)	0.099(16)	0.0018(3)
¹⁶⁶ Er	6228.54(18)	1.41(15)	0.026(3)	¹⁶⁹ Tm	396.758(4)	0.099(10)	0.00178(18)
¹⁶⁷ Er	6229.62(3)	1.54(9)	0.0279(16)	¹⁶⁹ Tm	400.1150(20)	0.717(19)	0.0129(3)
¹⁶⁷ Er	6360.23(3)	1.3(3)	0.024(5)	¹⁶⁹ Tm	400.6640(20)	0.20(5)	0.0036(9)
¹⁶⁷ Er	6677.27(3)	1.02(6)	0.0185(11)	¹⁶⁹ Tm	408.3570(10)	0.239(13)	0.00429(23)
Thulium (Z=69), At. Wt.=168.93421(2), $\sigma_{\gamma}^Z = 105.0(20)$							
¹⁶⁹ Tm	38.713	0.279(6)	0.00500(11)	¹⁶⁹ Tm	413.1330(10)	0.162(17)	0.0029(3)
¹⁶⁹ Tm	63.9550(20)	0.17(8)	0.0030(14)	¹⁶⁹ Tm	424.6940(20)	0.556(25)	0.0100(5)
¹⁶⁹ Tm	66.098	0.51(10)	0.0091(18)	¹⁶⁹ Tm	426.783(3)	0.186(18)	0.0033(3)
¹⁶⁹Tm	68.649	1.75(23)	0.031(4)	¹⁶⁹ Tm	429.0390(20)	0.308(24)	0.0055(4)
¹⁶⁹ Tm	69.9880(10)	0.19(7)	0.0034(13)	¹⁶⁹ Tm	440.5100(20)	0.13(3)	0.0023(5)
¹⁶⁹Tm	75.83	0.94(8)	0.0169(14)	¹⁶⁹ Tm	442.1490(10)	0.51(4)	0.0091(7)
¹⁶⁹Tm	87.5210(10)	1.29(3)	0.0231(5)	¹⁶⁹Tm	446.328(3)	1.62(4)	0.0291(7)
¹⁶⁹ Tm	87.5700(10)	0.29(6)	0.0052(11)	¹⁶⁹ Tm	454.2720(20)	0.295(20)	0.0053(4)
¹⁶⁹ Tm	89.905	0.116(21)	0.0021(4)	¹⁶⁹Tm	456.0460(10)	1.16(4)	0.0208(7)
¹⁶⁹ Tm	105.162	0.780(23)	0.0140(4)	¹⁶⁹ Tm	457.4070(10)	0.48(12)	0.0086(22)
¹⁶⁹ Tm	107.9560(10)	0.110(13)	0.00197(23)	¹⁶⁹ Tm	457.4100(20)	0.557(25)	0.0100(5)
¹⁶⁹ Tm	111.0050(10)	0.327(16)	0.0059(3)	¹⁶⁹ Tm	468.4740(20)	0.45(4)	0.0081(7)
¹⁶⁹ Tm	114.544	3.19(6)	0.0572(11)	¹⁶⁹ Tm	468.7760(20)	0.41(8)	0.0074(14)
¹⁶⁹ Tm	130.027	0.940(25)	0.0169(5)	¹⁶⁹ Tm	472.6610(10)	0.60(5)	0.0108(9)
¹⁶⁹ Tm	144.4790(10)	1.2(4)	0.022(7)	¹⁶⁹ Tm	473.5790(10)	0.15(4)	0.0027(7)
¹⁶⁹ Tm	144.48	5.96(11)	0.1069(20)	¹⁶⁹ Tm	477.027(4)	0.240(25)	0.0043(5)
¹⁶⁹Tm	149.7180(10)	7.11(12)	0.1275(22)	¹⁶⁹ Tm	481.3490(20)	0.109(22)	0.0020(4)
¹⁶⁹ Tm	153.6680(10)	0.098(15)	0.0018(3)	¹⁶⁹ Tm	485.210(4)	0.140(22)	0.0025(4)
¹⁶⁹ Tm	156.0030(10)	0.119(17)	0.0021(3)	¹⁶⁹ Tm	496.5720(20)	0.80(3)	0.0144(5)
¹⁶⁹ Tm	161.7200(10)	0.270(17)	0.0048(3)	¹⁶⁹ Tm	499.0260(20)	0.40(8)	0.0072(14)
¹⁶⁹Tm	165.735	3.29(6)	0.0590(11)	¹⁶⁹Tm	499.5560(20)	0.88(3)	0.0158(5)
¹⁶⁹ Tm	171.8550(10)	0.391(18)	0.0070(3)	¹⁶⁹Tm	505.018(7)	0.90(3)	0.0161(5)
¹⁶⁹ Tm	176.5240(10)	0.34(3)	0.0061(5)	¹⁶⁹ Tm	505.341(9)	0.84(3)	0.0151(5)
¹⁶⁹Tm	180.993	3.85(14)	0.0691(25)	¹⁶⁹Tm	512.1370(20)	1.96(5)	0.0352(9)
¹⁶⁹ Tm	198.2340(10)	0.094(21)	0.0017(4)	¹⁶⁹ Tm	512.6080(20)	0.108(22)	0.0019(4)
¹⁶⁹Tm	198.5260(10)	0.96(3)	0.0172(5)	¹⁶⁹ Tm	517.053(4)	0.15(3)	0.0027(5)
¹⁶⁹ Tm	204.448	8.72(19)	0.156(3)	¹⁶⁹ Tm	523.3590(20)	0.48(3)	0.0086(5)
¹⁶⁹ Tm	204.7820(10)	0.25(7)	0.0045(13)	¹⁶⁹ Tm	532.4280(20)	0.59(3)	0.0106(5)
¹⁶⁹Tm	219.706	3.64(6)	0.0653(11)	¹⁶⁹ Tm	532.858(3)	0.12(3)	0.0022(5)
¹⁶⁹ Tm	231.8330(10)	0.60(3)	0.0108(5)	¹⁶⁹Tm	535.8280(10)	1.18(4)	0.0212(7)
¹⁶⁹Tm	235.1890(10)	1.18(4)	0.0212(7)	¹⁶⁹ Tm	537.9910(20)	1.00(4)	0.0179(7)
¹⁶⁹ Tm	237.2390(10)	5.52(10)	0.0990(18)	¹⁶⁹ Tm	551.5140(20)	1.29(25)	0.023(5)
¹⁶⁹Tm	242.6220(10)	1.28(4)	0.0230(7)	¹⁶⁹ Tm	562.4440(20)	0.85(3)	0.0152(5)
¹⁶⁹ Tm	256.4550(10)	0.096(15)	0.0017(3)	¹⁶⁹Tm	565.2770(20)	1.58(4)	0.0283(7)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁶⁹ Tm	569.1730(20)	1.02(3)	0.0183(5)	¹⁶⁹ Tm	886.5560(20)	0.230(24)	0.0041(4)
¹⁶⁹ Tm	569.5440(20)	0.44(9)	0.0079(16)	¹⁶⁹ Tm	890.047(3)	0.17(4)	0.0030(7)
¹⁶⁹ Tm	573.017(4)	0.39(7)	0.0070(13)	¹⁶⁹ Tm	920.507(9)	0.113(24)	0.0020(4)
¹⁶⁹ Tm	573.017(4)	0.30(9)	0.0054(16)	¹⁶⁹ Tm	928.265(4)	0.37(3)	0.0066(5)
¹⁶⁹ Tm	581.2690(20)	0.32(7)	0.0057(13)	¹⁶⁹ Tm	943.522(4)	0.24(3)	0.0043(5)
¹⁶⁹ Tm	585.1540(10)	0.60(4)	0.0108(7)	¹⁶⁹ Tm	956.145(3)	0.33(6)	0.0059(11)
¹⁶⁹ Tm	589.0850(10)	0.58(10)	0.0104(18)	¹⁶⁹ Tm	959.201(4)	0.28(3)	0.0050(5)
¹⁶⁹ Tm	590.2270(20)	1.27(10)	0.0228(18)	¹⁶⁹ Tm	959.220(9)	0.45(9)	0.0081(16)
¹⁶⁹ Tm	599.1890(20)	0.155(25)	0.0028(5)	¹⁶⁹ Tm	973.121(12)	0.10(4)	0.0018(7)
¹⁶⁹ Tm	601.9780(20)	0.13(3)	0.0023(5)	¹⁶⁹ Tm	987.453(3)	0.30(3)	0.0054(5)
¹⁶⁹ Tm	603.9900(20)	1.40(5)	0.0251(9)	¹⁶⁹ Tm	995.714(4)	0.106(23)	0.0019(4)
¹⁶⁹ Tm	610.0310(20)	0.18(4)	0.0032(7)	¹⁶⁹ Tm	998.253(4)	0.200(25)	0.0036(5)
¹⁶⁹ Tm	611.6590(10)	0.83(4)	0.0149(7)	¹⁶⁹ Tm	1000.898(10)	0.23(4)	0.0041(7)
¹⁶⁹ Tm	619.423(3)	0.23(4)	0.0041(7)	¹⁶⁹ Tm	1018.431(10)	0.28(6)	0.0050(11)
¹⁶⁹ Tm	621.812(3)	0.12(3)	0.0022(5)	¹⁶⁹ Tm	1027.820(12)	0.26(4)	0.0047(7)
¹⁶⁹ Tm	623.1420(10)	0.27(4)	0.0048(7)	¹⁶⁹ Tm	1040.1330(10)	0.25(7)	0.0045(13)
¹⁶⁹ Tm	632.4310(20)	0.74(3)	0.0133(5)	¹⁶⁹ Tm	1043.108(12)	0.19(4)	0.0034(7)
¹⁶⁹ Tm	637.900(3)	1.25(4)	0.0224(7)	¹⁶⁹ Tm	1045.353(12)	0.18(4)	0.0032(7)
¹⁶⁹ Tm	637.9020(20)	1.8(3)	0.032(5)	¹⁶⁹ Tm	1061.868(14)	0.49(10)	0.0088(18)
¹⁶⁹ Tm	640.7790(20)	0.70(3)	0.0126(5)	¹⁶⁹ Tm	1070.969(6)	0.30(6)	0.0054(11)
¹⁶⁹ Tm	648.7440(20)	0.24(4)	0.0043(7)	¹⁶⁹ Tm	1101.996(3)	0.10(3)	0.0018(5)
¹⁶⁹ Tm	650.3720(10)	1.45(5)	0.0260(9)	¹⁶⁹ Tm	1140.192(4)	0.62(12)	0.0111(22)
¹⁶⁹ Tm	658.913(5)	1.56(5)	0.0280(9)	¹⁶⁹ Tm	1154.112(12)	0.18(4)	0.0032(7)
¹⁶⁹ Tm	664.9160(10)	0.30(4)	0.0054(7)	¹⁶⁹ Tm	1171.966(11)	0.14(3)	0.0025(5)
¹⁶⁹ Tm	669.656(4)	0.31(4)	0.0056(7)	¹⁶⁹ Tm	1178.905(4)	0.56(4)	0.0100(7)
¹⁶⁹ Tm	670.753(7)	0.12(4)	0.0022(7)	¹⁶⁹ Tm	1184.563(14)	0.20(3)	0.0036(5)
¹⁶⁹ Tm	679.5820(20)	0.15(3)	0.0027(5)	¹⁶⁹ Tm	1210.678(11)	0.36(7)	0.0065(13)
¹⁶⁹ Tm	680.5480(20)	0.41(3)	0.0074(5)	¹⁶⁹ Tm	1226.345(12)	0.120(22)	0.0022(4)
¹⁶⁹ Tm	693.2840(10)	0.30(3)	0.0054(5)	¹⁶⁹ Tm	1238.136(10)	0.107(21)	0.0019(4)
¹⁶⁹ Tm	694.085(13)	~0.1	~0.002	¹⁶⁹ Tm	1265.057(12)	0.210(24)	0.0038(4)
¹⁶⁹ Tm	703.6280(10)	1.32(4)	0.0237(7)	¹⁶⁹ Tm	1354.71(7)	0.128(23)	0.0023(4)
¹⁶⁹ Tm	707.8490(10)	0.50(10)	0.0090(18)	¹⁶⁹ Tm	4641.4(4)	0.32(3)	0.0057(5)
¹⁶⁹ Tm	709.381(3)	0.107(21)	0.0019(4)	¹⁶⁹ Tm	4732.6(4)	0.58(5)	0.0104(9)
¹⁶⁹ Tm	710.7670(20)	0.60(3)	0.0108(5)	¹⁶⁹ Tm	4773.8(8)	0.16(3)	0.0029(5)
¹⁶⁹ Tm	711.1330(20)	0.33(7)	0.0059(13)	¹⁶⁹ Tm	4922.1(5)	0.26(3)	0.0047(5)
¹⁶⁹ Tm	714.433(5)	0.089(20)	0.0016(4)	¹⁶⁹ Tm	4987.0(6)	0.16(3)	0.0029(5)
¹⁶⁹ Tm	719.2610(20)	1.01(3)	0.0181(5)	¹⁶⁹ Tm	5061.6(8)	0.103(21)	0.0018(4)
¹⁶⁹ Tm	720.8210(20)	0.57(3)	0.0102(5)	¹⁶⁹ Tm	5075.3(5)	0.39(4)	0.0070(7)
¹⁶⁹ Tm	724.585(3)	0.68(3)	0.0122(5)	¹⁶⁹ Tm	5124.1(5)	0.28(4)	0.0050(7)
¹⁶⁹ Tm	739.794(4)	0.108(18)	0.0019(3)	¹⁶⁹ Tm	5149.1(6)	0.31(4)	0.0056(7)
¹⁶⁹ Tm	744.765(7)	0.124(19)	0.0022(3)	¹⁶⁹ Tm	5158.2(6)	0.47(5)	0.0084(9)
¹⁶⁹ Tm	748.2310(20)	0.102(20)	0.0018(4)	¹⁶⁹ Tm	5216.5(9)	0.092(25)	0.0017(5)
¹⁶⁹ Tm	781.278(7)	0.20(4)	0.0036(7)	¹⁶⁹ Tm	5326.80(11)	0.18(3)	0.0032(5)
¹⁶⁹ Tm	781.279(7)	0.19(4)	0.0034(7)	¹⁶⁹ Tm	5353.72(11)	0.19(3)	0.0034(5)
¹⁶⁹ Tm	781.832(4)	0.090(20)	0.0016(4)	¹⁶⁹ Tm	5381.18(11)	0.18(3)	0.0032(5)
¹⁶⁹ Tm	784.900(4)	0.18(4)	0.0032(7)	¹⁶⁹ Tm	5399.03(11)	0.143(25)	0.0026(5)
¹⁶⁹ Tm	790.216(4)	0.17(3)	0.0030(5)	¹⁶⁹ Tm	5412.95(11)	0.39(5)	0.0070(9)
¹⁶⁹ Tm	800.424(6)	0.122(23)	0.0022(4)	¹⁶⁹ Tm	5423.08(11)	0.24(3)	0.0043(5)
¹⁶⁹ Tm	810.7260(20)	0.157(21)	0.0028(4)	¹⁶⁹ Tm	5431.26(11)	0.23(3)	0.0041(5)
¹⁶⁹ Tm	815.624(4)	0.76(3)	0.0136(5)	¹⁶⁹ Tm	5443.88(11)	0.150(25)	0.0027(5)
¹⁶⁹ Tm	818.5070(20)	0.233(20)	0.0042(4)	¹⁶⁹ Tm	5451.91(11)	0.148(25)	0.0027(5)
¹⁶⁹ Tm	824.0610(20)	0.318(22)	0.0057(4)	¹⁶⁹ Tm	5513.01(11)	0.16(5)	0.0029(9)
¹⁶⁹ Tm	844.677(9)	0.147(18)	0.0026(3)	¹⁶⁹ Tm	5683.40(11)	0.104(21)	0.0019(4)
¹⁶⁹ Tm	854.337(4)	1.41(4)	0.0253(7)	¹⁶⁹ Tm	5728.48(11)	0.26(3)	0.0047(5)
¹⁶⁹ Tm	866.522(6)	0.353(24)	0.0063(4)	¹⁶⁹ Tm	5731.36(11)	1.17(22)	0.021(4)
¹⁶⁹ Tm	869.401(4)	0.235(23)	0.0042(4)	¹⁶⁹ Tm	5737.51(11)	1.42(7)	0.0255(13)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁶⁹ Tm	5858.03(11)	0.41(4)	0.0074(7)	¹⁷⁴ Yb	363.938(6)	0.80(12)	0.0140(21)
¹⁶⁹ Tm	5898.56(11)	0.35(4)	0.0063(7)	¹⁶⁸ Yb	378.616(3)	0.033(6)	0.00058(11)
¹⁶⁹ Tm	5908.27(11)	0.49(4)	0.0088(7)	¹⁷⁴ Yb	389.422(5)	0.032(5)	0.00056(9)
¹⁶⁹ Tm	5941.47(11)	1.51(7)	0.0271(13)	¹⁷⁴ Yb	392.114(11)	0.097(12)	0.00170(21)
¹⁶⁹ Tm	5943.09(11)	1.03(20)	0.018(4)	¹⁷⁴ Yb	396.329(20)d	1.42(5)	0.0249[<0.1%]
¹⁶⁹ Tm	6001.61(11)	0.99(10)	0.0178(18)	¹⁷² Yb	399.17(4)	0.111(12)	0.00194(21)
¹⁶⁹ Tm	6354.59(11)	0.42(4)	0.0075(7)	¹⁷⁴ Yb	400.996(15)	0.015(4)	0.00026(7)
¹⁶⁹ Tm	6387.37(11)	1.48(7)	0.0265(13)	¹⁷⁴ Yb	405.156(6)	0.040(6)	0.00070(11)
¹⁶⁹ Tm	6442.10(11)	0.47(3)	0.0084(5)	¹⁷⁴ Yb	406.05(14)	0.111(14)	0.00194(25)
¹⁶⁹ Tm	6553.10(11)	0.65(13)	0.0117(23)	¹⁷⁴ Yb	406.548(5)	0.118(18)	0.0021(3)
Ytterbium (Z=70), At.Wt.=173.04(3), $\sigma_{\gamma}^Z = 34.9(8)$							
¹⁷⁰ Yb	19.3940(20)	0.021(5)	0.00037(9)	¹⁷³ Yb	409.38(7)	0.031(5)	0.00054(9)
¹⁷⁴ Yb	41.2180(20)	1.1(3)	0.019(5)	¹⁷⁴ Yb	411.48(11)	0.021(4)	0.00037(7)
¹⁷⁴ Yb	46.7510(20)	0.25(8)	0.0044(14)	¹⁷⁴ Yb	423.219(11)	0.045(7)	0.00079(12)
¹⁶⁸ Yb	62.7190(10)	0.064(12)	0.00112(21)	¹⁷⁴ Yb	428.613(12)	0.61(7)	0.0107(12)
¹⁷⁰ Yb	66.720(10)	0.024(6)	0.00042(11)	¹⁷⁴ Yb	436.173(5)	0.52(6)	0.0091(11)
¹⁶⁸ Yb	75.0400(10)	0.015(3)	0.00026(5)	¹⁷⁴ Yb	436.472(16)	0.037(8)	0.00065(14)
¹⁷³ Yb	76.996	0.40(4)	0.0070(7)	¹⁷⁴ Yb	452.80(14)	0.019(3)	0.00033(5)
¹⁷¹ Yb	78.7430(10)	0.67(10)	0.0117(18)	¹⁷⁴ Yb	453.299(6)	0.031(6)	0.00054(11)
¹⁷³ Yb	86.11(7)	0.164(18)	0.0029(3)	¹⁷⁴ Yb	465.033(11)	0.06(4)	0.0011(7)
¹⁶⁸ Yb	87.3840(10)	0.016(3)	0.00028(5)	¹⁷⁴ Yb	468.079(19)	0.022(4)	0.00039(7)
¹⁷⁴ Yb	87.9690(20)	0.26(6)	0.0046(11)	¹⁷⁴ Yb	476.606(11)	0.015(4)	0.00026(7)
¹⁷³ Yb	88.26(11)	0.044(8)	0.00077(14)	¹⁷⁴ Yb	476.643(8)	0.015(4)	0.00026(7)
¹⁷⁴ Yb	89.9570(20)	0.066(16)	0.0012(3)	¹⁷⁴ Yb	477.391(5)	0.75(8)	0.0131(14)
¹⁷³ Yb	93.60(6)	0.109(13)	0.00191(23)	¹⁷⁴ Yb	482.071(11)	0.23(3)	0.0040(5)
¹⁷⁴ Yb	95.2730(20)	0.20(5)	0.0035(9)	¹⁷¹ Yb	490.444(8)	0.0172(24)	0.00030(4)
¹⁷⁴ Yb	100.759(4)	0.019(7)	0.00033(12)	¹⁷⁴ Yb	496.414(11)	0.023(7)	0.00040(12)
¹⁷³ Yb	102.60(5)	0.44(5)	0.0077(9)	¹⁷⁴ Yb	497.717(10)	0.022(5)	0.00039(9)
¹⁷⁴ Yb	104.5260(20)	0.43(11)	0.0075(19)	¹⁷⁴ Yb	498.315(9)	0.076(11)	0.00133(19)
¹⁷⁴ Yb	113.805(4)d	0.417(14)	0.00730[<0.1%]	¹⁷⁴ Yb	505.05(5)	0.030(8)	0.00053(14)
¹⁷⁶ Yb	125.23(18)	0.007(3)	1.2(5)E-4	¹⁷⁴ Yb	511.784(11)	0.34(5)	0.0060(9)
¹⁷³ Yb	138.27(6)	0.058(7)	0.00102(12)	¹⁷⁴ Yb	514.868(7)d	9.0(9)	0.158[100%]
¹⁷⁴ Yb	142.0240(20)	0.032(8)	0.00056(14)	¹⁷⁴ Yb	518.491(11)	0.037(9)	0.00065(16)
¹⁷⁴ Yb	142.478(3)	0.021(5)	0.00037(9)	¹⁷¹ Yb	528.289(7)	0.024(3)	0.00042(5)
¹⁶⁸ Yb	144.5760(10)	0.016(3)	0.00028(5)	¹⁷⁴ Yb	534.735(9)	0.50(6)	0.0088(11)
¹⁷³ Yb	148.72(9)	0.031(5)	0.00054(9)	¹⁷⁴ Yb	548.841(12)	0.020(7)	0.00035(12)
¹⁶⁸ Yb	156.8980(10)	0.038(7)	0.00067(12)	¹⁷⁴ Yb	553.002(11)	0.091(13)	0.00159(23)
¹⁷⁴ Yb	163.012(5)	0.132(25)	0.0023(4)	¹⁷⁴ Yb	556.090(8)	0.066(11)	0.00116(19)
¹⁷⁴ Yb	172.167(4)	0.118(22)	0.0021(4)	¹⁷¹ Yb	558.935(8)	0.020(3)	0.00035(5)
¹⁷³ Yb	175.30(5)	0.58(6)	0.0102(11)	¹⁷⁴ Yb	565.242(11)	0.039(8)	0.00068(14)
¹⁷¹ Yb	181.529(3)	0.53(6)	0.0093(11)	¹⁷³ Yb	570.30(19)	0.028(6)	0.00049(11)
¹⁶⁸ Yb	191.2140(10)	0.22(4)	0.0039(7)	¹⁷⁴ Yb	571.915(8)	0.047(7)	0.00082(12)
¹⁷³ Yb	198.29(12)	0.023(4)	0.00040(7)	¹⁶⁸ Yb	572.700(7)	0.049(8)	0.00086(14)
¹⁷³ Yb	223.00(8)	0.029(4)	0.00051(7)	¹⁶⁸ Yb	576.398(10)	0.024(4)	0.00042(7)
¹⁷⁴ Yb	231.502(6)	0.060(8)	0.00105(14)	¹⁷¹ Yb	576.4(3)	0.020(3)	0.00035(5)
¹⁷⁴ Yb	232.435(3)	0.025(4)	0.00044(7)	¹⁷⁴ Yb	577.28(5)	0.046(8)	0.00081(14)
¹⁷³ Yb	243.68(19)	0.018(4)	0.00032(7)	¹⁶⁸ Yb	590.695(10)	0.090(15)	0.0016(3)
¹⁷⁴ Yb	246.778(14)	0.024(7)	0.00042(12)	¹⁷¹ Yb	602.469(5)	0.030(4)	0.00053(7)
¹⁷⁴ Yb	255.338(5)	0.033(10)	0.00058(18)	¹⁷⁴ Yb	602.841(8)	0.072(10)	0.00126(18)
¹⁷⁴ Yb	267.538(5)	0.073(10)	0.00128(18)	¹⁷⁴ Yb	618.09(4)	0.020(4)	0.00035(7)
¹⁷³ Yb	274.90(7)	0.044(6)	0.00077(11)	¹⁶⁸ Yb	622.127(11)	0.034(6)	0.00060(11)
¹⁷⁴ Yb	282.522(14)d	0.666(22)	0.0117[<0.1%]	¹⁶⁸ Yb	623.026(7)	0.035(6)	0.00061(11)
¹⁷¹ Yb	287.138(3)	0.062(11)	0.00109(19)	¹⁷⁴ Yb	624.692(9)	0.026(4)	0.00046(7)
¹⁷⁴ Yb	288.626(17)	0.016(3)	0.00028(5)	¹⁷⁴ Yb	635.22(4)	0.078(13)	0.00137(23)
¹⁷⁴ Yb	311.276(5)	0.26(4)	0.0046(7)	¹⁶⁸ Yb	635.348(7)	0.103(17)	0.0018(3)
¹⁷³ Yb	341.27(16)	0.026(5)	0.00046(9)	¹⁶⁸ Yb	635.418(7)	0.103(17)	0.0018(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁷³ Yb	1552.0(3)	0.032(9)	0.00056(16)	¹⁷⁴ Yb	4138.6(19)	0.023(6)	0.00040(11)
¹⁷¹ Yb	1553.54(25)	0.026(5)	0.00046(9)	¹⁷⁴ Yb	4174.9(13)	0.088(21)	0.0015(4)
¹⁷¹ Yb	1584.114(12)	0.037(6)	0.00065(11)	¹⁷⁴ Yb	4195.0(4)	0.058(14)	0.00102(25)
¹⁷¹ Yb	1589.06(4)	0.037(5)	0.00065(9)	¹⁷⁴ Yb	4454.3(4)	0.026(6)	0.00046(11)
¹⁷¹ Yb	1599.939(16)	0.125(16)	0.0022(3)	¹⁷⁴ Yb	4465.9(4)	0.040(10)	0.00070(18)
¹⁷¹ Yb	1608.522(9)	0.081(11)	0.00142(19)	¹⁷³ Yb	4716.5(7)	0.027(8)	0.00047(14)
¹⁷¹ Yb	1621.960(12)	0.030(4)	0.00053(7)	¹⁷⁴ Yb	4830.2(4)	0.25(6)	0.0044(11)
¹⁷¹ Yb	1631.792(20)	0.054(7)	0.00095(12)	¹⁷⁴ Yb	5011.0(4)	0.18(4)	0.0032(7)
¹⁷³ Yb	1638.36(17)	0.22(3)	0.0039(5)	¹⁷⁴ Yb	5266.3(4)	1.4(6)	0.025(11)
¹⁷³ Yb	1679.70(14)	0.161(19)	0.0028(3)	¹⁷⁴ Yb	5307.5(4)	0.020(5)	0.00035(9)
¹⁷¹ Yb	1696.12(3)	0.029(4)	0.00051(7)	¹⁷¹ Yb	5539.05(5)	0.083(11)	0.00145(19)
¹⁷¹ Yb	1715.35(4)	0.090(11)	0.00158(19)	¹⁷¹ Yb	5691.58(9)	0.020(3)	0.00035(5)
¹⁷³ Yb	1730.9(3)	0.030(8)	0.00053(14)	¹⁷⁰ Yb	5712.5(6)	0.056(9)	0.00098(16)
¹⁷¹ Yb	1742.889(10)	0.024(5)	0.00042(9)	¹⁷¹ Yb	5824.85(6)	0.0172(23)	0.00030(4)
¹⁷¹ Yb	1770.58(4)	0.073(22)	0.0013(4)	¹⁷¹ Yb	6009.65(6)	0.0148(19)	0.00026(3)
¹⁷³ Yb	1775.1(3)	0.052(11)	0.00091(19)	¹⁶⁸ Yb	6779.90(11)	0.058(7)	0.00102(12)
¹⁷¹ Yb	1786.76(3)	0.027(4)	0.00047(7)	Lutetium (Z=71), At.Wt.=174.967(1), $\sigma_{\gamma}^Z = 76.6(23)$			
¹⁷¹ Yb	1815.84(3)	0.073(10)	0.00128(18)	¹⁷⁵ Lu	38.7460(10)	0.38(12)	0.0066(21)
¹⁷¹ Yb	1849.32(4)	0.046(6)	0.00081(11)	¹⁷⁵ Lu	46.4590(10)	0.26(7)	0.0045(12)
¹⁷³ Yb	1859.2(3)	0.051(10)	0.00089(18)	¹⁷⁵ Lu	66.2400(10)	0.28(4)	0.0048(7)
¹⁷¹ Yb	1877.64(3)	0.035(5)	0.00061(9)	¹⁷⁵ Lu	71.5170(10)	3.96(22)	0.069(4)
¹⁷³ Yb	1920.6(3)	0.040(10)	0.00070(18)	¹⁷⁵ Lu	73.1430(10)	0.160(20)	0.0028(4)
¹⁷¹ Yb	1930.76(5)	0.070(9)	0.00123(16)	¹⁷⁶ Lu	88.36(4)	7.1(4) s⁻¹g⁻¹	Abundant
¹⁷¹ Yb	1956.39(3)	0.028(4)	0.00049(7)	¹⁷⁶ Lu	94.129(8)	0.72(4)	0.0125(7)
¹⁷¹ Yb	1968.29(3)	0.061(14)	0.00107(25)	¹⁷⁶ Lu	111.705(12)	1.03(5)	0.0178(9)
¹⁷¹ Yb	1997.515(21)	0.044(7)	0.00077(12)	¹⁷⁵ Lu	112.9220(10)	1.15(7)	0.0199(12)
¹⁷³ Yb	2003.14(25)	0.045(10)	0.00079(18)	¹⁷⁶ Lu	112.9500(10)d	3.47(16)	0.060[<0.1%]
¹⁷¹ Yb	2009.50(5)	0.074(12)	0.00130(21)	¹⁷⁶ Lu	115.651(8)	0.144(22)	0.0025(4)
¹⁷¹ Yb	2024.16(3)	0.081(12)	0.00142(21)	¹⁷⁶ Lu	119.836(3)	1.32(22)	0.023(4)
¹⁷³ Yb	2093.9(3)	0.026(8)	0.00046(14)	¹⁷⁶ Lu	121.620(3)	5.24(17)	0.091(3)
¹⁷¹ Yb	2102.90(3)	0.040(5)	0.00070(9)	¹⁷⁵ Lu	129.7730(10)	0.18(3)	0.0031(5)
¹⁷¹ Yb	2115.56(4)	0.039(7)	0.00068(12)	¹⁷⁶ Lu	135.802(19)	0.37(3)	0.0064(5)
¹⁷¹ Yb	2133.85(7)	0.043(6)	0.00075(11)	¹⁷⁶ Lu	138.607(5)	6.79(24)	0.118(4)
¹⁷³ Yb	2171.4(3)	0.059(12)	0.00103(21)	¹⁷⁵ Lu	139.3830(10)	0.25(4)	0.0043(7)
¹⁷¹ Yb	2195.09(5)	0.066(11)	0.00116(19)	¹⁷⁶ Lu	144.745(5)	1.33(8)	0.0230(14)
¹⁷¹ Yb	2234.17(10)	0.042(11)	0.00074(19)	¹⁷⁶ Lu	145.870(4)	1.52(9)	0.0263(16)
¹⁷¹ Yb	2238.19(3)	0.052(12)	0.00091(21)	¹⁷⁶ Lu	147.165(5)	4.96(19)	0.086(3)
¹⁷¹ Yb	2263.11(3)	0.042(11)	0.00074(19)	¹⁷⁶ Lu	147.167(5)	3.7(7)	0.064(12)
¹⁷¹ Yb	2296.47(4)	0.035(7)	0.00061(12)	¹⁷⁶ Lu	150.392(3)	13.8(4)	0.239(7)
¹⁷¹ Yb	2327.57(8)	0.094(19)	0.0016(3)	¹⁷⁵ Lu	153.4670(10)	0.55(5)	0.0095(9)
¹⁷³ Yb	2388.7(4)	0.036(10)	0.00063(18)	¹⁷⁶ Lu	162.492(4)	5.32(17)	0.092(3)
¹⁷¹ Yb	2401.37(3)	0.20(3)	0.0035(5)	¹⁷⁶ Lu	168.605(6)	0.97(5)	0.0168(9)
¹⁷⁴ Yb	3632.3(10)	0.40(10)	0.0070(18)	¹⁷⁶ Lu	171.869(7)	1.74(6)	0.0301(10)
¹⁷⁴ Yb	3661.2(14)	0.043(10)	0.00075(18)	¹⁷⁵ Lu	182.4220(10)	0.46(10)	0.0080(17)
¹⁷⁴ Yb	3714.7(5)	0.23(6)	0.0040(11)	¹⁷⁶ Lu	185.593(8)	3.42(12)	0.0592(21)
¹⁷⁴ Yb	3740.8(14)	0.043(10)	0.00075(18)	¹⁷⁶ Lu	187.970(23)	1.39(6)	0.0241(10)
¹⁷⁴ Yb	3776.2(23)	0.040(10)	0.00070(18)	¹⁷⁵ Lu	188.2870(10)	0.29(4)	0.0050(7)
¹⁷⁴ Yb	3782.9(19)	0.057(14)	0.00100(25)	¹⁷⁶ Lu	191.492(9)	0.62(12)	0.0107(21)
¹⁷⁴ Yb	3823.8(14)	0.026(6)	0.00046(11)	¹⁷⁵ Lu	192.2120(10)	1.08(14)	0.0187(24)
¹⁷⁴ Yb	3842.1(14)	0.074(18)	0.0013(3)	¹⁷⁶ Lu	195.565(8)	0.63(5)	0.0109(9)
¹⁷⁴ Yb	3854.4(11)	0.085(16)	0.0015(3)	¹⁷⁵ Lu	197.550(14)	0.30(14)	0.0052(24)
¹⁷³ Yb	3868.0(4)	0.103(14)	0.00180(25)	¹⁷⁵ Lu	201.5680(10)	0.78(12)	0.0135(21)
¹⁷⁴ Yb	3885.0(4)	0.72(17)	0.013(3)	¹⁷⁶ Lu	201.83(4)	37.9(22)	Abundant
¹⁷⁴ Yb	3929.3(4)	0.38(9)	0.0067(16)	¹⁷⁶ Lu	207.797(8)	1.00(5)	0.0173(9)
¹⁷⁴ Yb	3978.2(19)	0.020(5)	0.00035(9)	¹⁷⁶ Lu	208.3660(10)d	6.0(3)	0.104[<0.1%]
¹⁷⁴ Yb	4129.6(19)	0.026(6)	0.00046(11)	¹⁷⁶ Lu	209.492(24)	0.298(25)	0.0052(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁷⁶ Lu	212.841(15)	0.16(3)	0.0028(5)	¹⁷⁶ Lu	864.52(8)	0.191(16)	0.0033(3)
¹⁷⁶ Lu	213.965(8)	0.34(6)	0.0059(10)	¹⁷⁶ Lu	899.12(6)	0.423(25)	0.0073(4)
¹⁷⁵ Lu	217.0030(10)	0.35(10)	0.0061(17)	¹⁷⁶ Lu	907.86(6)	0.42(3)	0.0073(5)
¹⁷⁵ Lu	219.2830(20)	0.20(8)	0.0035(14)	¹⁷⁶ Lu	907.961(18)	0.35(5)	0.0061(9)
¹⁷⁵Lu	225.4030(10)	1.73(8)	0.0300(14)	¹⁷⁶ Lu	916.24(4)	0.439(25)	0.0076(4)
¹⁷⁵ Lu	227.9970(10)	0.57(7)	0.0099(12)	¹⁷⁵ Lu	1000.846(18)	0.15(10)	0.0026(17)
¹⁷⁶ Lu	228.708(10)	0.178(21)	0.0031(4)	¹⁷⁶ Lu	1036.39(8)	0.169(16)	0.0029(3)
¹⁷⁵ Lu	233.7410(20)	0.41(10)	0.0071(17)	¹⁷⁶ Lu	1061.97(6)	0.45(4)	0.0078(7)
¹⁷⁶ Lu	235.892(15)	0.81(4)	0.0140(7)	¹⁷⁶ Lu	1080.24(6)	0.68(4)	0.0118(7)
¹⁷⁵ Lu	238.6710(10)	0.20(6)	0.0035(10)	¹⁷⁶ Lu	1088.11(4)	0.83(4)	0.0144(7)
¹⁷⁶ Lu	244.310(12)	0.45(8)	0.0078(14)	¹⁷⁶ Lu	1215.36(13)	0.139(14)	0.00241(24)
¹⁷⁶ Lu	247.255(15)	0.247(23)	0.0043(4)	¹⁷⁶ Lu	1233.84(6)	0.187(19)	0.0032(3)
¹⁷⁵ Lu	251.1990(20)	0.16(3)	0.0028(5)	¹⁷⁶ Lu	1305.18(8)	0.36(3)	0.0062(5)
¹⁷⁶Lu	259.401(16)	1.89(8)	0.0327(14)	¹⁷⁶ Lu	1381.01(6)	0.30(3)	0.0052(5)
¹⁷⁵ Lu	263.7290(10)	0.59(10)	0.0102(17)	¹⁷⁶ Lu	4866.8(5)	0.25(5)	0.0043(9)
¹⁷⁶ Lu	264.581(6)	0.76(11)	0.0132(19)	¹⁷⁶ Lu	5016.6(5)	0.215(18)	0.0037(3)
¹⁷⁶Lu	268.788(5)	3.64(13)	0.0630(23)	¹⁷⁶ Lu	5023.6(3)	0.176(24)	0.0030(4)
¹⁷⁵ Lu	277.6830(10)	0.20(6)	0.0035(10)	¹⁷⁶ Lu	5319.45(24)	0.167(19)	0.0029(3)
¹⁷⁵ Lu	284.6410(10)	0.75(6)	0.0130(10)	¹⁷⁶ Lu	5323.12(13)	0.145(15)	0.0025(3)
¹⁷⁶ Lu	301.098(6)	0.73(4)	0.0126(7)	¹⁷⁵ Lu	5331.80(20)	0.16(4)	0.0028(7)
¹⁷⁶ Lu	306.84(4)	45.2(24) s ⁻¹ g ⁻¹	Abundant	¹⁷⁵ Lu	5331.94(20)	0.19(4)	0.0033(7)
¹⁷⁵Lu	310.1870(10)	1.49(8)	0.0258(14)	¹⁷⁶ Lu	5343.91(25)	0.26(3)	0.0045(5)
¹⁷⁶ Lu	313.350(8)	0.40(3)	0.0069(5)	¹⁷⁶ Lu	5465.7(3)	0.218(16)	0.0038(3)
¹⁷⁶Lu	319.036(8)	3.83(13)	0.0663(23)	¹⁷⁶ Lu	5570.12(10)	0.385(24)	0.0067(4)
¹⁷⁶ Lu	322.865(19)	0.31(3)	0.0054(5)	¹⁷⁶ Lu	5601.87(25)	0.327(25)	0.0057(4)
¹⁷⁶ Lu	329.59(3)	0.181(21)	0.0031(4)	¹⁷⁶ Lu	5728.00(10)	0.23(3)	0.0040(5)
¹⁷⁵ Lu	335.8480(20)	1.32(8)	0.0229(14)	¹⁷⁶ Lu	5769.72(10)	0.184(18)	0.0032(3)
¹⁷⁶ Lu	336.323(15)	0.19(3)	0.0033(5)	¹⁷⁶ Lu	6803.92(9)	0.38(8)	0.0066(14)
¹⁷⁶ Lu	346.37(3)	0.35(6)	0.0061(10)	Hafnium (Z=72), At.Wt.=178.49(2), σ_γ^Z =119(3)			
¹⁷⁶ Lu	348.084(9)	0.84(4)	0.0145(7)	¹⁷⁸ Hf	45.8570(10)	1.21(7)	0.0205(12)
¹⁷⁶ Lu	360.096(10)	0.29(9)	0.0050(16)	¹⁷⁷Hf	62.820(21)	5.26(16)	0.089(3)
¹⁷⁶ Lu	364.58(4)	0.62(3)	0.0107(5)	¹⁷⁷Hf	93.182(6)	13.3(9)	0.226(15)
¹⁷⁶Lu	367.433(11)	2.23(8)	0.0386(14)	¹⁷⁹ Hf	93.3240(20)	0.80(5)	0.0136(9)
¹⁷⁶ Lu	393.389(11)	0.54(3)	0.0094(5)	¹⁷⁸ Hf	105.8940(20)	0.335(10)	0.00569(17)
¹⁷⁶ Lu	413.665(13)	0.93(4)	0.0161(7)	¹⁷⁷ Hf	122.8970(10)	0.432(16)	0.0073(3)
¹⁷⁶ Lu	430.452(15)	0.147(21)	0.0025(4)	¹⁷⁴ Hf	125.7(10)	0.2000(20)	0.00340(3)
¹⁷⁶ Lu	436.505(13)	0.145(20)	0.0025(4)	¹⁷⁷ Hf	144.530(3)	0.384(13)	0.00652(22)
¹⁷⁶Lu	457.944(15)	8.3(3)	0.144(5)	¹⁷⁸ Hf	161.1890(20)	0.57(10)	0.0097(17)
¹⁷⁶ Lu	475.46(3)	0.287(16)	0.0050(3)	¹⁷⁸ Hf	193.3100(10)	1.1(3)	0.019(5)
¹⁷⁵ Lu	520.5500(20)	0.20(4)	0.0035(7)	¹⁷⁸ Hf	202.2840(20)	0.65(13)	0.0110(22)
¹⁷⁵ Lu	527.5090(20)	0.32(5)	0.0055(9)	¹⁷⁷Hf	213.439(7)	29.3(7)	0.497(12)
¹⁷⁶ Lu	544.602(18)	0.210(13)	0.00364(23)	¹⁷⁸Hf	214.3410(20)	5.7(6)	0.097(10)
¹⁷⁶ Lu	547.866(16)	0.306(17)	0.0053(3)	¹⁷⁸Hf	214.3410(20)d	16.3(3)	0.277[99%]
¹⁷⁶ Lu	550.288(15)	0.490(21)	0.0085(4)	¹⁷⁹ Hf	215.426(8)	2.77(17)	0.047(3)
¹⁷⁶ Lu	552.073(15)	0.67(3)	0.0116(5)	¹⁷⁹ Hf	235.020(7)	0.38(9)	0.0065(15)
¹⁷⁵ Lu	563.9420(20)	0.51(4)	0.0088(7)	¹⁷⁸ Hf	239.1660(10)	0.293(24)	0.0050(4)
¹⁷⁵ Lu	578.198(3)	0.20(8)	0.0035(14)	¹⁷⁷ Hf	244.3130(20)	0.58(4)	0.0098(7)
¹⁷⁶ Lu	606.65(7)	0.182(15)	0.0032(3)	¹⁷⁷ Hf	244.544(13)	0.97(14)	0.0165(24)
¹⁷⁶ Lu	671.908(15)	0.259(21)	0.0045(4)	¹⁷⁷ Hf	245.2950(20)	0.58(4)	0.0098(7)
¹⁷⁶ Lu	689.77(6)	0.31(5)	0.0054(9)	¹⁷⁷ Hf	256.6010(20)	0.426(20)	0.0072(3)
¹⁷⁶ Lu	695.033(16)	0.296(25)	0.0051(4)	¹⁷⁸ Hf	258.6230(20)	0.44(10)	0.0075(17)
¹⁷⁵ Lu	709.553(4)	0.21(7)	0.0036(12)	¹⁷⁷ Hf	273.166(3)	0.305(16)	0.0052(3)
¹⁷⁶ Lu	716.470(17)	0.189(16)	0.0033(3)	¹⁷⁷ Hf	277.2080(20)	0.47(3)	0.0080(5)
¹⁷⁶Lu	761.564(20)	2.60(9)	0.0450(16)	¹⁷⁷ Hf	289.5570(20)	0.67(4)	0.0114(7)
¹⁷⁵ Lu	834.810(3)	0.20(11)	0.0035(19)	¹⁷⁸Hf	303.9880(20)	3.38(9)	0.0574(15)
¹⁷⁵ Lu	838.643(3)	0.89(10)	0.0154(17)	¹⁷⁷Hf	325.559(4)	6.69(17)	0.114(3)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁷⁹ Hf	332.275(11)	0.73(17)	0.012(3)	¹⁷⁷ Hf	5260.9(5)	0.36(6)	0.0061(10)
¹⁷⁷ Hf	339.1990(20)	1.28(6)	0.0217(10)	¹⁷⁷ Hf	5294.9(5)	0.34(5)	0.0058(9)
¹⁷⁷ Hf	348.369(4)	0.60(8)	0.0102(14)	¹⁷⁷ Hf	5575.22(16)	0.41(4)	0.0070(7)
¹⁷⁷ Hf	426.380(5)	0.35(3)	0.0059(5)	¹⁷⁹ Hf	5647.71(11)	0.38(4)	0.0065(7)
¹⁷⁷ Hf	497.893(3)	1.11(11)	0.0188(19)	¹⁸⁰ Hf	5649.60(21)	0.33(18)	0.006(3)
¹⁷⁶ Hf	508.29(9)	1.05(6)	0.0178(10)	¹⁸⁰ Hf	5695.48(17)	1.09(9)	0.0185(15)
¹⁷⁷ Hf	547.374(5)	0.40(4)	0.0068(7)	¹⁷⁸ Hf	5723.809(22)	1.97(10)	0.0334(17)
¹⁷⁷ Hf	596.894(4)	0.34(13)	0.0058(22)	¹⁷⁷ Hf	5807.42(16)	0.35(5)	0.0059(9)
¹⁷⁸ Hf	729.515(4)	0.53(5)	0.0090(9)	¹⁷⁷ Hf	6111.85(16)	0.92(6)	0.0156(10)
¹⁷⁷ Hf	921.822(5)	0.84(5)	0.0143(9)	¹⁷⁷ Hf	6357.14(16)	0.32(5)	0.0054(9)
¹⁷⁷ Hf	961.919(5)	0.76(7)	0.0129(12)	Tantalum (Z=73), At. Wt.=180.9479(1), σ_γ^Z=20.6(5)			
¹⁷⁷ Hf	970.066(7)	0.32(8)	0.0054(14)	¹⁸¹ Ta	47.8120(20)	0.13(3)	0.0022(5)
¹⁷⁸ Hf	1003.650(4)	0.89(5)	0.0151(9)	¹⁸¹ Ta	54.4710(20)	0.052(13)	0.00087(22)
¹⁷⁷ Hf	1016.663(6)	0.30(13)	0.0051(22)	¹⁸¹ Ta	59.693(3)	0.042(13)	0.00070(22)
¹⁷⁹ Hf	1059.66(4)	0.32(3)	0.0054(5)	¹⁸¹ Ta	71.900(4)	0.060(15)	0.00100(25)
¹⁷⁹ Hf	1065.45(3)	1.94(5)	0.0329(9)	¹⁸¹ Ta	72.932(4)	0.054(15)	0.00090(25)
¹⁷⁷ Hf	1077.844(5)	2.40(6)	0.0407(10)	¹⁸¹ Ta	73.519(4)	0.06(3)	0.0010(5)
¹⁷⁷ Hf	1081.454(6)	2.82(7)	0.0479(12)	¹⁸¹ Ta	74.2680(20)	0.077(22)	0.0013(4)
¹⁷⁷Hf	1102.824(5)	2.96(8)	0.0503(14)	¹⁸¹ Ta	76.549(6)	0.029(13)	0.00049(22)
¹⁷⁷ Hf	1143.737(7)	1.84(6)	0.0312(10)	¹⁸¹ Ta	82.876(4)	0.029(13)	0.00049(22)
¹⁷⁷Hf	1167.072(6)	3.95(10)	0.0671(17)	¹⁸¹ Ta	92.480(3)	0.065(9)	0.00109(15)
¹⁷⁷Hf	1174.635(5)	4.8(7)	0.081(12)	¹⁸¹ Ta	94.1680(20)	0.051(7)	0.00085(12)
¹⁷⁷ Hf	1175.357(7)	2.6(5)	0.044(9)	¹⁸¹ Ta	95.156(3)	0.081(9)	0.00136(15)
¹⁷⁷ Hf	1183.504(8)	1.42(5)	0.0241(9)	¹⁸¹ Ta	97.467(3)	0.065(9)	0.00109(15)
¹⁷⁹ Hf	1197.92(8)	0.44(6)	0.0075(10)	¹⁸¹ Ta	97.8320(20)	0.139(7)	0.00233(12)
¹⁷⁷ Hf	1205.975(5)	1.26(23)	0.021(4)	¹⁸¹ Ta	99.8310(20)	0.127(7)	0.00213(12)
¹⁷⁷Hf	1207.213(5)	3.9(3)	0.066(5)	¹⁸¹ Ta	100.5540(20)	0.060(11)	0.00100(18)
¹⁷⁷ Hf	1226.532(6)	1.30(5)	0.0221(9)	¹⁸¹ Ta	104.1130(20)	0.037(6)	0.00062(10)
¹⁷⁷Hf	1229.287(8)	4.26(11)	0.0723(19)	¹⁸¹ Ta	107.863(3)	0.131(14)	0.00219(23)
¹⁷⁷ Hf	1232.172(5)	1.35(6)	0.0229(10)	¹⁸¹Ta	114.3150(10)	0.280(9)	0.00469(15)
¹⁷⁷ Hf	1247.379(5)	0.49(4)	0.0083(7)	¹⁸¹ Ta	114.3760(20)	0.110(20)	0.0018(3)
¹⁷⁷ Hf	1254.913(7)	0.40(4)	0.0068(7)	¹⁸¹ Ta	114.674(3)	0.193(20)	0.0032(3)
¹⁷⁷ Hf	1269.372(6)	2.26(7)	0.0384(12)	¹⁸¹ Ta	118.8950(20)	0.108(8)	0.00181(13)
¹⁷⁷ Hf	1291.282(6)	0.99(5)	0.0168(9)	¹⁸¹ Ta	119.516(3)	0.039(6)	0.00065(10)
¹⁷⁷ Hf	1310.071(5)	1.45(5)	0.0246(9)	¹⁸¹ Ta	119.6980(20)	0.038(6)	0.00064(10)
¹⁷⁷ Hf	1330.109(5)	2.08(8)	0.0353(14)	¹⁸¹ Ta	121.5340(20)	0.031(3)	0.00052(5)
¹⁷⁷ Hf	1333.832(5)	1.71(9)	0.0290(15)	¹⁸¹ Ta	122.613(3)	0.037(6)	0.00062(10)
¹⁷⁷ Hf	1340.447(6)	2.38(10)	0.0404(17)	¹⁸¹ Ta	122.675(3)	0.092(4)	0.00154(7)
¹⁷⁷ Hf	1344.841(5)	0.59(5)	0.0100(9)	¹⁸¹ Ta	122.9730(20)	0.075(9)	0.00126(15)
¹⁷⁷ Hf	1403.267(20)	0.51(4)	0.0087(7)	¹⁸¹ Ta	125.126(3)	0.030(4)	0.00050(7)
¹⁷⁷ Hf	1420.651(6)	1.81(8)	0.0307(14)	¹⁸¹Ta	133.8770(20)	0.63(7)	0.0106(12)
¹⁷⁷ Hf	1496.448(21)	0.44(3)	0.0075(5)	¹⁸¹ Ta	139.4560(20)	0.094(10)	0.00157(17)
¹⁷⁷ Hf	1542.416(7)	0.55(8)	0.0093(14)	¹⁸¹ Ta	139.6610(20)	0.029(3)	0.00049(5)
¹⁷⁷ Hf	1649.794(6)	0.367(22)	0.0062(4)	¹⁸¹ Ta	141.2450(20)	0.062(9)	0.00104(15)
¹⁷⁸ Hf	1649.81(10)	0.46(4)	0.0078(7)	¹⁸¹ Ta	142.261(5)	0.042(13)	0.00070(22)
¹⁷⁷ Hf	1725.094(10)	0.46(5)	0.0078(9)	¹⁸¹ Ta	143.156(7)	0.061(9)	0.00102(15)
¹⁷⁷ Hf	1848.821(8)	0.46(5)	0.0078(9)	¹⁸¹ Ta	146.7740(20)	0.141(4)	0.00236(7)
¹⁸⁰ Hf	1895.38(16)	0.54(5)	0.0092(9)	¹⁸¹ Ta	154.0850(20)	0.082(3)	0.00137(5)
¹⁷⁷ Hf	1904.272(10)	0.71(6)	0.0121(10)	¹⁸¹ Ta	156.0880(20)	0.233(6)	0.00390(10)
¹⁷⁷ Hf	1927.998(7)	0.30(5)	0.0051(9)	¹⁸¹ Ta	156.2300(20)	0.046(3)	0.00077(5)
¹⁷⁷ Hf	1957.294(12)	0.31(4)	0.0053(7)	¹⁸¹ Ta	159.048(3)	0.0449(23)	0.00075(4)
¹⁷⁸ Hf	3497.81(25)	0.31(5)	0.0053(9)	¹⁸¹ Ta	167.413(3)	0.031(3)	0.00052(5)
¹⁷⁸ Hf	4336.18(4)	0.35(4)	0.0059(7)	¹⁸¹ Ta	168.130(4)	0.033(9)	0.00055(15)
¹⁷⁸ Hf	4343.69(4)	0.44(5)	0.0075(9)	¹⁸¹ Ta	171.580(3)d	0.005400(11)	9.044E-5[65%]
¹⁷⁹ Hf	4915.2(6)	0.35(5)	0.0059(9)	¹⁸¹ Ta	171.580(3)	0.029(4)	0.00049(7)
¹⁷⁷ Hf	5068.3(5)	0.32(5)	0.0054(9)	¹⁸¹Ta	173.2050(20)	1.210(25)	0.0203(4)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁸² W	1556.18(13)	0.014(3)	2.3(5)E-4	¹⁸⁶ W	3920.2(4)	0.017(3)	0.00028(5)
¹⁸³ W	1569.9(3)	0.013(3)	2.1(5)E-4	¹⁸⁶ W	3964.87(18)	0.034(9)	0.00056(15)
¹⁸³ W	1765.47(9)	0.0105(22)	1.7(4)E-4	¹⁸² W	4014.17(5)	0.050(10)	0.00082(16)
¹⁸³ W	1919.4(4)	0.019(4)	0.00031(7)	¹⁸⁶ W	4018.1(5)	0.029(6)	0.00048(10)
¹⁸³ W	1945.14(15)	0.020(3)	0.00033(5)	¹⁸² W	4026.21(10)	0.019(3)	0.00031(5)
¹⁸³ W	1949.69(7)	0.0097(21)	1.6(4)E-4	¹⁸² W	4064.48(9)	0.018(3)	0.00030(5)
¹⁸³ W	1995.48(21)	0.0103(20)	1.7(3)E-4	¹⁸⁶ W	4082.8(5)	0.051(11)	0.00084(18)
¹⁸³ W	2014.85(5)	0.0104(15)	1.71(25)E-4	¹⁸⁶ W	4119.24(10)	0.059(4)	0.00097(7)
¹⁸³ W	2035.64(17)	0.025(3)	0.00041(5)	¹⁸⁶ W	4136.61(17)	0.034(5)	0.00056(8)
¹⁸³ W	2135.08(21)	0.013(3)	2.1(5)E-4	¹⁸⁶ W	4158.13(21)	0.043(5)	0.00071(8)
¹⁸³ W	2183.29(8)	0.022(3)	0.00036(5)	¹⁸² W	4162.33(17)	0.0122(15)	2.01(25)E-4
¹⁸³ W	2284.32(19)	0.018(4)	0.00030(7)	¹⁸⁴ W	4219.2(8)	0.034(16)	0.0006(3)
¹⁸⁶ W	2293.1(7)	0.011(3)	1.8(5)E-4	¹⁸² W	4246.61(4)	0.043(4)	0.00071(7)
¹⁸⁶ W	2367.1(4)	0.030(16)	0.0005(3)	¹⁸⁶ W	4249.66(7)	0.115(6)	0.00190(10)
¹⁸³ W	2369.9(3)	0.018(4)	0.00030(7)	¹⁸² W	4304.65(6)	0.020(3)	0.00033(5)
¹⁸⁶ W	2481.30(25)	0.031(4)	0.00051(7)	¹⁸⁶ W	4331.63(8)	0.040(4)	0.00066(7)
¹⁸⁶ W	2556.0(3)	0.021(4)	0.00035(7)	¹⁸² W	4367.18(4)	0.026(3)	0.00043(5)
¹⁸⁶ W	2584.20(18)	0.031(4)	0.00051(7)	¹⁸² W	4379.77(5)	0.017(3)	0.00028(5)
¹⁸⁶ W	2689.5(3)	0.024(4)	0.00040(7)	¹⁸⁶ W	4384.20(9)	0.057(5)	0.00094(8)
¹⁸⁶ W	2708.4(3)	0.026(4)	0.00043(7)	¹⁸⁶ W	4448.10(9)	0.048(3)	0.00079(5)
¹⁸⁶ W	2727.5(4)	0.021(11)	0.00035(18)	¹⁸² W	4460.59(9)	0.0124(23)	2.0(4)E-4
¹⁸⁶ W	2738.4(3)	0.032(4)	0.00053(7)	¹⁸⁴ W	4469.1(6)	0.022(10)	0.00036(16)
¹⁸⁶ W	2760.3(3)	0.033(4)	0.00054(7)	¹⁸⁶ W	4491.51(10)	0.036(10)	0.00059(16)
¹⁸⁶ W	2831.98(20)	0.023(4)	0.00038(7)	¹⁸² W	4518.11(5)	0.039(5)	0.00064(8)
¹⁸⁶ W	2849.3(3)	0.033(4)	0.00054(7)	¹⁸⁴ W	4535.5(3)	0.08(4)	0.0013(7)
¹⁸⁶ W	2939.4(4)	0.014(4)	2.3(7)E-4	¹⁸⁶ W	4557.49(11)	0.025(5)	0.00041(8)
¹⁸⁶ W	3055.01(20)	0.0290(25)	0.00048(4)	¹⁸² W	4562.86(14)	0.026(3)	0.00043(5)
¹⁸⁶ W	3097.3(4)	0.015(3)	2.5(5)E-4	¹⁸⁴ W	4573.7(3)	0.104(9)	0.00171(15)
¹⁸⁶ W	3114.78(20)	0.025(3)	0.00041(5)	¹⁸⁶ W	4574.94(8)	0.152(10)	0.00251(16)
¹⁸⁶ W	3148.2(5)	0.086(19)	0.0014(3)	¹⁸⁶ W	4626.35(7)	0.124(7)	0.00204(12)
¹⁸⁶ W	3153.9(10)	0.061(20)	0.0010(3)	¹⁸² W	4634.64(13)	0.015(4)	2.5(7)E-4
¹⁸⁶ W	3191.92(25)	0.037(3)	0.00061(5)	¹⁸⁶ W	4650.40(7)	0.052(5)	0.00086(8)
¹⁸⁶ W	3207.0(3)	0.030(4)	0.00049(7)	¹⁸⁶ W	4684.40(8)	0.150(7)	0.00247(12)
¹⁸⁶ W	3225.15(17)	0.042(6)	0.00069(10)	¹⁸² W	4719.90(5)	0.0189(25)	0.00031(4)
¹⁸⁶ W	3267.1(5)	0.0101(24)	1.7(4)E-4	¹⁸⁴ W	4748.7(4)	0.06(3)	0.0010(5)
¹⁸⁶ W	3314.4(4)	0.015(3)	2.5(5)E-4	¹⁸⁴ W	4931.79(25)	0.0119(23)	2.0(4)E-4
¹⁸⁶ W	3376.15(18)	0.041(4)	0.00068(7)	¹⁸⁴ W	4980.5(9)	0.017(8)	0.00028(13)
¹⁸⁶ W	3423.0(4)	0.030(3)	0.00049(5)	¹⁸⁴ W	4986.2(3)	0.019(9)	0.00031(15)
¹⁸⁶ W	3443.2(4)	0.039(12)	0.00064(20)	¹⁸³ W	5015.52(20)	0.0162(20)	0.00027(3)
¹⁸⁶ W	3452.8(9)	0.055(10)	0.00091(16)	¹⁸⁴ W	5091.05(25)	0.07(3)	0.0012(5)
¹⁸⁶ W	3469.40(14)	0.103(6)	0.00170(10)	¹⁸³ W	5116.55(10)	0.0114(16)	1.9(3)E-4
¹⁸⁶ W	3492.67(17)	0.051(4)	0.00084(7)	¹⁸² W	5164.43(3)	0.19(3)	0.0031(5)
¹⁸⁶ W	3510.72(19)	0.033(4)	0.00054(7)	¹⁸² W	5256.22(4)	0.0122(12)	2.01(20)E-4
¹⁸⁶ W	3529.69(18)	0.040(4)	0.00066(7)	¹⁸⁶ W	5261.68(6)	0.86(4)	0.0142(7)
¹⁸⁶ W	3534.56(17)	0.063(5)	0.00104(8)	¹⁸³ W	5285.00(8)	0.0115(14)	1.90(23)E-4
¹⁸⁶ W	3561.14(14)	0.060(4)	0.00099(7)	¹⁸⁶ W	5320.72(6)	0.605(21)	0.0100(4)
¹⁸⁶ W	3577.2(4)	0.016(4)	0.00026(7)	¹⁸⁶ W	5466.50(6)	0.023(4)	0.00038(7)
¹⁸³ W	3696.2(4)	0.011(3)	1.8(5)E-4	¹⁸³ W	5534.37(11)	0.011(4)	1.8(7)E-4
¹⁸⁶ W	3710.1(4)	0.034(8)	0.00056(13)	¹⁸⁴ W	5754.53(21)	0.0112(18)	1.8(3)E-4
¹⁸⁶ W	3739.05(17)	0.069(4)	0.00114(7)	¹⁸³ W	5796.19(9)	0.023(9)	0.00038(15)
¹⁸⁶ W	3760.9(3)	0.026(3)	0.00043(5)	¹⁸³ W	5797.50(9)	0.0161(23)	0.00027(4)
¹⁸⁶ W	3774.59(21)	0.026(3)	0.00043(5)	¹⁸³ W	6024.82(7)	0.036(3)	0.00059(5)
¹⁸⁶ W	3804.7(4)	0.020(3)	0.00033(5)	¹⁸² W	6144.28(3)	0.174(11)	0.00287(18)
¹⁸⁶ W	3847.8(4)	0.051(4)	0.00084(7)	¹⁸³ W	6189.75(7)	0.0264(24)	0.00044(4)
¹⁸³ W	3864.4(4)	0.011(3)	1.8(5)E-4	¹⁸² W	6190.78(3)	0.45(4)	0.0074(7)
¹⁸⁶ W	3886.4(3)	0.014(3)	2.3(5)E-4	¹⁸³ W	6289.64(7)	0.0235(19)	0.00039(3)
¹⁸⁶ W	3901.8(3)	0.024(3)	0.00040(5)				

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁸⁷ Re	290.665(6)	3.5(4)	0.057(7)	¹⁸⁷ Re	4916.3(3)	0.102(21)	0.0017(3)
¹⁸⁷ Re	291.492(8)	0.94(7)	0.0153(11)	¹⁸⁷ Re	4958.7(5)	0.14(3)	0.0023(5)
¹⁸⁷ Re	299.130(9)	0.151(14)	0.00246(23)	¹⁸⁷ Re	4973.1(5)	0.15(3)	0.0024(5)
¹⁸⁷ Re	300.210(4)	0.70(5)	0.0114(8)	¹⁸⁷ Re	4987.9(4)	0.17(4)	0.0028(7)
¹⁸⁵ Re	307.673(16)	0.34(3)	0.0055(5)	¹⁸⁷ Re	5000.8(4)	0.17(4)	0.0028(7)
¹⁸⁵ Re	316.457(9)	2.21(10)	0.0360(16)	¹⁸⁵ Re	5007.0(5)	0.27(4)	0.0044(7)
¹⁸⁷ Re	317.38(5)	0.083(17)	0.0014(3)	¹⁸⁷ Re	5012.60(25)	0.18(3)	0.0029(5)
¹⁸⁷ Re	318.37(3)	0.25(3)	0.0041(5)	¹⁸⁷ Re	5020.6(4)	0.098(23)	0.0016(4)
¹⁸⁵ Re	319.374(9)	0.18(3)	0.0029(5)	¹⁸⁵ Re	5027.9(4)	0.29(5)	0.0047(8)
¹⁸⁷ Re	352.11(3)	0.116(16)	0.0019(3)	¹⁸⁵ Re	5048.8(6)	0.096(23)	0.0016(4)
¹⁸⁵ Re	355.646(17)	0.115(16)	0.0019(3)	¹⁸⁷ Re	5049.3(3)	0.16(3)	0.0026(5)
¹⁸⁵ Re	358.11(10)	0.236(19)	0.0038(3)	¹⁸⁷ Re	5073.28(23)	0.43(5)	0.0070(8)
¹⁸⁵ Re	360.36(7)	0.449(25)	0.0073(4)	¹⁸⁷ Re	5080.3(4)	0.098(23)	0.0016(4)
¹⁸⁷ Re	362.712(9)	0.46(3)	0.0075(5)	¹⁸⁵ Re	5080.7(8)	0.094(23)	0.0015(4)
¹⁸⁵ Re	363.612(8)	0.16(4)	0.0026(7)	¹⁸⁷ Re	5134.8(3)	0.25(6)	0.0041(10)
¹⁸⁷ Re	376.816(10)	0.083(16)	0.0014(3)	¹⁸⁵ Re	5137.6(6)	0.39(4)	0.0063(7)
¹⁸⁵ Re	378.384(9)	0.54(3)	0.0088(5)	¹⁸⁷ Re	5167.6(3)	0.14(3)	0.0023(5)
¹⁸⁵ Re	390.854(23)	1.15(5)	0.0187(8)	¹⁸⁵ Re	5176.3(5)	0.18(3)	0.0029(5)
¹⁸⁷ Re	406.555(9)	0.18(4)	0.0029(7)	¹⁸⁷ Re	5224.37(7)	0.081(20)	0.0013(3)
¹⁸⁵ Re	407.05(16)	0.102(24)	0.0017(4)	¹⁸⁵ Re	5276.7(5)	0.14(3)	0.0023(5)
¹⁸⁵ Re	410.74(15)	0.10(3)	0.0016(5)	¹⁸⁷ Re	5314.86(9)	0.083(20)	0.0014(3)
¹⁸⁵ Re	411.496(10)	0.14(3)	0.0023(5)	¹⁸⁷ Re	5348.62(6)	0.20(3)	0.0033(5)
¹⁸⁵ Re	413.19(5)	0.16(4)	0.0026(7)	¹⁸⁵ Re	5353.10(13)	0.13(3)	0.0021(5)
¹⁸⁷ Re	423.525(21)	0.12(3)	0.0020(5)	¹⁸⁷ Re	5371.95(6)	0.090(19)	0.0015(3)
¹⁸⁷ Re	426.112(9)	0.13(3)	0.0021(5)	¹⁸⁵ Re	5493.19(13)	0.114(18)	0.0019(3)
¹⁸⁵ Re	439.09(23)	0.14(5)	0.0023(8)	¹⁸⁵ Re	5601.53(13)	0.109(18)	0.0018(3)
¹⁸⁵ Re	469.79(10)	0.09(3)	0.0015(5)	¹⁸⁷ Re	5614.74(6)	0.092(17)	0.0015(3)
¹⁸⁵ Re	479.6(3)	0.30(13)	0.0049(21)	¹⁸⁵ Re	5644.95(15)	0.088(16)	0.0014(3)
¹⁸⁷ Re	493.23(6)	0.10(3)	0.0016(5)	¹⁸⁷ Re	5688.91(6)	0.120(17)	0.0020(3)
¹⁸⁵ Re	496.57(14)	0.15(4)	0.0024(7)	¹⁸⁷ Re	5702.21(6)	0.100(16)	0.0016(3)
¹⁸⁷ Re	518.575(9)	0.24(6)	0.0039(10)	¹⁸⁵ Re	5708.74(13)	0.115(17)	0.0019(3)
¹⁸⁵ Re	550.77(23)	0.15(4)	0.0024(7)	¹⁸⁵ Re	5709.49(20)	0.098(24)	0.0016(4)
¹⁸⁷ Re	556.81(6)	0.13(4)	0.0021(7)	¹⁸⁷ Re	5715.61(6)	0.086(16)	0.0014(3)
¹⁸⁵ Re	585.4(3)	0.18(3)	0.0029(5)	¹⁸⁵ Re	5856.86(13)	0.140(15)	0.00228(24)
¹⁸⁵ Re	608.25(14)	0.25(3)	0.0041(5)	¹⁸⁷ Re	5871.65(6)	0.299(23)	0.0049(4)
¹⁸⁷ Re	609.04(3)	0.25(3)	0.0041(5)	¹⁸⁵ Re	5910.44(13)	0.60(4)	0.0098(7)
¹⁸⁵ Re	645.02(14)	0.18(3)	0.0029(5)	¹⁸⁵ Re	6005.30(13)	0.081(11)	0.00132(18)
¹⁸⁵ Re	680.49(10)	0.34(3)	0.0055(5)	¹⁸⁵ Re	6032.96(13)	0.090(12)	0.00146(20)
¹⁸⁵ Re	759.94(14)	0.17(5)	0.0028(8)	¹⁸⁵ Re	6079.87(13)	0.155(13)	0.00252(21)
¹⁸⁵ Re	761.47(23)	0.17(5)	0.0028(8)	¹⁸⁵ Re	6120.22(13)	0.182(16)	0.0030(3)
¹⁸⁵ Re	796.1(3)	0.31(3)	0.0050(5)	Osmium (Z=76), At.Wt.=190.23(3), $\sigma_{\gamma}^Z = 16.0(11)$			
¹⁸⁵ Re	3933.7(8)	0.09(4)	0.0015(7)	¹⁸⁴ Os	37.18(13)	0.034(6)	0.00054(10)
¹⁸⁵ Re	4079.0(8)	0.14(3)	0.0023(5)	¹⁹⁰ Os	57.480(10)	0.10(3)	0.0016(5)
¹⁸⁵ Re	4099.8(10)	0.13(3)	0.0021(5)	¹⁹⁰ Os	57.74(6)	0.081(6)	0.00129(10)
¹⁸⁵ Re	4129.4(8)	0.100(24)	0.0016(4)	¹⁸⁸ Os	59.079(16)	0.046(5)	0.00073(8)
¹⁸⁵ Re	4178.1(5)	0.088(22)	0.0014(4)	¹⁹⁰ Os	67.24(20)	0.021(4)	0.00033(6)
¹⁸⁵ Re	4455.7(23)	0.11(3)	0.0018(5)	¹⁹² Os	73.43(4)	0.174(8)	0.00277(13)
¹⁸⁵ Re	4611.3(5)	0.081(20)	0.0013(3)	¹⁸⁴ Os	90.95(15)	0.030(15)	0.00048(24)
¹⁸⁵ Re	4631.7(23)	0.085(23)	0.0014(4)	¹⁹² Os	131.26(5)	0.0291(17)	0.00046(3)
¹⁸⁵ Re	4663.7(4)	0.24(3)	0.0039(5)	¹⁹⁰ Os	138.070(10)	0.0239(16)	0.000381(25)
¹⁸⁵ Re	4743.5(8)	0.113(21)	0.0018(3)	¹⁹² Os	138.92(3)d	0.0467(22)	0.00074[1.1%]
¹⁸⁵ Re	4773.7(5)	0.18(3)	0.0029(5)	¹⁸⁷ Os	155.10(4)	1.19(3)	0.0190(5)
¹⁸⁵ Re	4860.7(5)	0.37(4)	0.0060(7)	¹⁸⁴ Os	158.40(10)	0.025(7)	0.00040(11)
¹⁸⁵ Re	4871.7(8)	0.11(3)	0.0018(5)	¹⁹⁰ Os	172.50(10)	0.025(4)	0.00040(6)
¹⁸⁷ Re	4888.6(3)	0.141(25)	0.0023(4)	¹⁹⁰ Os	175.80(4)	0.189(8)	0.00301(13)
¹⁸⁷ Re	4893.4(3)	0.081(17)	0.0013(3)	¹⁸⁶ Os	177.42(20)	0.021(4)	0.00033(6)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁸⁹ Os	182.02(10)	0.027(7)	0.00043(11)	¹⁹⁰ Os	434.16(12)	0.032(4)	0.00051(6)
¹⁹⁰ Os	182.30(10)	0.043(5)	0.00069(8)	¹⁹⁰ Os	442.18(12)	0.022(4)	0.00035(6)
¹⁸⁹Os	186.7180(20)	2.08(5)	0.0331(8)	¹⁸⁹ Os	447.79(7)	0.0213(19)	0.00034(3)
¹⁹⁰ Os	194.25(8)	0.028(3)	0.00045(5)	¹⁹⁰ Os	453.69(24)	0.022(5)	0.00035(8)
¹⁸⁹ Os	198.084(21)	0.056(7)	0.00089(11)	¹⁸⁸ Os	454.794(21)	0.028(9)	0.00045(14)
¹⁹² Os	204.42(4)	0.081(4)	0.00129(6)	¹⁹² Os	455.47(24)	0.025(5)	0.00040(8)
¹⁸⁴ Os	222.38(14)	0.021(7)	0.00033(11)	¹⁸⁸ Os	469.682(21)	0.040(5)	0.00064(8)
¹⁸⁹ Os	223.810(7)	0.052(4)	0.00083(6)	¹⁹² Os	471.60(25)	0.021(5)	0.00033(8)
¹⁹⁰ Os	229.93(4)	0.072(4)	0.00115(6)	¹⁹⁰ Os	475.33(16)	0.032(6)	0.00051(10)
¹⁹⁰ Os	235.24(3)	0.184(6)	0.00293(10)	¹⁸⁷Os	478.04(4)	0.523(14)	0.00833(22)
¹⁹⁰ Os	239.890(10)	0.080(4)	0.00127(6)	¹⁹⁰ Os	480.85(12)	0.043(7)	0.00069(11)
¹⁹² Os	242.41(4)	0.069(4)	0.00110(6)	¹⁹⁰ Os	485.87(20)	0.027(7)	0.00043(11)
¹⁹² Os	254.39(5)	0.0368(22)	0.00059(4)	¹⁸⁷ Os	487.62(12)	0.044(7)	0.00070(11)
¹⁹² Os	265.71(3)	0.101(3)	0.00161(5)	¹⁹⁰ Os	495.68(9)	0.035(7)	0.00056(11)
¹⁸⁸Os	272.82(4)	0.242(6)	0.00386(10)	¹⁹⁰ Os	499.77(8)	0.054(5)	0.00086(8)
¹⁹⁰ Os	275.34(3)	0.173(5)	0.00276(8)	¹⁸⁸ Os	505.861(20)	0.021(4)	0.00033(6)
¹⁹⁰ Os	291.650(10)	0.047(3)	0.00075(5)	¹⁸⁴ Os	512.84(5)	0.084(8)	0.00134(13)
¹⁹⁰ Os	295.030(10)	0.030(5)	0.00048(8)	¹⁸⁷ Os	514.76(9)	0.038(4)	0.00061(6)
¹⁹² Os	295.41(5)	0.055(4)	0.00088(6)	¹⁸⁴ Os	521.9(3)	0.024(5)	0.00038(8)
¹⁹⁰ Os	304.71(6)	0.073(4)	0.00116(6)	¹⁹⁰Os	527.60(3)	0.300(10)	0.00478(16)
¹⁹⁰ Os	305.020(10)	0.022(4)	0.00035(6)	¹⁹⁰ Os	537.75(4)	0.121(6)	0.00193(10)
¹⁹² Os	307.080(10)	0.026(3)	0.00041(5)	¹⁸⁴ Os	538.8(4)	0.023(7)	0.00037(11)
¹⁹⁰ Os	307.21(10)	0.026(3)	0.00041(5)	¹⁸⁴ Os	539.40(24)	0.022(4)	0.00035(6)
¹⁹⁰ Os	314.72(10)	0.039(3)	0.00062(5)	¹⁹⁰ Os	545.29(13)	0.031(4)	0.00049(6)
¹⁹⁰ Os	316.45(11)	0.030(4)	0.00048(6)	¹⁸⁸ Os	550.17(5)	0.021(4)	0.00033(6)
¹⁸⁷Os	322.98(6)	0.242(9)	0.00386(14)	¹⁸⁹Os	557.978(5)	0.84(3)	0.0134(5)
¹⁹⁰ Os	332.690(10)	0.055(5)	0.00088(8)	¹⁸⁹Os	569.344(20)	0.694(25)	0.0111(4)
¹⁹⁰ Os	339.61(5)	0.055(3)	0.00088(5)	¹⁸⁴ Os	589.87(19)	0.034(5)	0.00054(8)
¹⁸⁸ Os	343.473(20)	0.051(16)	0.00081(25)	¹⁸⁹ Os	605.26(3)	0.113(4)	0.00180(6)
¹⁹⁰ Os	343.61(6)	0.046(3)	0.00073(5)	¹⁸⁷ Os	623.92(11)	0.036(4)	0.00057(6)
¹⁹⁰ Os	345.92(10)	0.034(4)	0.00054(6)	¹⁸⁹ Os	630.985(23)	0.023(4)	0.00037(6)
¹⁸⁸ Os	346.871(25)	0.025(8)	0.00040(13)	¹⁸⁷Os	633.14(4)	0.585(16)	0.00932(25)
¹⁸⁷ Os	347.24(17)	0.023(4)	0.00037(6)	¹⁸⁷Os	635.02(5)	0.405(12)	0.00645(19)
¹⁹⁰ Os	349.25(6)	0.051(4)	0.00081(6)	¹⁹⁰ Os	636.7(3)	0.028(6)	0.00045(10)
¹⁹⁰ Os	352.56(9)	0.041(5)	0.00065(8)	¹⁹² Os	655.61(13)	0.025(3)	0.00040(5)
¹⁸⁹ Os	353.85(5)	0.0213(24)	0.00034(4)	¹⁹⁰ Os	664.18(9)	0.036(4)	0.00057(6)
¹⁹⁰ Os	355.80(10)	0.025(4)	0.00040(6)	¹⁸⁷ Os	672.64(11)	0.045(4)	0.00072(6)
¹⁸⁹ Os	358.71(5)	0.033(4)	0.00053(6)	¹⁸⁹ Os	725.11(5)	0.081(5)	0.00129(8)
¹⁹⁰ Os	359.01(7)	0.047(4)	0.00075(6)	¹⁸⁹ Os	768.653(15)	0.037(3)	0.00059(5)
¹⁸⁹Os	361.137(6)	0.466(15)	0.00742(24)	¹⁹⁰ Os	768.67(10)	0.046(5)	0.00073(8)
¹⁹⁰ Os	362.36(15)	0.040(9)	0.00064(14)	¹⁹² Os	786.64(15)	0.033(4)	0.00053(6)
¹⁹⁰ Os	365.04(12)	0.035(5)	0.00056(8)	¹⁸⁷ Os	810.60(11)	0.035(3)	0.00056(5)
¹⁹⁰ Os	366.33(5)	0.097(6)	0.00155(10)	¹⁸⁷ Os	824.43(11)	0.052(4)	0.00083(6)
¹⁸⁹Os	371.261(5)	0.574(14)	0.00914(22)	¹⁸⁷ Os	826.79(10)	0.029(3)	0.00046(5)
¹⁹⁰ Os	397.270(10)	0.038(6)	0.00061(10)	¹⁸⁹ Os	829.07(3)	0.056(6)	0.00089(10)
¹⁸⁹ Os	397.394(14)	0.115(5)	0.00183(8)	¹⁸⁷ Os	829.62(12)	0.109(16)	0.00174(25)
¹⁸⁶ Os	400.84(22)	0.022(6)	0.00035(10)	¹⁸⁷ Os	844.68(14)	0.024(4)	0.00038(6)
¹⁹⁰ Os	403.25(5)	0.065(4)	0.00104(6)	¹⁸⁹ Os	928.06(5)	0.085(5)	0.00135(8)
¹⁸⁹ Os	407.175(22)	0.060(7)	0.00096(11)	¹⁸⁷ Os	931.31(8)	0.073(5)	0.00116(8)
¹⁸⁹ Os	407.517(15)	0.134(5)	0.00213(8)	¹⁹² Os	951.14(5)	0.089(4)	0.00142(6)
¹⁸⁸ Os	410.602(21)	0.028(9)	0.00045(14)	¹⁸⁷ Os	987.33(13)	0.031(4)	0.00049(6)
¹⁹⁰ Os	413.23(4)	0.103(5)	0.00164(8)	¹⁸⁹ Os	987.41(7)	0.071(6)	0.00113(10)
¹⁹⁰ Os	423.76(7)	0.044(4)	0.00070(6)	¹⁸⁹ Os	1011.09(10)	0.031(4)	0.00049(6)
¹⁸⁶ Os	427.07(17)	0.022(4)	0.00035(6)	¹⁸⁷ Os	1017.84(20)	0.043(4)	0.00069(6)
¹⁸⁴ Os	431.45(20)	0.09(3)	0.0014(5)	¹⁸⁹ Os	1103.08(8)	0.047(5)	0.00075(8)
¹⁸⁹ Os	431.68(3)	0.036(4)	0.00057(6)	¹⁸⁹ Os	1114.77(5)	0.060(5)	0.00096(8)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁸⁹ Os	1117.79(8)	0.033(5)	0.00053(8)	¹⁸⁷ Os	3176.9(3)	0.025(5)	0.00040(8)
¹⁸⁷ Os	1149.77(8)	0.079(6)	0.00126(10)	¹⁹² Os	3980.58(25)	0.035(4)	0.00056(6)
¹⁸⁹ Os	1154.47(16)	0.029(9)	0.00046(14)	¹⁸⁸ Os	4222.8(5)	0.052(6)	0.00083(10)
¹⁹⁰ Os	1155.76(15)	0.042(5)	0.00067(8)	¹⁹² Os	4530.27(22)	0.090(8)	0.00143(13)
¹⁸⁷ Os	1174.82(20)	0.038(7)	0.00061(11)	¹⁹⁰ Os	4556.2(3)	0.035(7)	0.00056(11)
¹⁸⁹ Os	1174.95(9)	0.080(6)	0.00127(10)	¹⁹⁰ Os	4666.6(3)	0.024(6)	0.00038(10)
¹⁸⁷ Os	1191.92(17)	0.034(5)	0.00054(8)	¹⁹² Os	4694.4(3)	0.025(5)	0.00040(8)
¹⁸⁹ Os	1195.95(11)	0.077(6)	0.00123(10)	¹⁸⁷ Os	4749.98(22)	0.042(6)	0.00067(10)
¹⁸⁷ Os	1209.62(13)	0.063(6)	0.00100(10)	¹⁸⁷ Os	4812.6(3)	0.049(7)	0.00078(11)
¹⁸⁹ Os	1213.91(13)	0.031(6)	0.00049(10)	¹⁸⁷ Os	4919.6(3)	0.037(3)	0.00059(5)
¹⁸⁹ Os	1249.14(6)	0.035(5)	0.00056(8)	¹⁸⁷ Os	4959.35(25)	0.021(5)	0.00033(8)
¹⁸⁹ Os	1254.76(20)	0.041(5)	0.00065(8)	¹⁹⁰ Os	5010.7(3)	0.029(6)	0.00046(10)
¹⁸⁹ Os	1265.85(12)	0.029(5)	0.00046(8)	¹⁹⁰ Os	5036.9(3)	0.041(6)	0.00065(10)
¹⁸⁹ Os	1301.17(8)	0.035(5)	0.00056(8)	¹⁸⁷ Os	5096.77(22)	0.037(7)	0.00059(11)
¹⁸⁷ Os	1307.9(3)	0.025(3)	0.00040(5)	¹⁹⁰ Os	5146.63(14)	0.409(20)	0.0065(3)
¹⁸⁹ Os	1311.29(8)	0.031(3)	0.00049(5)	¹⁸⁷ Os	5172.38(25)	0.031(6)	0.00049(10)
¹⁸⁷ Os	1322.72(14)	0.037(4)	0.00059(6)	¹⁸⁷ Os	5223.66(21)	0.0215(21)	0.00034(3)
¹⁸⁷ Os	1332.35(20)	0.05(3)	0.0008(5)	¹⁸⁷ Os	5250.4(7)	0.029(6)	0.00046(10)
¹⁸⁷ Os	1332.53(25)	0.040(4)	0.00064(6)	¹⁹² Os	5277.11(22)	0.116(15)	0.00185(24)
¹⁸⁹ Os	1382.66(11)	0.026(3)	0.00041(5)	¹⁸⁹ Os	5315.8(3)	0.024(7)	0.00038(11)
¹⁸⁹ Os	1383.59(23)	0.026(4)	0.00041(6)	¹⁹⁰ Os	5341.4(3)	0.074(12)	0.00118(19)
¹⁸⁹ Os	1384.7(4)	0.023(5)	0.00037(8)	¹⁸⁸ Os	5364.5(4)	0.028(7)	0.00045(11)
¹⁸⁹ Os	1412.00(13)	0.0272(22)	0.00043(4)	¹⁸⁷ Os	5366.38(21)	0.028(7)	0.00045(11)
¹⁸⁹ Os	1429.31(11)	0.028(5)	0.00045(8)	¹⁸⁸ Os	5371.8(4)	0.023(7)	0.00037(11)
¹⁸⁷ Os	1435.74(14)	0.055(10)	0.00088(16)	¹⁸⁸ Os	5416.0(4)	0.053(20)	0.0008(3)
¹⁸⁹ Os	1436.94(14)	0.045(6)	0.00072(10)	¹⁸⁸ Os	5483.1(4)	0.049(8)	0.00078(13)
¹⁸⁷ Os	1452.88(19)	0.024(4)	0.00038(6)	¹⁸⁷ Os	5484.35(24)	0.049(8)	0.00078(13)
¹⁸⁷ Os	1457.56(11)	0.059(5)	0.00094(8)	¹⁸⁹ Os	5502.8(6)	0.021(6)	0.00033(10)
¹⁸⁷ Os	1465.36(13)	0.048(5)	0.00076(8)	¹⁸⁷ Os	5528.34(22)	0.038(7)	0.00061(11)
¹⁸⁹ Os	1489.05(8)	0.031(6)	0.00049(10)	¹⁸⁹ Os	5529.1(7)	0.045(8)	0.00072(13)
¹⁸⁹ Os	1512.11(19)	0.039(7)	0.00062(11)	¹⁸⁷ Os	5573.17(15)	0.052(6)	0.00083(10)
¹⁸⁹ Os	1546.20(9)	0.049(7)	0.00078(11)	¹⁹² Os	5583.70(20)	0.076(7)	0.00121(11)
¹⁸⁷ Os	1574.48(14)	0.031(6)	0.00049(10)	¹⁸⁹ Os	5599.6(7)	0.024(5)	0.00038(8)
¹⁸⁹ Os	1616.03(11)	0.033(6)	0.00053(10)	¹⁸⁷ Os	5641.20(23)	0.023(4)	0.00037(6)
¹⁸⁹ Os	1672.42(8)	0.035(6)	0.00056(10)	¹⁹⁰ Os	5674.5(4)	0.038(7)	0.00061(11)
¹⁸⁹ Os	1680.73(16)	0.053(6)	0.00084(10)	¹⁸⁹ Os	5680.3(3)	0.045(9)	0.00072(14)
¹⁸⁹ Os	1732.0(3)	0.024(5)	0.00038(8)	¹⁹⁰ Os	5683.87(21)	0.167(13)	0.00266(21)
¹⁸⁹ Os	1770.5(5)	0.026(3)	0.00041(5)	¹⁸⁷ Os	5702.93(15)	0.050(8)	0.00080(13)
¹⁸⁷ Os	1802.35(13)	0.035(5)	0.00056(8)	¹⁸⁶ Os	5703.4(7)	0.050(8)	0.00080(13)
¹⁸⁹ Os	1883.37(19)	0.027(9)	0.00043(14)	¹⁸⁹ Os	5749.8(10)	0.026(6)	0.00041(10)
¹⁸⁷ Os	1957.46(13)	0.027(6)	0.00043(10)	¹⁸⁹ Os	5782.7(3)	0.024(6)	0.00038(10)
¹⁸⁷ Os	2011.29(20)	0.021(5)	0.00033(8)	¹⁸⁹ Os	5873.5(3)	0.030(6)	0.00048(10)
¹⁸⁷ Os	2022.95(14)	0.053(6)	0.00084(10)	¹⁸⁹ Os	5881.67(19)	0.035(6)	0.00056(10)
¹⁸⁷ Os	2098.77(22)	0.0208(24)	0.00033(4)	¹⁸⁸ Os	5885.7(4)	0.041(7)	0.00065(11)
¹⁸⁷ Os	2131.44(14)	0.052(6)	0.00083(10)	¹⁸⁷ Os	5920.60(14)	0.044(6)	0.00070(10)
¹⁸⁷ Os	2193.17(24)	0.031(6)	0.00049(10)	¹⁸⁹ Os	5933.06(13)	0.096(8)	0.00153(13)
¹⁸⁷ Os	2214.6(3)	0.039(7)	0.00062(11)	¹⁸⁴ Os	6155.8(3)	0.044(6)	0.00070(10)
¹⁸⁷ Os	2261.21(14)	0.077(7)	0.00123(11)	¹⁸⁹ Os	6246.81(12)	0.026(3)	0.00041(5)
¹⁸⁷ Os	2286.54(14)	0.052(8)	0.00083(13)	¹⁸⁹ Os	6409.53(14)	0.026(3)	0.00041(5)
¹⁸⁷ Os	2306.04(21)	0.0215(18)	0.00034(3)	¹⁸⁴ Os	6587.21(25)	0.093(13)	0.00148(21)
¹⁸⁷ Os	2505.13(24)	0.040(5)	0.00064(8)	¹⁸⁹ Os	7234.19(11)	0.044(4)	0.00070(6)
¹⁸⁷ Os	2606.38(21)	0.023(5)	0.00037(8)	¹⁸⁹ Os	7792.14(11)	0.034(3)	0.00054(5)
¹⁸⁷ Os	2623.10(21)	0.023(5)	0.00037(8)	¹⁸⁷ Os	7834.30(8)	0.0247(23)	0.00039(4)
¹⁸⁷ Os	2817.11(25)	0.026(5)	0.00041(8)	¹⁸⁷ Os	7989.40(7)	0.0208(14)	0.000331(22)
¹⁸⁷ Os	3021.7(3)	0.026(3)	0.00041(5)	Iridium (Z=77), At.Wt.=192.217(3), σ_γ^Z=425(5)			
¹⁸⁷ Os	3069.9(3)	0.028(5)	0.00045(8)	¹⁹¹ Ir	23.9670(20)	0.170(14)	0.00268(22)
¹⁸⁷ Os	3110.00(18)	0.0273(19)	0.00043(3)				

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁹¹ Ir	26.2260(20)	0.132(9)	0.00208(14)	¹⁹¹ Ir	136.7910(10)	2.20(21)	0.035(3)
¹⁹³ Ir	39.2160(10)	0.17(11)	0.0027(17)	¹⁹¹ Ir	138.2480(20)	0.53(7)	0.0084(11)
¹⁹³ Ir	43.1190(10)	0.9(3)	0.014(5)	¹⁹³ Ir	138.6880(10)	0.8(5)	0.013(8)
¹⁹¹Ir	48.0570(10)	5.7(4)	0.090(6)	¹⁹¹ Ir	139.736(5)	0.27(4)	0.0043(6)
¹⁹¹ Ir	49.379(4)	0.122(10)	0.00192(16)	¹⁹¹ Ir	140.257(6)	0.32(5)	0.0050(8)
¹⁹¹ Ir	49.9560(20)	0.115(9)	0.00181(14)	¹⁹¹ Ir	140.814(6)	0.16(5)	0.0025(8)
¹⁹¹ Ir	50.782(8)	0.132(11)	0.00208(17)	¹⁹³ Ir	143.5940(10)	0.6(3)	0.009(5)
¹⁹¹ Ir	54.3210(20)	0.54(20)	0.009(3)	¹⁹¹ Ir	144.849(4)	0.57(9)	0.0090(14)
¹⁹³ Ir	54.4030(10)	0.12(8)	0.0019(13)	¹⁹¹Ir	144.903(5)	3.1(4)	0.049(6)
¹⁹¹Ir	58.8440(10)	5.3(3)	0.084(5)	¹⁹³ Ir	145.2220(10)	0.11(7)	0.0017(11)
¹⁹¹Ir	66.822(8)	1.31(13)	0.0207(20)	¹⁹¹ Ir	148.821(3)	1.08(12)	0.0170(19)
¹⁹¹ Ir	69.252(3)	0.25(7)	0.0039(11)	¹⁹¹ Ir	148.822(3)	1.08(12)	0.0170(19)
¹⁹³ Ir	69.4740(20)	0.19(14)	0.0030(22)	¹⁹³Ir	148.9340(10)	1.4(9)	0.022(14)
¹⁹¹ Ir	72.0240(20)	0.6(3)	0.009(5)	¹⁹¹ Ir	151.450(5)	0.26(5)	0.0041(8)
¹⁹¹ Ir	72.328(4)	0.28(9)	0.0044(14)	¹⁹¹Ir	151.5640(20)	2.89(20)	0.046(3)
¹⁹¹ Ir	77.369(3)	0.38(11)	0.0060(17)	¹⁹³ Ir	152.4080(10)	0.37(23)	0.006(4)
¹⁹¹Ir	77.9470(10)	4.8(4)	0.076(6)	¹⁹³ Ir	152.942(11)	0.55(13)	0.0087(20)
¹⁹³ Ir	82.3350(10)	0.5(3)	0.008(5)	¹⁹³ Ir	153.0550(10)	0.5(3)	0.008(5)
¹⁹¹ Ir	83.965(8)	0.18(9)	0.0028(14)	¹⁹¹ Ir	156.0870(20)	1.02(12)	0.0161(19)
¹⁹¹Ir	84.2740(20)	7.7(4)	0.121(6)	¹⁹¹Ir	156.654(3)	2.76(12)	0.0435(19)
¹⁹¹ Ir	90.857(3)	0.20(4)	0.0032(6)	¹⁹¹ Ir	158.180(4)	0.15(4)	0.0024(6)
¹⁹³ Ir	93.1630(10)	0.3(3)	0.005(5)	¹⁹³ Ir	160.8250(20)	0.34(11)	0.0054(17)
¹⁹¹ Ir	95.056(6)	0.24(5)	0.0038(8)	¹⁹³ Ir	160.9980(10)	0.4(3)	0.006(5)
¹⁹¹ Ir	95.470(4)	0.9(3)	0.014(5)	¹⁹³ Ir	162.7740(20)	0.24(15)	0.0038(24)
¹⁹³ Ir	95.5690(10)	0.8(5)	0.013(8)	¹⁹¹ Ir	162.850(6)	0.14(3)	0.0022(5)
¹⁹¹ Ir	97.347(3)	0.25(5)	0.0039(8)	¹⁹³ Ir	165.3800(20)	0.27(23)	0.004(4)
¹⁹¹ Ir	97.348(4)	0.36(14)	0.0057(22)	¹⁹³ Ir	165.4500(20)	0.35(22)	0.006(4)
¹⁹¹ Ir	98.524(4)	0.32(5)	0.0050(8)	¹⁹¹ Ir	166.089(5)	0.89(10)	0.0140(16)
¹⁹¹ Ir	99.603(6)	0.24(13)	0.0038(20)	¹⁹¹ Ir	166.435(4)	0.24(4)	0.0038(6)
¹⁹³ Ir	100.4030(20)	0.13(8)	0.0020(13)	¹⁹¹Ir	169.196(3)	3.05(13)	0.0481(20)
¹⁹¹ Ir	104.043(9)	0.13(4)	0.0020(6)	¹⁹¹ Ir	169.542(5)	0.52(7)	0.0082(11)
¹⁹¹ Ir	105.159(3)	0.14(6)	0.0022(10)	¹⁹¹ Ir	169.542(4)	0.52(7)	0.0082(11)
¹⁹¹ Ir	107.015(3)	0.20(7)	0.0032(11)	¹⁹³ Ir	169.5660(10)	0.24(15)	0.0038(24)
¹⁹¹ Ir	107.132(4)	0.23(6)	0.0036(10)	¹⁹³ Ir	169.8760(10)	0.15(9)	0.0024(14)
¹⁹¹Ir	108.0300(20)	2.62(12)	0.0413(19)	¹⁹¹ Ir	172.839(3)	0.53(24)	0.008(4)
¹⁹¹ Ir	108.658(4)	0.11(3)	0.0017(5)	¹⁹¹ Ir	174.139(8)	0.21(4)	0.0033(6)
¹⁹¹ Ir	110.352(3)	0.53(7)	0.0084(11)	¹⁹³ Ir	176.6510(20)	0.15(10)	0.0024(16)
¹⁹¹ Ir	111.025(3)	0.99(11)	0.0156(17)	¹⁹¹ Ir	176.812(3)	0.6(4)	0.009(6)
¹⁹³Ir	112.2310(10)	1.7(4)	0.027(6)	¹⁹¹ Ir	177.919(7)	0.28(6)	0.0044(10)
¹⁹³ Ir	115.4730(10)	0.5(3)	0.008(5)	¹⁹¹Ir	179.0380(20)	2.1(5)	0.033(8)
¹⁹³ Ir	117.8790(10)	0.4(3)	0.006(5)	¹⁹¹ Ir	183.626(3)	1.0(4)	0.016(6)
¹⁹¹ Ir	118.268(3)	0.15(3)	0.0024(5)	¹⁹³ Ir	184.6870(20)	0.92(22)	0.015(4)
¹⁹¹ Ir	118.7820(10)	0.56(7)	0.0088(11)	¹⁹¹ Ir	187.521(3)	0.43(5)	0.0068(8)
¹⁹¹ Ir	121.139(3)	0.17(7)	0.0027(11)	¹⁹¹ Ir	188.204(3)	0.52(23)	0.008(4)
¹⁹¹ Ir	122.596(3)	0.41(7)	0.0065(11)	¹⁹¹ Ir	189.100(7)	0.47(18)	0.007(3)
¹⁹³ Ir	123.8450(10)	1.0(6)	0.016(10)	¹⁹¹ Ir	193.718(3)	0.83(11)	0.0131(17)
¹⁹¹Ir	126.958(3)	1.86(10)	0.0293(16)	¹⁹³ Ir	193.9300(20)	0.21(13)	0.0033(20)
¹⁹³ Ir	132.8790(20)	0.18(10)	0.0028(16)	¹⁹¹ Ir	195.433(4)	0.27(7)	0.0043(11)
¹⁹¹ Ir	133.925(6)	0.19(5)	0.0030(8)	¹⁹³ Ir	195.5270(10)	0.21(13)	0.0033(20)
¹⁹³ Ir	136.1000(20)	0.17(11)	0.0027(17)	¹⁹¹ Ir	197.061(7)	0.73(19)	0.012(3)
¹⁹¹Ir	136.1250(10)	6.5(9)	0.102(14)	¹⁹³ Ir	198.8370(20)	0.15(9)	0.0024(14)
¹⁹¹ Ir	136.213(3)	4.0(5)	0.063(8)	¹⁹¹ Ir	199.174(7)	1.07(18)	0.017(3)
				¹⁹¹ Ir	199.418(5)	0.14(4)	0.0022(6)
				¹⁹¹ Ir	201.111(5)	0.21(6)	0.0033(10)
				¹⁹¹ Ir	203.015(3)	0.27(4)	0.0043(6)
				¹⁹¹Ir	206.220(4)	3.70(18)	0.058(3)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁹¹ Ir	207.301(5)	0.50(6)	0.0079(10)	¹⁹¹ Ir	299.476(8)	0.13(4)	0.0020(6)
¹⁹¹ Ir	208.440(6)	0.70(9)	0.0110(14)	¹⁹¹ Ir	302.905(8)	1.20(11)	0.0189(17)
¹⁹¹ Ir	210.352(5)	0.75(8)	0.0118(13)	¹⁹¹ Ir	305.448(4)	0.45(10)	0.0071(16)
¹⁹¹ Ir	210.354(5)	0.75(8)	0.0118(13)	¹⁹³ Ir	308.9740(10)	0.6(4)	0.009(6)
¹⁹¹ Ir	210.354(5)	2.1(4)	0.033(6)	¹⁹¹ Ir	310.010(6)	0.26(8)	0.0041(13)
¹⁹³ Ir	212.3460(20)	0.15(10)	0.0024(16)	¹⁹¹ Ir	310.08(10)	0.61(10)	0.0096(16)
¹⁹¹ Ir	215.117(5)	0.23(4)	0.0036(6)	¹⁹³ Ir	311.4960(10)	0.16(10)	0.0025(16)
¹⁹¹ Ir	215.5110(20)	0.24(4)	0.0038(6)	¹⁹¹ Ir	311.630(6)	0.23(6)	0.0036(10)
¹⁹¹ Ir	216.1940(20)	0.65(9)	0.0102(14)	¹⁹³ Ir	314.0520(10)	0.26(17)	0.004(3)
¹⁹¹ Ir	216.905(4)	5.57(24)	0.088(4)	¹⁹¹ Ir	316.061(7)	2.4(4)	0.038(6)
¹⁹¹ Ir	221.90(10)	0.83(16)	0.0131(25)	¹⁹¹ Ir	322.510(5)	0.51(11)	0.0080(17)
¹⁹¹ Ir	223.176(6)	0.18(3)	0.0028(5)	¹⁹³ Ir	328.448(14)d	9.1(3)	0.143[1.8%]
¹⁹³ Ir	224.0830(20)	0.18(11)	0.0028(17)	¹⁹¹ Ir	333.864(6)	1.53(10)	0.0241(16)
¹⁹³ Ir	225.4180(20)	0.12(7)	0.0019(11)	¹⁹³ Ir	337.5240(20)	0.62(21)	0.010(3)
¹⁹¹ Ir	226.2980(20)	4.0(4)	0.063(6)	¹⁹³ Ir	340.8130(20)	0.8(5)	0.013(8)
¹⁹³ Ir	226.6390(10)	0.20(12)	0.0032(19)	¹⁹¹ Ir	351.689(4)	10.9(4)	0.172(6)
¹⁹¹ Ir	226.722(5)	0.19(4)	0.0030(6)	¹⁹³ Ir	353.9610(10)	0.5(3)	0.008(5)
¹⁹³ Ir	228.0650(20)	0.12(8)	0.0019(13)	¹⁹¹ Ir	358.320(8)	0.34(9)	0.0054(14)
¹⁹¹ Ir	229.771(11)	0.48(11)	0.0076(17)	¹⁹¹ Ir	365.440(7)	1.15(10)	0.0181(16)
¹⁹¹ Ir	231.683(3)	0.95(13)	0.0150(20)	¹⁹³ Ir	371.5020(20)	2.11(12)	0.0333(19)
¹⁹¹ Ir	232.907(4)	0.20(4)	0.0032(6)	¹⁹¹ Ir	384.659(6)	0.50(12)	0.0079(19)
¹⁹³ Ir	234.8190(20)	0.44(13)	0.0069(20)	¹⁹³ Ir	390.9620(10)	0.6(4)	0.009(6)
¹⁹¹ Ir	241.867(7)	0.65(13)	0.0102(20)	¹⁹³ Ir	405.3660(20)	0.11(7)	0.0017(11)
¹⁹³ Ir	245.1090(20)	0.14(9)	0.0022(14)	¹⁹³ Ir	407.3150(20)	0.13(8)	0.0020(13)
¹⁹³ Ir	245.4920(20)	0.33(22)	0.005(4)	¹⁹³ Ir	411.988(10)	0.12(8)	0.0019(13)
¹⁹¹ Ir	246.169(3)	0.15(4)	0.0024(6)	¹⁹¹ Ir	418.138(6)	3.45(15)	0.0544(24)
¹⁹¹ Ir	246.800(4)	0.32(9)	0.0050(14)	¹⁹¹ Ir	432.716(6)	1.85(7)	0.0292(11)
¹⁹³ Ir	248.6000(20)	0.24(15)	0.0038(24)	¹⁹³ Ir	458.3070(20)	0.41(25)	0.006(4)
¹⁹³ Ir	252.2750(10)	0.11(7)	0.0017(11)	¹⁹³ Ir	460.2560(20)	0.8(5)	0.013(8)
¹⁹¹ Ir	252.499(12)	0.5(3)	0.008(5)	¹⁹³ Ir	4365.1(3)	0.22(3)	0.0035(5)
¹⁹¹ Ir	254.277(4)	1.08(11)	0.0170(17)	¹⁹³ Ir	4368.5(4)	0.14(3)	0.0022(5)
¹⁹³ Ir	255.3130(20)	0.36(13)	0.0057(20)	¹⁹³ Ir	4383.5(4)	0.11(3)	0.0017(5)
¹⁹¹ Ir	258.320(5)	0.24(5)	0.0038(8)	¹⁹³ Ir	4395.64(18)	0.39(3)	0.0061(5)
¹⁹¹ Ir	261.953(6)	2.02(23)	0.032(4)	¹⁹³ Ir	4401.28(18)	0.35(3)	0.0055(5)
¹⁹¹ Ir	262.03(10)	3.05(18)	0.048(3)	¹⁹³ Ir	4426.1(3)	0.23(3)	0.0036(5)
¹⁹³ Ir	262.7290(10)	0.14(8)	0.0022(13)	¹⁹³ Ir	4442.1(8)	0.14(3)	0.0022(5)
¹⁹¹ Ir	263.573(6)	0.86(10)	0.0136(16)	¹⁹³ Ir	4455.3(4)	0.13(3)	0.0020(5)
¹⁹¹ Ir	264.008(7)	0.57(7)	0.0090(11)	¹⁹³ Ir	4460.5(4)	0.18(3)	0.0028(5)
¹⁹³ Ir	264.7680(20)	0.8(5)	0.013(8)	¹⁹¹ Ir	4495.88(21)	0.44(4)	0.0069(6)
¹⁹¹ Ir	267.415(4)	0.93(21)	0.015(3)	¹⁹¹ Ir	4505.9(4)	0.20(3)	0.0032(5)
¹⁹³ Ir	271.6810(20)	0.6(4)	0.009(6)	¹⁹¹ Ir	4521.3(4)	0.12(4)	0.0019(6)
¹⁹¹ Ir	273.235(8)	0.49(8)	0.0077(13)	¹⁹¹ Ir	4531.28(19)	0.61(5)	0.0096(8)
¹⁹¹ Ir	273.236(7)	0.72(17)	0.011(3)	¹⁹¹ Ir	4556.8(8)	0.18(7)	0.0028(11)
¹⁹¹ Ir	273.568(5)	0.18(6)	0.0028(10)	¹⁹¹ Ir	4563.5(9)	0.14(11)	0.0022(17)
¹⁹¹ Ir	275.0380(20)	0.74(16)	0.0117(25)	¹⁹¹ Ir	4571.8(5)	0.23(4)	0.0036(6)
¹⁹³ Ir	275.2990(10)	0.6(4)	0.009(6)	¹⁹³ Ir	4577.9(4)	0.16(3)	0.0025(5)
¹⁹¹ Ir	276.787(4)	0.55(12)	0.0087(19)	¹⁹³ Ir	4584.4(3)	0.21(4)	0.0033(6)
¹⁹¹ Ir	278.193(8)	0.42(5)	0.0066(8)	¹⁹¹ Ir	4591.30(17)	0.57(4)	0.0090(6)
¹⁹³ Ir	278.5040(10)	1.8(11)	0.028(17)	¹⁹¹ Ir	4601.64(24)	0.22(4)	0.0035(6)
¹⁹¹ Ir	284.074(6)	1.95(15)	0.0307(24)	¹⁹¹ Ir	4611.6(6)	0.11(7)	0.0017(11)
¹⁹¹ Ir	284.947(3)	0.52(7)	0.0082(11)	¹⁹³ Ir	4612.5(3)	0.19(3)	0.0030(5)
¹⁹³ Ir	288.4310(20)	0.12(7)	0.0019(11)	¹⁹³ Ir	4618.0(4)	0.13(3)	0.0020(5)
¹⁹¹ Ir	292.374(4)	0.42(12)	0.0066(19)	¹⁹¹ Ir	4640.0(6)	0.15(6)	0.0024(10)
¹⁹³ Ir	293.541(14)d	1.76(6)	0.0277[1.8%]	¹⁹³ Ir	4643.2(3)	0.33(5)	0.0052(8)
¹⁹³ Ir	294.5300(20)	0.41(25)	0.006(4)	¹⁹¹ Ir	4646.47(13)	0.26(5)	0.0041(8)
¹⁹¹ Ir	296.257(8)	0.65(17)	0.010(3)	¹⁹¹ Ir	4663.36(21)	0.18(3)	0.0028(5)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁹¹ Ir	4668.09(17)	0.36(3)	0.0057(5)	¹⁹¹ Ir	5129.21(12)	0.90(5)	0.0142(8)
¹⁹³ Ir	4678.7(3)	0.18(3)	0.0028(5)	¹⁹¹ Ir	5138.06(14)	0.39(4)	0.0061(6)
¹⁹¹ Ir	4711.6(4)	0.17(3)	0.0027(5)	¹⁹¹Ir	5147.51(12)	1.29(6)	0.0203(10)
¹⁹³ Ir	4712.8(3)	0.28(3)	0.0044(5)	¹⁹¹ Ir	5153.1(3)	0.26(3)	0.0041(5)
¹⁹¹ Ir	4729.1(3)	0.167(25)	0.0026(4)	¹⁹³ Ir	5158.23(22)	0.36(3)	0.0057(5)
¹⁹¹ Ir	4734.2(3)	0.45(9)	0.0071(14)	¹⁹¹ Ir	5166.92(13)	0.96(6)	0.0151(10)
¹⁹³ Ir	4734.52(23)	0.46(3)	0.0073(5)	¹⁹³ Ir	5178.4(3)	0.34(4)	0.0054(6)
¹⁹¹ Ir	4750.18(15)	0.38(3)	0.0060(5)	¹⁹¹ Ir	5184.38(25)	0.20(6)	0.0032(10)
¹⁹¹ Ir	4755.28(20)	0.39(3)	0.0061(5)	¹⁹³ Ir	5185.2(4)	0.34(4)	0.0054(6)
¹⁹¹ Ir	4765.66(17)	0.245(24)	0.0039(4)	¹⁹¹ Ir	5194.52(24)	0.34(5)	0.0054(8)
¹⁹¹ Ir	4779.82(15)	0.32(3)	0.0050(5)	¹⁹¹ Ir	5198.64(21)	0.38(4)	0.0060(6)
¹⁹¹ Ir	4801.4(3)	0.12(3)	0.0019(5)	¹⁹¹ Ir	5219.92(17)	0.72(5)	0.0114(8)
¹⁹¹ Ir	4809.72(23)	0.44(4)	0.0069(6)	¹⁹¹ Ir	5248.02(23)	0.20(3)	0.0032(5)
¹⁹¹ Ir	4817.3(3)	0.28(4)	0.0044(6)	¹⁹¹ Ir	5261.14(17)	0.51(4)	0.0080(6)
¹⁹¹ Ir	4826.1(4)	0.11(3)	0.0017(5)	¹⁹¹ Ir	5283.60(13)	0.85(6)	0.0134(10)
¹⁹³ Ir	4826.9(4)	0.20(4)	0.0032(6)	¹⁹¹ Ir	5304.44(13)	0.73(5)	0.0115(8)
¹⁹¹ Ir	4838.3(4)	0.15(4)	0.0024(6)	¹⁹¹ Ir	5313.6(3)	0.15(4)	0.0024(6)
¹⁹³ Ir	4839.34(20)	0.41(4)	0.0065(6)	¹⁹³ Ir	5316.6(3)	0.20(4)	0.0032(6)
¹⁹¹ Ir	4849.6(3)	0.15(3)	0.0024(5)	¹⁹¹ Ir	5327.53(19)	0.71(5)	0.0112(8)
¹⁹¹ Ir	4854.8(5)	0.28(5)	0.0044(8)	¹⁹¹ Ir	5332.49(20)	0.54(5)	0.0085(8)
¹⁹³ Ir	4855.5(3)	0.48(4)	0.0076(6)	¹⁹¹ Ir	5347.1(3)	0.18(3)	0.0028(5)
¹⁹¹ Ir	4859.30(23)	0.45(4)	0.0071(6)	¹⁹¹ Ir	5357.09(16)	1.03(6)	0.0162(10)
¹⁹¹ Ir	4866.97(12)	0.68(4)	0.0107(6)	¹⁹¹ Ir	5376.11(14)	0.288(24)	0.0045(4)
¹⁹¹ Ir	4875.03(18)	0.33(4)	0.0052(6)	¹⁹¹ Ir	5384.82(20)	0.224(22)	0.0035(4)
¹⁹¹ Ir	4893.82(23)	0.35(3)	0.0055(5)	¹⁹¹ Ir	5400.78(16)	0.40(3)	0.0063(5)
¹⁹¹ Ir	4898.53(19)	0.41(4)	0.0065(6)	¹⁹¹ Ir	5420.57(23)	0.201(22)	0.0032(4)
¹⁹¹ Ir	4916.5(3)	0.29(5)	0.0046(8)	¹⁹¹ Ir	5431.34(12)	0.78(4)	0.0123(6)
¹⁹³ Ir	4921.1(4)	0.18(4)	0.0028(6)	¹⁹¹ Ir	5448.60(17)	0.51(4)	0.0080(6)
¹⁹¹ Ir	4932.9(3)	0.11(4)	0.0017(6)	¹⁹¹ Ir	5458.91(18)	0.60(5)	0.0095(8)
¹⁹¹ Ir	4938.9(3)	0.25(9)	0.0039(14)	¹⁹¹ Ir	5463.9(4)	0.31(7)	0.0049(11)
¹⁹¹ Ir	4942.92(18)	0.52(4)	0.0082(6)	¹⁹³ Ir	5467.0(3)	0.59(7)	0.0093(11)
¹⁹¹ Ir	4949.40(24)	0.31(4)	0.0049(6)	¹⁹¹ Ir	5483.9(4)	0.17(6)	0.0027(10)
¹⁹¹ Ir	4955.2(3)	0.15(7)	0.0024(11)	¹⁹³ Ir	5487.40(21)	0.58(4)	0.0091(6)
¹⁹¹ Ir	4966.5(3)	0.20(3)	0.0032(5)	¹⁹¹ Ir	5490.1(5)	0.19(3)	0.0030(5)
¹⁹¹ Ir	4972.12(17)	0.35(3)	0.0055(5)	¹⁹¹ Ir	5495.27(23)	0.22(3)	0.0035(5)
¹⁹¹ Ir	4980.57(15)	0.82(4)	0.0129(6)	¹⁹¹ Ir	5517.04(17)	0.76(4)	0.0120(6)
¹⁹¹ Ir	4985.93(14)	0.58(3)	0.0091(5)	¹⁹¹Ir	5534.73(12)	1.39(6)	0.0219(10)
¹⁹¹ Ir	4993.32(15)	0.40(4)	0.0063(6)	¹⁹¹ Ir	5552.18(21)	0.163(22)	0.0026(4)
¹⁹¹ Ir	5003.4(3)	0.35(4)	0.0055(6)	¹⁹¹Ir	5564.54(14)	1.71(8)	0.0270(13)
¹⁹³ Ir	5013.8(5)	0.21(4)	0.0033(6)	¹⁹¹ Ir	5569.4(3)	0.67(4)	0.0106(6)
¹⁹¹ Ir	5020.51(15)	0.66(6)	0.0104(10)	¹⁹³ Ir	5576.98(7)	0.121(24)	0.0019(4)
¹⁹¹ Ir	5028.52(15)	0.67(6)	0.0106(10)	¹⁹¹ Ir	5595.63(13)	0.72(4)	0.0114(6)
¹⁹¹ Ir	5037.5(3)	0.22(4)	0.0035(6)	¹⁹¹ Ir	5612.55(12)	1.06(5)	0.0167(8)
¹⁹¹ Ir	5042.35(23)	0.57(6)	0.0090(10)	¹⁹³ Ir	5630.33(7)	0.315(24)	0.0050(4)
¹⁹¹ Ir	5046.4(6)	0.12(3)	0.0019(5)	¹⁹³ Ir	5642.90(7)	0.293(25)	0.0046(4)
¹⁹¹ Ir	5053.15(23)	0.26(3)	0.0041(5)	¹⁹¹ Ir	5654.27(14)	0.39(3)	0.0061(5)
¹⁹³ Ir	5058.0(3)	0.20(3)	0.0032(5)	¹⁹¹ Ir	5661.00(20)	0.38(3)	0.0060(5)
¹⁹¹ Ir	5066.5(3)	0.15(3)	0.0024(5)	¹⁹¹Ir	5667.81(3)	2.68(10)	0.0423(16)
¹⁹³ Ir	5071.99(21)	0.28(3)	0.0044(5)	¹⁹¹ Ir	5681.1(3)	0.165(19)	0.0026(3)
¹⁹¹ Ir	5085.45(20)	0.266(25)	0.0042(4)	¹⁹¹Ir	5689.06(3)	1.73(7)	0.0273(11)
¹⁹¹ Ir	5091.10(18)	0.37(5)	0.0058(8)	¹⁹¹ Ir	5708.62(3)	0.122(17)	0.0019(3)
¹⁹³ Ir	5091.19(17)	0.52(3)	0.0082(5)	¹⁹¹ Ir	5727.2(3)	0.27(4)	0.0043(6)
¹⁹¹ Ir	5104.6(4)	0.14(3)	0.0022(5)	¹⁹³Ir	5728.97(7)	1.15(5)	0.0181(8)
¹⁹³ Ir	5109.0(3)	0.19(3)	0.0030(5)	¹⁹¹ Ir	5746.80(3)	0.190(18)	0.0030(3)
¹⁹¹ Ir	5109.6(6)	0.11(7)	0.0017(11)	¹⁹¹ Ir	5757.18(3)	0.49(6)	0.0077(10)
¹⁹³ Ir	5117.9(4)	0.12(3)	0.0019(5)	¹⁹³ Ir	5757.65(7)	0.42(4)	0.0066(6)
¹⁹¹ Ir	5123.3(3)	0.20(3)	0.0032(5)	-----	-----	-----	-----

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁹¹ Ir	5783.01(3)	1.34(6)	0.0211(10)	¹⁹⁵ Pt	2311.44(3)	0.134(4)	0.00208(6)
¹⁹³ Ir	5788.12(7)	0.43(4)	0.0068(6)	¹⁹⁵ Pt	2527.81(3)	0.07(3)	0.0011(5)
¹⁹¹ Ir	5808.33(3)	0.48(3)	0.0076(5)	¹⁹⁵ Pt	4949.0(4)	0.069(20)	0.0011(3)
¹⁹¹ Ir	5817.7(4)	0.113(25)	0.0018(4)	¹⁹⁶ Pt	5098.1(7)	0.093(6)	0.00144(9)
¹⁹³ Ir	5821.51(7)	0.48(3)	0.0076(5)	¹⁹⁵ Pt	5098.5(7)	0.10(3)	0.0016(5)
¹⁹¹ Ir	5829.70(3)	0.16(5)	0.0025(8)	¹⁹⁵ Pt	5173.4(3)	0.136(6)	0.00211(9)
¹⁹¹ Ir	5866.29(3)	0.73(6)	0.0115(10)	¹⁹⁵ Pt	5185.3(3)	0.085(5)	0.00132(8)
¹⁹¹ Ir	5866.97(3)	0.79(5)	0.0125(8)	¹⁹⁵ Pt	5254.70(8)	0.41(3)	0.0064(5)
¹⁹¹ Ir	5905.67(3)	0.45(4)	0.0071(6)	¹⁹⁵ Pt	5261.0(6)	0.097(14)	0.00151(22)
¹⁹¹ Ir	5909.64(3)	0.23(3)	0.0036(5)	¹⁹⁵ Pt	5306.9(3)	0.118(14)	0.00183(22)
¹⁹³ Ir	5917.68(7)	0.34(3)	0.0054(5)	¹⁹⁵ Pt	5393.05(16)	0.113(10)	0.00176(16)
¹⁹³ Ir	5927.93(7)	0.33(3)	0.0052(5)	¹⁹⁵ Pt	5451.93(14)	0.078(7)	0.00121(11)
¹⁹³ Ir	5954.39(7)	0.74(4)	0.0117(6)	¹⁹⁵ Pt	5612.62(11)	0.14(3)	0.0022(5)
¹⁹¹ Ir	5958.28(3)	1.79(8)	0.0282(13)	¹⁹⁵ Pt	5722.40(9)	0.071(5)	0.00110(8)
¹⁹¹ Ir	5962.29(3)	0.75(4)	0.0118(6)	¹⁹⁵ Pt	5759.22(10)	0.084(12)	0.00130(19)
¹⁹¹ Ir	5972.13(3)	0.254(21)	0.0040(3)	¹⁹⁵ Pt	5952.95(7)	0.086(16)	0.00134(25)
¹⁹³ Ir	5984.28(7)	0.212(21)	0.0033(3)	¹⁹⁵ Pt	6003.37(8)	0.073(4)	0.00113(6)
¹⁹¹ Ir	6004.53(3)	0.257(21)	0.0041(3)	¹⁹⁵ Pt	6033.69(7)	0.109(6)	0.00169(9)
¹⁹³ Ir	6023.50(7)	0.171(17)	0.0027(3)	Gold (Z=79, At.Wt.=196.96655(2), $\sigma_{\gamma}^z = 98.65(9)$)			
¹⁹¹ Ir	6079.26(3)	0.29(9)	0.0046(14)	¹⁹⁷ Au	35.8240(10)	0.41(5)	0.0063(8)
¹⁹¹ Ir	6082.48(3)	2.62(11)	0.0413(17)	¹⁹⁷ Au	55.1810(10)	2.90(12)	0.0446(18)
¹⁹¹ Ir	6093.26(3)	0.56(4)	0.0088(6)	¹⁹⁷ Au	66.3950(10)	0.42(12)	0.0065(18)
Platinum (Z=78, At.Wt.=195.078(2), $\sigma_{\gamma}^z = 10.3(4)$)				¹⁹⁷ Au	75.171(6)	0.390(23)	0.0060(4)
¹⁹⁴ Pt	211.4060(20)	0.0293(10)	0.000455(16)	¹⁹⁷ Au	82.3560(10)	2.3(4)	0.035(6)
¹⁹⁵ Pt	326.353(3)	0.511(10)	0.00794(16)	¹⁹⁷ Au	82.5240(10)	1.4(3)	0.022(5)
¹⁹⁵ Pt	332.985(4)	2.580(25)	0.0401(4)	¹⁹⁷ Au	83.144(6)	0.17(7)	0.0026(11)
¹⁹⁵ Pt	355.6840(20)	6.17(6)	0.0958(9)	¹⁹⁷ Au	91.0050(10)	0.294(15)	0.00452(23)
¹⁹⁵ Pt	393.346(5)	0.066(4)	0.00103(6)	¹⁹⁷ Au	97.2500(20)	2.1(5)	0.032(8)
¹⁹⁵ Pt	446.624(4)	0.0963(21)	0.00150(3)	¹⁹⁷ Au	101.9390(10)	0.953(17)	0.0147(3)
¹⁹⁵ Pt	521.161(5)	0.338(10)	0.00525(16)	¹⁹⁷ Au	103.5610(10)	0.338(15)	0.00520(23)
¹⁹⁸ Pt	542.98(4)d	0.0390(3)	0.000606[45%]	¹⁹⁷ Au	108.9120(20)	0.270(14)	0.00415(22)
¹⁹⁵ Pt	672.894(3)	0.179(4)	0.00278(6)	¹⁹⁷ Au	122.6520(10)	0.81(13)	0.0125(20)
¹⁹⁵ Pt	779.608(5)	0.227(3)	0.00353(5)	¹⁹⁷ Au	123.7860(10)	0.83(13)	0.0128(20)
¹⁹⁵ Pt	1005.878(5)	0.139(3)	0.00216(5)	¹⁹⁷ Au	131.9340(20)	0.17(6)	0.0026(9)
¹⁹⁵ Pt	1047.007(11)	0.181(4)	0.00281(6)	¹⁹⁷ Au	132.850(4)	0.104(24)	0.0016(4)
¹⁹⁵ Pt	1091.334(6)	0.181(4)	0.00281(6)	¹⁹⁷ Au	135.612(6)	0.10(3)	0.0015(5)
¹⁹⁵ Pt	1248.774(10)	0.099(3)	0.00154(5)	¹⁹⁷ Au	137.448(6)	0.13(5)	0.0020(8)
¹⁹⁵ Pt	1305.57(3)	0.062(3)	0.00096(5)	¹⁹⁷ Au	137.7630(10)	0.347(24)	0.0053(4)
¹⁹⁵ Pt	1321.541(15)	0.081(3)	0.00126(5)	¹⁹⁷ Au	137.999(5)	0.17(5)	0.0026(8)
¹⁹⁵ Pt	1358.31(6)	0.076(4)	0.00118(6)	¹⁹⁷ Au	142.9270(20)	0.161(16)	0.00248(25)
¹⁹⁵ Pt	1439.35(5)	0.067(3)	0.00104(5)	¹⁹⁷ Au	144.6050(10)	0.18(4)	0.0028(6)
¹⁹⁵ Pt	1491.625(16)	0.135(4)	0.00210(6)	¹⁹⁷ Au	145.1540(10)	0.46(13)	0.0071(20)
¹⁹⁵ Pt	1497.950(11)	0.084(3)	0.00130(5)	¹⁹⁷ Au	146.3460(20)	0.43(4)	0.0066(6)
¹⁹⁵ Pt	1510.75(5)	0.083(3)	0.00129(5)	¹⁹⁷ Au	146.6700(10)	0.28(5)	0.0043(8)
¹⁹⁵ Pt	1531.84(3)	0.122(4)	0.00190(6)	¹⁹⁷ Au	154.7940(20)	0.38(6)	0.0058(9)
¹⁹⁵ Pt	1532.435(12)	0.066(18)	0.0010(3)	¹⁹⁷ Au	154.797(5)	0.239(10)	0.00368(15)
¹⁹⁵ Pt	1562.76(4)	0.083(3)	0.00129(5)	¹⁹⁷ Au	158.4360(10)	1.250(18)	0.0192(3)
¹⁹⁵ Pt	1677.223(15)	0.087(4)	0.00135(6)	¹⁹⁷ Au	158.479(11)	0.67(9)	0.0103(14)
¹⁹⁵ Pt	1713.67(10)	0.090(4)	0.00140(6)	¹⁹⁷ Au	164.7130(10)	0.21(3)	0.0032(5)
¹⁹⁵ Pt	1737.278(16)	0.087(4)	0.00135(6)	¹⁹⁷ Au	166.2280(10)	0.279(11)	0.00429(17)
¹⁹⁵ Pt	1802.269(10)	0.146(4)	0.00227(6)	¹⁹⁷ Au	168.3340(10)	3.60(22)	0.055(3)
¹⁹⁵ Pt	1825.685(8)	0.091(4)	0.00141(6)	¹⁹⁷ Au	169.9550(10)	0.126(25)	0.0019(4)
¹⁹⁵ Pt	1888.116(12)	0.080(4)	0.00124(6)	¹⁹⁷ Au	170.1030(10)	1.66(22)	0.026(3)
¹⁹⁵ Pt	1968.858(13)	0.103(4)	0.00160(6)	¹⁹⁷ Au	170.3990(20)	0.38(5)	0.0058(8)
¹⁹⁵ Pt	1978.46(3)	0.163(5)	0.00253(8)	¹⁹⁷ Au	175.3070(20)	0.10(8)	0.0015(12)
¹⁹⁵ Pt	2309.20(9)	0.066(14)	0.00103(22)	¹⁹⁷ Au	180.8640(10)	0.63(11)	0.0097(17)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁹⁷ Au	188.1670(20)	0.63(15)	0.0097(23)	¹⁹⁷ Au	411.802d	94.29(15)	1.453(23)
¹⁹⁷ Au	191.1870(20)	0.18(3)	0.0028(5)	¹⁹⁷ Au	418.8400(20)	0.70(9)	0.0108(14)
197Au	192.3920(10)	3.9(18)	0.06(3)	197Au	440.3290(20)	0.9(4)	0.014(6)
197Au	192.9440(10)	1.70(22)	0.026(3)	¹⁹⁷ Au	441.070(5)	0.7(5)	0.011(8)
¹⁹⁷ Au	202.9920(20)	0.229(6)	0.00352(9)	¹⁹⁷ Au	444.3910(20)	0.56(7)	0.0086(11)
¹⁹⁷ Au	204.1580(10)	0.513(10)	0.00789(15)	¹⁹⁷ Au	447.527(3)	0.10(4)	0.0015(6)
¹⁹⁷ Au	204.1620(10)	0.59(10)	0.0091(15)	¹⁹⁷ Au	448.562(7)	0.118(15)	0.00182(23)
¹⁹⁷ Au	206.2230(10)	0.199(6)	0.00306(9)	¹⁹⁷ Au	449.5700(20)	0.50(6)	0.0077(9)
¹⁹⁷ Au	213.0650(10)	0.094(13)	0.00145(20)	¹⁹⁷ Au	456.1570(20)	0.141(22)	0.0022(3)
¹⁹⁷ Au	214.858(3)	0.19(5)	0.0029(8)	¹⁹⁷ Au	456.287(4)	0.47(6)	0.0072(9)
197Au	214.9710(10)	9.0(12)	0.138(18)	¹⁹⁷ Au	458.0540(20)	0.29(4)	0.0045(6)
¹⁹⁷ Au	215.2950(20)	0.19(3)	0.0029(5)	¹⁹⁷ Au	458.370(4)	0.16(3)	0.0025(5)
¹⁹⁷ Au	218.8300(10)	0.141(22)	0.0022(3)	¹⁹⁷ Au	464.7620(20)	0.17(6)	0.0026(9)
¹⁹⁷ Au	219.4190(20)	0.42(4)	0.0065(6)	¹⁹⁷ Au	485.638(5)	0.16(3)	0.0025(5)
¹⁹⁷ Au	234.6000(20)	0.091(12)	0.00140(18)	¹⁹⁷ Au	502.407(8)	0.16(4)	0.0025(6)
197Au	236.0450(10)	4.1(5)	0.063(8)	¹⁹⁷ Au	509.175(4)	0.37(9)	0.0057(14)
¹⁹⁷ Au	236.1710(20)	0.26(6)	0.0040(9)	¹⁹⁷ Au	510.427(6)	0.19(7)	0.0029(11)
¹⁹⁷ Au	245.314(6)	0.111(18)	0.0017(3)	¹⁹⁷ Au	511.067(6)	0.111(22)	0.0017(3)
197Au	247.5730(10)	5.56(8)	0.0855(12)	¹⁹⁷ Au	511.5170(20)	0.68(11)	0.0105(17)
¹⁹⁷ Au	248.739(3)	0.111(16)	0.00171(25)	¹⁹⁷ Au	512.5790(20)	0.16(6)	0.0025(9)
¹⁹⁷ Au	260.8820(10)	0.83(13)	0.0128(20)	¹⁹⁷ Au	515.132(6)	0.104(14)	0.00160(22)
197Au	261.4040(10)	5.3(20)	0.08(3)	¹⁹⁷ Au	516.0620(10)	0.35(5)	0.0054(8)
¹⁹⁷ Au	266.6470(10)	0.26(3)	0.0040(5)	¹⁹⁷ Au	520.746(6)	0.19(8)	0.0029(12)
¹⁹⁷ Au	269.0730(20)	0.155(24)	0.0024(4)	¹⁹⁷ Au	522.351(4)	0.096(12)	0.00148(18)
¹⁹⁷ Au	271.1380(20)	0.104(16)	0.00160(25)	¹⁹⁷ Au	524.752(3)	0.27(8)	0.0042(12)
¹⁹⁷ Au	271.2280(20)	0.170(24)	0.0026(4)	¹⁹⁷ Au	525.1340(20)	0.35(4)	0.0054(6)
¹⁹⁷ Au	271.8940(10)	0.40(13)	0.0062(20)	197Au	529.1650(20)	1.9(10)	0.029(15)
¹⁹⁷ Au	276.072(3)	0.226(5)	0.00348(8)	¹⁹⁷ Au	529.954(4)	0.39(5)	0.0060(8)
¹⁹⁷ Au	277.2460(20)	0.277(6)	0.00426(9)	¹⁹⁷ Au	540.3010(20)	0.49(23)	0.008(4)
¹⁹⁷ Au	284.1090(20)	0.16(3)	0.0025(5)	¹⁹⁷ Au	542.3670(20)	0.104(14)	0.00160(22)
197Au	291.7240(20)	1.05(17)	0.016(3)	¹⁹⁷ Au	544.008(5)	0.52(5)	0.0080(8)
¹⁹⁷ Au	293.1210(20)	0.101(16)	0.00155(25)	¹⁹⁷ Au	548.9350(20)	0.67(9)	0.0103(14)
¹⁹⁷ Au	307.7180(10)	0.44(6)	0.0068(9)	¹⁹⁷ Au	552.467(3)	0.104(14)	0.00160(22)
¹⁹⁷ Au	311.9040(20)	0.47(6)	0.0072(9)	¹⁹⁷ Au	555.6890(20)	0.126(17)	0.0019(3)
¹⁹⁷ Au	314.913(3)	0.27(4)	0.0042(6)	¹⁹⁷ Au	565.784(5)	0.38(5)	0.0058(8)
¹⁹⁷ Au	324.900(5)	0.104(14)	0.00160(22)	¹⁹⁷ Au	565.810(3)	0.43(6)	0.0066(9)
197Au	328.4840(20)	1.48(19)	0.023(3)	¹⁹⁷ Au	571.683(3)	0.50(7)	0.0077(11)
¹⁹⁷ Au	328.740(10)	0.111(14)	0.00171(22)	¹⁹⁷ Au	573.388(13)	0.126(17)	0.0019(3)
¹⁹⁷ Au	333.8380(20)	0.111(14)	0.00171(22)	¹⁹⁷ Au	573.746(6)	0.096(14)	0.00148(22)
¹⁹⁷ Au	337.5330(10)	0.178(23)	0.0027(4)	¹⁹⁷ Au	573.960(4)	0.33(4)	0.0051(6)
¹⁹⁷ Au	339.2910(20)	0.090(25)	0.0014(4)	¹⁹⁷ Au	574.370(5)	0.148(20)	0.0023(3)
¹⁹⁷ Au	346.9050(20)	0.44(11)	0.0068(17)	¹⁹⁷ Au	574.381(3)	0.36(5)	0.0055(8)
¹⁹⁷ Au	347.8800(20)	0.111(14)	0.00171(22)	¹⁹⁷ Au	574.733(10)	0.104(14)	0.00160(22)
197Au	350.8280(10)	1.0(5)	0.015(8)	¹⁹⁷ Au	577.3020(20)	0.27(3)	0.0042(5)
¹⁹⁷ Au	355.5300(20)	0.31(4)	0.0048(6)	¹⁹⁷ Au	579.297(3)	0.53(8)	0.0082(12)
¹⁹⁷ Au	364.0240(20)	0.11(3)	0.0017(5)	¹⁹⁷ Au	584.800(10)	0.121(15)	0.00186(23)
¹⁹⁷ Au	364.030(6)	0.104(14)	0.00160(22)	¹⁹⁷ Au	593.184(8)	0.148(21)	0.0023(3)
¹⁹⁷ Au	368.2510(20)	0.133(21)	0.0020(3)	¹⁹⁷ Au	609.432(4)	0.111(9)	0.00171(14)
¹⁹⁷ Au	371.0790(20)	0.44(6)	0.0068(9)	¹⁹⁷ Au	612.7240(20)	0.104(14)	0.00160(22)
¹⁹⁷ Au	373.1450(20)	0.130(19)	0.0020(3)	¹⁹⁷ Au	612.799(6)	0.096(22)	0.0015(3)
¹⁹⁷ Au	378.2990(20)	0.178(23)	0.0027(4)	¹⁹⁷ Au	625.4280(20)	0.44(4)	0.0068(6)
197Au	381.1990(10)	3.0(4)	0.046(6)	¹⁹⁷ Au	631.660(9)	0.144(19)	0.0022(3)
¹⁹⁷ Au	383.284(4)	0.24(3)	0.0037(5)	¹⁹⁷ Au	632.275(3)	0.170(23)	0.0026(4)
¹⁹⁷ Au	393.884(5)	0.22(3)	0.0034(5)	¹⁹⁷ Au	635.166(3)	0.24(3)	0.0037(5)
¹⁹⁷ Au	396.104(4)	0.100(8)	0.00154(12)	¹⁹⁷ Au	640.669(3)	0.59(5)	0.0091(8)
¹⁹⁷ Au	398.295(6)	0.096(13)	0.00148(20)	¹⁹⁷ Au	647.293(5)	0.126(17)	0.0019(3)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0
¹⁹⁷ Au	655.528(4)	0.21(3)	0.0032(5)	¹⁹⁷ Au	913.776(4)	0.30(6)	0.0046(9)
¹⁹⁷ Au	655.569(3)	0.24(5)	0.0037(8)	¹⁹⁷ Au	916.435(6)	0.25(4)	0.0038(6)
¹⁹⁷ Au	659.2490(20)	0.25(6)	0.0038(9)	¹⁹⁷ Au	927.421(4)	0.31(12)	0.0048(18)
¹⁹⁷ Au	661.451(10)	0.093(19)	0.0014(3)	¹⁹⁷ Au	928.995(6)	0.126(22)	0.0019(3)
¹⁹⁷ Au	668.561(7)	0.163(22)	0.0025(3)	¹⁹⁷ Au	933.928(6)	0.47(14)	0.0072(22)
¹⁹⁷ Au	672.6550(10)	0.55(7)	0.0085(11)	¹⁹⁷ Au	946.453(5)	0.096(13)	0.00148(20)
¹⁹⁷ Au	673.503(8)	0.126(18)	0.0019(3)	¹⁹⁷ Au	947.971(6)	0.32(4)	0.0049(6)
¹⁹⁷ Au	678.208(10)	0.41(12)	0.0063(18)	¹⁹⁷ Au	952.503(7)	0.19(3)	0.0029(5)
¹⁹⁷ Au	680.391(6)	0.10(3)	0.0015(5)	¹⁹⁷ Au	971.8180(20)	0.13(4)	0.0020(6)
¹⁹⁷ Au	682.804(5)	0.111(15)	0.00171(23)	¹⁹⁷ Au	978.936(8)	0.141(20)	0.0022(3)
¹⁹⁷ Au	686.865(5)	0.218(18)	0.0034(3)	¹⁹⁷ Au	983.082(7)	0.096(14)	0.00148(22)
¹⁹⁷ Au	688.968(10)	0.155(24)	0.0024(4)	¹⁹⁷ Au	985.002(6)	0.104(25)	0.0016(4)
¹⁹⁷ Au	690.046(6)	0.388(20)	0.0060(3)	¹⁹⁷ Au	993.654(6)	0.21(5)	0.0032(8)
¹⁹⁷ Au	692.972(6)	0.094(18)	0.0014(3)	¹⁹⁷ Au	999.682(4)	0.23(3)	0.0035(5)
¹⁹⁷ Au	698.287(4)	0.15(5)	0.0023(8)	¹⁹⁷ Au	1000.447(4)	0.104(22)	0.0016(3)
¹⁹⁷ Au	702.474(5)	0.51(7)	0.0078(11)	¹⁹⁷ Au	1005.487(6)	0.133(24)	0.0020(4)
¹⁹⁷ Au	724.623(6)	0.115(18)	0.0018(3)	¹⁹⁷ Au	1006.100(3)	0.096(15)	0.00148(23)
¹⁹⁷ Au	728.239(6)	0.161(19)	0.0025(3)	¹⁹⁷ Au	1018.136(6)	0.11(3)	0.0017(5)
¹⁹⁷ Au	728.997(6)	0.111(20)	0.0017(3)	¹⁹⁷ Au	1018.426(4)	0.18(3)	0.0028(5)
¹⁹⁷ Au	732.221(10)	0.104(14)	0.00160(22)	¹⁹⁷ Au	1028.199(5)	0.10(3)	0.0015(5)
¹⁹⁷ Au	740.0000(20)	0.310(21)	0.0048(3)	¹⁹⁷ Au	1028.564(6)	0.46(7)	0.0071(11)
¹⁹⁷ Au	744.8580(20)	0.104(15)	0.00160(23)	¹⁹⁷ Au	1038.274(3)	0.184(14)	0.00283(22)
¹⁹⁷ Au	745.220(4)	0.33(6)	0.0051(9)	¹⁹⁷ Au	1046.323(7)	0.111(16)	0.00171(25)
¹⁹⁷ Au	746.073(5)	0.133(18)	0.0020(3)	¹⁹⁷ Au	1047.121(6)	0.155(20)	0.0024(3)
¹⁹⁷ Au	764.011(3)	0.3(3)	0.005(5)	¹⁹⁷ Au	1047.847(5)	0.096(14)	0.00148(22)
¹⁹⁷ Au	765.131(6)	0.163(22)	0.0025(3)	¹⁹⁷ Au	1049.231(6)	0.104(17)	0.0016(3)
¹⁹⁷ Au	767.886(5)	0.096(14)	0.00148(22)	¹⁹⁷ Au	1050.701(5)	0.28(5)	0.0043(8)
¹⁹⁷ Au	767.960(6)	0.096(14)	0.00148(22)	¹⁹⁷ Au	1054.055(5)	0.16(3)	0.0025(5)
¹⁹⁷ Au	770.858(5)	0.206(17)	0.0032(3)	¹⁹⁷ Au	1060.888(7)	0.19(3)	0.0029(5)
¹⁹⁷ Au	776.632(6)	0.118(19)	0.0018(3)	¹⁹⁷ Au	1064.436(8)	0.096(13)	0.00148(20)
¹⁹⁷ Au	783.230(5)	0.111(23)	0.0017(4)	¹⁹⁷ Au	1064.998(7)	0.15(4)	0.0023(6)
¹⁹⁷ Au	786.793(10)	0.261(15)	0.00402(23)	¹⁹⁷ Au	1076.761(5)	0.111(21)	0.0017(3)
¹⁹⁷ Au	788.131(13)	0.104(19)	0.0016(3)	¹⁹⁷ Au	1079.197(5)	0.24(4)	0.0037(6)
¹⁹⁷ Au	794.158(7)	0.178(24)	0.0027(4)	¹⁹⁷ Au	1081.54(4)	0.096(25)	0.0015(4)
¹⁹⁷ Au	796.217(5)	0.148(22)	0.0023(3)	¹⁹⁷ Au	1085.605(5)	0.19(3)	0.0029(5)
¹⁹⁷ Au	801.7050(20)	0.19(4)	0.0029(6)	¹⁹⁷ Au	1101.942(4)	0.170(23)	0.0026(4)
¹⁹⁷ Au	806.248(8)	0.13(3)	0.0020(5)	¹⁹⁷ Au	1106.951(5)	0.19(4)	0.0029(6)
¹⁹⁷ Au	810.100(7)	0.26(3)	0.0040(5)	¹⁹⁷ Au	1107.562(9)	0.52(10)	0.0080(15)
¹⁹⁷ Au	815.954(7)	0.104(20)	0.0016(3)	¹⁹⁷ Au	1109.196(4)	0.49(10)	0.0075(15)
¹⁹⁷ Au	822.572(5)	0.104(17)	0.0016(3)	¹⁹⁷ Au	1111.461(7)	0.37(6)	0.0057(9)
¹⁹⁷ Au	825.483(4)	0.31(5)	0.0048(8)	¹⁹⁷ Au	1114.585(6)	0.178(24)	0.0027(4)
¹⁹⁷ Au	831.470(5)	0.153(19)	0.0024(3)	¹⁹⁷ Au	1128.417(6)	0.141(19)	0.0022(3)
¹⁹⁷ Au	833.906(6)	0.104(16)	0.00160(25)	¹⁹⁷ Au	1132.895(8)	0.25(5)	0.0038(8)
¹⁹⁷ Au	836.432(3)	0.76(3)	0.0117(5)	¹⁹⁷ Au	1148.562(6)	0.27(4)	0.0042(6)
¹⁹⁷ Au	838.156(5)	0.13(3)	0.0020(5)	¹⁹⁷ Au	1150.671(9)	0.25(4)	0.0038(6)
¹⁹⁷ Au	839.516(5)	0.73(20)	0.011(3)	¹⁹⁷ Au	1157.2330(20)	0.13(4)	0.0020(6)
¹⁹⁷ Au	846.216(7)	0.104(24)	0.0016(4)	¹⁹⁷ Au	1179.882(7)	0.12(5)	0.0018(8)
¹⁹⁷ Au	854.178(6)	0.093(18)	0.0014(3)	¹⁹⁷ Au	1183.796(6)	0.32(5)	0.0049(8)
¹⁹⁷ Au	854.650(4)	0.148(25)	0.0023(4)	¹⁹⁷ Au	1187.936(4)	0.15(4)	0.0023(6)
¹⁹⁷ Au	863.082(6)	0.148(25)	0.0023(4)	¹⁹⁷ Au	1189.904(10)	0.10(3)	0.0015(5)
¹⁹⁷ Au	868.771(4)	0.364(15)	0.00560(23)	¹⁹⁷ Au	1195.597(6)	0.148(22)	0.0023(3)
¹⁹⁷ Au	872.827(4)	0.096(18)	0.0015(3)	¹⁹⁷ Au	1200.827(8)	0.104(16)	0.00160(25)
¹⁹⁷ Au	877.308(4)	0.21(5)	0.0032(8)	¹⁹⁷ Au	1210.691(4)	0.20(3)	0.0031(5)
¹⁹⁷ Au	885.638(6)	0.17(3)	0.0026(5)	¹⁹⁷ Au	1216.453(5)	0.21(3)	0.0032(5)
¹⁹⁷ Au	891.613(3)	0.096(23)	0.0015(4)	¹⁹⁷ Au	1225.938(6)	0.27(4)	0.0042(6)
¹⁹⁷ Au	898.612(4)	0.15(3)	0.0023(5)	¹⁹⁷ Au	1239.572(5)	0.49(8)	0.0075(12)
¹⁹⁷ Au	902.478(6)	0.38(6)	0.0058(9)	¹⁹⁷ Au	1252.166(9)	0.126(23)	0.0019(4)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
¹⁹⁷ Au	1272.140(5)	0.096(16)	0.00148(25)	¹⁹⁷ Au	5722.94(10)	0.55(16)	0.0085(25)
¹⁹⁷ Au	1274.975(5)	0.26(4)	0.0040(6)	¹⁹⁷ Au	5767.01(10)	0.09(3)	0.0014(5)
¹⁹⁷ Au	1281.377(7)	0.49(12)	0.0075(18)	¹⁹⁷ Au	5808.50(10)	0.24(9)	0.0037(14)
¹⁹⁷ Au	1283.442(7)	0.35(11)	0.0054(17)	¹⁹⁷ Au	5839.57(10)	0.16(8)	0.0025(12)
¹⁹⁷ Au	1297.124(6)	0.43(10)	0.0066(15)	¹⁹⁷ Au	5879.74(10)	0.30(8)	0.0046(12)
¹⁹⁷ Au	1301.041(6)	0.15(6)	0.0023(9)	Mercury (Z=80), At.Wt.=200.59(2), σ_γ^Z = 384(8)			
¹⁹⁷ Au	1304.825(5)	0.25(5)	0.0038(8)	¹⁹⁶ Hg	133.98(5)d	0.0155(4)	2.34E-4[1.4%]
¹⁹⁷ Au	1306.851(5)	0.70(9)	0.0108(14)	¹⁹⁶ Hg	308.07(11)	0.79(7)	0.0119(11)
¹⁹⁷ Au	1308.164(4)	0.118(25)	0.0018(4)	199 Hg	367.947(9)	251(5)	3.79(8)
¹⁹⁷ Au	1316.318(5)	0.21(4)	0.0032(6)	²⁰¹ Hg	439.50(8)	0.52(7)	0.0079(11)
¹⁹⁷ Au	1324.356(14)	0.19(3)	0.0029(5)	¹⁹⁹ Hg	540.927(7)	2.75(9)	0.0415(14)
¹⁹⁷ Au	1335.515(12)	0.16(4)	0.0025(6)	¹⁹⁹ Hg	579.295(11)	7.64(23)	0.115(4)
¹⁹⁷ Au	1338.164(5)	0.118(22)	0.0018(3)	¹⁹⁹ Hg	661.403(11)	22.3(5)	0.337(8)
¹⁹⁷ Au	1344.153(6)	0.16(3)	0.0025(5)	¹⁹⁹ Hg	688.953(7)	2.83(11)	0.0428(17)
¹⁹⁷ Au	1361.477(5)	0.27(4)	0.0042(6)	¹⁹⁹ Hg	851.30(5)	2.69(9)	0.0406(14)
¹⁹⁷ Au	1363.345(4)	0.26(4)	0.0040(6)	¹⁹⁹ Hg	886.153(10)	13.5(11)	0.204(17)
¹⁹⁷ Au	1379.390(6)	0.141(22)	0.0022(3)	¹⁹⁹ Hg	1147.222(11)	7.79(23)	0.118(4)
¹⁹⁷ Au	1396.133(6)	0.141(22)	0.0022(3)	¹⁹⁹ Hg	1202.328(10)	12.0(3)	0.181(5)
¹⁹⁷ Au	1431.641(6)	0.15(4)	0.0023(6)	¹⁹⁹ Hg	1205.717(11)	13.5(5)	0.204(8)
¹⁹⁷ Au	1431.949(4)	0.23(4)	0.0035(6)	¹⁹⁹ Hg	1225.476(11)	12.3(3)	0.186(5)
¹⁹⁷ Au	1445.373(5)	0.14(3)	0.0022(5)	¹⁹⁹ Hg	1254.099(12)	7.56(23)	0.114(4)
¹⁹⁷ Au	1487.130(4)	0.20(4)	0.0031(6)	¹⁹⁹ Hg	1262.941(11)	21.5(5)	0.325(8)
¹⁹⁷ Au	1487.599(7)	0.20(4)	0.0031(6)	¹⁹⁹ Hg	1273.497(10)	10.6(3)	0.160(5)
¹⁹⁷ Au	1530.698(6)	0.30(5)	0.0046(8)	¹⁹⁹ Hg	1350.354(10)	4.10(16)	0.0619(24)
¹⁹⁷ Au	1554.420(5)	0.25(9)	0.0038(14)	¹⁹⁹ Hg	1362.971(10)	5.93(19)	0.090(3)
¹⁹⁷ Au	4951.85(10)	0.156(16)	0.00240(25)	¹⁹⁹ Hg	1407.942(20)	9.53(23)	0.144(4)
¹⁹⁷ Au	4957.83(10)	0.63(11)	0.0097(17)	¹⁹⁹ Hg	1467.92(5)	3.31(13)	0.0500(20)
¹⁹⁷ Au	4975.87(10)	0.161(16)	0.00248(25)	¹⁹⁹ Hg	1488.825(11)	2.92(14)	0.0441(21)
¹⁹⁷ Au	4981.55(10)	0.09(3)	0.0014(5)	¹⁹⁹ Hg	1514.903(10)	2.68(13)	0.0405(20)
¹⁹⁷ Au	4998.68(10)	0.31(4)	0.0048(6)	¹⁹⁹ Hg	1557.65(9)	2.6(8)	0.039(12)
¹⁹⁷ Au	5007.08(10)	0.113(15)	0.00174(23)	¹⁹⁹ Hg	1557.94(4)	2.87(14)	0.0434(21)
¹⁹⁷ Au	5025.11(10)	0.113(16)	0.00174(25)	199 Hg	1570.273(12)	29.6(7)	0.447(11)
¹⁹⁷ Au	5036.63(10)	0.18(7)	0.0028(11)	¹⁹⁹ Hg	1604.322(11)	4.07(17)	0.061(3)
¹⁹⁷ Au	5040.15(10)	0.18(7)	0.0028(11)	199 Hg	1693.296(11)	56.2(16)	0.849(24)
¹⁹⁷ Au	5080.60(10)	0.152(15)	0.00234(23)	¹⁹⁹ Hg	1718.299(12)	8.47(23)	0.128(4)
¹⁹⁷ Au	5088.46(10)	0.50(8)	0.0077(12)	¹⁹⁹ Hg	1758.97(6)	3.33(14)	0.0503(21)
¹⁹⁷ Au	5102.85(10)	0.87(13)	0.0134(20)	¹⁹⁹ Hg	2002.083(13)	24.3(9)	0.367(14)
¹⁹⁷ Au	5110.17(10)	0.156(11)	0.00240(17)	¹⁹⁹ Hg	2271.90(3)	6.05(23)	0.091(4)
¹⁹⁷ Au	5116.11(10)	0.161(13)	0.00248(20)	¹⁹⁹ Hg	2296.310(23)	2.89(17)	0.044(3)
¹⁹⁷ Au	5140.74(10)	0.395(18)	0.0061(3)	¹⁹⁹ Hg	2639.85(3)	11.6(3)	0.175(5)
¹⁹⁷ Au	5148.90(10)	0.46(8)	0.0071(12)	¹⁹⁹ Hg	2818.26(5)	3.42(16)	0.0517(24)
¹⁹⁷ Au	5153.21(10)	0.119(14)	0.00183(22)	¹⁹⁹ Hg	2901.25(5)	4.63(19)	0.070(3)
¹⁹⁷ Au	5174.08(10)	0.334(16)	0.00514(25)	¹⁹⁹ Hg	2920.90(4)	4.99(23)	0.075(4)
¹⁹⁷ Au	5205.39(10)	0.16(6)	0.0025(9)	¹⁹⁹ Hg	3186.21(5)	11.3(4)	0.171(6)
¹⁹⁷ Au	5218.35(10)	0.272(20)	0.0042(3)	¹⁹⁹ Hg	3216.63(9)	2.93(17)	0.044(3)
¹⁹⁷ Au	5225.49(10)	0.42(9)	0.0065(14)	¹⁹⁹ Hg	3269.19(5)	3.96(18)	0.060(3)
¹⁹⁷ Au	5246.72(10)	0.51(20)	0.008(3)	¹⁹⁹ Hg	3288.85(4)	13.3(4)	0.201(6)
¹⁹⁷ Au	5271.86(10)	0.38(20)	0.006(3)	¹⁹⁹ Hg	4373.37(8)	3.70(23)	0.056(4)
¹⁹⁷ Au	5279.44(10)	0.524(20)	0.0081(3)	¹⁹⁹ Hg	4575.36(6)	4.23(23)	0.064(4)
¹⁹⁷ Au	5302.86(10)	0.19(10)	0.0029(15)	¹⁹⁹ Hg	4675.44(9)	13.0(4)	0.196(6)
¹⁹⁷ Au	5355.00(10)	0.401(16)	0.00617(25)	199 Hg	4739.43(5)	30.1(8)	0.455(12)
¹⁹⁷ Au	5473.96(10)	0.21(6)	0.0032(9)	¹⁹⁹ Hg	4759.09(6)	12.4(4)	0.187(6)
¹⁹⁷ Au	5493.81(10)	0.42(10)	0.0065(15)	¹⁹⁹ Hg	4811.64(9)	3.70(23)	0.056(4)
¹⁹⁷ Au	5524.66(10)	0.80(14)	0.0123(22)	¹⁹⁹ Hg	4842.07(6)	20.0(6)	0.302(9)
¹⁹⁷ Au	5540.41(10)	0.17(6)	0.0026(9)	¹⁹⁹ Hg	4954.47(5)	4.01(23)	0.061(4)
¹⁹⁷ Au	5620.62(10)	0.34(9)	0.0052(14)	¹⁹⁹ Hg	4974.98(7)	5.22(23)	0.079(4)
197 Au	5710.52(10)	1.27(17)	0.020(3)	-----			

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	<i>E</i> _γ (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	<i>k</i> ₀	^A Z	<i>E</i> _γ (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	<i>k</i> ₀
¹⁹⁹ Hg	5050.07(5)	20.0(6)	0.302(9)	²⁰³ Tl	818.14(8)	0.0279(10)	0.000414(15)
¹⁹⁹ Hg	5388.43(5)	17.5(5)	0.264(8)	²⁰³Tl	873.16(8)	0.168(4)	0.00249(6)
¹⁹⁹Hg	5658.24(4)	27.5(7)	0.415(11)	²⁰³ Tl	931.39(8)	0.0257(12)	0.000381(18)
¹⁹⁹Hg	5967.02(4)	62.5(15)	0.944(23)	²⁰³Tl	949.88(8)	0.0479(15)	0.000710(22)
¹⁹⁹ Hg	6309.96(4)	4.0(3)	0.060(5)	²⁰³ Tl	1013.27(9)	0.0217(12)	0.000322(18)
¹⁹⁹ Hg	6397.37(4)	3.7(3)	0.056(5)	²⁰³ Tl	1063.00(9)	0.0185(10)	0.000274(15)
¹⁹⁹ Hg	6457.98(4)	23.1(8)	0.349(12)	²⁰³ Tl	1093.02(8)	0.0353(12)	0.000523(18)
Thallium (Z=81), At.Wt.=204.3833(2), $\sigma_{\gamma}^Z = 3.44(6)$				²⁰³Tl	1110.37(8)	0.0413(12)	0.000612(18)
²⁰³ Tl	77.07(22)	0.011(5)	1.6(7)E-4	²⁰³Tl	1121.29(7)	0.0600(17)	0.000890(25)
²⁰³ Tl	132.11(14)	0.0062(10)	9.2(15)E-5	²⁰³ Tl	1134.01(9)	0.0133(7)	1.97(10)E-4
²⁰³Tl	139.94(9)	0.400(7)	0.00593(10)	²⁰³Tl	1155.43(7)	0.0605(17)	0.000897(25)
²⁰³ Tl	145.88(10)	0.0054(5)	8.0(7)E-5	²⁰³ Tl	1182.6(4)	0.0052(12)	7.7(18)E-5
²⁰³ Tl	152.93(11)	0.0144(6)	2.14(9)E-4	²⁰³Tl	1234.69(7)	0.0746(25)	0.00111(4)
²⁰³Tl	154.01(9)	0.0926(17)	0.001373(25)	²⁰³Tl	1478.77(8)	0.0544(22)	0.00081(3)
²⁰³ Tl	157.32(10)	0.0061(5)	9.0(7)E-5	²⁰³ Tl	1706.20(16)	0.0091(15)	1.35(22)E-4
²⁰³ Tl	171.88(9)	0.0109(5)	1.62(7)E-4	²⁰³Tl	1741.01(8)	0.0548(25)	0.00081(4)
²⁰³ Tl	178.78(11)	0.0050(5)	7.4(7)E-5	²⁰³ Tl	1756.27(12)	0.027(3)	0.00040(4)
²⁰³Tl	198.33(8)	0.0408(10)	0.000605(15)	²⁰³ Tl	4076.7(6)	0.0072(15)	1.07(22)E-4
²⁰⁵ Tl	265.86(9)	0.0210(7)	0.000311(10)	²⁰³ Tl	4101.4(4)	0.0086(25)	1.3(4)E-4
²⁰³ Tl	284.81(12)	0.0052(5)	7.7(7)E-5	²⁰³ Tl	4115.08(17)	0.0222(17)	0.000329(25)
²⁰³ Tl	286.88(11)	0.0058(5)	8.6(7)E-5	²⁰³Tl	4195.98(14)	0.0373(22)	0.00055(3)
²⁰³Tl	292.26(8)	0.0983(20)	0.00146(3)	²⁰³Tl	4225.47(17)	0.045(3)	0.00067(4)
²⁰⁵ Tl	304.86(9)	0.0225(12)	0.000334(18)	²⁰³ Tl	4286.3(8)	0.0057(15)	8.5(22)E-5
²⁰³ Tl	310.31(9)	0.0245(12)	0.000363(18)	²⁰³ Tl	4309.00(24)	0.0210(22)	0.00031(3)
²⁰³Tl	318.88(8)	0.325(6)	0.00482(9)	²⁰³ Tl	4343.56(12)	0.034(3)	0.00050(4)
²⁰³ Tl	325.85(8)	0.0301(10)	0.000446(15)	²⁰³ Tl	4402.60(15)	0.0208(15)	0.000308(22)
²⁰³ Tl	330.09(9)	0.0267(10)	0.000396(15)	²⁰³ Tl	4439.3(3)	0.0094(15)	1.39(22)E-4
²⁰⁵ Tl	330.09(9)	0.0267(10)	0.000396(15)	²⁰³Tl	4495.74(13)	0.043(4)	0.00064(6)
²⁰³ Tl	331.76(9)	0.0371(10)	0.000550(15)	²⁰³Tl	4540.62(15)	0.0413(25)	0.00061(4)
²⁰³ Tl	336.96(10)	0.0080(6)	1.19(9)E-4	²⁰³ Tl	4570.0(3)	0.0180(20)	0.00027(3)
²⁰³Tl	347.96(8)	0.361(10)	0.00535(15)	²⁰³ Tl	4600.95(16)	0.0292(22)	0.00043(3)
²⁰⁵ Tl	369.18(7)	0.016(3)	2.4(4)E-4	²⁰³Tl	4687.58(12)	0.098(4)	0.00145(6)
²⁰³ Tl	369.65(24)	0.0047(12)	7.0(18)E-5	²⁰³Tl	4705.83(14)	0.058(3)	0.00086(4)
²⁰³ Tl	383.99(8)	0.0341(12)	0.000506(18)	²⁰³ Tl	4715.3(4)	0.0131(20)	1.9(3)E-4
²⁰³ Tl	389.48(11)	0.0079(7)	1.17(10)E-4	²⁰³Tl	4752.24(11)	0.148(5)	0.00219(7)
²⁰³Tl	395.62(8)	0.0862(20)	0.00128(3)	²⁰³ Tl	4804.4(4)	0.0138(20)	2.0(3)E-4
²⁰³ Tl	416.91(17)	0.0069(12)	1.02(18)E-4	²⁰³Tl	4841.40(15)	0.090(4)	0.00133(6)
²⁰³ Tl	418.27(11)	0.0141(12)	2.09(18)E-4	²⁰³ Tl	4867.5(6)	0.0074(20)	1.1(3)E-4
²⁰³Tl	424.81(8)	0.1200(25)	0.00178(4)	²⁰³Tl	4913.57(11)	0.164(5)	0.00243(7)
²⁰³Tl	471.90(8)	0.116(3)	0.00172(4)	²⁰³ Tl	4980.97(20)	0.036(3)	0.00053(4)
²⁰³ Tl	483.29(12)	0.0082(10)	1.22(15)E-4	²⁰³Tl	5014.61(15)	0.058(3)	0.00086(4)
²⁰³Tl	488.11(8)	0.096(4)	0.00142(6)	²⁰³Tl	5130.50(23)	0.058(4)	0.00086(6)
²⁰³ Tl	489.26(24)	0.008(3)	1.2(4)E-4	²⁰³Tl	5180.38(12)	0.141(5)	0.00209(7)
²⁰³ Tl	563.21(8)	0.0356(15)	0.000528(22)	²⁰³ Tl	5238.4(3)	0.0156(20)	2.3(3)E-4
²⁰³ Tl	587.01(10)	0.0109(10)	1.62(15)E-4	²⁰³Tl	5261.48(13)	0.084(4)	0.00125(6)
²⁰³ Tl	591.13(9)	0.0225(10)	0.000334(15)	²⁰³Tl	5279.86(12)	0.207(6)	0.00307(9)
²⁰³Tl	624.46(8)	0.0413(10)	0.000612(15)	²⁰³Tl	5404.41(12)	0.147(5)	0.00218(7)
²⁰³ Tl	626.54(8)	0.0388(10)	0.000575(15)	²⁰³Tl	5451.07(14)	0.079(3)	0.00117(4)
²⁰³ Tl	629.12(8)	0.0388(10)	0.000575(15)	²⁰³ Tl	5520.3(4)	0.0183(25)	0.00027(4)
²⁰⁵ Tl	649.30(15)	0.0106(10)	1.57(15)E-4	²⁰³Tl	5533.35(13)	0.131(5)	0.00194(7)
²⁰³ Tl	678.01(8)	0.0361(15)	0.000535(22)	²⁰³Tl	5603.28(13)	0.282(10)	0.00418(15)
²⁰³ Tl	714.86(24)	0.0074(12)	1.10(18)E-4	²⁰³Tl	5641.57(12)	0.316(7)	0.00469(10)
²⁰³Tl	732.09(9)	0.064(3)	0.00095(4)	²⁰⁵ Tl	5852.5(5)	0.0072(15)	1.07(22)E-4
²⁰³Tl	737.12(8)	0.118(5)	0.00175(7)	²⁰⁵ Tl	5867.8(4)	0.0091(17)	1.35(25)E-4
²⁰³ Tl	764.13(9)	0.0316(12)	0.000469(18)	²⁰³ Tl	5890.2(4)	0.0067(17)	9.9(25)E-5
²⁰⁵ Tl	803.30(20)d	3.5(6)E-6	5.2E-8[90%]	²⁰³Tl	5917.48(16)	0.084(4)	0.00125(6)

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
²⁰³ Tl	6025.21(24)	0.0222(25)	0.00033(4)	²⁰⁹ Bi	1175.48(12)	0.00048(7)	7.0(10)E-6
²⁰³ Tl	6118.79(23)	0.0232(20)	0.00034(3)	²⁰⁹ Bi	1203.52(11)	0.00077(12)	1.12(17)E-5
²⁰³Tl	6166.61(14)	0.166(6)	0.00246(9)	²⁰⁹ Bi	1203.61(8)	2.1(8)E-4	3.0(12)E-6
²⁰³Tl	6183.05(15)	0.081(4)	0.00120(6)	²⁰⁹ Bi	1203.61(10)	2.1(8)E-4	3.0(12)E-6
²⁰⁵ Tl	6197.8(4)	0.0109(17)	1.62(25)E-4	²⁰⁹ Bi	1211.11(15)	0.00031(5)	4.5(7)E-6
²⁰³Tl	6222.57(16)	0.065(4)	0.00096(6)	²⁰⁹ Bi	1226.30(6)	0.00042(7)	6.1(10)E-6
²⁰³ Tl	6336.11(22)	0.0245(22)	0.00036(3)	²⁰⁹ Bi	1337.09(6)	0.00156(21)	2.3(3)E-5
²⁰⁵ Tl	6504.3(6)	0.0040(10)	5.9(15)E-5	²⁰⁹ Bi	1360.16(15)	2.0(4)E-4	2.9(6)E-6
²⁰³Tl	6514.57(15)	0.129(5)	0.00191(7)	²⁰⁹ Bi	1397.83(11)	0.00033(5)	4.8(7)E-6
²⁰³ Tl	6654.71(25)	0.0104(12)	1.54(18)E-4	²⁰⁹ Bi	1430.29(14)	0.00027(4)	3.9(6)E-6
Lead (Z=82), At.Wt.=207.2(1), σ_γ^Z=0.154(7)				²⁰⁹ Bi	1465.52(14)	0.00026(4)	3.8(6)E-6
²⁰⁶ Pb	569.702d	0.0014(3)	2.0E-5[100%]	²⁰⁹ Bi	1484.30(8)	0.00034(5)	4.9(7)E-6
²⁰⁴ Pb	6729.38(9)	0.00320(10)	4.68(15)E-5	²⁰⁹ Bi	1596.43(7)	0.00073(10)	1.06(15)E-5
²⁰⁶ Pb	6737.62(10)	0.00691(19)	1.01(3)E-4	²⁰⁹ Bi	1625.78(17)	2.1(4)E-4	3.0(6)E-6
²⁰⁷Pb	7367.78(7)	0.137(3)	0.00200(4)	²⁰⁹ Bi	1658.34(7)	0.00035(5)	5.1(7)E-6
Bismuth (Z=83), At.Wt.=208.98038(2), σ_γ^Z=0.0338(7)				²⁰⁹ Bi	1708.84(9)	0.00071(10)	1.03(15)E-5
²⁰⁹ Bi	46.58(12)	0.00043(9)	6.2(13)E-6	²⁰⁹ Bi	1708.92(10)	2.2(8)E-4	3.2(12)E-6
²⁰⁹ Bi	63.59(5)	1.8(4)E-4	2.6(6)E-6	²⁰⁹ Bi	1756.35(14)	2.4(4)E-4	3.5(6)E-6
²⁰⁹ Bi	64.94(6)	2.1(13)E-4	3.0(19)E-6	²⁰⁹ Bi	1824.97(15)	0.00054(8)	7.8(12)E-6
²⁰⁹ Bi	65.24(20)	1.8(4)E-4	2.6(6)E-6	²⁰⁹ Bi	1839.74(13)	0.00046(7)	6.7(10)E-6
²⁰⁹ Bi	91.29(5)	0.0005(3)	7(4)E-6	²⁰⁹ Bi	2026.66(15)	0.00037(7)	5.4(10)E-6
²⁰⁹ Bi	92.48(13)	2.5(4)E-4	3.6(6)E-6	²⁰⁹ Bi	2496.69(16)	0.00034(7)	4.9(10)E-6
²⁰⁹ Bi	116.49(9)	0.00054(21)	8(3)E-6	²⁰⁹Bi	2505.35(7)	0.0021(3)	3.0(4)E-5
²⁰⁹ Bi	154.86(6)	2.5(4)E-4	3.6(6)E-6	²⁰⁹ Bi	2570.29(7)	0.00031(5)	4.5(7)E-6
²⁰⁹ Bi	154.89(5)	0.0013(5)	1.9(7)E-5	²⁰⁹ Bi	2598.33(8)	0.00166(24)	2.4(4)E-5
²⁰⁹Bi	162.19(11)	0.008(3)	1.2(4)E-4	²⁰⁹ Bi	2614.55(12)	0.00027(5)	3.9(7)E-6
²⁰⁹ Bi	162.27(6)	0.00162(21)	2.3(3)E-5	²⁰⁹ Bi	2624.34(7)	0.00154(21)	2.2(3)E-5
²⁰⁹ Bi	183.04(6)	1.8(8)E-4	2.6(12)E-6	²⁰⁹Bi	2828.29(7)	0.00179(24)	2.6(4)E-5
²⁰⁹ Bi	311.23(11)	2.0(4)E-4	2.9(6)E-6	²⁰⁹ Bi	2898.17(8)	0.00080(12)	1.16(17)E-5
²⁰⁹Bi	319.78(4)	0.0115(14)	1.67(20)E-4	²⁰⁹ Bi	3081.27(10)	0.00145(20)	2.1(3)E-5
²⁰⁹ Bi	347.92(9)	2.1(4)E-4	3.0(6)E-6	²⁰⁹ Bi	3141.75(8)	0.00041(7)	5.9(10)E-6
²⁰⁹ Bi	347.93(5)	1.8(8)E-4	2.6(12)E-6	²⁰⁹ Bi	3214.64(8)	0.00061(9)	8.8(13)E-6
²⁰⁹ Bi	392.82(9)	2.4(4)E-4	3.5(6)E-6	²⁰⁹ Bi	3230.66(10)	2.1(4)E-4	3.0(6)E-6
²⁰⁹ Bi	408.77(7)	0.00043(7)	6.2(10)E-6	²⁰⁹ Bi	3268.99(9)	2.2(5)E-4	3.2(7)E-6
²⁰⁹ Bi	563.06(7)	2.1(8)E-4	3.0(12)E-6	²⁰⁹ Bi	3356.60(8)	0.00167(24)	2.4(4)E-5
²⁰⁹ Bi	563.14(7)	0.00051(7)	7.4(10)E-6	²⁰⁹ Bi	3396.16(7)	0.00170(24)	2.5(4)E-5
²⁰⁹ Bi	610.92(11)	1.8(4)E-4	2.6(6)E-6	²⁰⁹ Bi	3407.4(3)	2.5(5)E-4	3.6(7)E-6
²⁰⁹ Bi	644.36(8)	2.5(4)E-4	3.6(6)E-6	²⁰⁹ Bi	3610.84(6)	2.1(5)E-4	3.0(7)E-6
²⁰⁹ Bi	645.82(6)	0.00047(7)	6.8(10)E-6	²⁰⁹ Bi	3632.77(7)	0.00136(20)	2.0(3)E-5
²⁰⁹Bi	673.97(5)	0.0026(4)	3.8(6)E-5	²⁰⁹Bi	4054.57(6)	0.0137(18)	2.0(3)E-4
²⁰⁹ Bi	769.21(6)	0.00078(10)	1.13(15)E-5	²⁰⁹ Bi	4101.76(6)	0.0089(12)	1.29(17)E-4
²⁰⁹ Bi	774.91(10)	0.00054(21)	8(3)E-6	²⁰⁹Bi	4165.36(5)	0.00173(24)	2.5(4)E-5
²⁰⁹ Bi	774.92(7)	0.00141(20)	2.0(3)E-5	²⁰⁹ Bi	4171.05(9)	0.0171(22)	2.5(3)E-4
²⁰⁹ Bi	808.77(7)	0.00042(16)	6.1(23)E-6	²⁰⁹ Bi	4256.65(5)	0.0024(3)	3.5(4)E-5
²⁰⁹ Bi	808.79(7)	0.00119(16)	1.73(23)E-5	²⁰⁹ Bi	4284.80(6)	0.00042(7)	6.1(10)E-6
²⁰⁹ Bi	826.98(13)	2.0(3)E-4	2.9(4)E-6	Thorium (Z=90), At.Wt.=232.0381(1), σ_γ^Z=7.35(3)			
²⁰⁹ Bi	855.45(14)	1.8(4)E-4	2.6(6)E-6	²³² Th	39.92(13)	0.0029(4)	3.8(5)E-5
²⁰⁹ Bi	900.07(7)	0.00035(13)	5.1(19)E-6	²³² Th	44.36(14)	0.0031(4)	4.0(5)E-5
²⁰⁹ Bi	900.22(9)	0.00102(14)	1.48(20)E-5	²³² Th	53.71(12)	0.0139(10)	1.82(13)E-4
²⁰⁹ Bi	912.77(10)	0.00034(5)	4.9(7)E-6	²³² Th	57.41(15)	0.0068(9)	8.9(12)E-5
²⁰⁹ Bi	971.82(7)	0.00026(9)	3.8(13)E-6	²³²Th	63.810(10)	10.7(5) s⁻¹g⁻¹	Abundant
²⁰⁹ Bi	971.83(9)	0.00072(9)	1.04(13)E-5	²³² Th	77.09(15)	0.09(3)	0.0012(4)
²⁰⁹ Bi	1012.53(7)	0.00064(9)	9.3(13)E-6	²³²Th	140.880(10)	0.85(18) s⁻¹g⁻¹	Abundant
²⁰⁹ Bi	1013.03(13)	2.1(8)E-4	3.0(12)E-6	²³² Th	201.75(12)	0.0079(8)	1.03(10)E-4
²⁰⁹ Bi	1118.21(19)	2.1(4)E-4	3.0(6)E-6	²³² Th	211.86(11)	0.0191(17)	2.49(22)E-4
²⁰⁹ Bi	1156.34(14)	2.0(4)E-4	2.9(6)E-6	²³² Th	229.08(11)	0.0163(13)	2.13(17)E-4

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k ₀	^A Z	E _{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k ₀
²³² Th	256.25(11)	0.093(17)	0.00121(22)	²³² Th	780.8(3)	0.0052(15)	6.8(20)E-5
²³² Th	263.06(14)	0.0073(17)	9.5(22)E-5	²³² Th	785.86(22)	0.0097(18)	1.27(24)E-4
²³² Th	277.48(11)	0.0312(25)	0.00041(3)	²³² Th	797.79(9)	0.0416(20)	0.00054(3)
²³² Th	281.40(11)	0.0170(14)	2.22(18)E-4	²³² Th	808.53(11)	0.0212(14)	0.000277(18)
²³² Th	286.16(25)	0.0028(7)	3.7(9)E-5	²³² Th	814.75(10)	0.0196(13)	0.000256(17)
²³² Th	311.91(10)	0.0187(10)	2.44(13)E-4	²³² Th	834.83(14)	0.059(5)	0.00077(7)
²³² Th	316.64(10)	0.0397(18)	0.000518(24)	²³² Th	846.0(5)	0.013(3)	1.7(4)E-4
²³² Th	319.08(10)	0.082(3)	0.00107(4)	²³² Th	849.4(7)	0.005(3)	7(4)E-5
²³² Th	320.98(13)	0.0072(8)	9.4(10)E-5	²³² Th	860.61(13)	0.047(5)	0.00061(7)
²³² Th	327.80(10)	0.0269(16)	0.000351(21)	²³² Th	869.69(14)	0.0138(11)	1.80(14)E-4
²³² Th	329.88(11)	0.0221(17)	0.000289(22)	²³² Th	872.13(11)	0.0268(15)	0.000350(20)
²³² Th	331.37(11)	0.0291(19)	0.000380(25)	²³² Th	907.44(14)	0.0081(10)	1.06(13)E-4
²³² Th	335.92(10)	0.089(4)	0.00116(5)	²³² Th	913.74(17)	0.0063(10)	8.2(13)E-5
²³² Th	354.27(10)	0.0408(20)	0.00053(3)	²³² Th	918.70(13)	0.0096(10)	1.25(13)E-4
²³² Th	365.28(16)	0.0060(9)	7.8(12)E-5	²³² Th	941.79(13)	0.0103(11)	1.35(14)E-4
²³² Th	366.79(16)	0.0061(9)	8.0(12)E-5	²³² Th	968.78(9)	0.132(6)	0.00172(8)
²³² Th	370.35(15)	0.0044(8)	5.7(10)E-5	²³² Th	996.7(3)	0.0067(16)	8.8(21)E-5
²³² Th	384.7(3)	0.0030(8)	3.9(10)E-5	²³² Th	1013.84(11)	0.037(3)	0.00048(4)
²³² Th	427.24(17)	0.0040(7)	5.2(9)E-5	²³² Th	1031.1(3)	0.0040(10)	5.2(13)E-5
²³² Th	432.15(13)	0.0076(8)	9.9(10)E-5	²³² Th	1034.27(11)	0.0165(14)	2.15(18)E-4
²³² Th	472.30(10)	0.165(8)	0.00215(10)	²³² Th	1044.58(14)	0.0112(12)	1.46(16)E-4
²³² Th	506.22(13)	0.0075(11)	9.8(14)E-5	²³² Th	1055.60(14)	0.0105(12)	1.37(16)E-4
²³² Th	522.73(10)	0.102(5)	0.00133(7)	²³² Th	1096.9(4)	0.0050(13)	6.5(17)E-5
²³² Th	531.58(10)	0.0404(23)	0.00053(3)	²³² Th	1100.98(11)	0.0211(16)	0.000276(21)
²³² Th	535.08(17)	0.0062(11)	8.1(14)E-5	²³² Th	1116.9(3)	0.0060(12)	7.8(16)E-5
²³² Th	539.66(10)	0.061(3)	0.00080(4)	²³² Th	1125.46(19)	0.0079(13)	1.03(17)E-4
²³² Th	548.23(11)	0.042(10)	0.00055(13)	²³² Th	1145.37(17)	0.0123(15)	1.61(20)E-4
²³² Th	553.36(13)	0.011(3)	1.4(4)E-4	²³² Th	1152.1(4)	0.0052(15)	6.8(20)E-5
²³² Th	556.93(11)	0.040(10)	0.00052(13)	²³² Th	1154.5(4)	0.0056(15)	7.3(20)E-5
²³² Th	561.25(11)	0.033(8)	0.00043(10)	²³² Th	1164.6(4)	0.0047(13)	6.1(17)E-5
²³² Th	566.63(10)	0.19(5)	0.0025(7)	²³² Th	1184.9(6)	0.0036(13)	4.7(17)E-5
²³² Th	569.15(16)	0.008(3)	1.0(4)E-4	²³² Th	2485.2(3)	0.0090(17)	1.18(22)E-4
²³² Th	578.02(9)	0.105(5)	0.00137(7)	²³² Th	2503.5(3)	0.0107(18)	1.40(24)E-4
²³² Th	580.16(19)	0.0125(21)	1.6(3)E-4	²³² Th	2524.7(4)	0.0087(16)	1.14(21)E-4
²³² Th	583.27(9)	0.279(11)	0.00364(14)	²³² Th	2543.3(5)	0.013(3)	1.7(4)E-4
²³² Th	586.02(10)	0.045(3)	0.00059(4)	²³² Th	2546.8(8)	0.0076(23)	1.0(3)E-4
²³² Th	593.23(10)	0.043(3)	0.00056(4)	²³² Th	2551.9(4)	0.010(4)	1.3(5)E-4
²³² Th	605.41(10)	0.054(4)	0.00071(5)	²³² Th	2557.8(5)	0.0069(17)	9.0(22)E-5
²³² Th	612.45(9)	0.018(3)	2.4(4)E-4	²³² Th	2590.0(10)	0.0069(20)	9(3)E-5
²³² Th	622.95(11)	0.0125(15)	1.63(20)E-4	²³² Th	2596.76(23)	0.0118(18)	1.54(24)E-4
²³² Th	632.09(12)	0.0105(9)	1.37(12)E-4	²³² Th	2630.1(3)	0.0071(19)	9.3(25)E-5
²³² Th	659.56(16)	0.0173(20)	2.3(3)E-4	²³² Th	2640.8(4)	0.0110(18)	1.44(24)E-4
²³² Th	662.0(3)	0.0101(18)	1.32(24)E-4	²³² Th	2653.2(3)	0.010(4)	1.3(5)E-4
²³² Th	665.11(10)	0.084(4)	0.00110(5)	²³² Th	2659.39(21)	0.013(4)	1.7(5)E-4
²³² Th	681.81(9)	0.079(4)	0.00103(5)	²³² Th	2671.7(6)	0.0085(18)	1.11(24)E-4
²³² Th	684.96(13)	0.0117(16)	1.53(21)E-4	²³² Th	2689.4(8)	0.008(3)	1.0(4)E-4
²³² Th	696.57(14)	0.0139(17)	1.82(22)E-4	²³² Th	2703.55(24)	0.014(5)	1.8(7)E-4
²³² Th	703.1(5)	0.0073(18)	9.5(24)E-5	²³² Th	2712.56(22)	0.013(4)	1.7(5)E-4
²³² Th	705.17(11)	0.050(4)	0.00065(5)	²³² Th	2719.67(18)	0.016(3)	2.1(4)E-4
²³² Th	714.23(10)	0.052(3)	0.00068(4)	²³² Th	2732.7(5)	0.008(3)	1.0(4)E-4
²³² Th	721.60(22)	0.0073(15)	9.5(20)E-5	²³² Th	2739.8(3)	0.0072(14)	9.4(18)E-5
²³² Th	735.25(14)	0.0123(16)	1.61(21)E-4	²³² Th	2744.7(3)	0.0081(15)	1.06(20)E-4
²³² Th	741.02(15)	0.0122(16)	1.59(21)E-4	²³² Th	2758.3(4)	0.0063(14)	8.2(18)E-5
²³² Th	752.05(16)	0.0142(19)	1.85(25)E-4	²³² Th	2771.3(4)	0.0030(12)	3.9(16)E-5
²³² Th	768.58(23)	0.0091(15)	1.19(20)E-4	²³² Th	2784.5(3)	0.0075(15)	9.8(20)E-5
²³² Th	777.8(4)	0.0034(14)	4.4(18)E-5	²³² Th	2807.08(18)	0.0110(17)	1.44(22)E-4

TABLE 7.3

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k ₀	^A Z	E _γ (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k ₀
²³² Th	2821.9(3)	0.0110(20)	1.4(3)E-4	²³² Th	4045.00(13)	0.0118(9)	1.54(12)E-4
²³² Th	2824.9(3)	0.0144(22)	1.9(3)E-4	²³² Th	4073.33(19)	0.0060(7)	7.8(9)E-5
²³² Th	2838.0(3)	0.0059(15)	7.7(20)E-5	²³² Th	4201.85(16)	0.0110(9)	1.44(12)E-4
²³² Th	2851.0(3)	0.0077(15)	1.01(20)E-4	²³² Th	4215.0(4)	0.0033(5)	4.3(7)E-5
²³² Th	2880.86(17)	0.0093(14)	1.21(18)E-4	²³² Th	4246.78(15)	0.0093(7)	1.21(9)E-4
²³² Th	2924.3(3)	0.0082(11)	1.07(14)E-4	²³² Th	4450.54(21)	0.0043(5)	5.6(7)E-5
²³² Th	2945.0(4)	0.0033(9)	4.3(12)E-5	²³² Th	4769.66(25)	0.0047(7)	6.1(9)E-5
²³² Th	2970.49(21)	0.0064(10)	8.4(13)E-5	²³² Th	4787.0(6)	0.0037(7)	4.8(9)E-5
²³² Th	2980.69(18)	0.0084(11)	1.10(14)E-4	Uranium (Z=92), At.Wt.=238.02891(3), σ_γ^z=3.374(20)			
²³² Th	2989.93(25)	0.0066(10)	8.6(13)E-5	¹³⁹ Ba ^d	29.9660(10)d	0.0381(11)	0.000485[<0.1%]
²³² Th	3009.9(3)	0.0051(10)	6.7(13)E-5	²³⁵ U	31.60(5)	0.10(3) s⁻¹g⁻¹	Abundant
²³² Th	3044.7(4)	0.0031(12)	4.0(16)E-5	²³⁵ U	34.70(10)	0.2100(15) s⁻¹g⁻¹	Abundant
²³² Th	3056.43(23)	0.0084(12)	1.10(16)E-4	²³⁵ U	41.4(3)	0.17(12) s⁻¹g⁻¹	Abundant
²³² Th	3070.6(4)	0.0039(12)	5.1(16)E-5	²³⁵ U	41.96(15)	0.35(6) s⁻¹g⁻¹	Abundant
²³² Th	3087.34(17)	0.0086(24)	1.1(3)E-4	²³⁸ U	43.5330(10)d	0.110(3)	0.00140[53%]
²³² Th	3118.4(9)	0.0040(10)	5.2(13)E-5	²³⁵ U	51.22(10)	0.20(4) s⁻¹g⁻¹	Abundant
²³² Th	3127.73(25)	0.0058(11)	7.6(14)E-5	²³⁵ U	54.25(5)	0.1700(12) s⁻¹g⁻¹	Abundant
²³² Th	3132.80(17)	0.0087(10)	1.14(13)E-4	²³⁵ U	72.70(20)	0.630(5) s⁻¹g⁻¹	Abundant
²³² Th	3148.23(10)	0.0208(14)	0.000272(18)	²³⁸ U	74.6640(10)d	1.300(3)	0.01655[53%]
²³² Th	3173.87(19)	0.0089(10)	1.16(13)E-4	²³⁵ U	75.02(5)	0.35(6) s⁻¹g⁻¹	Abundant
²³² Th	3184.94(17)	0.0079(10)	1.03(13)E-4	²³⁵ U	76.198(4)	0.046(6) s⁻¹g⁻¹	Abundant
²³² Th	3196.66(12)	0.0171(13)	2.23(17)E-4	²³⁵ U	96.090(20)	0.52(7) s⁻¹g⁻¹	Abundant
²³² Th	3230.47(23)	0.0123(12)	1.61(16)E-4	²³⁸ Np ^d	106.1230(20)d	0.723(11)	0.00920[<0.1%]
²³² Th	3245.2(5)	0.0030(8)	3.9(10)E-5	²³⁵ U	109.160(20)	8.9(3) s⁻¹g⁻¹	Abundant
²³² Th	3260.9(3)	0.0056(9)	7.3(12)E-5	²³⁵ U	115.45(5)	0.17(6) s⁻¹g⁻¹	Abundant
²³² Th	3276.3(4)	0.0063(10)	8.2(13)E-5	²³⁵ U	120.35(5)	0.1500(11) s⁻¹g⁻¹	Abundant
²³² Th	3287.94(14)	0.0165(14)	2.15(18)E-4	²³⁸ U	127.301(5)	0.0099(20)	1.26(25)E-4
²³² Th	3294.9(3)	0.0051(9)	6.7(12)E-5	²³⁸ U	133.7990(10)	0.38(8)	0.0048(10)
²³² Th	3326.21(17)	0.0102(10)	1.33(13)E-4	²³⁵ U	136.55(5)	0.0690(5) s⁻¹g⁻¹	Abundant
²³² Th	3341.90(13)	0.0168(13)	2.19(17)E-4	²³⁵ U	140.76(4)	1.27(12) s⁻¹g⁻¹	Abundant
²³² Th	3363.3(3)	0.0051(8)	6.7(10)E-5	²³⁵ U	143.760(20)	63.0(7) s⁻¹g⁻¹	Abundant
²³² Th	3377.84(13)	0.0135(12)	1.76(16)E-4	²³⁵ U	150.930(20)	0.46(6) s⁻¹g⁻¹	Abundant
²³² Th	3391.3(3)	0.0044(8)	5.7(10)E-5	²³⁵ U	163.330(20)	29.2(3) s⁻¹g⁻¹	Abundant
²³² Th	3398.09(13)	0.0191(14)	2.49(18)E-4	²³⁸ U	169.089(10)	0.012(4)	1.5(5)E-4
²³² Th	3436.17(12)	0.0211(15)	0.000276(20)	²³⁵ U	182.61(5)	1.96(12) s⁻¹g⁻¹	Abundant
²³² Th	3448.42(10)	0.0233(16)	0.000304(21)	²³⁵ U	185.715(5)	329(4) s⁻¹g⁻¹	Abundant
²³² Th	3461.45(24)	0.0069(10)	9.0(13)E-5	²³⁸ U	193.956(15)	0.0039(20)	5.0(25)E-5
²³² Th	3473.00(8)	0.057(3)	0.00074(4)	²³⁵ U	194.940(10)	3.62(7) s⁻¹g⁻¹	Abundant
²³² Th	3502.4(3)	0.0049(9)	6.4(12)E-5	²³⁵ U	198.900(20)	0.24(4) s⁻¹g⁻¹	Abundant
²³² Th	3509.43(14)	0.0170(14)	2.22(18)E-4	²³⁵ U	202.110(20)	6.21(13) s⁻¹g⁻¹	Abundant
²³² Th	3524.9(5)	0.0120(12)	1.57(16)E-4	²³⁵ U	205.311(10)	28.8(4) s⁻¹g⁻¹	Abundant
²³² Th	3530.96(13)	0.0397(24)	0.00052(3)	²³⁸ Np ^d	209.7530(20)d	0.0909(13)	0.001157[<0.1%]
²³² Th	3548.5(3)	0.0038(8)	5.0(10)E-5	²³⁵ U	215.28(3)	0.167(17) s⁻¹g⁻¹	Abundant
²³² Th	3602.66(19)	0.0119(10)	1.55(13)E-4	²³⁵ U	221.380(20)	0.69(6) s⁻¹g⁻¹	Abundant
²³² Th	3614.88(23)	0.0057(7)	7.4(9)E-5	²³⁸ Np ^d	228.1830(10)d	0.286(5)	0.00364[<0.1%]
²³² Th	3635.17(20)	0.0073(8)	9.5(10)E-5	²³⁵ U	228.78(5)	0.0400(3) s⁻¹g⁻¹	Abundant
²³² Th	3653.0(4)	0.0034(6)	4.4(8)E-5	²³⁵ U	233.50(3)	0.17(3) s⁻¹g⁻¹	Abundant
²³² Th	3712.29(24)	0.0049(6)	6.4(8)E-5	²³⁵ U	240.87(3)	0.43(4) s⁻¹g⁻¹	Abundant
²³² Th	3724.86(16)	0.0086(8)	1.12(10)E-4	²³⁵ U	243.60(20)	0.023(3)	0.00029(4)
²³² Th	3735.59(12)	0.0115(9)	1.50(12)E-4	²³⁵ U	246.84(4)	0.305(17) s⁻¹g⁻¹	Abundant
²³² Th	3746.40(16)	0.0072(7)	9.4(9)E-5	²³⁸ U	250.062(7)	0.034(12)	0.00043(15)
²³² Th	3755.05(13)	0.0098(9)	1.28(12)E-4	²³⁵ U	275.129	0.30(3) s⁻¹g⁻¹	Abundant
²³² Th	3802.96(17)	0.0071(7)	9.3(9)E-5	²³⁵ U	275.43(10)	0.40(12) s⁻¹g⁻¹	Abundant
²³² Th	3861.50(22)	0.0057(7)	7.4(9)E-5	²³⁸ Np ^d	277.5990(10)d	0.382(6)	0.00486[<0.1%]
²³² Th	3946.42(10)	0.0268(15)	0.000350(20)				
²³² Th	3971.83(22)	0.0041(5)	5.4(7)E-5				
²³² Th	4016.6(3)	0.0037(6)	4.8(8)E-5				

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.3. ADOPTED PROMPT AND DECAY GAMMA RAYS FROM THERMAL NEUTRON CAPTURE FOR ALL ELEMENTS (cont.)

^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀	^A Z	E _γ (keV)	σ _γ ^Z (E _γ) (b)	k ₀
²³⁵ U	289.56(4)	0.0400(3) s⁻¹g⁻¹	Abundant	²³⁸ U	853.23(4)	0.055(12)	0.00070(15)
²³⁵ U	291.65(3)	0.23(3) s⁻¹g⁻¹	Abundant	²³⁸ U	893.30(10)	0.016(4)	2.0(5)E-4
²³⁸ U	292.5870(20)	0.016(6)	2.0(8)E-4	²³⁵ U	909.06(6)	0.026(4)	0.00033(5)
²³⁵ UP ^p	297.00(10)	0.220(20)	0.00280(25)	²³⁵ U	943.14(7)	0.082(10)	0.00104(13)
²³⁵ U	300.00(10)	0.016(3)	2.0(4)E-4	²³⁸ U	961.06(4)	0.0039(20)	5.0(25)E-5
²³⁸ Np ^d	315.880(3)d	0.0425(8)	0.000541[<0.1%]	²³⁸ U	990.49(3)	0.010(4)	1.3(5)E-4
²³⁸ Np ^d	334.3100(20)d	0.0550(8)	0.000700[<0.1%]	²³⁸ U	1007.03(6)	0.0079(20)	1.01(25)E-4
²³⁵ U	345.90(3)	0.23(3) s⁻¹g⁻¹	Abundant	²³⁸ U	1007.03(6)	0.0079(20)	1.01(25)E-4
²³⁵ U	387.82(3)	0.23(3) s⁻¹g⁻¹	Abundant	²³⁵ U	1014.1(10)	0.026(4)	0.00033(5)
²³⁸ U	451.213(23)	0.010(4)	1.3(5)E-4	²³⁸ U	1021.25(4)	0.0079(20)	1.01(25)E-4
²³⁸ U	478.79(8)	0.012(4)	1.5(5)E-4	²³⁸ U	1021.25(4)	0.0079(20)	1.01(25)E-4
²³⁸ U	496.753(11)	0.034(8)	0.00043(10)	²³⁸ U	1029.32(5)	0.037(8)	0.00047(10)
²³⁸ U	521.849(7)	0.073(3)	0.00093(4)	²³⁸ U	1048.85(8)	0.012(4)	1.5(5)E-4
²³⁸ U	535.45(5)	0.028(6)	0.00036(8)	²³⁸ U	1060.82(8)	0.016(4)	2.0(5)E-4
²³⁸ U	537.26(3)	0.0079(20)	1.01(25)E-4	²³⁸ U	1062.48(6)	0.0079(20)	1.01(25)E-4
¹³⁹ Ba ^d	537.261(9)d	0.066(3)	0.00084[<0.1%]	²³⁸ U	1066.82(12)	0.030(6)	0.00038(8)
²³⁸ U	539.278(12)	0.099(20)	0.00126(25)	²³⁸ U	1089.50(5)	0.014(4)	1.8(5)E-4
²³⁸ U	542.085(12)	0.024(6)	0.00031(8)	²³⁸ U	1110.27(6)	0.010(4)	1.3(5)E-4
²³⁸ U	552.069(5)	0.207(5)	0.00264(6)	²³⁸ U	1149.8(3)	0.010(4)	1.3(5)E-4
²³⁸ U	554.054(8)	0.085(20)	0.00108(25)	²³⁸ U	1152.80(6)	0.010(4)	1.3(5)E-4
²³⁸ U	554.10(8)	0.028(6)	0.00036(8)	²³⁸ U	1155.05(4)	0.010(4)	1.3(5)E-4
²³⁸ U	562.027(22)	0.032(10)	0.00041(13)	²³⁵ UP	1279.01(10)	0.200(10)	0.00255(13)
²³⁸ U	563.17(3)	0.014(4)	1.8(5)E-4	²³⁸ U	2998.5(5)	0.012(4)	1.5(5)E-4
²³⁸ U	580.340(13)	0.043(10)	0.00055(13)	²³⁸ U	3089.4(5)	0.0071(24)	9(3)E-5
²³⁸ U	582.034(9)	0.016(4)	2.0(5)E-4	²³⁸ U	3114.2(5)	0.007(3)	9(4)E-5
²³⁸ U	588.88(3)	0.024(6)	0.00031(8)	²³⁸ U	3121.7(5)	0.008(3)	1.0(4)E-4
²³⁸ U	590.39(3)	0.034(12)	0.00043(15)	²³⁸ U	3175.2(5)	0.0067(22)	9(3)E-5
²³⁸ U	592.309(13)	0.045(12)	0.00057(15)	²³⁸ U	3191.7(5)	0.0047(16)	6.0(20)E-5
²³⁸ U	593.612(5)	0.108(24)	0.0014(3)	²³⁸ U	3197.2(5)	0.016(6)	2.0(8)E-4
²³⁸ U	600.284(10)	0.030(8)	0.00038(10)	²³⁸ U	3220.1(5)	0.012(4)	1.5(5)E-4
²³⁸ U	605.581(9)	0.053(12)	0.00067(15)	²³⁸ U	3233.2(5)	0.010(3)	1.3(4)E-4
²³⁸ U	611.38(3)	0.014(4)	1.8(5)E-4	²³⁸ U	3286.12(20)	0.0040(3)	5.1(4)E-5
²³⁸ U	612.253(5)	0.23(5)	0.0029(6)	²³⁸ U	3296.5(3)	0.0070(5)	8.9(6)E-5
²³⁸ U	629.722(9)	0.073(20)	0.00093(25)	²³⁸ U	3312.8(5)	0.0040(10)	5.1(13)E-5
²³⁸ U	638.505(12)	0.041(12)	0.00052(15)	²³⁸ U	3445.44(6)	0.0045(3)	5.7(4)E-5
²³⁸ U	669.385(13)	0.0039(20)	5.0(25)E-5	²³⁸ U	3564.45(9)	0.0042(4)	5.3(5)E-5
²³⁸ U	673.307(12)	0.010(4)	1.3(5)E-4	²³⁸ U	3583.10(7)	0.042(3)	0.00053(4)
²³⁸ U	681.355(9)	0.012(4)	1.5(5)E-4	²³⁸ U	3611.78(9)	0.0146(10)	1.86(13)E-4
²³⁸ U	687.853(8)	0.028(8)	0.00036(10)	²³⁸ U	3639.39(6)	0.0122(8)	1.55(10)E-4
²³⁸ U	689.907(11)	0.043(10)	0.00055(13)	²³⁸ U	3651.36(6)	0.0069(5)	8.8(6)E-5
²³⁸ U	715.832(9)	0.022(6)	0.00028(8)	²³⁸ U	3739.59(13)	0.0038(3)	4.8(4)E-5
²³⁸ U	767.86(21)	0.020(6)	0.00025(8)	²³⁸ U	3844.56(21)	0.0068(5)	8.7(6)E-5
²³⁸ U	787.15(7)	0.020(6)	0.00025(8)	²³⁸ U	3982.69(5)	0.0259(14)	0.000330(18)
²³⁸ U	794.21(8)	0.020(6)	0.00025(8)	²³⁸ U	3991.25(5)	0.0241(12)	0.000307(15)
²³⁸ U	799.12(7)	0.0079(20)	1.01(25)E-4	²³⁸ U	4060.35(5)	0.186(3)	0.00237(4)
²³⁸ U	819.868(21)	0.010(4)	1.3(5)E-4	²³⁸ U	4067.02(5)	0.0073(4)	9.3(5)E-5
²³⁸ U	828.04(21)	0.024(6)	0.00031(8)				
²³⁸ U	831.837(19)	0.053(12)	0.00067(15)				
²³⁸ U	842.42(8)	0.024(6)	0.00031(8)				

^d Decay or fission product.

^p Prompt fission to ¹³⁴Te.

'Abundant': See explanation in Section 7.6.2 in the text.

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁵⁶ Fe	14.411(14)	0.149(3)	0.00809(16)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
⁷¹ Ga	16.43(3)	0.078(5)	0.00339(22)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁵¹ V	17.152(6)	0.260(20)	0.0155(12)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁹³ Nb	17.810(7)	0.0579(14)	0.00189(5)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹¹⁵ In	22.796(7)	7(3)	0.18(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁵⁵ Mn	26.560(20)	3.42(4)	0.1887(22)	846.754(13.10), 1810.72(3.62), 83.884(3.11)
¹²⁷ I	27.3620(10)	0.43(4)	0.0103(10)	133.6110(1.42), 442.901(0.600), 58.1100(0.28)
¹⁵⁹ Tb	29.0170(20)	0.21(4)	0.0040(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁸¹ Br	29.1130(10)	0.1680(20)	0.00637(8)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
³⁹ K	29.8300(10)	1.380(20)	0.1070(16)	770.3050(0.903), 1158.887(0.1600), 5380.018(0.146)
¹³⁹ La	29.9640(10)	0.169(8)	0.00369(17)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹³⁹ Ba	29.9660(10)d	0.0381(11)	0.000485[0.1%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²⁷ Al	30.6380(10)	0.0798(20)	0.00896(22)	1778.92(0.232), 7724.027(0.0493), 3033.896(0.0179)
¹⁵⁹ Tb	32.652(3)	0.19(3)	0.0036(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁵⁹ Tb	33.1590(10)	0.22(4)	0.0042(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁷⁹ Br	37.0520(20)d	0.428(12)	0.0162[7.4%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁷⁹ Br	37.054(3)	0.160(10)	0.0061(4)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹²³ Sb	40.8040(10)	0.10(3)	0.0025(8)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁷⁴ Yb	41.2180(20)	1.1(3)	0.019(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁵⁹ Tb	41.8900(10)	0.64(10)	0.0122(19)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
²³⁸ U	43.5330(10)d	0.110(3)	0.00140[53%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷⁵ As	44.4250(10)	0.560(20)	0.0227(8)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
⁷⁵ As	46.0980(10)	0.337(15)	0.0136(6)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁸² W	46.4840(10)	0.192(10)	0.00316(16)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁷⁴ Yb	46.7510(20)	0.25(8)	0.0044(14)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁹¹ Ir	48.0570(10)	5.7(4)	0.090(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁵¹ Eu	48.31(17)	181(70)	3.6(14)	89.847(1430), 77.23(187)
¹³³ Cs	48.790(20)	0.345(10)	0.00787(23)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁶⁴ Dy	50.4310(20)	33.9(15)	0.63(3)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁵⁹ Tb	50.8690(10)	0.60(15)	0.011(3)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁰³ Rh	51.50(3)	16.0(4)	0.471(12)	180.87(22.6), 97.14(19.5), 217.82(7.38)
¹⁰³ Rh	51.50(3)d	5.2(3)	0.153[90%]	180.87(22.6), 97.14(19.5), 51.50(16.0)
⁴⁵ Sc	52.0110(10)	0.87(3)	0.0586(20)	227.773(7.13), 147.011(6.08), 142.528(4.88)
¹²⁷ I	52.385(3)	0.167(19)	0.0040(5)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
¹⁸² W	52.5290(10)	0.128(11)	0.00211(18)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁵⁹ Tb	54.1290(10)	0.60(15)	0.011(3)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹³⁹ La	54.9440(10)	0.143(7)	0.00312(15)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁹⁷ Au	55.1810(10)	2.90(12)	0.0446(18)	410(94), 214.9710(9.0), 247.5730(5.56)
¹²⁷ I	58.1100(20)	0.28(4)	0.0067(10)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
¹⁹¹ Ir	58.8440(10)	5.3(3)	0.084(5)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁵ Re	59.0100(20)	5.5(8)	0.090(13)	63.5820(8.0), 155.041(7.16), 137.157(5.29)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁸⁶ W	59.03(4)	0.208(7)	0.00343(12)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁷⁹ Br	59.471(4)	0.202(5)	0.00766(19)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁵⁹ Tb	59.6430(10)	0.48(6)	0.0092(11)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁸⁵ Rb	59.75(6)	0.010(4)	0.00035(14)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹³³ Cs	60.0300(10)	0.443(14)	0.0101(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁴¹ Pr	60.0630(20)	0.134(14)	0.0029(3)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹¹⁵ In	60.9160(10)	15.8(11)	0.42(3)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹²¹ Sb	61.4130(10)	0.75(18)	0.019(5)	564.24(2.700), 78.0910(0.48), 121.4970(0.40)
¹⁷⁷ Hf	62.820(21)	5.26(16)	0.089(3)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹³⁹ La	63.1790(10)	0.208(8)	0.00454(17)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁸⁷ Re	63.5820(20)	8.0(14)	0.130(23)	155.041(7.16), 59.0100(5.5), 137.157(5.29)
¹⁵⁹ Tb	63.6860(10)	1.46(16)	0.028(3)	75.0500(1.78), 64.1100(1.2), 41.8900(0.64)
¹⁵⁹ Tb	64.1100(20)	1.2(3)	0.023(6)	75.0500(1.78), 63.6860(1.46), 41.8900(0.64)
¹⁴¹ Pr	64.5050(20)	0.137(6)	0.00295(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁹¹ Ir	66.822(8)	1.31(13)	0.0207(20)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁴¹ Pr	68.6110(20)	0.116(6)	0.00249(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁶⁹ Tm	68.649	1.75(23)	0.031(4)	200(8.72), 149.7180(7.11), 140(5.96)
¹²¹ Sb	71.4670(10)	0.095(22)	0.0024(6)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁷⁵ Lu	71.5170(10)	3.96(22)	0.069(4)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸⁶ W	72.002(4)d	1.32(3)	0.0218[1.4%]	685.73(3.24), 479.550(2.59), 134.247(1.050)
¹⁰⁹ Ag	72.67(5)	0.9(15)	0.03(4)	198.72(7.75), 235.62(4.62), 78.91(3.90)
²³⁸ U	74.6640(10)d	1.30000(14)	0.0165511[53%]	106.1230(0.723), 277.5990(0.382), 133.7990(0.38)
¹⁸⁷ Re	74.8630(20)	1.29(8)	0.0210(13)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁷⁵ As	74.8720(10)	0.12(3)	0.0049(12)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁵⁹ Tb	75.0500(10)	1.78(18)	0.034(3)	63.6860(1.46), 64.1100(1.2), 41.8900(0.64)
¹⁶⁹ Tm	75.83	0.94(8)	0.0169(14)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁷³ Yb	76.996	0.40(4)	0.0070(7)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²³² Th	77.09(15)	0.09(3)	0.0012(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁵¹ Eu	77.23(4)	187(13)	3.7(3)	89.847(1430), 48.31(181)
¹⁸⁶ W	77.39(3)	0.134(5)	0.00221(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁹¹ Ir	77.9470(10)	4.8(4)	0.076(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
³¹ P	78.083(20)	0.059(3)	0.0058(3)	512.646(0.079), 636.663(0.0311), 3899.89(0.0294)
¹²¹ Sb	78.0910(10)	0.48(11)	0.012(3)	564.24(2.700), 61.4130(0.75), 121.4970(0.40)
¹⁷¹ Yb	78.7430(10)	0.67(10)	0.0117(18)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁵⁹ Tb	78.8670(10)	0.19(4)	0.0036(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁰⁷ Ag	78.91(4)	3.90(12)	0.110(3)	198.72(7.75), 235.62(4.62), 117.45(3.85)
¹⁵⁹ Tb	79.099(6)	0.43(6)	0.0082(11)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁵⁷ Gd	79.5100(10)	4010(100)	77.3(19)	181.931(7200), 944.174(3090), 962.104(2050)
¹⁶⁷ Er	79.8040(10)	18.2(8)	0.330(14)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
¹⁰⁹ Ag	79.91(6)	1.0(16)	0.03(5)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁶⁵ Ho	80.574(8)d	3.87(5)	0.0711[1.3%]	136.6650(14.5), 116.8360(8.1), 426.012(2.88)
¹⁶¹ Dy	80.64(7)	16.5(5)	0.308(9)	184.257(146), 538.609(69.2), 496.931(44.9)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁹⁷ Au	82.3560(10)	2.3(4)	0.035(6)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁹⁷ Au	82.5240(10)	1.4(3)	0.022(5)	410(94), 214.9710(9.0), 247.5730(5.56)
⁵⁵ Mn	83.884(23)	3.11(5)	0.172(3)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
¹⁹¹ Ir	84.2740(20)	7.7(4)	0.121(6)	351.689(10.9), 328.448(9.1), 136.1250(6.5)
¹⁴¹ Pr	84.998(3)	0.207(11)	0.00445(24)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁰³ Rh	85.19(3)	3.2(3)	0.094(9)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹¹⁵ In	85.5690(20)	22.1(16)	0.58(4)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁷³ Yb	86.11(7)	0.164(18)	0.0029(3)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁷⁵ As	86.7880(10)	0.579(11)	0.0234(4)	559.10(2.00), 165.0490(0.996), 44.4250(0.560)
¹⁸⁵ Re	87.264(3)	0.84(4)	0.0137(7)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁶⁹ Tm	87.5210(10)	1.29(3)	0.0231(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹²³ Sb	87.601	0.212(8)	0.00528(20)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁷⁴ Yb	87.9690(20)	0.26(6)	0.0046(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹²¹ Sb	88.2690(10)	0.083(19)	0.0021(5)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁹¹ Ir	88.7340(10)	3.67(24)	0.058(4)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁵⁵ Gd	88.9670(10)	1380(40)	26.6(8)	181.931(7200), 79.5100(4010), 944.174(3090)
⁶⁵ Cu	89.08(4)	0.0970(17)	0.00463(8)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
¹⁵⁹ Tb	89.4080(20)	0.21(3)	0.0040(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁵¹ Eu	89.847(6)	1430(30)	28.5(6)	77.23(187), 48.31(181)
¹⁹¹ Ir	90.7030(20)	1.25(15)	0.0197(24)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
²³ Na	90.9920(10)	0.235(3)	0.0310(4)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹⁸⁷ Re	92.4640(20)	1.07(6)	0.0174(10)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁷⁷ Hf	93.182(6)	13.3(9)	0.226(15)	213.439(29.3), 214.3410(16.3), 325.559(6.69)
¹⁵⁹ Tb	93.3060(20)	0.218(25)	0.0042(5)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁷⁴ Yb	95.2730(20)	0.20(5)	0.0035(9)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹¹⁵ In	96.036(5)	11.4(14)	0.30(4)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹¹⁵ In	96.062(3)	24.6(18)	0.65(5)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁰³ Rh	97.14(3)	19.5(4)	0.574(12)	180.87(22.6), 51.50(16.0), 217.82(7.38)
¹⁹⁷ Au	97.2500(20)	2.1(5)	0.032(8)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁵⁹ Tb	97.503(3)	0.50(6)	0.0095(11)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁸² W	99.0790(10)	0.155(13)	0.00256(21)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁹³ Nb	99.4070(10)	0.196(9)	0.0064(3)	255.9290(0.176), 253.115(0.1320), 113.4010(0.117)
¹⁰³ Rh	100.74(4)	4.96(10)	0.146(3)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁹⁷ Au	101.9390(10)	0.953(17)	0.0147(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁷³ Yb	102.60(5)	0.44(5)	0.0077(9)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁷¹ Ga	103.25(3)d	0.0526(11)	0.00229[100%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁷⁴ Yb	104.5260(20)	0.43(11)	0.0075(19)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵⁵ Mn	104.611(23)	1.74(3)	0.0960(17)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
¹²¹ Sb	105.8160(10)	0.21(5)	0.0052(12)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁸⁷ Re	105.8620(20)	1.77(8)	0.0288(13)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁰⁹ Ag	105.95(6)	0.87(13)	0.024(4)	198.72(7.75), 235.62(4.62), 78.91(3.90)
²³⁸ Np	106.1230(20)d	0.723(11)	0.00920[0.6%]	74.6640(1.30000), 277.5990(0.382), 133.7990(0.38)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁸² W	107.9320(10)	0.144(12)	0.00237(20)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁹¹ Ir	108.0300(20)	2.62(12)	0.0413(19)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸³ W	111.216(9)	0.195(6)	0.00321(10)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁹³ Ir	112.2310(10)	1.7(4)	0.027(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷¹ Ga	112.36(3)	0.155(3)	0.00674(13)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁷⁶ Lu	112.9500(10)d	3.47(16)	0.060[0.2%]	150.392(13.8), 457.944(8.3), 138.607(6.79)
⁹³ Nb	113.4010(10)	0.117(3)	0.00382(10)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹³³ Cs	113.7650(20)	0.777(15)	0.0177(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁷⁴ Yb	113.805(4)d	0.417(14)	0.00730[0.3%]	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁸¹ Ta	114.3150(10)	0.280(9)	0.00469(15)	270.4030(2.60), 173.2050(1.210), 402.623(1.180)
¹⁶⁹ Tm	114.544	3.19(6)	0.0572(11)	200(8.72), 149.7180(7.11), 140(5.96)
¹²¹ Sb	114.8680(10)	0.31(7)	0.0077(17)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
⁶⁴ Zn	115.225(18)	0.167(3)	0.00774(14)	1077.335(0.356), 7863.55(0.1410), 1883.12(0.0718)
¹³³ Cs	116.3740(20)	1.39(12)	0.032(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹³³ Cs	116.612(4)	1.44(12)	0.033(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁷⁵ As	116.7550(10)	0.107(18)	0.0043(7)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁶⁵ Ho	116.8360(10)	8.1(4)	0.149(7)	136.6650(14.5), 80.574(3.87), 426.012(2.88)
¹⁰⁹ Ag	117.45(8)	3.85(7)	0.1082(20)	198.72(7.75), 235.62(4.62), 78.91(3.90)
⁷⁵ As	120.2580(10)	0.402(8)	0.0163(3)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹³³ Cs	120.588(3)	0.414(10)	0.00944(23)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹²¹ Sb	121.4970(10)	0.40(9)	0.0100(22)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁷⁶ Lu	121.620(3)	5.24(17)	0.091(3)	150.392(13.8), 457.944(8.3), 138.607(6.79)
⁵⁶ Fe	122.077(14)	0.096(3)	0.00521(16)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
⁷⁵ As	122.2470(10)	0.227(5)	0.00918(20)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹²⁷ I	124.2810(20)	0.180(13)	0.0043(3)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
⁵¹ V	124.453(4)	0.23(5)	0.014(3)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁵¹ V	125.082(3)	1.61(4)	0.0958(24)	1434.10(4.81), 6517.282(0.78), 645.703(0.769)
¹¹⁵ In	126.3720(20)	4.0(3)	0.106(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁴¹ Pr	126.8460(20)	0.307(15)	0.0066(3)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁹¹ Ir	126.958(3)	1.86(10)	0.0293(16)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁰³ Rh	127.20(3)	5.27(21)	0.155(6)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁸⁶ W	127.43(4)	0.129(5)	0.00213(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹³³ Cs	127.5000(20)d	0.310(11)	7.1E-03[11%]	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁶⁹ Tm	130.027	0.940(25)	0.0169(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹³³ Cs	130.2320(20)	1.410(21)	0.0322(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹²⁷ I	133.6110(10)	1.42(10)	0.0339(24)	442.901(0.600), 27.3620(0.43), 58.1100(0.28)
²³⁸ U	133.7990(10)	0.38(8)	0.0048(10)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁸¹ Ta	133.8770(20)	0.63(7)	0.0106(12)	270.4030(2.60), 173.2050(1.210), 402.623(1.180)
¹⁸⁶ W	134.247(7)d	1.050(20)	0.0173[1.4%]	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁰³ Rh	134.54(3)	6.8(4)	0.200(12)	180.87(22.6), 97.14(19.5), 51.50(16.0)
⁷⁵ As	135.4110(10)	0.156(4)	0.00631(16)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁵⁹ Tb	135.5970(20)	0.39(4)	0.0074(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁹¹ Ir	136.1250(10)	6.5(9)	0.102(14)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹¹ Ir	136.213(3)	4.0(5)	0.063(8)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁶⁵ Ho	136.6650(20)	14.5(7)	0.266(13)	116.8360(8.1), 80.574(3.87), 426.012(2.88)
¹⁹¹ Ir	136.7910(10)	2.20(21)	0.035(3)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁵ Re	137.157(8)d	5.29(3)	0.0861[0.4%]	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹¹⁵ In	138.326(8)d	5.11(18)	0.135[30%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁷⁶ Lu	138.607(5)	6.79(24)	0.118(4)	150.392(13.8), 457.944(8.3), 208.3660(6.0)
⁷⁶ Se	139.2270(10)	0.543(9)	0.0208(4)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
²⁰³ Tl	139.94(9)	0.400(7)	0.00593(10)	347.96(0.361), 318.88(0.325), 5641.57(0.316)
¹⁴¹ Pr	140.9050(20)	0.479(10)	0.01030(22)	176.8630(1.06), 1575.6(0.426), 5666.170(0.379)
¹⁸⁷ Re	141.760(4)	1.46(8)	0.0238(13)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁴⁵ Sc	142.528(8)d	4.88(7)	0.329[99%]	227.773(7.13), 147.011(6.08), 295.243(3.97)
¹⁸⁵ Re	144.152(5)	1.8(3)	0.029(5)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁶⁹ Tm	144.4790(10)	1.2(4)	0.022(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁶⁹ Tm	144.48	5.96(11)	0.1069(20)	200(8.72), 149.7180(7.11), 237.2390(5.52)
⁷⁵ As	144.5480(10)	0.1000(22)	0.00404(9)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁹¹ Ir	144.903(5)	3.1(4)	0.049(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷¹ Ga	145.14(3)	0.466(7)	0.0203(3)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁸⁶ W	145.79(3)	0.970(21)	0.0160(4)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁷⁶ Lu	145.870(4)	1.52(9)	0.0263(16)	150.392(13.8), 457.944(8.3), 138.607(6.79)
⁴⁵ Sc	147.011(10)	6.08(9)	0.410(6)	227.773(7.13), 142.528(4.88), 295.243(3.97)
¹⁷⁶ Lu	147.165(5)	4.96(19)	0.086(3)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁷⁶ Lu	147.167(5)	3.7(7)	0.064(12)	150.392(13.8), 457.944(8.3), 138.607(6.79)
⁵¹ V	147.846(3)	0.253(6)	0.0151(4)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹²¹ Sb	148.238	0.26(6)	0.0065(15)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁹³ Ir	148.9340(10)	1.4(9)	0.022(14)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁶⁵ Ho	149.309(3)	2.25(12)	0.0413(22)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
¹⁶⁹ Tm	149.7180(10)	7.11(12)	0.1275(22)	200(8.72), 140(5.96), 237.2390(5.52)
¹⁷⁶ Lu	150.392(3)	13.8(4)	0.239(7)	457.944(8.3), 138.607(6.79), 208.3660(6.0)
¹⁹¹ Ir	151.5640(20)	2.89(20)	0.046(3)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁵ Re	151.688(3)	1.15(7)	0.0187(11)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹²⁷ I	153.011(3)	0.209(14)	0.0050(3)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
¹⁵⁹ Tb	153.6870(20)	0.44(5)	0.0084(10)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
²⁰³ Tl	154.01(9)	0.0926(17)	0.001373(25)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁸⁷ Re	155.041(4)d	7.16(25)	0.117[2.0%]	63.5820(8.0), 59.0100(5.5), 137.157(5.29)
¹⁸⁷ Os	155.10(4)	1.19(3)	0.0190(5)	186.7180(2.08), 557.978(0.84), 569.344(0.694)
¹²³ Sb	155.1780(10)	0.081(9)	0.00202(22)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹³⁹ La	155.560(5)	0.192(7)	0.00419(15)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁹¹ Ir	156.654(3)	2.76(12)	0.0435(19)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷⁵ As	157.7450(10)	0.117(24)	0.0047(10)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁹⁷ Au	158.4360(10)	1.250(18)	0.0192(3)	410(94), 214.9710(9.0), 247.5730(5.56)
⁵⁹ Co	158.517(17)	1.200(15)	0.0617(8)	229.879(7.18), 277.161(6.77), 555.972(5.76)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹¹⁶ Sn	158.65(6)	0.0145(3)	0.000370(8)	1293.591(0.1340), 1171.28(0.0879), 1229.64(0.0673)
⁶³ Cu	159.281(5)	0.648(10)	0.0309(5)	278.250(0.893), 7915.62(0.869), 7637.40(0.54)
¹²⁷ I	160.7570(10)	0.187(16)	0.0045(4)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
⁷⁶ Se	161.9220(10)d	0.855(23)	0.0328[99%]	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
²⁰⁹ Bi	162.19(11)	0.008(3)	1.2E-04(4)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
¹⁸² W	162.315(8)	0.187(5)	0.00308(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹¹⁵ In	162.393(3)d	15.8(8)	0.417[100%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁷⁶ Lu	162.492(4)	5.32(17)	0.092(3)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹³⁹ La	162.659(3)	0.489(18)	0.0107(4)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁷⁵ As	165.0490(10)	0.996(16)	0.0403(7)	559.10(2.00), 86.7880(0.579), 44.4250(0.560)
¹⁶⁹ Tm	165.735	3.29(6)	0.0590(11)	200(8.72), 149.7180(7.11), 140(5.96)
¹³⁸ Ba	165.8570(10)d	0.074(8)	0.00163[21%]	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹⁹ F	166.700(20)	0.000413(18)	6.6E-05(3)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁴⁰ Ar	167.30(20)	0.53(5)	0.040(4)	4745.3(0.36), 1186.8(0.34), 516.0(0.167)
¹⁸⁷ Re	167.327(3)	1.46(6)	0.0238(10)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁹⁷ Au	168.3340(10)	3.60(22)	0.055(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁰³ Rh	169.16(5)	2.88(19)	0.085(6)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁹¹ Ir	169.196(3)	3.05(13)	0.0481(20)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹⁷ Au	170.1030(10)	1.66(22)	0.026(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹¹⁵ In	171.059(5)	3.44(25)	0.091(7)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁷⁶ Lu	171.869(7)	1.74(6)	0.0301(10)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸¹ Ta	173.2050(20)	1.210(25)	0.0203(4)	270.4030(2.60), 402.623(1.180), 133.8770(0.63)
¹¹⁵ In	173.886(6)	4.1(3)	0.108(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹³³ Cs	174.3040(20)	0.420(11)	0.00958(25)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁷⁰ Ge	175.05(3)	0.164(4)	0.00684(17)	595.851(1.100), 867.899(0.553), 608.353(0.250)
¹⁷³ Yb	175.30(5)	0.58(6)	0.0102(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹³³ Cs	176.4040(20)	2.47(4)	0.0563(9)	205.615(1.560), 510.795(1.54), 307.015(1.45)
¹⁴¹ Pr	176.8630(20)	1.06(4)	0.0228(9)	140.9050(0.479), 1575.6(0.426), 5666.170(0.379)
¹⁰³ Rh	178.66(4)	3.27(14)	0.096(4)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁵⁹ Tb	178.881(3)	0.42(8)	0.0080(15)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁹¹ Ir	179.0380(20)	2.1(5)	0.033(8)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁰³ Rh	180.87(3)	22.6(15)	0.67(4)	97.14(19.5), 51.50(16.0), 217.82(7.38)
¹⁶⁹ Tm	180.993	3.85(14)	0.0691(25)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁷¹ Yb	181.529(3)	0.53(6)	0.0093(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁵⁷ Gd	181.931(4)	7200(300)	139(6)	79.5100(4010), 944.174(3090), 962.104(2050)
¹⁴¹ Pr	182.786(4)	0.377(14)	0.0081(3)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
⁷¹ Ga	184.09(3)	0.1040(21)	0.00452(9)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁶⁴ Dy	184.257(4)	146(15)	2.7(3)	538.609(69.2), 496.931(44.9), 185.19(39.1)
¹⁶⁷ Er	184.2850(10)	56(5)	1.01(9)	815.9890(42.5), 198.2440(29.9), 79.8040(18.2)
¹⁶¹ Dy	185.19(9)	39.1(12)	0.729(22)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁷⁶ Lu	185.593(8)	3.42(12)	0.0592(21)	150.392(13.8), 457.944(8.3), 138.607(6.79)
⁶⁵ Cu	185.96(4)	0.244(3)	0.01164(14)	278.250(0.893), 7915.62(0.869), 159.281(0.648)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹¹⁵ In	186.2100(20)	26.6(18)	0.70(5)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁸⁹ Os	186.7180(20)	2.08(5)	0.0331(8)	155.10(1.19), 557.978(0.84), 569.344(0.694)
¹³³ Cs	186.8400(20)	0.282(9)	0.00643(21)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁶⁹ Ga	187.84(3)	0.1080(21)	0.00469(9)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁷⁶ Lu	187.970(23)	1.39(6)	0.0241(10)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸⁷ Re	188.813(6)	0.98(10)	0.0159(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁶⁸ Yb	191.2140(10)	0.22(4)	0.0039(7)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁰⁷ Ag	191.39(3)	1.81(5)	0.0509(14)	198.72(7.75), 235.62(4.62), 78.91(3.90)
⁷¹ Ga	192.11(3)	0.194(3)	0.00843(13)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁹⁷ Au	192.3920(10)	3.9(18)	0.06(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁰⁷ Ag	192.90(3)	2.20(6)	0.0618(17)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁹⁷ Au	192.9440(10)	1.70(22)	0.026(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁵⁹ Tb	193.431(4)	0.37(4)	0.0071(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁷¹ Ga	194.66(4)	0.1070(21)	0.00465(9)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁷⁹ Br	195.602(4)	0.434(14)	0.0165(5)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁸⁷ Rb	196.34(3)	0.00964(19)	0.000342(7)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
⁷¹ Ga	197.94(5)	0.1330(24)	0.00578(10)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁶⁷ Er	198.2440(10)	29.9(16)	0.54(3)	184.2850(56), 815.9890(42.5), 79.8040(18.2)
¹³³ Cs	198.3010(20)	1.100(19)	0.0251(4)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
²⁰³ Tl	198.33(8)	0.0408(10)	0.000605(15)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁶⁹ Tm	198.5260(10)	0.96(3)	0.0172(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁰⁹ Ag	198.72(4)	7.75(13)	0.218(4)	235.62(4.62), 78.91(3.90), 117.45(3.85)
¹⁵⁵ Gd	199.2130(10)	2020(60)	38.9(12)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁸⁵ Re	199.337(16)	0.91(4)	0.0148(7)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁸⁷ Re	199.513(5)	1.02(10)	0.0166(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁷⁶ Se	200.4530(20)	0.233(9)	0.0089(4)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁸⁶ W	201.44(5)	0.319(8)	0.00526(13)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹²¹ Sb	201.5950(10)	0.091(3)	0.00226(8)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
⁸⁹ Y	202.53(3)	0.289(7)	0.00985(24)	6080.171(0.76), 776.613(0.659), 574.106(0.174)
⁶³ Cu	202.950(8)	0.193(3)	0.00920(14)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
¹⁶⁹ Tm	204.448	8.72(19)	0.156(3)	149.7180(7.11), 140(5.96), 237.2390(5.52)
¹⁸⁶ W	204.83(4)	0.148(4)	0.00244(7)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹³³ Cs	205.615(3)	1.560(25)	0.0356(6)	176.4040(2.47), 510.795(1.54), 307.015(1.45)
¹⁹¹ Ir	206.220(4)	3.70(18)	0.058(3)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁰⁷ Ag	206.46(3)	3.58(7)	0.1006(20)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁸⁷ Re	207.853(4)	4.44(21)	0.072(3)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁷⁶ Lu	208.3660(10)d	6.0(3)	0.104[0.2%]	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸⁷ Re	208.843(7)	0.98(10)	0.0159(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
²³⁸ Np	209.7530(20)d	0.0909(13)	0.001157[0.6%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁹¹ Ir	210.354(5)	2.1(4)	0.033(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁵ Re	210.698(4)	1.50(10)	0.0244(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁷⁵ As	211.1470(10)	0.113(3)	0.00457(12)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁵⁵ Mn	212.039(21)	2.13(3)	0.1175(17)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁷¹ Ga	212.58(4)	0.0583(12)	0.00253(5)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁷⁷ Hf	213.439(7)	29.3(7)	0.497(12)	214.3410(16.3), 93.182(13.3), 325.559(6.69)
¹⁷⁸ Hf	214.3410(20)	5.7(6)	0.097(10)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹⁷⁸ Hf	214.3410(20)d	16.3(3)	0.277[99%]	213.439(29.3), 93.182(13.3), 325.559(6.69)
¹⁸⁵ Re	214.647(4)	2.53(14)	0.0412(23)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁹⁷ Au	214.9710(10)	9.0(12)	0.138(18)	410(94), 247.5730(5.56), 261.4040(5.3)
¹⁰⁷ Ag	215.15(4)	1.55(3)	0.0435(8)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁰³ Rh	215.340(22)	5.20(12)	0.153(4)	180.87(22.6), 97.14(19.5), 51.50(16.0)
⁴⁵ Sc	216.44(4)	2.49(4)	0.168(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
¹⁰³ Rh	216.54(8)	5.0(10)	0.15(3)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁹¹ Ir	216.905(4)	5.57(24)	0.088(4)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁰³ Rh	217.82(3)	7.38(13)	0.217(4)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹³⁹ La	218.225(22)	0.78(3)	0.0170(7)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹³³ Cs	218.341(3)	0.309(9)	0.00705(21)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁷⁹ Br	219.377(3)	0.399(14)	0.0151(5)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁶⁹ Tm	219.706	3.64(6)	0.0653(11)	200(8.72), 149.7180(7.11), 140(5.96)
¹³³ Cs	219.7530(20)	0.344(9)	0.00784(21)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁶⁵ Ho	221.186(4)	2.05(11)	0.0377(20)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
⁷⁹ Br	223.627(3)	0.153(5)	0.00580(19)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁷⁵ Lu	225.4030(10)	1.73(8)	0.0300(14)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸⁶ W	225.86(4)	0.113(17)	0.0019(3)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁹¹ Ir	226.2980(20)	4.0(4)	0.063(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁷ Re	227.083(6)	1.78(12)	0.0290(20)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁴⁵ Sc	227.773(12)	7.13(11)	0.481(7)	147.011(6.08), 142.528(4.88), 295.243(3.97)
²³⁸ Np	228.1830(10)d	0.286(5)	0.00364[0.6%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁴⁵ Sc	228.716(12)	3.31(5)	0.223(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁵⁹ Co	229.879(17)	7.18(8)	0.369(4)	277.161(6.77), 555.972(5.76), 447.711(3.41)
¹²¹ Sb	233.1690(10)	0.0996(24)	0.00248(6)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
⁷⁹ Br	234.320(3)	0.205(10)	0.0078(4)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹³³ Cs	234.3340(20)	1.070(23)	0.0244(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁶⁹ Tm	235.1890(10)	1.18(4)	0.0212(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹¹⁵ In	235.275(4)	4.9(3)	0.129(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁰⁹ Ag	235.62(4)	4.62(7)	0.1298(20)	198.72(7.75), 78.91(3.90), 117.45(3.85)
¹³⁹ La	235.771(8)	0.111(4)	0.00242(9)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁷⁵ As	235.8770(10)	0.181(4)	0.00732(16)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁹⁷ Au	236.0450(10)	4.1(5)	0.063(8)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁸⁷ Re	236.627(4)	1.45(10)	0.0236(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁰⁷ Ag	236.85(4)	1.95(3)	0.0548(8)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁰⁹ Ag	236.89(7)	1.3(9)	0.037(25)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁶⁹ Tm	237.2390(10)	5.52(10)	0.0990(18)	200(8.72), 149.7180(7.11), 140(5.96)
¹³⁹ La	237.660(4)	0.320(12)	0.0070(3)	1596.21(5.84), 487.021(2.79), 815.772(1.430)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁷⁶ Se	238.9980(10)	2.06(3)	0.0791(12)	613.724(2.14), 520.6370(1.260), 161.9220(0.855)
¹⁶⁵ Ho	239.132(4)	2.25(12)	0.0413(22)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
¹⁶⁹ Tm	242.6220(10)	1.28(4)	0.0230(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁵⁹ Tb	242.973(12)	0.219(24)	0.0042(5)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁷⁹ Br	244.237(3)	0.45(3)	0.0171(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁸¹ Br	244.8310(10)	0.15(5)	0.0057(19)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁷⁹ Br	245.203(4)	0.80(3)	0.0303(11)	776.517(0.990), 554.3480(0.838), 619.106(0.515)
¹¹⁰ Cd	245.3(3)	274(25)	7.4(7)	558.32(1860), 651.19(358)
¹³³ Cs	245.8620(20)	0.740(15)	0.0169(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁹⁷ Au	247.5730(10)	5.56(8)	0.0855(12)	410(94), 214.9710(9.0), 261.4040(5.3)
¹⁵⁹ Tb	248.062(5)	0.30(3)	0.0057(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁷¹ Ga	248.89(4)	0.136(8)	0.0059(4)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁷⁶ Se	249.7880(10)	0.538(9)	0.0206(4)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁸⁷ Re	251.243(5)	1.80(23)	0.029(4)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁸³ W	252.854(11)	0.101(3)	0.00166(5)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁹³ Nb	253.115(5)	0.1320(19)	0.00431(6)	99.4070(0.196), 255.9290(0.176), 113.4010(0.117)
⁵⁹ Co	254.379(17)	1.290(16)	0.0663(8)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁸⁵ Re	254.998(4)	1.15(5)	0.0187(8)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁹³ Nb	255.9290(20)	0.176(3)	0.00574(10)	99.4070(0.196), 253.115(0.1320), 113.4010(0.117)
²³² Th	256.25(11)	0.093(17)	0.00121(22)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁸⁵ Re	257.447(9)	0.87(23)	0.014(4)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁰⁷ Ag	259.17(3)	1.560(25)	0.0438(7)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁷⁶ Lu	259.401(16)	1.89(8)	0.0327(14)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹³³ Cs	261.1640(20)	0.401(11)	0.00914(25)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁹⁷ Au	261.4040(10)	5.3(20)	0.08(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁹¹ Ir	261.953(6)	2.02(23)	0.032(4)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹¹ Ir	262.03(10)	3.05(18)	0.048(3)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷⁵ As	263.8940(10)	0.18(4)	0.0073(16)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁰³ Rh	266.84(3)	2.66(17)	0.078(5)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁰⁹ Ag	267.08(3)	2.73(6)	0.0767(17)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁷⁶ Lu	268.788(5)	3.64(13)	0.0630(23)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁸¹ Ta	270.4030(20)	2.60(6)	0.0435(10)	173.2050(1.210), 402.623(1.180), 133.8770(0.63)
⁵⁵ Mn	271.198(22)	0.94(6)	0.052(3)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁷⁹ Br	271.374(3)	0.462(7)	0.0175(3)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹³⁹ La	272.306(4)	0.502(19)	0.0110(4)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁸⁸ Os	272.82(4)	0.242(6)	0.00386(10)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹¹⁵ In	272.9660(20)	33.1(24)	0.87(6)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁸⁶ W	273.10(5)	0.272(7)	0.00448(12)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁸⁷ Re	274.298(5)	0.80(6)	0.0130(10)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁷⁹ Br	274.532(5)	0.158(3)	0.00599(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁵⁹ Co	277.161(17)	6.77(8)	0.348(4)	229.879(7.18), 555.972(5.76), 447.711(3.41)
²³² Th	277.48(11)	0.0312(25)	0.00041(3)	583.27(0.279), 566.63(0.19), 472.30(0.165)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²³⁸ Np	277.5990(10)d	0.382(6)	0.00486[0.6%]	74.6640(1.30000), 106.1230(0.723), 133.7990(0.38)
⁶³ Cu	278.250(14)	0.893(15)	0.0426(7)	7915.62(0.869), 159.281(0.648), 7637.40(0.54)
¹⁹³ Ir	278.5040(10)	1.8(11)	0.028(17)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁷⁴ Yb	282.522(14)d	0.666(22)	0.0117[0.3%]	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹²¹ Sb	282.6500(10)	0.274(7)	0.00682(17)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
⁶⁰ Ni	282.917(18)	0.211(3)	0.01089(15)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
¹³⁶ Ba	283.58(6)	0.0404(12)	0.00089(3)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹⁹¹ Ir	284.074(6)	1.95(15)	0.0307(24)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁶⁷ Er	284.6560(20)	13.7(12)	0.248(22)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
¹¹⁵ In	284.914(4)	4.5(3)	0.119(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁷⁴ Se	286.5710(20)	0.280(6)	0.01075(23)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
⁸¹ Br	287.7390(20)	0.253(4)	0.00960(15)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹³⁹ La	288.255(5)	0.73(3)	0.0159(7)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁸⁷ Re	290.665(6)	3.5(4)	0.057(7)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁸⁷ Re	291.492(8)	0.94(7)	0.0153(11)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
¹⁹⁷ Au	291.7240(20)	1.05(17)	0.016(3)	410(94), 214.9710(9.0), 247.5730(5.56)
²⁰³ Tl	292.26(8)	0.0983(20)	0.00146(3)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁹³ Nb	293.206(4)	0.0651(16)	0.00212(5)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹⁹³ Ir	293.541(14)d	1.76(6)	0.0277[1.8%]	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷⁹ Br	294.349(3)	0.1160(22)	0.00440(8)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁰⁷ Ag	294.39(3)	2.05(12)	0.058(3)	198.72(7.75), 235.62(4.62), 78.91(3.90)
⁵¹ V	295.023(14)	0.164(4)	0.00976(24)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁴⁵ Sc	295.243(10)	3.97(11)	0.268(7)	227.773(7.13), 147.011(6.08), 142.528(4.88)
²³⁵ U	297.00(10)	0.220(20)	0.00280(25)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷⁶ Se	297.2160(20)	0.337(7)	0.0129(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹¹⁵ In	298.664(3)	9.4(7)	0.248(18)	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁰⁷ Ag	299.95(3)	1.15(5)	0.0323(14)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹²⁷ I	301.906(5)	0.17(6)	0.0041(14)	133.6110(1.42), 442.901(0.600), 27.3620(0.43)
¹⁹¹ Ir	302.905(8)	1.20(11)	0.0189(17)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁷⁸ Hf	303.9880(20)	3.38(9)	0.0574(15)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹³³ Cs	307.015(4)	1.45(3)	0.0331(7)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁹³ Nb	309.915(8)	0.0690(17)	0.00225(6)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹⁷⁵ Lu	310.1870(10)	1.49(8)	0.0258(14)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁶⁹ Tm	311.0190(10)	2.50(5)	0.0448(9)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁷⁴ Yb	311.276(5)	0.26(4)	0.0046(7)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵⁵ Mn	314.398(20)	1.460(20)	0.0805(11)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁷⁹ Br	314.982(3)	0.460(9)	0.0174(3)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
²³⁸ Np	315.880(3)d	0.0425(8)	0.000541[0.6%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁹¹ Ir	316.061(7)	2.4(4)	0.038(6)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁸⁵ Re	316.457(9)	2.21(10)	0.0360(16)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
²³² Th	316.64(10)	0.0397(18)	0.000518(24)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁶⁹ Ga	318.87(3)	0.0592(14)	0.00257(6)	834.08(1.65), 2201.91(0.52), 629.96(0.490)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²⁰³ Tl	318.88(8)	0.325(6)	0.00482(9)	139.94(0.400), 347.96(0.361), 5641.57(0.316)
¹⁷⁶ Lu	319.036(8)	3.83(13)	0.0663(23)	150.392(13.8), 457.944(8.3), 138.607(6.79)
²³² Th	319.08(10)	0.082(3)	0.00107(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
²⁰⁹ Bi	319.78(4)	0.0115(14)	1.67E-04(20)	4171.05(0.0171), 4054.57(0.0137), 4101.76(0.0089)
¹⁸⁷ Os	322.98(6)	0.242(9)	0.00386(14)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁷⁷ Hf	325.559(4)	6.69(17)	0.114(3)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹⁹³ Ir	328.448(14)d	9.1(3)	0.143[1.8%]	351.689(10.9), 84.2740(7.7), 136.1250(6.5)
¹⁹⁷ Au	328.4840(20)	1.48(19)	0.023(3)	410(94), 214.9710(9.0), 247.5730(5.56)
¹³⁹ La	328.762(8)d	1.250(18)	0.0273[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁰⁷ Ag	328.99(3)	0.795(12)	0.0223(3)	198.72(7.75), 235.62(4.62), 78.91(3.90)
²³² Th	331.37(11)	0.0291(19)	0.000380(25)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹²¹ Sb	332.2860(10)	0.101(3)	0.00251(8)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁹⁵ Pt	332.985(4)	2.580(25)	0.0401(4)	355.6840(6.17)
¹⁰³ Rh	333.44(3)	3.27(8)	0.0963(24)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁹¹ Ir	333.864(6)	1.53(10)	0.0241(16)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁴⁹ Sm	333.97(4)	4790(60)	96.5(12)	439.40(2860), 737.44(597), 505.51(528)
²³⁸ Np	334.3100(20)d	0.0550(8)	0.000700[0.6%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹¹⁵ In	335.450(10)	9.1(7)	0.240(18)	1293.54(131), 1097.30(87.3), 416.86(43.0)
²³² Th	335.92(10)	0.089(4)	0.00116(5)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁹³ Nb	337.527(7)	0.054(6)	0.00176(20)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
⁵⁸ Ni	339.420(11)	0.1670(21)	0.00862(11)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
¹⁵⁹ Tb	339.487(5)	0.35(4)	0.0067(8)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁴⁸ Ti	341.706(5)	1.840(21)	0.1165(13)	1381.745(5.18), 6760.084(2.97), 6418.426(1.96)
⁷⁹ Br	343.405(3)	0.118(4)	0.00448(15)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁶³ Cu	343.898(14)	0.215(4)	0.01025(19)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
⁸¹ Br	345.0060(10)	0.154(4)	0.00584(15)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
²⁰³ Tl	347.96(8)	0.361(10)	0.00535(15)	139.94(0.400), 318.88(0.325), 5641.57(0.316)
¹⁶⁴ Dy	349.248(10)	14.7(6)	0.274(11)	184.257(146), 538.609(69.2), 496.931(44.9)
²⁰ Ne	350.72(6)	0.0198(4)	0.00297(6)	2035.67(0.0245), 4374.13(0.01910), 2793.94(0.00900)
¹⁹⁷ Au	350.8280(10)	1.0(5)	0.015(8)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁹¹ Ir	351.689(4)	10.9(4)	0.172(6)	328.448(9.1), 84.2740(7.7), 136.1250(6.5)
⁵⁶ Fe	352.347(12)	0.273(3)	0.01481(16)	7631.136(0.653), 7645.5450(0.549), 6018.532(0.227)
²³² Th	354.27(10)	0.0408(20)	0.00053(3)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁹⁵ Pt	355.6840(20)	6.17(6)	0.0958(9)	332.985(2.580)
¹³³ Cs	356.157(4)	0.445(12)	0.0101(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁵⁹ Tb	357.748(5)	0.26(3)	0.0050(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁰⁹ Ag	360.41(3)	1.55(3)	0.0435(8)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁸⁹ Os	361.137(6)	0.466(15)	0.00742(24)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁷⁴ Yb	363.938(6)	0.80(12)	0.0140(21)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁹¹ Ir	365.440(7)	1.15(10)	0.0181(16)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁷⁹ Br	366.604(4)	0.233(6)	0.00884(23)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁷⁶ Lu	367.433(11)	2.23(8)	0.0386(14)	150.392(13.8), 457.944(8.3), 138.607(6.79)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁹⁹ Hg	367.947(9)	251(5)	3.79(8)	5967.02(62.5), 1693.296(56.2), 4739.43(30.1)
¹⁸⁹ Os	371.261(5)	0.574(14)	0.00914(22)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁹³ Ir	371.5020(20)	2.11(12)	0.0333(19)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁶⁵ Ho	371.772(5)	1.56(8)	0.0287(15)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
¹³³ Cs	377.311(5)	0.310(9)	0.00707(21)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁰⁷ Ag	380.90(3)	1.59(3)	0.0447(8)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁹⁷ Au	381.1990(10)	3.0(4)	0.046(6)	410(94), 214.9710(9.0), 247.5730(5.56)
¹⁶⁹ Tm	384.0790(20)	1.95(5)	0.0350(9)	200(8.72), 149.7180(7.11), 140(5.96)
¹¹⁵ In	385.111(8)	12.1(9)	0.319(24)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁶⁵ Cu	385.77(3)	0.1310(18)	0.00625(9)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
¹⁶⁴ Dy	385.9840(20)	34.8(10)	0.649(19)	184.257(146), 538.609(69.2), 496.931(44.9)
²⁴ Mg	389.670(21)	0.00586(24)	0.00073(3)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
⁷¹ Ga	390.66(4)	0.0476(12)	0.00207(5)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁸⁵ Re	390.854(23)	1.15(5)	0.0187(8)	63.5820(8.0), 155.041(7.16), 59.0100(5.5)
⁵⁹ Co	391.218(15)	1.080(14)	0.0555(7)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁷¹ Ga	393.28(3)	0.1340(23)	0.00582(10)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
²⁰³ Tl	395.62(8)	0.0862(20)	0.00128(3)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁷⁴ Yb	396.329(20)d	1.42(5)	0.0249[0.3%]	514.868(9.0), 639.261(1.43), 5266.3(1.4)
¹⁸¹ Ta	402.623(3)	1.180(23)	0.0198(4)	270.4030(2.60), 173.2050(1.210), 133.8770(0.63)
¹⁶⁹ Tm	411.5060(20)	2.37(5)	0.0425(9)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁶⁴ Dy	411.651(5)	35.1(10)	0.655(19)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁹⁷ Au	411.802d	94.29(15)	1.453[0.5%]	214.9710(9.0), 247.5730(5.56), 261.4040(5.3)
¹⁶⁴ Dy	414.985(7)	31(5)	0.58(9)	184.257(146), 538.609(69.2), 496.931(44.9)
¹¹⁵ In	416.86(3)d	43.0(18)	1.13[30%]	1293.54(131), 1097.30(87.3), 272.9660(33.1)
¹⁹¹ Ir	418.138(6)	3.45(15)	0.0544(24)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
⁵¹ V	419.475(13)	0.249(6)	0.0148(4)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁸⁵ Rb	421.50(3)	0.0259(5)	0.000918(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹³⁹ La	422.66(4)	0.370(14)	0.0081(3)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
²⁰³ Tl	424.81(8)	0.1200(25)	0.00178(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁸³ Kr	425.30(11)	2.960(19)	0.1070(7)	881.74(20.8), 1213.42(8.28), 1463.86(7.10)
¹⁶⁵ Ho	426.012(5)	2.88(15)	0.053(3)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
⁷⁵ As	426.5750(10)	0.100(3)	0.00404(12)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁷⁴ Yb	428.613(12)	0.61(7)	0.0107(12)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹³⁹ La	432.493(12)d	0.1780(18)	0.00388[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁹¹ Ir	432.716(6)	1.85(7)	0.0292(11)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹¹⁵ In	433.723(8)	6.0(4)	0.158(11)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁵⁹ Co	435.677(17)	0.789(10)	0.0406(5)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁷⁴ Yb	436.173(5)	0.52(6)	0.0091(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵¹ V	436.627(13)	0.397(9)	0.0236(5)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁴⁹ Sm	439.40(4)	2860(150)	58(3)	333.97(4790), 737.44(597), 505.51(528)
⁷⁶ Se	439.4510(20)	0.319(8)	0.0122(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹³³ Cs	442.8430(20)	0.316(12)	0.0072(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹²⁷ I	442.901(10)d	0.595(4)	0.0142[51%]	133.6110(1.42), 27.3620(0.43), 58.1100(0.28)
¹⁶⁹ Tm	446.328(3)	1.62(4)	0.0291(7)	200(8.72), 149.7180(7.11), 140(5.96)
⁵⁹ Co	447.711(19)	3.41(4)	0.1754(21)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁶⁴ Dy	447.893(7)	17.4(5)	0.324(9)	184.257(146), 538.609(69.2), 496.931(44.9)
¹³³ Cs	450.345(3)	0.99(5)	0.0226(11)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁵⁹ Tb	451.617(10)	0.21(3)	0.0040(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁵⁵ Mn	454.378(21)	0.388(7)	0.0214(4)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
¹³⁸ Ba	454.73(5)	0.0853(22)	0.00188(5)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹⁶⁹ Tm	456.0460(10)	1.16(4)	0.0208(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁷⁶ Lu	457.944(15)	8.3(3)	0.144(5)	150.392(13.8), 138.607(6.79), 208.3660(6.0)
⁹³ Nb	458.467(10)	0.0240(5)	0.000783(16)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹³⁷ Ba	462.78(4)	0.0660(16)	0.00146(4)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹⁵⁹ Tb	464.264(17)	0.192(21)	0.0037(4)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
⁵⁸ Ni	464.978(12)	0.843(10)	0.0435(5)	8998.414(1.49), 8533.509(0.721), 6837.50(0.458)
⁶⁵ Cu	465.14(3)	0.1350(21)	0.00644(10)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
¹⁶⁴ Dy	465.416(6)	38.0(10)	0.709(19)	184.257(146), 538.609(69.2), 496.931(44.9)
⁷⁹ Br	468.980(3)	0.29(3)	0.0110(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁰³ Rh	470.40(3)	2.61(7)	0.0769(21)	180.87(22.6), 97.14(19.5), 51.50(16.0)
⁷⁵ As	471.0000(10)	0.203(5)	0.00821(20)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹¹⁵ In	471.349(11)	4.3(3)	0.113(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
²⁰³ Tl	471.90(8)	0.116(3)	0.00172(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²³ Na	472.202(9)d	0.478(4)	0.0630[100%]	1368.66(0.530), 2754.13(0.530), 90.9920(0.235)
²³² Th	472.30(10)	0.165(8)	0.00215(10)	583.27(0.279), 566.63(0.19), 968.78(0.132)
⁷⁵ As	473.1540(10)	0.176(5)	0.00712(20)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁴⁰ Ce	475.04(4)	0.082(7)	0.00177(15)	661.99(0.241), 4766.10(0.113), 4291.08(0.053)
¹⁰¹ Ru	475.0950(20)	0.98(9)	0.029(3)	539.538(1.53), 686.907(0.52), 631.22(0.30)
¹⁶⁴ Dy	477.061(6)	22(7)	0.41(13)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁶⁴ Dy	477.08(4)	15.8(5)	0.295(9)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁷⁴ Yb	477.391(5)	0.75(8)	0.0131(14)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁰ B	477.595(3)	716(25)	201(7)	
¹⁸⁷ Os	478.04(4)	0.523(14)	0.00833(22)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁸⁶ W	479.550(22)d	2.59(5)	0.0427[1.4%]	685.73(3.24), 72.002(1.32), 134.247(1.050)
¹⁷⁴ Yb	482.071(11)	0.23(3)	0.0040(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵⁹ Co	484.257(16)	0.804(11)	0.0413(6)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹³⁹ La	487.021(12)d	2.79(4)	0.0609[0.9%]	1596.21(5.84), 815.772(1.430), 328.762(1.250)
⁸⁵ Rb	487.89(4)	0.0494(12)	0.00175(4)	556.82(0.0913), 555.61(0.0407), 872.94(0.0321)
²⁰³ Tl	488.11(8)	0.096(4)	0.00142(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹¹⁵ In	492.532(11)	3.31(24)	0.087(6)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁷³ Ge	492.933(5)	0.133(3)	0.00555(13)	595.851(1.100), 867.899(0.553), 608.353(0.250)
¹³⁹ La	495.620(13)	0.081(3)	0.00177(7)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁰⁹ Ag	495.71(3)	1.080(18)	0.0303(5)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁶⁴ Dy	496.931(5)	44.9(11)	0.837(21)	184.257(146), 538.609(69.2), 185.19(39.1)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁵⁹ Co	497.269(16)	2.16(4)	0.1111(21)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁹³ Nb	499.426(8)	0.0648(18)	0.00211(6)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹⁶⁹ Tm	499.5560(20)	0.88(3)	0.0158(5)	200(8.72), 149.7180(7.11), 140(5.96)
⁷⁰ Ge	499.87(3)	0.162(6)	0.00676(25)	595.851(1.100), 867.899(0.553), 608.353(0.250)
¹³³ Cs	502.840(3)	0.256(13)	0.0058(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁶⁹ Tm	505.018(7)	0.90(3)	0.0161(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁴⁹ Sm	505.51(3)	528(80)	10.6(16)	333.97(4790), 439.40(2860), 737.44(597)
⁶⁹ Ga	508.19(3)	0.349(6)	0.0152(3)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹³³ Cs	510.795(3)	1.54(3)	0.0351(7)	176.4040(2.47), 205.615(1.560), 307.015(1.45)
¹⁷⁴ Yb	511.784(11)	0.34(5)	0.0060(9)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁰⁵ Pd	511.843(20)	4.00(4)	0.1139(11)	717.356(0.777), 616.192(0.629)
¹⁶⁹ Tm	512.1370(20)	1.96(5)	0.0352(9)	200(8.72), 149.7180(7.11), 140(5.96)
⁸¹ Br	512.488(20)	0.21(3)	0.0080(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
³¹ P	512.646(19)	0.079(4)	0.0077(4)	78.083(0.059), 636.663(0.0311), 3899.89(0.0294)
¹⁷⁴ Yb	514.868(7)d	9.0(9)	0.158[100%]	639.261(1.43), 396.329(1.42), 5266.3(1.4)
⁴⁰ Ar	516.0(3)	0.167(17)	0.0127(13)	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
³⁵ Cl	517.0730(10)	7.58(5)	0.648(4)	1164.8650(8.91), 6110.842(6.59), 1951.1400(6.33)
⁹³ Nb	518.113(12)	0.0579(13)	0.00189(4)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
⁷⁶ Se	518.1810(20)	0.273(7)	0.0105(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹³³ Cs	519.101(4)	0.349(18)	0.0080(4)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁴⁰ Ca	519.66(5)	0.0503(13)	0.00380(10)	1942.67(0.352), 6419.59(0.176), 4418.52(0.0708)
⁷⁶ Se	520.6370(20)	1.260(18)	0.0484(7)	613.724(2.14), 238.9980(2.06), 161.9220(0.855)
²³⁸ U	521.849(7)	0.073(3)	0.00093(4)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²³² Th	522.73(10)	0.102(5)	0.00133(7)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁰⁹ Ag	524.47(3)	0.804(11)	0.0226(3)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹³³ Cs	525.356(4)	0.39(3)	0.0089(7)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
¹⁵⁹ Tb	525.933(17)	0.22(3)	0.0042(6)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)
¹⁹⁰ Os	527.60(3)	0.300(10)	0.00478(16)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁹⁷ Au	529.1650(20)	1.9(10)	0.029(15)	410(94), 214.9710(9.0), 247.5730(5.56)
¹³³ Cs	529.504(6)	0.519(23)	0.0118(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
²³² Th	531.58(10)	0.0404(23)	0.00053(3)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁷⁴ Yb	534.735(9)	0.50(6)	0.0088(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁶⁹ Tm	535.8280(10)	1.18(4)	0.0212(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁰⁹ Ag	536.13(3)	1.090(16)	0.0306(5)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹²⁹ Xe	536.17(9)	1.71(24)	0.039(6)	667.79(6.7), 772.72(1.78), 630.29(1.41)
⁸⁵ Rb	536.48(4)	0.0167(5)	0.000592(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹³⁹ Ba	537.261(9)d	0.066(3)	0.00084[0.1%]	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁶⁹ Tm	537.9910(20)	1.00(4)	0.0179(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁰³ Rh	538.04(3)	2.43(7)	0.0716(21)	180.87(22.6), 97.14(19.5), 51.50(16.0)
¹⁶⁴ Dy	538.609(8)	69.2(19)	1.29(4)	184.257(146), 496.931(44.9), 185.19(39.1)
⁸⁵ Rb	538.66(4)	0.0169(5)	0.000599(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹³³ Cs	539.180(4)	0.360(11)	0.00821(25)	176.4040(2.47), 205.615(1.560), 510.795(1.54)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²³⁸ U	539.278(12)	0.099(20)	0.00126(25)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁴⁵ Sc	539.437(20)	0.738(19)	0.0497(13)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁹⁹ Ru	539.538(15)	1.53(13)	0.046(4)	475.0950(0.98), 686.907(0.52), 631.22(0.30)
²³² Th	539.66(10)	0.061(3)	0.00080(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁷⁹ Br	542.515(6)	0.114(5)	0.00432(19)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁶⁵ Ho	542.780(4)	1.94(13)	0.0356(24)	136.6650(14.5), 116.8360(8.1), 80.574(3.87)
¹⁴¹ Pr	546.448(15)	0.148(4)	0.00318(9)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²³² Th	548.23(11)	0.042(10)	0.00055(13)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹³⁹ La	549.01(3)	0.098(4)	0.00214(9)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁰⁹ Ag	549.56(3)	1.540(24)	0.0433(7)	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹⁶⁹ Tm	551.5140(20)	1.29(25)	0.023(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁸⁶ W	551.52(4)d	0.603(14)	0.00994[1.4%]	685.73(3.24), 479.550(2.59), 72.002(1.32)
²³⁸ U	552.069(5)	0.207(5)	0.00264(6)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²³⁸ U	554.054(8)	0.085(20)	0.00108(25)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁸¹ Br	554.3480(20)d	0.838(8)	0.0318[1.0%]	776.517(0.990), 245.203(0.80), 619.106(0.515)
⁴⁵ Sc	554.44(4)	1.82(4)	0.123(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁸⁵ Rb	555.61(3)d	0.0407(10)	0.00144[98%]	556.82(0.0913), 487.89(0.0494), 872.94(0.0321)
¹⁰³ Rh	555.81(4)d	3.14(9)	0.092[98%]	180.87(22.6), 97.14(19.5), 51.50(16.0)
⁵⁹ Co	555.972(13)	5.76(6)	0.296(3)	229.879(7.18), 277.161(6.77), 447.711(3.41)
⁸⁵ Rb	556.82(3)	0.0913(24)	0.00324(9)	487.89(0.0494), 555.61(0.0407), 872.94(0.0321)
¹¹⁵ In	556.845(21)	4.7(3)	0.124(8)	1293.54(131), 1097.30(87.3), 416.86(43.0)
²³² Th	556.93(11)	0.040(10)	0.00052(13)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁸⁶ W	557.16(5)	0.125(5)	0.00206(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁴¹ Pr	557.75(3)	0.15(4)	0.0032(9)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁸⁹ Os	557.978(5)	0.84(3)	0.0134(5)	186.7180(2.08), 155.10(1.19), 569.344(0.694)
¹¹³ Cd	558.32(3)	1860(30)	50.1(8)	651.19(358), 245.3(274)
⁷⁵ As	559.10(5)d	2.00(10)	0.081[1.3%]	165.0490(0.996), 86.7880(0.579), 44.4250(0.560)
¹⁴¹ Pr	560.495(23)	0.150(7)	0.00323(15)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
⁹¹ Zr	560.958(3)	0.0285(5)	0.000947(17)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
²³² Th	561.25(11)	0.033(8)	0.00043(10)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁹³ Nb	562.328(9)	0.0293(11)	0.00096(4)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹²¹ Sb	564.24(4)d	2.700(4)	0.06720[0.5%]	61.4130(0.75), 78.0910(0.48), 121.4970(0.40)
¹⁶⁹ Tm	565.2770(20)	1.58(4)	0.0283(7)	200(8.72), 149.7180(7.11), 140(5.96)
²³² Th	566.63(10)	0.19(5)	0.0025(7)	583.27(0.279), 472.30(0.165), 968.78(0.132)
¹³⁹ La	567.386(12)	0.335(13)	0.0073(3)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁶⁹ Tm	569.1730(20)	1.02(3)	0.0183(5)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁸⁹ Os	569.344(20)	0.694(25)	0.0111(4)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁴¹ Pr	570.111(14)	0.112(5)	0.00241(11)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁴¹ Pr	573.28(4)	0.12(3)	0.0026(7)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
⁸⁹ Y	574.106(20)	0.174(7)	0.00593(24)	6080.171(0.76), 776.613(0.659), 202.53(0.289)
¹⁸⁶ W	577.30(5)	0.191(5)	0.00315(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²³² Th	578.02(9)	0.105(5)	0.00137(7)	583.27(0.279), 566.63(0.19), 472.30(0.165)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁷⁶ Se	578.8550(20)	0.243(5)	0.00933(19)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
⁶³ Cu	579.75(3)	0.0898(15)	0.00428(7)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
²³⁸ U	580.340(13)	0.043(10)	0.00055(13)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²³² Th	583.27(9)	0.279(11)	0.00364(14)	566.63(0.19), 472.30(0.165), 968.78(0.132)
¹⁹ F	583.561(16)	0.00356(12)	0.000568(19)	1633.53(0.0096), 656.006(0.00197), 665.207(0.00149)
¹⁶⁴ Dy	583.982(5)	24(7)	0.45(13)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁴⁹ Sm	584.27(3)	480(70)	9.7(14)	333.97(4790), 439.40(2860), 737.44(597)
⁴⁵ Sc	584.785(13)	1.77(3)	0.1193(20)	227.773(7.13), 147.011(6.08), 142.528(4.88)
²⁴ Mg	585.00(3)	0.0314(11)	0.00392(14)	3916.84(0.0320), 2828.172(0.0240), 1808.668(0.0180)
²³² Th	586.02(10)	0.045(3)	0.00059(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁶⁹ Tm	590.2270(20)	1.27(10)	0.0228(18)	200(8.72), 149.7180(7.11), 140(5.96)
²³⁸ U	592.309(13)	0.045(12)	0.00057(15)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²³² Th	593.23(10)	0.043(3)	0.00056(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
²³⁸ U	593.612(5)	0.108(24)	0.0014(3)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹³⁹ La	595.099(12)	0.103(4)	0.00225(9)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁷³ Ge	595.851(5)	1.100(24)	0.0459(10)	867.899(0.553), 608.353(0.250), 175.05(0.164)
⁷¹ Ga	601.21(6)d	0.471(22)	0.0205[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹²³ Te	602.729(17)	2.46(16)	0.058(4)	722.772(0.52), 645.819(0.263)
¹⁶⁹ Tm	603.9900(20)	1.40(5)	0.0251(9)	200(8.72), 149.7180(7.11), 140(5.96)
²³² Th	605.41(10)	0.054(4)	0.00071(5)	583.27(0.279), 566.63(0.19), 472.30(0.165)
²³⁸ U	605.581(9)	0.053(12)	0.00067(15)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷³ Ge	608.353(4)	0.250(6)	0.01043(25)	595.851(1.100), 867.899(0.553), 175.05(0.164)
¹¹⁵ In	608.422(11)	3.51(25)	0.093(7)	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁶³ Cu	608.766(23)	0.270(6)	0.0129(3)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
²³⁸ U	612.253(5)	0.23(5)	0.0029(6)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷⁷ Se	613.724(3)	2.14(5)	0.0821(19)	238.9980(2.06), 520.6370(1.260), 161.9220(0.855)
¹⁰⁵ Pd	616.192(20)	0.629(9)	0.0179(3)	511.843(4.00), 717.356(0.777)
⁷⁹ Br	616.3(5)d	0.39(4)	0.0148[62%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁴³ Nd	618.062(19)	13.4(3)	0.282(6)	696.499(33.3), 814.12(4.98), 864.301(4.27)
¹⁸⁶ W	618.26(4)d	0.746(17)	0.0123[1.4%]	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁸¹ Br	619.106(4)d	0.515(5)	0.01953[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁴¹ Pr	619.29(4)	0.152(4)	0.00327(9)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²⁰³ Tl	624.46(8)	0.0413(10)	0.000612(15)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁸⁶ W	625.519(10)d	0.129(3)	0.00213[1.4%]	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹³⁸ Ba	627.29(5)	0.294(6)	0.00649(13)	1435.77(0.308), 818.514(0.212), 4095.84(0.155)
⁴⁵ Sc	627.462(18)	2.23(5)	0.150(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
¹⁰¹ Ru	627.970(22)	0.176(16)	0.0053(5)	539.538(1.53), 475.0950(0.98), 686.907(0.52)
²³⁸ U	629.722(9)	0.073(20)	0.00093(25)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷¹ Ga	629.96(5)d	0.490(22)	0.0213[2.4%]	834.08(1.65), 2201.91(0.52), 601.21(0.471)
¹⁴¹ Pr	630.04(3)	0.16(6)	0.0034(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹³¹ Xe	630.29(4)	1.41(11)	0.0325(25)	667.79(6.7), 772.72(1.78), 536.17(1.71)
¹⁰¹ Ru	631.22(4)	0.30(3)	0.0090(9)	539.538(1.53), 475.0950(0.98), 686.907(0.52)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁶⁷ Er	631.7050(20)	7.9(3)	0.143(5)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
¹⁸⁷ Os	633.14(4)	0.585(16)	0.00932(25)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁴¹ Pr	633.34(4)	0.113(4)	0.00243(9)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁸⁷ Os	635.02(5)	0.405(12)	0.00645(19)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
³¹ P	636.663(21)	0.0311(14)	0.00304(14)	512.646(0.079), 78.083(0.059), 3899.89(0.0294)
¹⁶⁹ Tm	637.900(3)	1.25(4)	0.0224(7)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁶⁹ Tm	637.9020(20)	1.8(3)	0.032(5)	200(8.72), 149.7180(7.11), 140(5.96)
²³⁸ U	638.505(12)	0.041(12)	0.00052(15)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁸⁵ Rb	638.93(5)	0.0101(13)	0.00036(5)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹⁷⁴ Yb	639.261(9)	1.43(17)	0.025(3)	514.868(9.0), 396.329(1.42), 5266.3(1.4)
¹³³ Cs	645.453(5)	0.248(13)	0.0057(3)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁵¹ V	645.703(13)	0.769(17)	0.0457(10)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁴¹ Pr	645.720(24)	0.311(7)	0.00669(15)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹²³ Te	645.819(20)	0.263(22)	0.0062(5)	602.729(2.46), 722.772(0.52)
⁶³ Cu	648.80(3)	0.102(3)	0.00486(14)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
¹⁶⁹ Tm	650.3720(10)	1.45(5)	0.0260(9)	200(8.72), 149.7180(7.11), 140(5.96)
⁶⁹ Ga	651.09(3)	0.1030(22)	0.00448(10)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹¹³ Cd	651.19(3)	358(5)	9.65(13)	558.32(1860), 245.3(274)
¹⁹ F	656.006(18)	0.00197(7)	0.000314(11)	1633.53(0.0096), 583.561(0.00356), 665.207(0.00149)
⁷⁵ As	657.05(5)d	0.279(14)	0.0113[1.3%]	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁰⁹ Ag	657.50(10)d	1.86(5)	0.0523[99%]	198.72(7.75), 235.62(4.62), 78.91(3.90)
¹³⁹ La	658.278(12)	0.103(4)	0.00225(9)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁶⁹ Tm	658.913(5)	1.56(5)	0.0280(9)	200(8.72), 149.7180(7.11), 140(5.96)
⁷⁹ Br	660.561(4)	0.082(3)	0.00311(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁴⁰ Ce	661.99(5)	0.241(15)	0.0052(3)	4766.10(0.113), 475.04(0.082), 4291.08(0.053)
²³² Th	665.11(10)	0.084(4)	0.00110(5)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁹ F	665.207(18)	0.00149(6)	2.38E-04(10)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
¹³¹ Xe	667.79(6)	6.7(5)	0.155(12)	772.72(1.78), 536.17(1.71), 630.29(1.41)
²⁰⁹ Bi	673.97(5)	0.0026(4)	3.8E-05(6)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
²³² Th	681.81(9)	0.079(4)	0.00103(5)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁸⁶ W	685.73(4)d	3.24(7)	0.0534[1.4%]	479.550(2.59), 72.002(1.32), 134.247(1.050)
⁹⁹ Ru	686.907(17)	0.52(5)	0.0156(15)	539.538(1.53), 475.0950(0.98), 631.22(0.30)
²³⁸ U	689.907(11)	0.043(10)	0.00055(13)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷⁹ Br	689.994(16)	0.083(4)	0.00315(15)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁶⁹ Ga	690.943(24)	0.305(4)	0.01326(17)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁵⁶ Fe	691.960(19)	0.1370(18)	0.00743(10)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
¹²¹ Sb	692.65(4)d	0.146(5)	0.00363[0.5%]	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
⁷⁷ Se	694.914(4)	0.443(10)	0.0170(4)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁴³ Nd	696.499(10)	33.3(23)	0.70(5)	618.062(13.4), 814.12(4.98), 864.301(4.27)
⁸¹ Br	698.374(5)d	0.337(3)	0.01278[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹⁴¹ Pr	698.65(3)	0.22(6)	0.0047(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁶⁹ Tm	703.6280(10)	1.32(4)	0.0237(7)	200(8.72), 149.7180(7.11), 140(5.96)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²³² Th	705.17(11)	0.050(4)	0.00065(5)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹³⁹ La	708.244(14)	0.134(5)	0.00292(11)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
²³² Th	714.23(10)	0.052(3)	0.00068(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁵⁹ Co	717.310(18)	0.845(14)	0.0435(7)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁰⁵ Pd	717.356(22)	0.777(9)	0.0221(3)	511.843(4.00), 616.192(0.629)
¹⁶⁹ Tm	719.2610(20)	1.01(3)	0.0181(5)	200(8.72), 149.7180(7.11), 140(5.96)
⁹⁵ Mo	719.528(14)	0.310(10)	0.0098(3)	778.221(2.02), 849.85(0.43), 847.603(0.324)
¹³⁹ La	722.538(14)	0.212(8)	0.00463(17)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹²³ Te	722.772(25)	0.52(4)	0.0123(10)	602.729(2.46), 645.819(0.263)
¹⁶⁷ Er	730.6580(10)	11.6(4)	0.210(7)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
²⁰³ Tl	732.09(9)	0.064(3)	0.00095(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²⁰³ Tl	737.12(8)	0.118(5)	0.00175(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁴² Ce	737.43(7)	0.026(3)	0.00056(7)	661.99(0.241), 4766.10(0.113), 475.04(0.082)
¹⁴⁹ Sm	737.44(4)	597(8)	12.03(16)	333.97(4790), 439.40(2860), 505.51(528)
¹⁶⁷ Er	741.3650(20)	6.72(24)	0.122(4)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
¹⁴² Nd	742.106(22)	3.8(4)	0.080(8)	696.499(33.3), 618.062(13.4), 814.12(4.98)
¹⁴¹ Pr	746.973(14)	0.146(4)	0.00314(9)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
⁵⁰ Cr	749.09(3)	0.569(9)	0.0332(5)	834.849(1.38), 8884.36(0.78), 7938.46(0.424)
¹³⁹ La	751.637(18)d	0.2650(23)	0.00578[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁷⁶ Lu	761.564(20)	2.60(9)	0.0450(16)	150.392(13.8), 457.944(8.3), 138.607(6.79)
¹⁷⁴ Yb	767.169(9)	0.151(25)	0.0026(4)	514.868(9.0), 639.261(1.43), 396.329(1.42)
³⁹ K	770.3050(20)	0.903(12)	0.0700(9)	29.8300(1.380), 1158.887(0.1600), 5380.018(0.146)
¹³¹ Xe	772.72(4)	1.78(14)	0.041(3)	667.79(6.7), 536.17(1.71), 630.29(1.41)
¹⁸⁶ W	772.89(5)d	0.490(10)	0.00808[1.4%]	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁸¹ Br	776.517(3)d	0.990(10)	0.0375[1.0%]	554.3480(0.838), 245.203(0.80), 619.106(0.515)
⁸⁹ Y	776.613(18)	0.659(9)	0.0225(3)	6080.171(0.76), 202.53(0.289), 574.106(0.174)
⁹⁵ Mo	778.221(10)	2.02(6)	0.0638(19)	849.85(0.43), 847.603(0.324), 719.528(0.310)
¹⁵⁷ Gd	780.174(10)	1010(22)	19.5(4)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁸⁶ W	782.12(6)	0.22(3)	0.0036(5)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁵⁹ Co	785.628(21)	2.41(7)	0.124(4)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁷¹ Ga	786.17(16)d	0.160(22)	0.0070[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
³⁵ Cl	786.3020(10)	3.420(3)	0.2923(3)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
³⁵ Cl	788.4280(10)	5.42(5)	0.463(4)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
¹⁸³ W	792.059(16)	0.119(6)	0.00196(10)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁵¹ V	793.546(13)	0.199(5)	0.0118(3)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
²³² Th	797.79(9)	0.0416(20)	0.00054(3)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁶⁷ Zn	805.79(3)	0.045(3)	0.00209(14)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
¹⁷⁴ Yb	811.427(9)	0.92(16)	0.016(3)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁴³ Nd	814.12(3)	4.98(12)	0.1046(25)	696.499(33.3), 618.062(13.4), 864.301(4.27)
¹³⁹ La	815.772(19)d	1.430(12)	0.0312[0.9%]	1596.21(5.84), 487.021(2.79), 328.762(1.250)
¹⁶⁷ Er	815.9890(20)	42.5(15)	0.77(3)	184.2850(56), 198.2440(29.9), 79.8040(18.2)
¹⁸⁶ W	816.13(5)	0.104(4)	0.00171(7)	685.73(3.24), 479.550(2.59), 72.002(1.32)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹³⁵ Ba	818.514(12)	0.212(4)	0.00468(9)	1435.77(0.308), 627.29(0.294), 4095.84(0.155)
¹¹⁵ In	818.70(20)d	17.8(7)	0.470[30%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹⁶⁷ Er	821.1680(20)	6.2(3)	0.112(5)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
⁵¹ V	823.184(13)	0.320(8)	0.0190(5)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁷⁴ Yb	825.22(7)	0.154(24)	0.0027(4)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁸¹ Br	827.828(6)d	0.285(3)	0.01081[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
²³⁸ U	831.837(19)	0.053(12)	0.00067(15)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
⁷¹ Ga	834.08(3)d	1.65(5)	0.0717[2.4%]	2201.91(0.52), 629.96(0.490), 601.21(0.471)
⁶⁸ Zn	834.77(3)	0.037(5)	0.00171(23)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
²³² Th	834.83(14)	0.059(5)	0.00077(7)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁵³ Cr	834.849(22)	1.38(3)	0.0804(17)	8884.36(0.78), 749.09(0.569), 7938.46(0.424)
⁹³ Nb	835.72(3)	0.0376(8)	0.00123(3)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
⁴⁰ Ar	837.7(3)	0.063(7)	0.0048(5)	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
¹⁸⁶ W	840.18(5)	0.143(5)	0.00236(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
³² S	840.993(13)	0.347(6)	0.0328(6)	5420.574(0.308), 2379.661(0.208), 3220.588(0.117)
⁵¹ V	845.948(13)	0.252(7)	0.0150(4)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁵⁵ Mn	846.754(20)d	13.10(4)	0.7226[12%]	1810.72(3.62), 26.560(3.42), 83.884(3.11)
⁹⁵ Mo	847.603(11)	0.324(9)	0.0102(3)	778.221(2.02), 849.85(0.43), 719.528(0.310)
⁹⁵ Mo	849.85(3)	0.43(3)	0.0136(10)	778.221(2.02), 847.603(0.324), 719.528(0.310)
⁸⁷ Sr	850.657(12)	0.275(4)	0.00951(14)	1836.067(1.030), 898.055(0.702)
²³⁸ U	853.23(4)	0.055(12)	0.00070(15)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁶⁷ Er	853.4810(10)	7.5(3)	0.136(5)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
⁹ Be	853.630(12)	0.00208(24)	0.00070(8)	6809.61(0.0058), 3367.448(0.00285), 2590.014(0.00191)
¹⁶⁹ Tm	854.337(4)	1.41(4)	0.0253(7)	200(8.72), 149.7180(7.11), 140(5.96)
⁶⁴ Zn	855.69(3)	0.066(6)	0.0031(3)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
¹⁷¹ Yb	857.621(7)	0.208(25)	0.0036(4)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²³² Th	860.61(13)	0.047(5)	0.00061(7)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁴³ Nd	864.301(10)	4.27(11)	0.0897(23)	696.499(33.3), 618.062(13.4), 814.12(4.98)
¹⁴¹ Pr	864.98(3)	0.14(3)	0.0030(7)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹³⁹ La	867.846(20)d	0.337(4)	0.00735[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁷³ Ge	867.899(5)	0.553(12)	0.0231(5)	595.851(1.100), 608.353(0.250), 175.05(0.164)
²³ Na	869.210(9)	0.1080(13)	0.01424(17)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹⁶ O	870.68(6)	1.77E-04(11)	3.35E-05(21)	2184.42(1.64E-04), 1087.75(1.58E-04), 3272.02(3.53E-05)
¹⁷⁴ Yb	871.695(9)	0.24(4)	0.0042(7)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁸⁵ Rb	872.94(4)	0.0321(5)	0.001138(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
²⁰³ Tl	873.16(8)	0.168(4)	0.00249(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²³ Na	874.389(6)	0.0760(11)	0.01002(15)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
⁵⁸ Ni	877.977(11)	0.236(3)	0.01219(15)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
⁸³ Kr	881.74(11)	20.8(3)	0.752(11)	1213.42(8.28), 1463.86(7.10), 425.30(2.960)
¹⁶¹ Dy	882.27(6)	18.3(6)	0.341(11)	184.257(146), 538.609(69.2), 496.931(44.9)
⁷⁶ Se	885.8270(20)	0.262(7)	0.0101(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁸⁶ W	891.59(6)	0.136(5)	0.00224(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁷¹ Ga	894.91(11)d	0.35(3)	0.0152[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁵⁷ Gd	897.502(10)	1200(50)	23.1(10)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁵⁷ Gd	897.611(10)	1090(50)	21.0(10)	181.931(7200), 79.5100(4010), 944.174(3090)
⁸⁷ Sr	898.055(11)	0.702(10)	0.0243(4)	1836.067(1.030), 850.657(0.275)
¹⁸³ W	903.274(17)	0.115(5)	0.00190(8)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁶⁷ Er	914.9420(10)	6.99(24)	0.127(4)	184.2850(56), 815.9890(42.5), 198.2440(29.9)
¹³⁹ La	919.550(23)d	0.1630(18)	0.00356[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹²¹ Sb	921.00(7)	0.075(4)	0.00187(10)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹³⁹ La	925.189(21)d	0.422(4)	0.00921[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁹¹ Zr	934.4640(10)	0.125(5)	0.00415(17)	1465.7(0.063), 1205.6(0.042), 2042.2(0.032)
²³⁵ U	943.14(7)	0.082(10)	0.00104(13)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹⁵⁷ Gd	944.174(10)	3090(70)	59.5(13)	181.931(7200), 79.5100(4010), 944.174(3090)
⁵⁹ Co	945.314(17)	0.98(4)	0.0504(21)	229.879(7.18), 277.161(6.77), 555.972(5.76)
²⁰³ Tl	949.88(8)	0.0479(15)	0.000710(22)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁹³ Nb	957.28(5)	0.0248(7)	0.000809(23)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
⁷³ Ge	961.055(7)	0.129(4)	0.00538(17)	595.851(1.100), 867.899(0.553), 608.353(0.250)
¹⁵⁷ Gd	962.104(10)	2050(130)	39.5(25)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁷¹ Yb	964.197(10)	0.229(25)	0.0040(4)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²³² Th	968.78(9)	0.132(6)	0.00172(8)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹¹⁵ Sn	972.619(17)	0.0158(5)	0.000403(13)	1293.591(0.1340), 1171.28(0.0879), 1229.64(0.0673)
²⁴ Mg	974.66(3)	0.00663(24)	0.00083(3)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
¹⁵⁷ Gd	977.121(10)	1440(21)	27.8(4)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁸² W	979.871(18)	0.102(10)	0.00168(16)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁷ Li	980.53(7)	0.00415(13)	0.00181(6)	2032.30(0.0381), 1051.90(0.00414)
²⁷ Al	982.951(10)	0.00902(14)	0.001013(16)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁹ F	983.538(20)	0.00116(4)	1.85E-04(6)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
¹⁴¹ Pr	992.00(4)	0.138(10)	0.00297(22)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁴¹ Pr	1006.361(22)	0.153(8)	0.00329(17)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
⁶⁸ Zn	1007.809(25)	0.056(7)	0.0026(3)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
²³² Th	1013.84(11)	0.037(3)	0.00048(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
²² Ne	1017.00(20)	0.0030(5)	0.00045(8)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
¹⁸² W	1026.373(17)	0.161(15)	0.00265(25)	685.73(3.24), 479.550(2.59), 72.002(1.32)
⁸⁵ Rb	1026.55(6)	0.0218(4)	0.000773(14)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
⁸⁵ Rb	1032.32(5)	0.0227(4)	0.000805(14)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹⁷¹ Yb	1039.150(7)	0.22(3)	0.0039(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁸¹ Br	1044.002(5)d	0.323(3)	0.01225[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹³⁸ Ba	1047.73(6)	0.0319(10)	0.000704(22)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
⁷¹ Ga	1050.69(5)d	0.119(13)	0.0052[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁷ Li	1051.90(7)	0.00414(12)	0.00181(5)	2032.30(0.0381), 980.53(0.00415)
¹⁹ F	1056.776(17)	0.00095(3)	1.52E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
³¹ P	1071.217(23)	0.0249(12)	0.00244(12)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
²⁰ Ne	1071.34(7)	0.0054(4)	0.00081(6)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁷¹ Yb	1076.246(6)	0.52(6)	0.0091(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁸⁵ Rb	1076.64(20)d	0.0301(5)	0.001067[0.08%]	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
⁶⁷ Zn	1077.335(16)	0.356(5)	0.01650(23)	115.225(0.167), 7863.55(0.1410), 1883.12(0.0718)
¹⁶ O	1087.75(6)	1.58E-04(7)	2.99E-05(13)	870.68(1.77E-04), 2184.42(1.64E-04), 3272.02(3.53E-05)
¹⁷¹ Yb	1093.674(9)	0.24(3)	0.0042(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹¹⁵ In	1097.30(20)d	87.3(17)	2.30[30%]	1293.54(131), 416.86(43.0), 272.9660(33.1)
⁷³ Ge	1101.282(6)	0.134(3)	0.00559(13)	595.851(1.100), 867.899(0.553), 608.353(0.250)
⁹⁶ Zr	1102.67(6)	0.0235(8)	0.00078(3)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
¹⁷⁷ Hf	1102.824(5)	2.96(8)	0.0503(14)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
⁸⁵ Rb	1105.52(10)	0.0151(3)	0.000535(11)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹⁵⁷ Gd	1107.612(9)	1830(40)	35.3(8)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁴² Ce	1107.66(5)	0.040(3)	0.00087(7)	661.99(0.241), 4766.10(0.113), 475.04(0.082)
²⁰³ Tl	1110.37(8)	0.0413(12)	0.000612(18)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁹³ Nb	1118.54(3)	0.022(7)	0.00072(23)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹⁵⁷ Gd	1119.163(10)	1180(30)	22.7(6)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁷¹ Yb	1119.780(8)	0.46(6)	0.0081(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²⁰³ Tl	1121.29(7)	0.0600(17)	0.000890(25)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²⁵ Mg	1129.575(23)	0.00891(25)	0.00111(3)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
¹⁴¹ Pr	1150.946(21)	0.141(5)	0.00303(11)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²⁰³ Tl	1155.43(7)	0.0605(17)	0.000897(25)	139.94(0.400), 347.96(0.361), 318.88(0.325)
³⁹ K	1158.887(10)	0.1600(25)	0.01240(19)	29.8300(1.380), 770.3050(0.903), 5380.018(0.146)
³⁵ Cl	1164.8650(10)	8.91(4)	0.762(3)	517.0730(7.58), 6110.842(6.59), 1951.1400(6.33)
¹⁷⁷ Hf	1167.072(6)	3.95(10)	0.0671(17)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹¹⁹ Sn	1171.28(6)	0.0879(13)	0.00224(3)	1293.591(0.1340), 1229.64(0.0673), 972.619(0.0158)
¹⁷⁷ Hf	1174.635(5)	4.8(7)	0.081(12)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹⁵⁷ Gd	1183.968(10)	958(60)	18.5(12)	181.931(7200), 79.5100(4010), 944.174(3090)
¹⁵⁷ Gd	1185.988(9)	1600(90)	30.8(17)	181.931(7200), 79.5100(4010), 944.174(3090)
⁴⁰ Ar	1186.8(3)	0.34(3)	0.0258(23)	167.30(0.53), 4745.3(0.36), 516.0(0.167)
¹⁵⁷ Gd	1187.122(9)	1420(90)	27.4(17)	181.931(7200), 79.5100(4010), 944.174(3090)
⁷³ Ge	1204.199(6)	0.141(4)	0.00588(17)	595.851(1.100), 867.899(0.553), 608.353(0.250)
⁹⁰ Zr	1205.6(7)	0.042(5)	0.00140(17)	934.4640(0.125), 1465.7(0.063), 2042.2(0.032)
⁹³ Nb	1206.26(5)	0.0284(10)	0.00093(3)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹⁷⁷ Hf	1207.213(5)	3.9(3)	0.066(5)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
⁸³ Kr	1213.42(12)	8.28(17)	0.299(6)	881.74(20.8), 1463.86(7.10), 425.30(2.960)
⁷⁵ As	1216.08(5)d	0.155(8)	0.0063[1.3%]	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁷⁷ Hf	1229.287(8)	4.26(11)	0.0723(19)	213.439(29.3), 214.3410(16.3), 93.182(13.3)
¹¹⁷ Sn	1229.64(6)	0.0673(13)	0.00172(3)	1293.591(0.1340), 1171.28(0.0879), 972.619(0.0158)
²⁰³ Tl	1234.69(7)	0.0746(25)	0.00111(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁵⁶ Fe	1260.448(19)	0.0684(11)	0.00371(6)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
⁶⁷ Zn	1261.15(3)	0.0431(10)	0.00200(5)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
¹³⁵ Ba	1261.52(7)	0.095(5)	0.00210(11)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹² C	1261.765(9)	0.00124(3)	0.000313(8)	4945.301(0.00261), 3683.920(0.00122)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²⁸ Si	1273.349(17)	0.0289(6)	0.00312(7)	3538.966(0.1190), 4933.889(0.1120), 2092.902(0.0331)
²³⁵ U	1279.01(10)	0.200(10)	0.00255(13)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹¹⁵ In	1293.54(15)d	131(3)	3.46[30%]	1097.30(87.3), 416.86(43.0), 272.9660(33.1)
¹¹⁵ Sn	1293.591(15)	0.1340(21)	0.00342(5)	1171.28(0.0879), 1229.64(0.0673), 972.619(0.0158)
⁷⁶ Se	1296.986(7)	0.240(7)	0.0092(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
⁸⁵ Rb	1304.48(4)	0.0204(5)	0.000723(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
¹⁷³ Yb	1308.53(11)	0.168(19)	0.0029(3)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁷⁷ Se	1308.632(5)	0.317(8)	0.0122(3)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁹ F	1309.126(17)	0.00076(3)	1.21E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁸¹ Br	1317.473(10)d	0.314(3)	0.01191[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
¹³¹ Xe	1317.93(8)	0.89(7)	0.0205(16)	667.79(6.7), 772.72(1.78), 536.17(1.71)
⁶⁷ Zn	1340.14(3)	0.0457(16)	0.00212(7)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
²³ Na	1368.66(3)d	0.530(8)	0.0699[2.3%]	2754.13(0.530), 472.202(0.478), 90.9920(0.235)
¹⁷⁴ Yb	1378.22(7)	0.42(12)	0.0074(21)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁴⁸ Ti	1381.745(5)	5.18(12)	0.328(8)	6760.084(2.97), 6418.426(1.96), 341.706(1.840)
¹⁹ F	1387.901(20)	0.00082(3)	1.31E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁹¹ Zr	1405.159(3)	0.0301(10)	0.00100(3)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
⁵¹ V	1434.10(3)d	4.81(10)	0.286[91%]	125.082(1.61), 6517.282(0.78), 645.703(0.769)
¹³⁷ Ba	1435.77(4)	0.308(7)	0.00680(15)	627.29(0.294), 818.514(0.212), 4095.84(0.155)
¹³⁷ Ba	1444.91(5)	0.0801(20)	0.00177(4)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
⁸³ Kr	1463.86(6)	7.10(8)	0.257(3)	881.74(20.8), 1213.42(8.28), 425.30(2.960)
⁷¹ Ga	1464.00(7)d	0.0609(19)	0.00265[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁹⁰ Zr	1465.7(7)	0.063(15)	0.0021(5)	934.4640(0.125), 1205.6(0.042), 2042.2(0.032)
⁸¹ Br	1474.880(10)d	0.1930(20)	0.00732[1.0%]	776.517(0.990), 554.3480(0.838), 245.203(0.80)
²⁰³ Tl	1478.77(8)	0.0544(22)	0.00081(3)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹¹⁵ In	1507.40(20)d	15.5(5)	0.409[30%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁵⁹ Co	1515.720(25)	1.740(25)	0.0895(13)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁷¹ Yb	1521.197(16)	0.193(24)	0.0034(4)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵¹ V	1558.843(18)	0.323(8)	0.0192(5)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁹⁹ Hg	1570.273(12)	29.6(7)	0.447(11)	367.947(251), 5967.02(62.5), 1693.296(56.2)
¹⁴¹ Pr	1575.6(5)d	0.426(12)	0.0092[1.8%]	176.8630(1.06), 140.9050(0.479), 5666.170(0.379)
⁴⁸ Ti	1585.941(5)	0.624(8)	0.0395(5)	1381.745(5.18), 6760.084(2.97), 6418.426(1.96)
¹³⁹ La	1596.21(4)d	5.84(9)	0.1274[0.9%]	487.021(2.79), 815.772(1.430), 328.762(1.250)
⁷¹ Ga	1596.68(8)d	0.0732(16)	0.00318[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
³⁵ Cl	1601.072(4)	1.210(7)	0.1034(6)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
⁵⁶ Fe	1612.786(18)	0.1530(22)	0.00830(12)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
²⁷ Al	1622.877(18)	0.00989(15)	0.001111(17)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁹ F	1633.53(3)d	0.0096(4)	0.00153[100%]	583.561(0.00356), 656.006(0.00197), 665.207(0.00149)
²³ Na	1636.293(21)	0.0250(7)	0.00330(9)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹⁷³ Yb	1638.36(17)	0.22(3)	0.0039(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁴ N	1678.281(14)	0.0063(3)	0.00136(7)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
¹⁷³ Yb	1679.70(14)	0.161(19)	0.0028(3)	514.868(9.0), 639.261(1.43), 396.329(1.42)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁹⁹ Hg	1693.296(11)	56.2(16)	0.849(24)	367.947(251), 5967.02(62.5), 4739.43(30.1)
⁵⁶ Fe	1725.288(21)	0.181(3)	0.00982(16)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
²⁰³ Tl	1741.01(8)	0.0548(25)	0.00081(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹¹⁵ In	1753.8(6)d	3.82(12)	0.101[30%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
⁵¹ V	1777.961(19)	0.169(13)	0.0101(8)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
²⁷ Al	1778.92(3)d	0.232(4)	0.0261[95%]	30.6380(0.0798), 7724.027(0.0493), 3033.896(0.0179)
⁵³ Cr	1784.70(4)	0.1760(20)	0.01026(12)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
²⁵ Mg	1808.668(22)	0.0180(5)	0.00224(6)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
⁵⁵ Mn	1810.72(4)d	3.62(11)	0.200[12%]	846.754(13.10), 26.560(3.42), 83.884(3.11)
⁵⁹ Co	1830.800(25)	1.700(23)	0.0874(12)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁸⁷ Sr	1836.067(21)	1.030(18)	0.0356(6)	898.055(0.702), 850.657(0.275)
¹⁹ F	1843.688(20)	0.000600(23)	9.6E-05(4)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁷¹ Ga	1861.09(6)d	0.0904(19)	0.00393[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
⁹⁰ Zr	1880.4(4)	0.016(4)	0.00053(13)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
⁶⁷ Zn	1883.12(3)	0.0718(18)	0.00333(8)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
¹⁴ N	1884.821(16)	0.01470(18)	0.00318(4)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
⁸⁵ Rb	1890.7(4)	0.017(4)	0.00060(14)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
⁸³ Kr	1897.79(8)	2.24(3)	0.0810(11)	881.74(20.8), 1213.42(8.28), 1463.86(7.10)
²⁰ Ne	1931.08(6)	0.00591(22)	0.00089(3)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
⁴⁰ Ca	1942.67(3)	0.352(7)	0.0266(5)	6419.59(0.176), 4418.52(0.0708), 2001.31(0.0659)
³⁵ Cl	1951.1400(20)	6.33(4)	0.541(3)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
¹⁰² Ru	1959.30(7)	0.210(19)	0.0063(6)	539.538(1.53), 475.0950(0.98), 686.907(0.52)
³⁵ Cl	1959.346(4)	4.10(3)	0.350(3)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
²² Ne	1979.89(6)	0.00306(17)	0.00046(3)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
¹⁴ N	1999.690(16)	0.00323(4)	0.000699(9)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
⁴⁰ Ca	2001.31(3)	0.0659(15)	0.00498(11)	1942.67(0.352), 6419.59(0.176), 4418.52(0.0708)
⁴⁰ Ca	2009.84(3)	0.0409(10)	0.00309(8)	1942.67(0.352), 6419.59(0.176), 4418.52(0.0708)
²³ Na	2025.139(22)	0.0341(8)	0.00450(11)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
⁷ Li	2032.30(4)	0.0381(8)	0.0166(4)	980.53(0.00415), 1051.90(0.00414)
²⁰ Ne	2035.67(20)	0.0245(25)	0.0037(4)	350.72(0.0198), 4374.13(0.01910), 2793.94(0.00900)
⁹⁰ Zr	2042.2(4)	0.032(8)	0.0011(3)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
²⁸ Si	2092.902(18)	0.0331(6)	0.00357(7)	3538.966(0.1190), 4933.889(0.1120), 1273.349(0.0289)
¹¹⁵ In	2112.1(4)d	24.1(7)	0.636[30%]	1293.54(131), 1097.30(87.3), 416.86(43.0)
¹¹⁵ Sn	2112.302(16)	0.0152(5)	0.000388(13)	1293.591(0.1340), 1171.28(0.0879), 1229.64(0.0673)
⁵⁵ Mn	2113.05(4)d	1.91(5)	0.105[12%]	846.754(13.10), 1810.72(3.62), 26.560(3.42)
³¹ P	2114.47(3)	0.0115(5)	0.00113(5)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
³¹ P	2151.52(4)	0.0100(5)	0.00098(5)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
³¹ P	2156.90(4)	0.0128(6)	0.00125(6)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
¹⁶ O	2184.42(7)	1.64E-04(7)	3.11E-05(13)	870.68(1.77E-04), 1087.75(1.58E-04), 3272.02(3.53E-05)
⁷¹ Ga	2201.91(13)d	0.52(4)	0.0226[2.4%]	834.08(1.65), 629.96(0.490), 601.21(0.471)
²³ Na	2208.40(3)	0.0259(9)	0.00341(12)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹³⁷ Ba	2217.84(8)	0.044(5)	0.00097(11)	1435.77(0.308), 627.29(0.294), 818.514(0.212)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹ H	2223.24835(9)	0.3326(7)	1.0000(21)	
⁵³ Cr	2239.04(8)	0.186(3)	0.01084(17)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
²⁷ Al	2282.794(9)	0.00890(17)	0.001000(19)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
³² S	2379.661(14)	0.208(5)	0.0197(5)	840.993(0.347), 5420.574(0.308), 3220.588(0.117)
¹⁷¹ Yb	2401.37(3)	0.20(3)	0.0035(5)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²³ Na	2414.457(21)	0.0237(5)	0.00312(7)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹⁹ F	2431.084(10)	0.000392(24)	6.3E-05(4)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
²⁴ Mg	2438.54(3)	0.00473(19)	0.000590(24)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
⁷¹ Ga	2491.6(3)d	0.17(4)	0.0074[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
²⁰⁹ Bi	2505.35(7)	0.0021(3)	3.0E-05(4)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
⁷¹ Ga	2507.40(12)d	0.28(4)	0.0122[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
²³ Na	2517.81(3)	0.0699(15)	0.00921(20)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
¹⁴ N	2520.457(17)	0.00441(24)	0.00095(5)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
¹³⁹ La	2521.40(5)d	0.2120(23)	0.00463[0.9%]	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁹ F	2529.212(18)	0.00061(3)	9.7E-05(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁹⁰ Zr	2557.8(8)	0.016(4)	0.00053(13)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
⁹⁰ Zr	2577.3(14)	0.016(4)	0.00053(13)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
³¹ P	2586.00(4)	0.0089(4)	0.00087(4)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
⁹ Be	2590.014(19)	0.00191(15)	0.00064(5)	6809.61(0.0058), 3367.448(0.00285), 853.630(0.00208)
²⁷ Al	2590.193(9)	0.00807(16)	0.000906(18)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
²³ Na	2752.271(23)	0.0654(12)	0.00862(16)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
²³ Na	2754.13(6)d	0.530(8)	0.0699[2.3%]	1368.66(0.530), 472.202(0.478), 90.9920(0.235)
⁴⁰ Ar	2771.9(8)	0.057(9)	0.0043(7)	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
²⁰ Ne	2793.94(5)	0.00900(11)	0.001352(17)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
²⁴ Mg	2828.172(25)	0.0240(8)	0.00299(10)	3916.84(0.0320), 585.00(0.0314), 1808.668(0.0180)
²⁰⁹ Bi	2828.29(7)	0.00179(24)	2.6E-05(4)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
³⁵ Cl	2863.819(12)	1.820(10)	0.1556(9)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
²⁰ Ne	2895.32(10)	0.00252(7)	0.000378(11)	2035.67(0.0245), 350.72(0.0198), 4374.13(0.01910)
³² S	2930.67(3)	0.0832(13)	0.00786(12)	840.993(0.347), 5420.574(0.308), 2379.661(0.208)
¹⁹ F	3014.568(10)	0.000405(15)	6.46E-05(24)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
²⁷ Al	3033.896(6)	0.0179(3)	0.00201(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
⁷¹ Ga	3034.6(4)d	0.15(3)	0.0065[2.4%]	834.08(1.65), 2201.91(0.52), 629.96(0.490)
²⁴ Mg	3054.00(3)	0.0083(3)	0.00103(4)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
³¹ P	3058.17(4)	0.0110(4)	0.00108(4)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
³⁵ Cl	3061.82(4)	1.130(7)	0.0966(6)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
¹³⁹ La	3082.979(24)	0.140(5)	0.00305(11)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
³² S	3220.588(17)	0.117(5)	0.0111(5)	840.993(0.347), 5420.574(0.308), 2379.661(0.208)
¹⁶ O	3272.02(8)	3.53E-05(23)	6.7E-06(4)	870.68(1.77E-04), 2184.42(1.64E-04), 1087.75(1.58E-04)
³¹ P	3273.98(4)	0.0083(3)	0.00081(3)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
²⁴ Mg	3301.41(3)	0.00620(24)	0.00077(3)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
⁹ Be	3367.448(25)	0.00285(22)	0.00096(7)	6809.61(0.0058), 853.630(0.00208), 2590.014(0.00191)
²⁴ Mg	3413.10(3)	0.00401(16)	0.000500(20)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁹ Be	3443.406(20)	0.00098(7)	0.000330(24)	6809.61(0.0058), 3367.448(0.00285), 853.630(0.00208)
²⁷ Al	3465.058(7)	0.0146(3)	0.00164(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁸⁶ W	3469.40(14)	0.103(6)	0.00170(10)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²³² Th	3473.00(8)	0.057(3)	0.00074(4)	583.27(0.279), 566.63(0.19), 472.30(0.165)
⁹⁰ Zr	3475.8(15)	0.019(5)	0.00063(17)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
¹⁹ F	3488.064(18)	0.00073(3)	1.16E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
³¹ P	3522.59(3)	0.0219(8)	0.00214(8)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
²³² Th	3530.96(13)	0.0397(24)	0.00052(3)	583.27(0.279), 566.63(0.19), 472.30(0.165)
¹⁴ N	3531.981(15)	0.0071(4)	0.00154(9)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
²⁸ Si	3538.966(22)	0.1190(20)	0.01284(22)	4933.889(0.1120), 2092.902(0.0331), 1273.349(0.0289)
²³⁸ U	3583.10(7)	0.042(3)	0.00053(4)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
²³ Na	3587.460(25)	0.0596(11)	0.00786(15)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
²⁷ Al	3591.189(8)	0.01000(21)	0.001123(24)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁷⁴ Yb	3632.3(10)	0.40(10)	0.0070(18)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹³⁸ Ba	3641.12(9)	0.0562(16)	0.00124(4)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹³⁹ La	3665.631(24)	0.135(5)	0.00295(11)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁴ N	3677.732(13)	0.0115(6)	0.00249(13)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
¹³⁹ La	3679.641(24)	0.139(5)	0.00303(11)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹² C	3683.920(9)	0.00122(3)	0.000308(8)	4945.301(0.00261), 1261.765(0.00124)
⁴⁰ Ar	3700.6(8)	0.065(7)	0.0049(5)	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
¹⁷⁴ Yb	3714.7(5)	0.23(6)	0.0040(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁴¹ Pr	3790.37(3)	0.140(6)	0.00301(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²⁵ Mg	3831.480(24)	0.00418(14)	0.000521(17)	3916.84(0.0320), 585.00(0.0314), 2828.172(0.0240)
¹⁷⁴ Yb	3885.0(4)	0.72(17)	0.013(3)	514.868(9.0), 639.261(1.43), 396.329(1.42)
³¹ P	3899.89(3)	0.0294(10)	0.00288(10)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
²⁴ Mg	3916.84(3)	0.0320(11)	0.00399(14)	585.00(0.0314), 2828.172(0.0240), 1808.668(0.0180)
¹⁷⁴ Yb	3929.3(4)	0.38(9)	0.0067(16)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁹ F	3964.872(20)	0.000435(18)	6.9E-05(3)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
²³ Na	3981.450(25)	0.0677(11)	0.00892(15)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
⁹⁰ Zr	3982.3(15)	0.015(4)	0.00050(13)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
²⁰⁹ Bi	4054.57(6)	0.0137(18)	2.0E-04(3)	4171.05(0.0171), 319.78(0.0115), 4101.76(0.0089)
²³⁸ U	4060.35(5)	0.186(3)	0.00237(4)	74.6640(1.30000), 106.1230(0.723), 277.5990(0.382)
¹³⁸ Ba	4095.84(9)	0.155(4)	0.00342(9)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
²⁰⁹ Bi	4101.76(6)	0.0089(12)	1.29E-04(17)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
²⁷ Al	4133.407(7)	0.0149(3)	0.00167(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
²⁰⁹ Bi	4165.36(5)	0.00173(24)	2.5E-05(4)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
²⁰⁹ Bi	4171.05(9)	0.0171(22)	2.5E-04(3)	4054.57(0.0137), 319.78(0.0115), 4101.76(0.0089)
⁵⁶ Fe	4218.27(5)	0.099(3)	0.00537(16)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
²⁰³ Tl	4225.47(17)	0.045(3)	0.00067(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁸⁶ W	4249.66(7)	0.115(6)	0.00190(10)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²⁰⁹ Bi	4256.65(5)	0.0024(3)	3.5E-05(4)	4171.05(0.0171), 4054.57(0.0137), 319.78(0.0115)
²⁷ Al	4259.534(7)	0.0153(3)	0.00172(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹⁴⁰ Ce	4291.08(4)	0.053(4)	0.00115(9)	661.99(0.241), 4766.10(0.113), 475.04(0.082)
¹⁴² Ce	4336.46(8)	0.0251(20)	0.00054(4)	661.99(0.241), 4766.10(0.113), 475.04(0.082)
²⁰ Ne	4374.13(6)	0.01910(22)	0.00287(3)	2035.67(0.0245), 350.72(0.0198), 2793.94(0.00900)
¹³⁹ La	4389.505(14)	0.255(10)	0.00556(22)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹³⁹ La	4416.22(3)	0.247(9)	0.00539(20)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁴⁰ Ca	4418.52(5)	0.0708(18)	0.00535(14)	1942.67(0.352), 6419.59(0.176), 2001.31(0.0659)
²⁰³ Tl	4495.74(13)	0.043(4)	0.00064(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹³⁹ La	4502.647(13)	0.164(6)	0.00358(13)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁴ N	4508.731(12)	0.0132(7)	0.00286(15)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
²⁰³ Tl	4540.62(15)	0.0413(25)	0.00061(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁹ F	4556.817(20)	0.000517(23)	8.2E-05(4)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
¹⁸⁴ W	4573.7(3)	0.104(9)	0.00171(15)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁸⁶ W	4574.94(8)	0.152(10)	0.00251(16)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁸⁶ W	4626.35(7)	0.124(7)	0.00204(12)	685.73(3.24), 479.550(2.59), 72.002(1.32)
³¹ P	4671.37(3)	0.0194(7)	0.00190(7)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
¹⁸⁶ W	4684.40(8)	0.150(7)	0.00247(12)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²⁰³ Tl	4687.58(12)	0.098(4)	0.00145(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²⁷ Al	4690.676(5)	0.01090(24)	0.00122(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁴¹ Pr	4692.120(22)	0.291(10)	0.00626(22)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²⁰³ Tl	4705.83(14)	0.058(3)	0.00086(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²⁷ Al	4733.844(11)	0.0126(3)	0.00142(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
¹⁹⁹ Hg	4739.43(5)	30.1(8)	0.455(12)	367.947(251), 5967.02(62.5), 1693.296(56.2)
⁴⁰ Ar	4745.3(8)	0.36(4)	0.027(3)	167.30(0.53), 1186.8(0.34), 516.0(0.167)
²⁰³ Tl	4752.24(11)	0.148(5)	0.00219(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁴⁰ Ce	4766.10(5)	0.113(8)	0.00244(17)	661.99(0.241), 475.04(0.082), 4291.08(0.053)
¹⁴¹ Pr	4801.22(3)	0.140(8)	0.00301(17)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁷⁴ Yb	4830.2(4)	0.25(6)	0.0044(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
²⁰³ Tl	4841.40(15)	0.090(4)	0.00133(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹³⁹ La	4842.695(7)	0.661(25)	0.0144(6)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
³² S	4869.61(3)	0.0650(13)	0.00614(12)	840.993(0.347), 5420.574(0.308), 2379.661(0.208)
¹³⁹ La	4888.606(7)	0.150(6)	0.00327(13)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
²⁰³ Tl	4913.57(11)	0.164(5)	0.00243(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
²⁸ Si	4933.889(24)	0.1120(23)	0.01209(25)	3538.966(0.1190), 2092.902(0.0331), 1273.349(0.0289)
¹² C	4945.301(3)	0.00261(5)	0.000659(13)	1261.765(0.00124), 3683.920(0.00122)
³⁵ Cl	4979.759(20)	1.230(10)	0.1051(9)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
¹⁷⁴ Yb	5011.0(4)	0.18(4)	0.0032(7)	514.868(9.0), 639.261(1.43), 396.329(1.42)
⁵⁵ Mn	5014.37(7)	0.737(20)	0.0407(11)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
²⁰³ Tl	5014.61(15)	0.058(3)	0.00086(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁹ F	5033.530(23)	0.00063(3)	1.00E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
¹⁴¹ Pr	5096.081(15)	0.208(8)	0.00447(17)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹³⁹ La	5097.726(6)	0.68(3)	0.0148(7)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
⁹³ Nb	5103.34(7)	0.0232(12)	0.00076(4)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
¹³⁹ La	5126.257(6)	0.114(4)	0.00249(9)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
²⁰³ Tl	5130.50(23)	0.058(4)	0.00086(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁴¹ Pr	5140.72(3)	0.269(11)	0.00579(24)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁶⁴ Dy	5142.29(3)	15.7(10)	0.293(19)	184.257(146), 538.609(69.2), 496.931(44.9)
⁵¹ V	5142.363(23)	0.200(6)	0.0119(4)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁹⁰ Os	5146.63(14)	0.409(20)	0.0065(3)	186.7180(2.08), 155.10(1.19), 557.978(0.84)
¹⁹¹ Ir	5147.51(12)	1.29(6)	0.0203(10)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹³⁹ La	5160.902(6)	0.089(5)	0.00194(11)	1596.21(5.84), 487.021(2.79), 815.772(1.430)
¹⁸² W	5164.43(3)	0.19(3)	0.0031(5)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²⁰³ Tl	5180.38(12)	0.141(5)	0.00209(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁵⁵ Mn	5180.89(8)	0.412(13)	0.0227(7)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁵⁹ Co	5181.77(7)	0.912(23)	0.0469(12)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁵¹ V	5210.143(19)	0.244(20)	0.0145(12)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
²⁰³ Tl	5261.48(13)	0.084(4)	0.00125(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁸⁶ W	5261.68(6)	0.86(4)	0.0142(7)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁷⁴ Yb	5266.3(4)	1.4(6)	0.025(11)	514.868(9.0), 639.261(1.43), 396.329(1.42)
¹⁴ N	5269.159(13)	0.0236(3)	0.00511(7)	5297.821(0.01680), 5533.395(0.0155), 1884.821(0.01470)
¹⁹ F	5279.360(20)	0.000421(20)	6.7E-05(3)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
²⁰³ Tl	5279.86(12)	0.207(6)	0.00307(9)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁴ N	5297.821(15)	0.01680(23)	0.00363(5)	5269.159(0.0236), 5533.395(0.0155), 1884.821(0.01470)
¹⁸⁶ W	5320.72(6)	0.605(21)	0.0100(4)	685.73(3.24), 479.550(2.59), 72.002(1.32)
³⁹ K	5380.018(16)	0.146(4)	0.0113(3)	29.8300(1.380), 770.3050(0.903), 1158.887(0.1600)
²⁰³ Tl	5404.41(12)	0.147(5)	0.00218(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
³² S	5420.574(24)	0.308(7)	0.0291(7)	840.993(0.347), 2379.661(0.208), 3220.588(0.117)
²⁰³ Tl	5451.07(14)	0.079(3)	0.00117(4)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁶⁸ Zn	5474.02(10)	0.042(5)	0.00195(23)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
⁹³ Nb	5496.24(10)	0.0205(14)	0.00067(5)	99.4070(0.196), 255.9290(0.176), 253.115(0.1320)
¹³³ Cs	5505.46(20)	0.333(22)	0.0076(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁵¹ V	5515.813(23)	0.39(4)	0.0232(24)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁵⁵ Mn	5527.08(8)	0.788(22)	0.0435(12)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
²⁰³ Tl	5533.35(13)	0.131(5)	0.00194(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁴ N	5533.395(14)	0.0155(8)	0.00335(17)	5269.159(0.0236), 5297.821(0.01680), 1884.821(0.01470)
⁷⁵ As	5533.94(3)	0.151(7)	0.0061(3)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
¹⁹¹ Ir	5534.73(12)	1.39(6)	0.0219(10)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹ F	5543.713(10)	0.000407(17)	6.5E-05(3)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
¹⁶⁴ Dy	5557.26(3)	28.7(14)	0.54(3)	184.257(146), 538.609(69.2), 496.931(44.9)
¹⁴ N	5562.057(13)	0.0084(5)	0.00182(11)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
¹⁹¹ Ir	5564.54(14)	1.71(8)	0.0270(13)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹³³ Cs	5572.00(25)	0.249(20)	0.0057(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
⁴⁰ Ar	5582.4(8)	0.077(8)	0.0058(6)	167.30(0.53), 4745.3(0.36), 1186.8(0.34)
⁷⁶ Se	5600.995(21)	0.301(14)	0.0116(5)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
⁷¹ Ga	5601.75(25)	0.063(4)	0.00274(17)	834.08(1.65), 2201.91(0.52), 629.96(0.490)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma} \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²⁰³ Tl	5603.28(13)	0.282(10)	0.00418(15)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁶⁴ Dy	5607.69(3)	35.9(16)	0.67(3)	184.257(146), 538.609(69.2), 496.931(44.9)
¹³³ Cs	5637.056(17)	0.277(21)	0.0063(5)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
²⁰³ Tl	5641.57(12)	0.316(7)	0.00469(10)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁹⁹ Hg	5658.24(4)	27.5(7)	0.415(11)	367.947(251), 5967.02(62.5), 1693.296(56.2)
⁵⁹ Co	5660.93(4)	1.89(6)	0.097(3)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁴¹ Pr	5666.170(6)	0.379(15)	0.0082(3)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
¹⁹¹ Ir	5667.81(3)	2.68(10)	0.0423(16)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹¹ Ir	5689.06(3)	1.73(7)	0.0273(11)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹⁷ Au	5710.52(10)	1.27(17)	0.020(3)	410(94), 214.9710(9.0), 247.5730(5.56)
³⁵ Cl	5715.244(21)	1.820(16)	0.1556(14)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
¹⁹³ Ir	5728.97(7)	1.15(5)	0.0181(8)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹³⁷ Ba	5730.81(6)	0.0617(20)	0.00136(4)	1435.77(0.308), 627.29(0.294), 818.514(0.212)
¹⁶⁹ Tm	5731.36(11)	1.17(22)	0.021(4)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁶⁹ Tm	5737.51(11)	1.42(7)	0.0255(13)	200(8.72), 149.7180(7.11), 140(5.96)
⁵⁹ Co	5742.53(4)	0.766(23)	0.0394(12)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁵¹ V	5752.064(22)	0.366(24)	0.0218(14)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹⁹¹ Ir	5783.01(3)	1.34(6)	0.0211(10)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁴¹ Pr	5843.026(5)	0.147(6)	0.00316(13)	176.8630(1.06), 140.9050(0.479), 1575.6(0.426)
²⁰³ Tl	5917.48(16)	0.084(4)	0.00125(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁵⁵ Mn	5920.39(8)	1.06(3)	0.0585(17)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁵⁶ Fe	5920.449(21)	0.225(5)	0.0122(3)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
¹⁶⁹ Tm	5941.47(11)	1.51(7)	0.0271(13)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁶⁹ Tm	5943.09(11)	1.03(20)	0.018(4)	200(8.72), 149.7180(7.11), 140(5.96)
¹⁹¹ Ir	5958.28(3)	1.79(8)	0.0282(13)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
¹⁹⁹ Hg	5967.02(4)	62.5(15)	0.944(23)	367.947(251), 1693.296(56.2), 4739.43(30.1)
⁵⁹ Co	5975.98(4)	2.9(4)	0.149(21)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁶⁹ Tm	6001.61(11)	0.99(10)	0.0178(18)	200(8.72), 149.7180(7.11), 140(5.96)
⁷⁶ Se	6006.973(21)	0.289(20)	0.0111(8)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
⁷¹ Ga	6007.25(14)	0.069(5)	0.00300(22)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁹ F	6016.802(16)	0.00094(4)	1.50E-04(6)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁵⁶ Fe	6018.532(20)	0.227(5)	0.0123(3)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
⁸⁹ Y	6080.171(22)	0.76(4)	0.0259(14)	776.613(0.659), 202.53(0.289), 574.106(0.174)
¹⁹¹ Ir	6082.48(3)	2.62(11)	0.0413(17)	351.689(10.9), 328.448(9.1), 84.2740(7.7)
³⁵ Cl	6110.842(18)	6.59(6)	0.563(5)	1164.8650(8.91), 517.0730(7.58), 1951.1400(6.33)
⁷¹ Ga	6111.72(24)	0.055(4)	0.00239(17)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
¹⁸² W	6144.28(3)	0.174(11)	0.00287(18)	685.73(3.24), 479.550(2.59), 72.002(1.32)
²⁰³ Tl	6166.61(14)	0.166(6)	0.00246(9)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹³³ Cs	6175.412(17)	0.252(16)	0.0057(4)	176.4040(2.47), 205.615(1.560), 510.795(1.54)
²⁰³ Tl	6183.05(15)	0.081(4)	0.00120(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
¹⁸² W	6190.78(3)	0.45(4)	0.0074(7)	685.73(3.24), 479.550(2.59), 72.002(1.32)
¹⁵⁹ Tb	6218.56(7)	0.190(22)	0.0036(4)	75.0500(1.78), 63.6860(1.46), 64.1100(1.2)

TABLE 7.4

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
²⁰³ Tl	6222.57(16)	0.065(4)	0.00096(6)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁹¹ Zr	6295.13(16)	0.0279(20)	0.00093(7)	934.4640(0.125), 1465.7(0.063), 1205.6(0.042)
¹⁴ N	6322.428(12)	0.01450(22)	0.00314(5)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
⁷¹ Ga	6358.61(14)	0.138(5)	0.00600(22)	834.08(1.65), 2201.91(0.52), 629.96(0.490)
²⁸ Si	6379.801(21)	0.0207(6)	0.00223(7)	3538.966(0.1190), 4933.889(0.1120), 2092.902(0.0331)
¹⁶⁹ Tm	6387.37(11)	1.48(7)	0.0265(13)	200(8.72), 149.7180(7.11), 140(5.96)
²³ Na	6395.478(15)	0.1000(20)	0.0132(3)	1368.66(0.530), 2754.13(0.530), 472.202(0.478)
⁴⁸ Ti	6418.426(14)	1.96(6)	0.124(4)	1381.745(5.18), 6760.084(2.97), 341.706(1.840)
⁴⁰ Ca	6419.59(5)	0.176(5)	0.0133(4)	1942.67(0.352), 4418.52(0.0708), 2001.31(0.0659)
⁵¹ V	6464.887(18)	0.43(4)	0.0256(24)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
¹³¹ Xe	6467.09(12)	1.33(19)	0.031(4)	667.79(6.7), 772.72(1.78), 536.17(1.71)
⁵⁹ Co	6485.99(3)	2.32(5)	0.119(3)	229.879(7.18), 277.161(6.77), 555.972(5.76)
²⁰³ Tl	6514.57(15)	0.129(5)	0.00191(7)	139.94(0.400), 347.96(0.361), 318.88(0.325)
⁵¹ V	6517.282(19)	0.78(4)	0.0464(24)	1434.10(4.81), 125.082(1.61), 645.703(0.769)
¹²¹ Sb	6523.52(7)	0.075(3)	0.00187(8)	564.24(2.700), 61.4130(0.75), 78.0910(0.48)
¹⁹ F	6600.175(16)	0.00096(3)	1.53E-04(5)	1633.53(0.0096), 583.561(0.00356), 656.006(0.00197)
⁷⁶ Se	6600.690(21)	0.623(20)	0.0239(8)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
³⁵ Cl	6619.615(19)	2.530(23)	0.2163(20)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
³⁵ Cl	6627.821(18)	1.470(16)	0.1257(14)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
⁵³ Cr	6645.61(8)	0.183(13)	0.0107(8)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁵⁹ Co	6706.01(3)	3.02(6)	0.155(3)	229.879(7.18), 277.161(6.77), 555.972(5.76)
¹⁵⁷ Gd	6750.11(5)	965(30)	18.6(6)	181.931(7200), 79.5100(4010), 944.174(3090)
⁴⁸ Ti	6760.084(14)	2.97(9)	0.188(6)	1381.745(5.18), 6418.426(1.96), 341.706(1.840)
⁵⁵ Mn	6783.74(12)	0.378(17)	0.0209(9)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
³¹ P	6785.504(24)	0.0267(15)	0.00261(15)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
⁷⁵ As	6808.872(8)	0.160(8)	0.0065(3)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
⁹ Be	6809.61(3)	0.0058(5)	0.00195(17)	3367.448(0.00285), 853.630(0.00208), 2590.014(0.00191)
⁷⁵ As	6810.898(8)	0.56(3)	0.0227(12)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
⁶² Ni	6837.50(3)	0.458(8)	0.0236(4)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
⁴⁵ Sc	6839.09(4)	0.95(4)	0.064(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁴⁵ Sc	6840.34(4)	0.76(11)	0.051(7)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁵¹ V	6874.157(19)	0.49(6)	0.029(4)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁵⁹ Co	6877.16(3)	3.02(6)	0.155(3)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁶⁶ Zn	6958.8(3)	0.043(3)	0.00199(14)	1077.335(0.356), 115.225(0.167), 7863.55(0.1410)
⁵⁹ Co	6985.41(3)	1.05(13)	0.054(7)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁶³ Cu	6988.68(5)	0.126(6)	0.0060(3)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
⁷⁵ As	7020.139(8)	0.104(7)	0.0042(3)	559.10(2.00), 165.0490(0.996), 86.7880(0.579)
⁵⁵ Mn	7057.89(9)	1.22(3)	0.0673(17)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁵³ Cr	7099.91(6)	0.146(9)	0.0085(5)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁵⁵ Mn	7159.63(10)	0.643(24)	0.0355(13)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁵¹ V	7162.898(15)	0.59(4)	0.0351(24)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
⁶³ Cu	7176.68(5)	0.0925(17)	0.00441(8)	278.250(0.893), 7915.62(0.869), 159.281(0.648)

CHAPTER 7. ADOPTED DATABASE AND USER TABLES

TABLE 7.4. ENERGY ORDERED TABLE OF THE MOST INTENSE THERMAL NEUTRON CAPTURE GAMMA RAYS (cont.)

^A Z	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	k_0	$E_{\gamma}, \sigma_{\gamma}^Z(E_{\gamma})$ for associated intense gamma rays
⁷⁶ Se	7179.492(21)	0.261(25)	0.0100(10)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
²⁸ Si	7199.199(23)	0.0125(4)	0.00135(4)	3538.966(0.1190), 4933.889(0.1120), 2092.902(0.0331)
⁵⁹ Co	7214.42(3)	1.38(3)	0.0710(15)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁵⁵ Mn	7243.52(9)	1.36(3)	0.0750(17)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁶³ Cu	7253.01(5)	0.1500(23)	0.00715(11)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
⁵⁵ Mn	7270.14(12)	0.362(15)	0.0200(8)	846.754(13.10), 1810.72(3.62), 26.560(3.42)
⁵⁶ Fe	7278.838(10)	0.137(4)	0.00743(22)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
¹⁴ N	7298.983(17)	0.00746(12)	0.00161(3)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
⁶³ Cu	7306.93(4)	0.321(17)	0.0153(8)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
⁵¹ V	7310.721(15)	0.227(9)	0.0135(5)	1434.10(4.81), 125.082(1.61), 6517.282(0.78)
²⁰⁷ Pb	7367.78(7)	0.137(3)	0.00200(4)	
³⁵ Cl	7413.968(18)	3.29(5)	0.281(4)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
⁷⁶ Se	7418.467(21)	0.350(13)	0.0134(5)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
³¹ P	7422.022(25)	0.0082(3)	0.00080(3)	512.646(0.079), 78.083(0.059), 636.663(0.0311)
⁵⁹ Co	7491.54(3)	1.16(3)	0.0596(15)	229.879(7.18), 277.161(6.77), 555.972(5.76)
⁶⁰ Ni	7536.637(25)	0.190(4)	0.00981(21)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
⁷⁹ Br	7577.04(8)	0.108(3)	0.00410(11)	776.517(0.990), 554.3480(0.838), 245.203(0.80)
⁸⁵ Rb	7624.07(11)	0.0114(5)	0.000404(18)	556.82(0.0913), 487.89(0.0494), 555.61(0.0407)
⁵⁶ Fe	7631.136(14)	0.653(13)	0.0354(7)	7645.5450(0.549), 352.347(0.273), 6018.532(0.227)
⁶³ Cu	7637.40(4)	0.54(7)	0.026(3)	278.250(0.893), 7915.62(0.869), 159.281(0.648)
⁵⁶ Fe	7645.5450(10)	0.549(11)	0.0298(6)	7631.136(0.653), 352.347(0.273), 6018.532(0.227)
²⁷ Al	7693.397(4)	0.0081(3)	0.00091(3)	1778.92(0.232), 30.6380(0.0798), 7724.027(0.0493)
²⁷ Al	7724.027(4)	0.0493(15)	0.00554(17)	1778.92(0.232), 30.6380(0.0798), 3033.896(0.0179)
³⁵ Cl	7790.330(18)	2.66(3)	0.227(3)	1164.8650(8.91), 517.0730(7.58), 6110.842(6.59)
⁶⁰ Ni	7819.517(21)	0.336(6)	0.0173(3)	8998.414(1.49), 464.978(0.843), 8533.509(0.721)
⁶⁴ Zn	7863.55(7)	0.1410(19)	0.00653(9)	1077.335(0.356), 115.225(0.167), 1883.12(0.0718)
⁶³ Cu	7915.62(4)	0.869(20)	0.0414(10)	278.250(0.893), 159.281(0.648), 7637.40(0.54)
⁵² Cr	7938.46(23)	0.424(11)	0.0247(6)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁴⁵ Sc	8175.176(21)	1.80(6)	0.121(4)	227.773(7.13), 147.011(6.08), 142.528(4.88)
¹⁴ N	8310.161(19)	0.00330(6)	0.000714(13)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
⁵⁰ Cr	8482.80(9)	0.169(7)	0.0098(4)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁵⁰ Cr	8510.77(8)	0.233(8)	0.0136(5)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁴⁵ Sc	8532.122(20)	0.89(4)	0.060(3)	227.773(7.13), 147.011(6.08), 142.528(4.88)
⁵⁸ Ni	8533.509(17)	0.721(13)	0.0372(7)	8998.414(1.49), 464.978(0.843), 6837.50(0.458)
⁵³ Cr	8884.36(5)	0.78(5)	0.045(3)	834.849(1.38), 749.09(0.569), 7938.46(0.424)
⁵⁸ Ni	8998.414(15)	1.49(3)	0.0769(15)	464.978(0.843), 8533.509(0.721), 6837.50(0.458)
⁵⁴ Fe	9297.68(19)	0.0747(25)	0.00405(14)	7631.136(0.653), 7645.5450(0.549), 352.347(0.273)
⁵³ Cr	9719.06(5)	0.260(18)	0.0152(10)	834.849(1.38), 8884.36(0.78), 749.09(0.569)
⁷⁷ Se	9883.35(3)	0.220(22)	0.0084(8)	613.724(2.14), 238.9980(2.06), 520.6370(1.260)
¹⁴ N	10829.120(12)	0.0113(8)	0.00244(17)	5269.159(0.0236), 5297.821(0.01680), 5533.395(0.0155)
³ He	20520.46	4.2E-11(12)	3.2E-11(9)	

Chapter 8

CD-ROM FOR THE PGAA-IAEA DATABASE

R.B. Firestone, V. Zerkin

8.1. INTRODUCTION

Both the database of prompt gamma rays from slow neutron capture for elemental analysis and the results of this CRP are available on the accompanying CD-ROM. The file *index.html* is the home page for the CD-ROM, and provides links to the following information:

- (a) The *CRP* — General information, papers and reports relevant to this CRP.
- (b) The *PGAA-IAEA database viewer* — An interactive program to display and search the PGAA database by isotope, energy or capture cross-section.
- (c) The *Database of Prompt Gamma Rays from Slow Neutron Capture for Elemental Analysis* — This report.
- (d) The *PGAA database files* — Adopted PGAA database and associated files in EXCEL, PDF and Text formats. The archival databases by Lone et al. [8.1] and by Reedy and Frankle [8.2, 8.3] are also available.
- (e) The *Evaluated Gamma-Ray Activation File (EGAF)* — The adopted PGAA database in ENSDF format. Data can be viewed with the Isotope Explorer 2.2 ENSDF Viewer (see (g)).
- (f) The *PGAA database evaluation* — ENSDF format versions of the adopted PGAA database, and the Budapest and ENSDF isotopic input

files. Decay scheme balance and statistical analysis summaries are provided.

- (g) The *Isotope Explorer 2.2 ENSDF viewer* — Windows software for viewing the level scheme drawings and tables provided in ENSDF format. The complete ENSDF database is included, as of December 2002.

The databases and viewers are discussed in greater detail in the following sections.

8.2. THE PGAA-IAEA DATABASE VIEWER

The PGAA-IAEA database viewer, which was developed by Zerkin (IAEA), is provided on the CD-ROM. This viewer is also available on the Internet from the Nuclear Data Service of the IAEA: <http://www-nds.iaea.org>, and contains HTML pages with large portions of JavaScript and GIF plots for the gamma emissions of each isotope. Such a design enables the viewer to be used on many platforms with standard Web browsers. The viewer also includes interactive plotting provided with the ZVView program, which can be used as a helper application. The ZVView programs for Windows and Linux are included in the CD-ROM.

The viewer can be opened in standard mode to view the database, or in advanced mode to search the database. Figure 8.1 shows a periodic table of

17-Chlorine (457) Cl-35 (386) Cl-37 (71)																	
3	4																
Li	Be																
11	12																
Na	Mg																
19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36
K	Ca	Sc	Ti	V	Cr	Mn	Fe	Co	Ni	Cu	Zn	Ga	Ge	As	Se	Br	Kr
37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54
Rb	Sr	Y	Zr	Nb	Mo	Tc	Ru	Rh	Pd	Ag	Cd	In	Sn	Sb	Te	I	Xe
55	56	57*	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86
Cs	Ba	La	Hf	Ta	W	Re	Os	Ir	Pt	Au	Hg	Tl	Pb	Bi	Po	At	Rn
87	88	89**	104	105	106	107	108	109	110	111	112						
Fr	Ra	Ac	Rf	Db	Sg	Bh	Hs	Mt	*	*	*						
* Lanthanides																	
58 59 60 61 62 63 64 65 66 67 68 69 70 71																	
Ce Pr Nd Pm Sm Eu Gd Tb Dy Ho Er Tm Yb Lu																	
** Actinides																	
90 91 92 93 94 95 96 97 98 99 100 101 102 103																	
Th Pa U Np Pu Am Cm Bk Cf Es Fm Md No Lr																	

FIG. 8.1. Periodic table of elements and isotopes displayed by the PGAA-IAEA viewer.

the PGAA elements, as obtained when the viewer is opened.

Clicking with the mouse on an element in the periodic table displays the isotopes of that element and the number of prompt gamma rays in the

database for each isotope. A new window is also opened, as shown in Fig. 8.2, which displays the isotopic and elemental data, as well as histograms of the gamma ray energies and intensities.

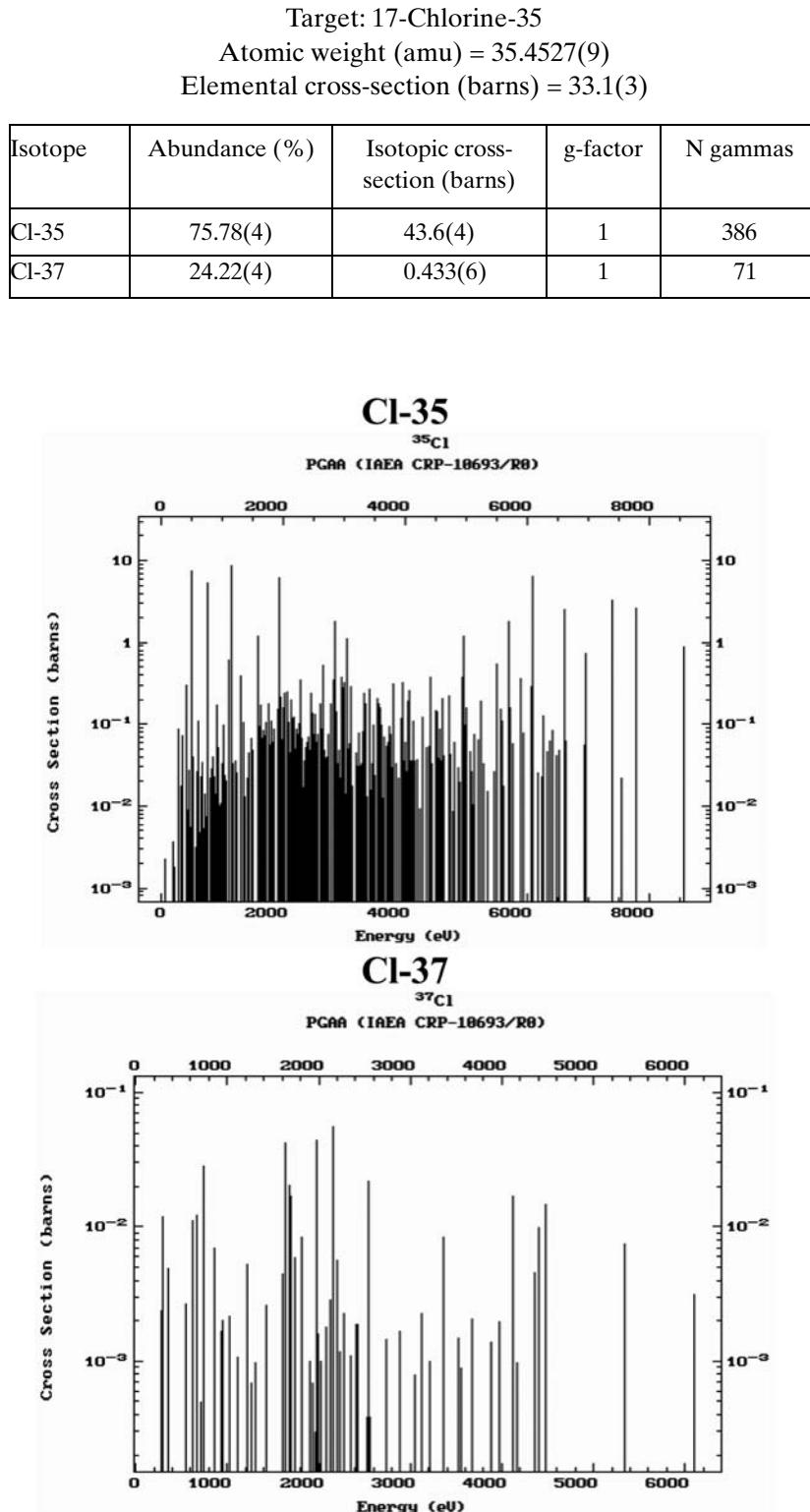


FIG. 8.2. Isotopic and elemental data, and histograms of gamma ray energies and intensities displayed with the PGAA-IAEA viewer.

8.2. THE PGAA-IAEA DATABASE VIEWER

Clicking on an isotope in the selected element box (the square on the left) opens a table of gamma ray energies, cross-sections, type (prompt or decay) and k_0 values, as shown in Fig. 8.3.

An advanced retrieval mode is available in which the viewer displays a gamma ray search window, as shown in Fig. 8.4. There are two options in this mode: either retrieve the whole database (about 35 000 lines) or retrieve a reduced version of it (about 1300 gamma lines). The reduced version

contains lines that are up to 10% of the most intense gamma ray emission for each element, but at least one gamma ray emission for each isotope is independent of the intensity.

The result of the search shown in Fig. 8.4 for gamma rays between 3000 and 3002 keV is displayed in a new window, as shown in Fig. 8.5. The PGAA databases can also be downloaded in Text format from the PGAA-IAEA viewer.

Target: 17-Chlorine-35
 Isotopic Abundance (%): 75.78(4)
 Isotopic Capture Cross-section (barns): 43.6(4)
 Number of Gammas: 386
 Westcott g-factor: 1

Sigma(b): Partial gamma ray production cross-section (barns)
 p – Prompt, d – Delayed, S – Stable

#	<i>E</i> (keV)	<i>Sigma</i> (<i>b</i>)	Type	<i>Half-life</i>	<i>k</i> 0
1	85.747(9)	2.3e-3(5)	p	Stable	6.9e-3(15)
2	204.380(8)	3.7e-3(8)	p	Stable	0.0111(24)
3	225.49(7)	1.58e-3(6)	p	Stable	4.74e-3(18)
4	225.89(5)	1.1e-3(5)	p	Stable	3.3e-3(15)
5	236.775(13)	1.8e-3(6)	p	Stable	5.4e-3(18)
6	292.177(8)	0.0893(10)	p	Stable	0.268(3)
7	302.64(4)	2.1e-3(11)	p	Stable	6e-3(3)
8	337.620(11)	0.018(6)	p	Stable	0.054(18)
9	342.314(7)	5.4e-3(9)	p	Stable	0.016(3)
10	358.291(6)	0.0736(20)	p	Stable	0.221(6)
11	369(4)	0.019(5)	p	Stable	0.057(15)
12	371.3(25)	1.4e-3(3)	p	Stable	4.2e-3(9)
13	376.4460(20)	1.3e-3(3)	p	Stable	3.9e-3(9)
14	427.89(10)	9.9e-3(16)	p	Stable	3e-2(5)
15	428.060(8)	3.9e-3(7)	p	Stable	0.0117(21)
16	435.964(13)	0.051(8)	p	Stable	0.153(24)
17	436.222(4)	0.309(20)	p	Stable	0.928(6)
18	455.58(11)	4.3e-3(21)	p	Stable	0.013(6)
19	459.46(8)	9e-3(3)	p	Stable	0.027(9)
20	463.72(4)	2e-3(16)	p	Stable	6e-3(5)
21	464.8(5)	4e-3(3)	p	Stable	0.012(9)
22	465.9(11)	5e-3(15)	p	Stable	0.015(5)
23	466.63(15)	1e-2(5)	p	Stable	3e-2(15)
24	468.359(7)	0.0274(20)	p	Stable	0.082(6)
25	478.4(25)	0.027(15)	p	Stable	8e-2(5)

FIG. 8.3. Display of a partial table of gamma ray energies, cross-sections, prompt or decay type, and k_0 value (the complete table contains 386 gamma rays).

Gamma-Ray Search

	Energy (keV)	Z	A	CS
<i>From</i>	<input checked="" type="checkbox"/> 3000	<input type="checkbox"/> 20	<input type="checkbox"/> 43	<input type="checkbox"/> 1e-4
<i>To</i>	<input checked="" type="checkbox"/> 3002	<input type="checkbox"/> 30	<input type="checkbox"/> 44	<input type="checkbox"/> 1e-3

Type: All Prompt Delayed

Sort by: Energy Cross-section

FIG. 8.4. Gamma ray search window: data can be selected from the entire database by energy, atomic number, mass number, delayed or prompt type, and/or cross-section, and the results can be sorted by energy or cross-section.

P G A A -						
n	Energy, keV	Isotope	Sigma, b	Type	Half-life	k_0
1	3001.07(5)	Cl-35	0.216(7)	p	S	0.649(21)
2	3001.17(13)	La-139	2.2e-3(23)	p	S	6.6e-3(7)
3	3001.55(5)	K-40	1.3e-5(3)	p	S	3.9e-5(9)
4	3001.89(15)	Ca-40	7.3e-4(19)	p	S	2.2e-3(6)
5	3001.97(13)	Sc-45	0.043(12)	p	S	0.13(4)

p – prompt, d – delayed, S – stable

FIG. 8.5. Display of results of a search for gamma rays with $E = 3000\text{--}3002 \text{ keV}$.

8.3. THE PGAA DATA FILES

The PGAA database and associated files are provided in various formats. Microsoft Excel format files include elemental data (atomic weights and elemental cross-sections), isotopic data (abundances, cross-sections and g factors) and gamma ray data (energies, cross-sections and k_0 values). Tables of isotopic data, decay parent data, gamma ray data, g factors and references from this report are provided in Adobe PDF and PostScript. Energies and cross-sections for adopted prompt and decay gamma rays, and for input ENSDF and Budapest gamma rays, are available in Text format.

8.4. THE EVALUATED GAMMA RAY ACTIVATION FILE

This file contains the recommended PGAA database in ENSDF format. The nuclear structure information associated with these data is also preserved, along with three data sets for neutron capture gamma rays: adopted PGAA, Budapest PGAA and the Reedy and Frankle data [8.2, 8.3]. The Evaluated Gamma-Ray Activation File can be viewed by means of the Isotope Explorer 2.2 ENSDF viewer (Section 8.6).

8.4. THE EVALUATED GAMMA RAY ACTIVATION FILE

8.5. PGAA DATABASE EVALUATION

Selecting an element in the HTML periodic table provides a detailed summary of the evaluation. The atomic abundances and cross-sections found by Mughabghab et al. are given for each isotope [8.4–8.6]. All Budapest and ENSDF input databases and the final adopted data are provided in ENSDF format. A summary of the initial matching of the Budapest data to the ENSDF data is given as a Text file for determining isotopic assignments. This file contains all of the gamma rays measured at Budapest, and was subsequently edited to select only those gamma rays that could be reliably placed in a known level scheme. Additional Text files show the least squares energy and intensity fits, as well as the intensity balance of the decay scheme for all relevant data sets. Summary HTML tables are provided that compare the adopted, ENSDF, Budapest, Reedy and Frankle [8.2, 8.3], and Lone et al. [8.1] data.

Presented is the total cross-section as deduced from the total measured gamma ray intensity feeding the ground state and/or de-exciting the capture state. This parameter can also be deduced in some cases from the gamma ray intensity of short lived radioisotopes. If the decay scheme is dominated by continuum or unobserved gamma rays that populate the ground state, this cross-section should be considered to be a lower limit. The agreement between the data of Mughabghab [8.4] and the current measurements was excellent in a good many cases. Data that exceed the values of Mughabghab may indicate that the adopted values are too low, particularly when the overall intensity balances are correct. The new cross-section results should be taken as a guide to the overall quality of the data; we do not recommend that these values be quoted until further analysis can be performed.

8.6. THE ISOTOPE EXPLORER 2.2 ENSDF VIEWER

The Isotope Explorer 2.2 ENSDF viewer by Firestone and Chu (Lawrence Berkeley National Laboratory, USA) and Ekström (Lund University, Sweden) can be installed on Windows PCs to display level scheme drawings and tables from the data provided in ENSDF format. A ‘tour’ of Isotope Explorer’s capabilities is provided, as shown in Fig. 8.6. Links are available to download and install the program, and a detailed user manual is included. The program is installed by going to the download link, clicking on the self-extracting program archive IE223.EXE (50 MB), choosing ‘OPEN’, and extracting the program and files to the selected directory. The application can be run from this directory or a short cut can be created to the extension ‘.ENS’ used for the PGAA ENSDF data. Associating this extension with Isotope Explorer in the PC will allow direct runs when opening the file. The ENSDF format files can also be read with a text editor, and an ENSDF format manual is provided.

When running Isotope Explorer directly from the executable file, the user is prompted to select an isotope. The program can be configured to select data from a local or an Internet database. A copy of the complete ENSDF file is included on the CD-ROM, which can be downloaded from the installation menu and used as the local database.

The user can open an ENSDF file directly from the Isotope Explorer file menu. Figure 8.7 shows an example of a level scheme display for the $^{24}\text{Mg}(n, \gamma)$ reaction. Only the lowest tier of gamma rays is shown, and the user must scroll through the display to see gamma rays from the capture state. Different displays can be chosen with the Addview menu. In Fig. 8.8, a tabular display is shown for the $^{24}\text{Mg}(n, \gamma)$ reaction data. Other features, including plots and chart generation, are described in the Isotope Explorer manual.

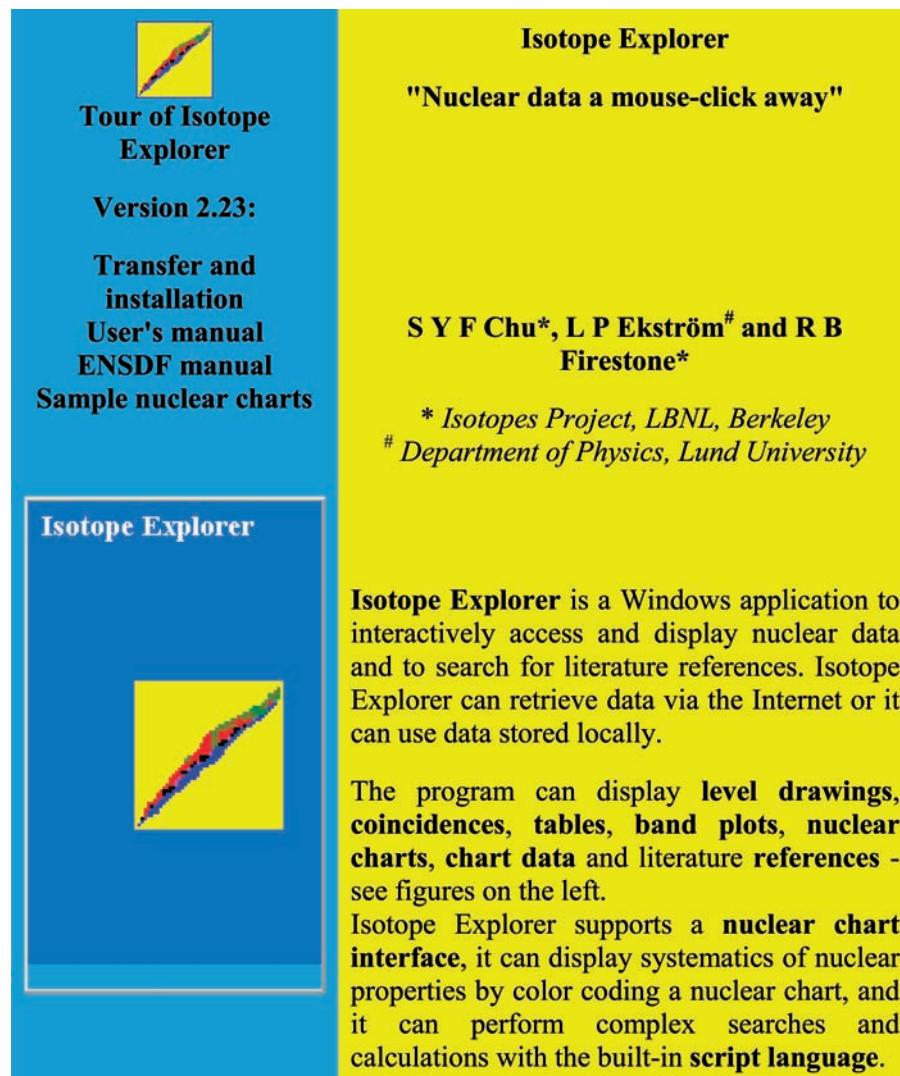
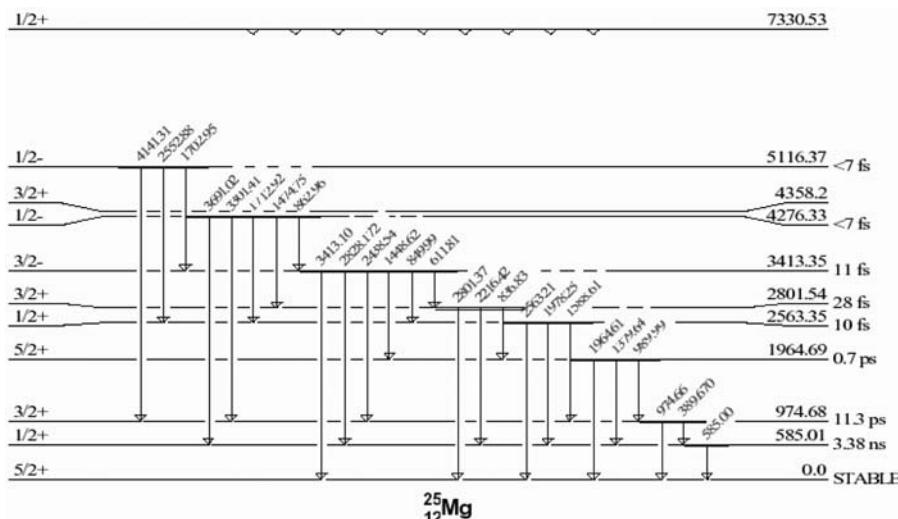


FIG. 8.6. Tour of Isotope Explorer 2.2.

FIG. 8.7. Level scheme displayed with Isotope Explorer (in this case for the $^{24}\text{Mg}(n, \gamma)$ reaction): gamma rays are displayed in tiers that can be scrolled through.

Gammas for $^{25}\text{Mg}; ^{24}\text{Mg}(n, \gamma)$ E=thermalGeneral Comments

SIGMAN=0.051 5 (1981MUZQ)

 I_{γ} Normalization: NORMALIZATION FROM 1992WA06.

E_{γ}	E_{level}	$J\pi_i$	$J\pi_f$	Mult \dagger	$\delta\dagger$	I_{γ}^{\dagger}	$T_{1/2}$
389.670 21	974.68 3	3/2+	1/2+	M1+E2	+0.13 3	0.00586 24	11.3 ps 3
585.00 3	585.01 3	1/2+	5/2+	E2(+M3)	~ 0	0.0314 11	3.38 ns 5
611.81 9	3413.35 3	3/2-	3/2+			1.2×10^{-5} 12	11 fs 4
836.83 6	2801.54 9	3/2+	5/2+	M1(+E2)	-0.03 3	1.58×10^{-4} 15	28 fs 7
849.99 4	3413.35 3	3/2-	1/2+			6.6×10^{-5} 11	11 fs 4
862.96 3	4276.33 4	1/2-	3/2-	[M1]		0.000410 21	<7 fs
974.66 3	974.68 3	3/2+	5/2+	M1+E2	+0.36 2	0.00663 24	11.3 ps 3
989.99 10	1964.69 10	5/2+	3/2+	M1+E2	-0.25 2	3.9×10^{-5} 8	0.7 ps 3
1379.64 9	1964.69 10	5/2+	1/2+	E2(+M3)	~ 0	8.4×10^{-5} 11	0.7 ps 3
1448.62 10	3413.35 3	3/2-	5/2+			1.2×10^{-5} 12	11 fs 4
1474.75 10	4276.33 4	1/2-	3/2+			1.2×10^{-5} 12	<7 fs
1588.61 4	2563.35 4	1/2+	3/2+			0.000250 23	10 fs 3
1702.95 15	5116.37 15	1/2-	3/2-	M1+E2	+0.09 7	3.2×10^{-5} 10	<7 fs
1712.92 4	4276.33 4	1/2-	1/2+	E1		0.00118 7	<7 fs
1964.61 10	1964.69 10	5/2+	5/2+	M1+E2	-0.60 10	8.1×10^{-5} 18	0.7 ps 3
1978.25 3	2563.35 4	1/2+	1/2+	M1		0.00111 5	10 fs 3
2214.06 15	7330.53 4	1/2+	1/2-	[E1]		0.00030 3	
2216.42 9	2801.54 9	3/2+	1/2+			1.9×10^{-4} 3	28 fs 7
2438.54 3	3413.35 3	3/2-	3/2+	E1(+M2)	~ 0	0.00473 19	11 fs 4
2552.88 15	5116.37 15	1/2-	1/2+	M1(+E2)	-0.19 9	2.4×10^{-5} 9	<7 fs
2563.21 4	2563.35 4	1/2+	5/2+	[E2]		5.5×10^{-5} 16	10 fs 3
2801.37 9	2801.54 9	3/2+	5/2+	M1+E2	-0.64 8	1.31×10^{-4} 16	28 fs 7
2828.172 25	3413.35 3	3/2-	1/2+	E1(+M2)	~ 0	0.0240 8	11 fs 4

FIG. 8.8. Tabular display as listed by Isotope Explorer of gamma ray data for the $^{24}\text{Mg}(n, \gamma)$ reaction.**REFERENCES TO CHAPTER 8**

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Appendix I

GAMMA RAY CROSS-SECTION DATA MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR

Z. Révay, G.L. Molnár

Table I.1 contains isotopic gamma ray energy and thermal neutron radiative cross-sections measured with the thermal neutron beam at the Budapest reactor. Only transitions with $\sigma_{\gamma}^Z(E)$ larger than 5% of the highest cross-section for

gamma rays with energies higher than 100 keV are listed for each element. The complete set of data is available on the CD-ROM accompanying this report. These data are discussed in greater detail in Chapter 6.

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR

E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)	E_{γ} (keV)	$\sigma_{\gamma}^Z(E_{\gamma})$ (b)
Hydrogen			
2223.2590(10)	0.3326(7)	3531.98(5)	0.00686(12)
Deuterium			
6250.2(1)	0.000492(25)	3677.80(5)	0.01140(15)
Lithium			
980.48(4)	0.00410(14)	4508.69(6)	0.01290(21)
1051.81(5)	0.00410(12)	5268.98(7)	0.0237(4)
2032.300(20)	0.0387(12)	5297.66(15)	0.0167(3)
7246.7(3)	0.0024(3)	5533.25(8)	0.01570(25)
Beryllium			
853.631(11)	0.00165(15)	5561.95(8)	0.00863(15)
2590.014(25)	0.00188(17)	6322.30(9)	0.0149(3)
3367.48(4)	0.0029(3)	7298.90(10)	0.00772(16)
3443.42(4)	0.00099(9)	8310.17(13)	0.00336(9)
6809.58(10)	0.0062(6)	9149.24(17)	0.00133(6)
Boron			
480(3)	713.0(23)	10829.10(21)	0.0107(4)
Carbon			
1261.71(6)	0.00123(3)	Oxygen	
3684.02(7)	0.00117(4)	870.68(3)	1.75(11)E-4
4945.30(7)	0.00270(8)	1087.71(3)	1.51(9)E-4
Nitrogen			
1678.24(3)	0.00625(9)	2184.38(4)	1.75(11)E-4
1681.17(4)	0.00130(4)	3272.11(7)	3.53(25)E-5
1884.85(3)	0.01450(18)	Fluorine	
1999.69(3)	0.00321(5)	166.61(3)	0.000405(20)
2520.45(4)	0.00425(8)	556.29(3)	2.01(10)E-4
2830.80(5)	0.00133(4)	583.493(22)	0.00352(15)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
1148.02(5)	0.000252(16)	1129.42(3)	0.0090(4)
1309.12(3)	0.00076(4)	1808.62(6)	0.0181(8)
1387.82(3)	0.00079(4)	2438.42(9)	0.00459(22)
1542.47(5)	0.000265(17)	2828.12(10)	0.0239(11)
1843.68(4)	0.00059(3)	2881.52(11)	0.00279(15)
2143.20(7)	1.94(14)E-4	3053.85(12)	0.0083(4)
2427.83(11)	1.87(18)E-4	3301.29(13)	0.0063(3)
2431.04(7)	0.00041(3)	3413.04(14)	0.00400(20)
2529.21(6)	0.00065(4)	3561.14(14)	0.00252(13)
3014.61(7)	0.000407(25)	3831.25(16)	0.00408(20)
3051.56(10)	0.000301(23)	3916.65(16)	0.0314(15)
3112.88(9)	2.17(16)E-4	5451.79(23)	0.00205(12)
3488.15(8)	0.00077(5)	8153.4(4)	0.00271(19)
3586.23(14)	0.00026(3)	Aluminium	
3589.42(15)	2.0(3)E-4	831.41(5)	0.00269(7)
3964.85(10)	0.00039(3)	982.94(4)	0.00902(14)
4556.90(11)	0.00044(3)	1013.57(4)	0.00555(10)
5033.53(11)	0.00070(4)	1408.27(4)	0.00640(13)
5279.42(13)	0.00042(4)	1526.12(4)	0.00339(9)
5291.46(15)	2.3(3)E-4	1589.59(4)	0.00247(7)
5543.70(13)	0.00039(4)	1622.90(3)	0.00989(15)
5616.88(16)	1.76(15)E-4	1927.44(4)	0.00262(7)
6017.04(11)	0.00094(6)	2108.19(4)	0.00549(11)
6600.39(11)	0.00099(5)	2138.82(4)	0.00424(9)
Sodium		2271.77(4)	0.00396(10)
90.979(16)	0.235(3)	2282.71(4)	0.00890(17)
472.222(13)	0.478(4)	2577.53(5)	0.00412(10)
869.221(17)	0.1080(13)	2590.10(5)	0.00807(16)
874.399(18)	0.0759(11)	2625.67(5)	0.00264(6)
1636.23(4)	0.0250(7)	2821.31(6)	0.00752(15)
2025.15(5)	0.0338(9)	3033.75(6)	0.0179(3)
2208.27(5)	0.0254(7)	3464.87(8)	0.0146(3)
2517.59(5)	0.0695(11)	3590.93(9)	0.01000(21)
2752.27(7)	0.0654(12)	3848.95(10)	0.00699(17)
3587.31(7)	0.0596(12)	3875.35(10)	0.00618(14)
3981.15(8)	0.0678(12)	4133.20(10)	0.0149(3)
6395.05(13)	0.1010(20)	4259.35(11)	0.0153(3)
Magnesium		4659.81(13)	0.00605(16)
389.64(3)	0.0058(3)	4690.48(13)	0.01090(24)
584.936(24)	0.0316(15)	4733.63(14)	0.0126(3)
974.61(3)	0.0067(3)	4902.89(14)	0.00716(18)
1003.05(3)	0.00165(8)	5133.99(15)	0.00722(23)

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TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
5410.79(16)	0.00481(19)	2379.50(4)	0.208(3)
5585.38(19)	0.00279(12)	2753.09(5)	0.0277(5)
6101.54(19)	0.00570(21)	2930.59(5)	0.0832(13)
6315.91(20)	0.00500(20)	3220.36(6)	0.1240(20)
7693.1(3)	0.0081(3)	3369.48(6)	0.0272(5)
7723.78(25)	0.0493(15)	4430.28(9)	0.0263(6)
Silicon			
1273.38(3)	0.0289(6)	4869.19(9)	0.0652(13)
2092.91(3)	0.0330(6)	5420.24(10)	0.309(7)
3538.98(5)	0.1180(20)	Chlorine	
3660.73(6)	0.00705(21)	517.077(8)	7.43(7)
4933.83(7)	0.1120(23)	786.18(15)	3.6(17)
5106.60(10)	0.0065(3)	788.37(21)	4.9(23)
6379.75(11)	0.0210(6)	1131.180(15)	0.634(10)
7199.02(13)	0.0127(4)	1162.56(5)	0.71(3)
7199.02(13)	0.0127(4)	1164.831(12)	8.92(7)
Phosphorus			
77.992(23)	0.059(3)	1601.055(14)	1.230(15)
512.650(18)	0.079(4)	1951.150(15)	6.49(5)
636.570(17)	0.0310(14)	1959.359(16)	4.18(4)
1071.154(20)	0.0248(12)	2676.11(3)	0.524(10)
1322.639(25)	0.00526(25)	2863.76(3)	1.830(25)
1676.81(3)	0.00402(20)	3061.76(3)	1.110(19)
1941.01(4)	0.00411(20)	4979.75(5)	1.260(24)
2114.32(4)	0.0114(5)	5517.13(8)	0.578(17)
2151.42(4)	0.0099(5)	5715.16(7)	1.86(4)
2156.74(4)	0.0127(6)	6110.71(7)	7.37(11)
2585.82(5)	0.0088(4)	6619.58(8)	2.75(4)
2885.89(5)	0.0064(3)	6627.87(8)	1.56(3)
3057.94(6)	0.0109(5)	6977.75(10)	0.794(21)
3273.87(7)	0.0084(4)	7413.92(10)	3.57(6)
3522.49(7)	0.0224(11)	7790.28(11)	2.89(6)
3899.65(8)	0.0301(14)	8578.58(15)	0.93(3)
Potassium			
4199.70(9)	0.0057(3)	770.325(23)	0.903(12)
4364.24(9)	0.0074(4)	1158.880(24)	0.1600(25)
4660.97(10)	0.0057(3)	1247.20(3)	0.0784(13)
4671.21(9)	0.0199(10)	1303.42(3)	0.0550(12)
5265.46(11)	0.0060(3)	1613.76(3)	0.1190(20)
5705.41(13)	0.00447(25)	1618.98(3)	0.1300(21)
6785.30(14)	0.0276(14)	2007.71(4)	0.0513(12)
7422.08(17)	0.0086(5)	2017.49(4)	0.0540(12)
Sulphur			
841.013(14)	0.348(6)	2039.94(4)	0.0519(13)
841.013(14)	0.348(6)	2047.33(4)	0.0537(13)

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TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
2073.67(4)	0.1370(24)	1693.35(5)	0.465(19)
2290.64(5)	0.0582(13)	1857.62(6)	0.393(17)
2545.92(6)	0.0536(12)	4974.54(10)	0.498(24)
3055.30(7)	0.0464(12)	5267.04(10)	0.38(3)
3545.64(9)	0.0746(18)	5896.90(17)	0.42(3)
4135.58(9)	0.0563(17)	6170.24(16)	0.47(5)
4360.22(9)	0.0776(21)	6317.64(25)	0.58(4)
5379.96(12)	0.146(4)	6349.4(3)	0.53(4)
5695.45(13)	0.114(3)	6556.82(14)	0.384(24)
5751.76(13)	0.108(3)	6839.73(11)	0.95(4)
Calcium			
519.56(8)	0.0503(13)	7117.01(18)	0.39(3)
1942.68(3)	0.352(7)	7635.42(20)	0.40(3)
2001.31(3)	0.0659(15)	8132.37(18)	0.48(3)
2009.84(3)	0.0409(10)	8175.07(10)	1.80(6)
3609.84(9)	0.0284(9)	8315.75(16)	0.41(3)
4418.50(12)	0.0708(18)	8532.07(12)	0.89(4)
5899.99(20)	0.0258(12)	Titanium	
6419.69(21)	0.176(5)	341.69(3)	1.840(21)
Scandium			
52.049(21)	0.87(3)	1381.74(3)	5.18(12)
142.627(16)	4.88(7)	1498.65(3)	0.297(5)
147.114(16)	6.08(9)	1585.95(3)	0.624(8)
216.475(17)	2.49(4)	1762.02(3)	0.311(4)
227.860(16)	7.13(11)	4881.24(6)	0.308(7)
228.806(16)	3.31(5)	6418.35(8)	1.96(6)
295.343(19)	3.97(11)	6555.87(9)	0.334(8)
486.054(21)	0.593(14)	6760.01(9)	2.97(9)
539.466(25)	0.738(19)	Vanadium	
547.14(3)	0.373(12)	125.23(3)	1.61(4)
554.555(23)	1.82(4)	148.09(3)	0.253(6)
584.80(3)	1.77(3)	295.196(25)	0.164(4)
627.477(22)	2.23(5)	419.624(24)	0.249(6)
721.78(3)	0.487(15)	436.765(23)	0.397(9)
773.834(22)	0.572(13)	645.789(22)	0.769(17)
807.74(3)	0.523(13)	793.614(23)	0.199(5)
860.66(3)	0.396(13)	823.26(3)	0.320(8)
1123.41(5)	0.380(14)	846.046(24)	0.252(7)
1166.60(4)	0.386(14)	1358.52(3)	0.151(5)
1285.31(9)	0.373(19)	1558.89(3)	0.323(8)
1335.04(3)	0.640(22)	1778.02(13)	0.169(13)
1618.16(7)	0.362(19)	2145.88(7)	0.140(4)
		4117.10(21)	0.094(4)
		5142.40(14)	0.200(6)

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TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
5210.18(16)	0.244(20)	2330.55(7)	0.191(8)
5515.90(17)	0.39(4)	3267.17(7)	0.188(6)
5752.27(14)	0.366(24)	3408.61(5)	0.303(10)
5892.46(15)	0.126(7)	4566.56(10)	0.197(9)
6465.09(18)	0.43(4)	4689.14(11)	0.120(9)
6517.62(15)	0.78(4)	4724.84(8)	0.281(10)
6874.48(20)	0.49(6)	4949.21(8)	0.274(10)
7163.17(18)	0.59(4)	5014.37(7)	0.737(20)
7294.13(23)	0.089(5)	5034.60(15)	0.108(8)
7310.98(21)	0.227(9)	5067.87(9)	0.265(12)
Chromium			
564.14(3)	0.1130(20)	5253.98(12)	0.132(13)
749.10(3)	0.569(9)	5527.08(8)	0.788(22)
834.80(3)	1.38(3)	5761.23(11)	0.200(12)
1784.41(4)	0.177(3)	5920.39(8)	1.06(3)
1898.90(4)	0.0851(21)	6104.29(12)	0.213(10)
2238.78(4)	0.185(3)	6783.74(12)	0.378(17)
2320.80(4)	0.136(3)	6929.22(13)	0.248(12)
5617.37(10)	0.132(5)	7057.89(9)	1.22(3)
6134.19(12)	0.078(4)	7159.63(10)	0.643(24)
7361.09(14)	0.091(4)	7243.52(9)	1.36(3)
7373.85(15)	0.080(4)	7270.14(12)	0.362(15)
7937.86(12)	0.424(11)	Iron	
8482.84(14)	0.168(7)	122.078(22)	0.096(3)
8510.68(14)	0.231(8)	352.332(16)	0.273(3)
Manganese			
83.884(23)	3.11(5)	366.737(16)	0.0497(7)
104.611(23)	1.74(3)	691.914(16)	0.1370(18)
188.521(22)	0.330(6)	898.14(3)	0.0540(10)
212.039(21)	2.13(3)	1018.860(21)	0.0507(11)
215.150(22)	0.168(3)	1260.353(21)	0.0684(11)
230.096(24)	0.193(4)	1612.77(3)	0.1530(22)
271.198(22)	0.94(6)	1725.255(24)	0.181(3)
314.398(20)	1.460(20)	2721.18(5)	0.0384(13)
335.502(24)	0.147(3)	3267.30(6)	0.0367(13)
375.192(22)	0.124(3)	3413.14(6)	0.0449(14)
454.378(21)	0.388(7)	3436.57(13)	0.045(4)
459.754(23)	0.210(5)	3854.17(7)	0.0333(12)
2043.99(5)	0.243(5)	4217.93(6)	0.099(3)
2062.81(4)	0.179(5)	4405.90(7)	0.0453(13)
2175.91(5)	0.111(4)	4809.70(8)	0.0416(13)
2294.42(7)	0.112(6)	5920.25(8)	0.225(5)
		6018.29(8)	0.227(5)

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TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
7278.83(10)	0.137(4)	6876.76(11)	3.02(6)
7631.05(9)	0.653(13)	6984.9(4)	1.05(13)
7645.48(9)	0.549(11)	7055.43(12)	0.666(19)
9297.90(21)	0.0747(25)	7203.02(13)	0.369(16)
Cobalt			
58.90(22)	0.392(4)	7214.09(12)	1.38(3)
158.519(12)	1.200(15)	7491.29(12)	1.16(3)
229.811(12)	7.18(8)	282.940(18)	0.211(3)
254.371(12)	1.290(16)	339.370(18)	0.1660(21)
277.199(11)	6.77(8)	464.972(18)	0.843(10)
391.221(12)	1.080(14)	877.984(19)	0.236(3)
435.671(12)	0.789(10)	5817.17(6)	0.1090(24)
447.717(11)	3.41(4)	6583.78(7)	0.0837(21)
461.064(15)	0.519(9)	6837.44(6)	0.458(8)
484.284(11)	0.804(11)	7536.56(8)	0.191(4)
497.264(13)	2.16(4)	7819.55(8)	0.337(6)
555.941(10)	5.76(6)	8120.60(9)	0.133(3)
710.493(16)	0.660(12)	8533.45(8)	0.721(13)
717.302(14)	0.845(14)	8998.31(9)	1.49(3)
726.616(21)	0.448(10)	Copper	
785.614(17)	2.41(7)	88.86(3)	0.0970(17)
901.148(18)	0.418(9)	159.02(3)	0.649(8)
930.47(5)	0.408(22)	185.66(3)	0.244(3)
1215.965(20)	0.520(9)	202.69(3)	0.1940(25)
1507.28(3)	0.463(9)	277.993(25)	0.893(12)
1515.695(25)	1.740(25)	343.651(25)	0.215(3)
1830.77(3)	1.700(23)	384.27(3)	0.0701(11)
1852.70(3)	0.456(10)	385.37(3)	0.1310(18)
2032.74(4)	0.393(11)	464.857(25)	0.1350(21)
3748.76(7)	0.415(13)	467.74(3)	0.0673(13)
4906.06(17)	0.43(3)	503.45(3)	0.0596(10)
5181.14(12)	0.912(23)	579.48(3)	0.0899(14)
5269.92(12)	0.404(11)	608.52(3)	0.266(5)
5602.39(10)	0.434(16)	648.53(3)	0.101(3)
5614.04(10)	0.399(15)	662.67(5)	0.067(5)
5638.55(10)	0.379(15)	5417.60(9)	0.0564(23)
5660.68(16)	1.89(6)	6009.96(18)	0.0453(25)
5742.16(9)	0.766(23)	6600.08(13)	0.078(5)
5925.39(10)	0.643(18)	6674.12(13)	0.0534(24)
5975.60(22)	2.9(4)	6679.64(11)	0.067(3)
6486.17(13)	2.32(5)	6987.99(9)	0.092(3)
6705.52(10)	3.02(6)	7175.93(12)	0.070(4)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
7252.10(11)	0.114(5)	194.67(3)	0.1060(21)
7306.25(9)	0.245(6)	198.00(3)	0.1330(24)
7571.23(14)	0.047(3)	211.08(3)	0.0343(8)
7636.75(9)	0.428(9)	212.58(3)	0.0582(12)
7915.00(9)	0.869(16)	229.06(3)	0.0377(10)
Zinc			
53.97(3)	0.0225(20)	248.95(4)	0.140(10)
61.2530(20)	0.055(5)	264.02(4)	0.0238(9)
93.386(22)	0.0343(8)	266.09(4)	0.0361(11)
115.256(23)	0.167(3)	315.95(4)	0.0275(9)
153.124(22)	0.0322(6)	318.87(3)	0.0592(14)
184.665(20)	0.0321(4)	374.37(4)	0.0303(10)
300.317(25)	0.0202(6)	390.64(3)	0.0477(12)
751.68(3)	0.0307(10)	393.26(3)	0.1340(23)
834.78(3)	0.0372(12)	411.11(3)	0.0384(11)
855.66(8)	0.066(6)	508.19(3)	0.349(6)
909.65(4)	0.0186(8)	651.09(3)	0.1030(22)
1007.806(25)	0.0557(15)	690.943(24)	0.305(4)
1077.336(17)	0.356(5)	1140.37(4)	0.0422(16)
1126.10(3)	0.0224(7)	1203.40(6)	0.0286(14)
1261.17(3)	0.0433(11)	1311.89(6)	0.0259(12)
1340.15(3)	0.0431(13)	4839.99(13)	0.040(3)
1673.46(5)	0.0255(11)	5194.5(3)	0.033(3)
1883.11(4)	0.0726(22)	5233.47(14)	0.0341(20)
2210.12(9)	0.0270(13)	5334.13(18)	0.0271(18)
4137.28(12)	0.0196(23)	5340.59(14)	0.0409(22)
5473.74(12)	0.040(4)	5488.31(17)	0.0296(19)
6867.51(17)	0.0243(17)	5601.79(15)	0.063(4)
6910.92(16)	0.0192(14)	6008.11(14)	0.070(5)
6958.45(12)	0.042(3)	6111.19(16)	0.056(4)
7069.17(17)	0.0217(14)	6128.73(23)	0.024(3)
7863.54(11)	0.141(5)	6360.02(13)	0.138(5)
88.97(3)	0.0306(9)	6513.06(18)	0.0325(20)
Gallium			
103.25(3)	0.0525(11)	Germanium	
112.46(3)	0.155(3)	175.05(3)	0.164(4)
145.24(3)	0.465(7)	253.22(3)	0.0609(16)
153.90(3)	0.0319(8)	325.74(3)	0.0649(18)
181.60(7)	0.037(4)	492.989(22)	0.133(3)
184.13(3)	0.1040(21)	499.966(22)	0.158(4)
187.84(3)	0.1080(21)	595.879(20)	1.100(24)
192.09(3)	0.194(3)	608.375(21)	0.250(6)
		701.490(24)	0.0642(19)
		708.14(3)	0.0821(23)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
867.940(23)	0.553(12)	7020.0(3)	0.104(7)
961.04(4)	0.129(4)	Selenium	
999.78(3)	0.0581(19)	87.87(3)	0.210(4)
1101.22(3)	0.134(3)	139.28(3)	0.542(9)
1105.56(3)	0.0708(20)	161.99(3)	0.855(22)
1204.14(4)	0.141(4)	200.50(4)	0.240(10)
1471.75(5)	0.083(3)	239.06(3)	2.06(3)
Arsenic		249.85(3)	0.539(9)
74.88(8)	0.12(3)	281.68(3)	0.125(4)
86.83(3)	0.579(11)	286.62(3)	0.280(6)
116.91(7)	0.107(18)	297.26(3)	0.338(7)
117.58(10)	0.071(18)	439.52(3)	0.320(8)
120.28(3)	0.402(8)	467.77(4)	0.128(4)
122.26(3)	0.227(5)	484.45(4)	0.125(4)
127.55(3)	0.096(3)	518.21(4)	0.274(7)
135.48(3)	0.156(4)	520.68(3)	1.270(19)
141.24(4)	0.0625(21)	578.85(3)	0.244(5)
144.60(3)	0.1000(22)	613.72(3)	2.14(5)
157.79(8)	0.117(24)	694.88(3)	0.444(10)
165.09(3)	0.996(16)	755.34(3)	0.186(4)
178.16(3)	0.0979(23)	817.86(4)	0.175(5)
187.94(4)	0.090(3)	885.40(4)	0.262(7)
198.70(3)	0.089(3)	888.84(4)	0.180(5)
211.18(3)	0.113(3)	1005.01(4)	0.118(5)
221.60(4)	0.0534(25)	1240.06(5)	0.109(5)
225.76(3)	0.0803(24)	1296.92(4)	0.241(7)
235.84(3)	0.181(4)	1308.60(4)	0.317(9)
263.88(5)	0.18(4)	1411.51(9)	0.117(6)
281.56(6)	0.085(20)	1713.48(6)	0.159(7)
297.55(4)	0.055(3)	1995.83(6)	0.123(6)
300.44(5)	0.051(3)	4526.6(3)	0.118(8)
352.41(4)	0.071(3)	4565.5(3)	0.163(12)
357.36(4)	0.074(3)	5025.57(12)	0.141(12)
363.94(4)	0.059(3)	5600.89(13)	0.287(14)
402.64(4)	0.061(3)	5795.65(17)	0.112(15)
426.62(3)	0.100(3)	6006.85(13)	0.269(16)
471.05(3)	0.203(5)	6232.01(17)	0.177(17)
473.21(3)	0.176(5)	6413.36(15)	0.184(15)
550.48(4)	0.071(3)	6600.67(12)	0.613(20)
6295.2(4)	0.064(6)	7179.51(15)	0.237(19)
6810.11(21)	0.160(8)	7418.52(14)	0.342(13)
6926.22(22)	0.061(4)	9188.42(21)	0.128(8)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
9883.30(22)	0.180(10)	7422.40(14)	0.0495(18)
Bromine			
59.57(3)	0.202(5)	7576.27(14)	0.108(3)
195.64(3)	0.434(14)	Rubidium	
211.62(4)	0.0454(21)	113.75(3)	0.00535(14)
219.37(3)	0.399(14)	196.34(3)	0.00964(19)
223.64(3)	0.153(5)	421.494(23)	0.0259(5)
234.32(3)	0.205(10)	487.89(3)	0.0494(12)
244.31(4)	0.45(3)	514.55(3)	0.00653(20)
245.23(3)	0.80(3)	536.50(3)	0.0167(5)
271.39(3)	0.462(7)	538.66(3)	0.0169(5)
274.54(3)	0.158(3)	555.61(3)	0.0407(10)
287.76(3)	0.253(4)	556.81(3)	0.0913(24)
294.32(3)	0.1160(22)	638.82(6)	0.0101(13)
299.95(16)	0.08(8)	691.57(3)	0.00725(18)
315.05(3)	0.460(9)	872.93(3)	0.0321(5)
343.42(4)	0.118(4)	881.53(4)	0.00480(17)
345.09(4)	0.154(4)	913.12(4)	0.00497(15)
366.58(4)	0.233(6)	1026.35(3)	0.0218(4)
389.10(4)	0.0486(13)	1032.32(3)	0.0227(4)
432.20(3)	0.0783(14)	1105.51(4)	0.0151(3)
452.69(6)	0.0679(24)	1304.45(4)	0.0204(5)
459.76(6)	0.0455(19)	1389.31(5)	0.00809(21)
468.91(4)	0.29(3)	1666.78(6)	0.00774(23)
512.22(5)	0.21(3)	6065.00(25)	0.0047(3)
542.39(4)	0.114(5)	6471.30(25)	0.0049(3)
549.45(3)	0.0593(14)	6520.7(3)	0.0064(4)
565.98(4)	0.0551(12)	6832.2(3)	0.0064(4)
608.70(4)	0.0438(13)	7346.0(3)	0.0059(3)
660.38(6)	0.082(3)	7624.1(3)	0.0114(5)
Strontium			
684.84(5)	0.050(3)	388.526(22)	0.0517(9)
689.87(4)	0.083(4)	585.610(20)	0.0704(14)
701.97(4)	0.0648(14)	850.671(17)	0.275(4)
715.93(4)	0.0420(23)	898.063(16)	0.703(10)
765.75(5)	0.0537(16)	1218.548(24)	0.0597(13)
830.72(4)	0.0413(12)	1717.81(3)	0.0672(15)
860.41(7)	0.0450(19)	1836.05(3)	1.030(18)
914.25(4)	0.0508(14)	3009.34(7)	0.0579(16)
976.41(4)	0.0459(13)	6266.82(17)	0.075(3)
1248.78(12)	0.0527(22)	6660.38(18)	0.064(3)
7030.72(15)	0.0447(22)	7527.58(20)	0.067(3)
7077.34(14)	0.0566(24)		

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Yttrium			
202.58(4)	0.291(4)	775.75(4)	0.0158(6)
574.13(4)	0.172(4)	835.75(4)	0.0376(8)
776.64(3)	0.659(9)	878.99(8)	0.0191(17)
1211.56(4)	0.0447(12)	883.74(5)	0.0192(7)
1371.09(6)	0.0400(12)	894.27(5)	0.0185(7)
4107.52(6)	0.0518(17)	896.96(6)	0.0144(7)
6080.12(7)	0.754(13)	911.61(5)	0.0176(7)
Zirconium			
160.94(10)	0.0111(7)	957.27(4)	0.0248(7)
266.78(7)	0.0091(5)	1121.9(3)	0.0106(13)
448.13(7)	0.0067(3)	1129.01(10)	0.0175(15)
560.91(6)	0.0285(5)	1192.10(7)	0.0137(7)
844.08(7)	0.0095(4)	1206.48(8)	0.0284(10)
912.71(7)	0.0117(5)	1223.01(10)	0.0121(7)
934.47(6)	0.125(5)	1228.40(11)	0.0114(7)
1102.67(6)	0.0235(8)	1239.54(10)	0.0096(7)
1132.10(7)	0.0100(7)	1291.47(8)	0.0097(7)
1206.89(8)	0.0417(25)	1392.82(9)	0.0105(8)
1405.02(6)	0.0301(10)	1459.99(10)	0.0095(6)
1847.78(15)	0.0084(8)	4739.39(23)	0.0153(9)
5262.7(4)	0.0064(8)	5070.5(3)	0.0102(8)
6294.86(18)	0.0279(20)	5103.62(24)	0.0232(12)
Niobium			
78.63(3)	0.0169(3)	5193.8(3)	0.0114(8)
99.41(3)	0.196(9)	5496.46(25)	0.0205(14)
113.39(3)	0.117(3)	5895.3(3)	0.0183(8)
161.24(3)	0.0190(5)	6831.7(3)	0.0175(8)
253.135(23)	0.1320(19)	7186.6(3)	0.0089(6)
255.957(23)	0.176(3)	Molybdenum	
293.223(25)	0.0651(16)	608.753(18)	0.121(4)
309.926(25)	0.0690(17)	719.523(17)	0.310(10)
329.19(3)	0.0108(4)	736.814(16)	0.119(4)
337.48(4)	0.054(6)	778.221(10)	2.02(6)
458.47(3)	0.0240(5)	787.398(15)	0.168(5)
499.48(3)	0.0648(18)	847.605(12)	0.324(9)
518.16(3)	0.0579(13)	1091.298(25)	0.201(6)
527.64(5)	0.0127(7)	1200.13(4)	0.124(4)
562.29(5)	0.0293(11)	1497.65(5)	0.122(4)
689.78(4)	0.0164(6)	6918.7(4)	0.106(6)
751.69(5)	0.0143(6)	Ruthenium	
755.30(5)	0.0123(6)	475.0950(10)	0.98(9)
		539.522(11)	1.53(13)
		627.974(16)	0.176(16)
		631.24(3)	0.30(3)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
686.890(13)	0.52(5)	717.349(14)	0.777(9)
822.610(19)	0.137(12)	1045.77(3)	0.321(7)
1046.4980(20)	0.103(9)	1050.30(3)	0.360(8)
1103.03(3)	0.100(9)	1127.99(3)	0.323(6)
1341.52(3)	0.137(12)	1572.57(9)	0.22(3)
1362.02(7)	0.111(13)	Silver	
1627.24(3)	0.129(12)	78.91(4)	3.90(12)
1959.33(3)	0.210(19)	105.61(5)	0.76(4)
6627.84(14)	0.093(9)	113.51(6)	0.52(3)
7790.53(16)	0.132(13)	117.45(3)	3.84(7)
Rhodium		191.39(3)	1.81(5)
51.34(4)	14.6(16)	192.90(3)	2.20(6)
85.19(3)	3.2(3)	195.34(4)	0.50(3)
96.99(3)	20.1(4)	198.52(3)	7.75(13)
100.68(3)	4.96(10)	201.31(6)	0.45(3)
127.21(3)	5.27(21)	206.46(3)	3.58(7)
134.54(3)	6.8(4)	215.15(4)	1.55(3)
169.26(7)	2.88(19)	235.62(3)	4.62(7)
177.64(4)	1.85(12)	236.85(4)	1.95(3)
180.73(3)	22.6(12)	259.17(3)	1.560(25)
185.93(3)	1.50(5)	267.08(3)	2.73(6)
202.69(5)	1.6(3)	270.00(4)	0.565(25)
212.92(3)	1.27(3)	286.91(4)	0.400(25)
215.35(3)	6.74(12)	294.39(3)	2.05(12)
217.75(3)	7.38(13)	299.95(3)	1.15(5)
266.60(3)	2.66(14)	328.99(3)	0.795(12)
269.17(3)	1.42(11)	338.742(25)	0.595(10)
323.79(10)	1.54(19)	349.95(3)	0.70(4)
333.44(3)	3.27(8)	357.77(5)	0.561(22)
374.79(3)	1.300(25)	360.39(3)	1.55(3)
420.61(3)	2.06(4)	378.12(5)	0.744(20)
440.52(3)	2.23(10)	380.90(3)	1.59(3)
470.41(3)	2.61(7)	408.61(3)	0.459(9)
482.24(3)	1.78(6)	465.37(6)	0.46(3)
786.94(4)	1.16(3)	495.714(25)	1.080(18)
5917.04(14)	1.31(4)	524.473(25)	0.804(11)
Palladium		536.125(24)	1.090(16)
113.47(3)	0.335(5)	549.560(23)	1.540(24)
245.128(24)	0.250(4)	586.81(3)	0.459(8)
325.310(23)	0.208(3)	593.88(3)	0.484(11)
511.847(13)	4.00(4)	620.08(4)	0.40(5)
616.219(15)	0.628(9)	626.41(4)	0.39(6)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
632.95(3)	0.42(12)	471.92(4)	2.43(14)
657.741(22)	2.36(3)	476.13(8)	1.05(7)
724.75(4)	0.393(14)	492.52(5)	1.87(11)
750.77(3)	0.529(11)	518.06(5)	1.74(11)
1013.11(3)	0.698(13)	521.62(7)	1.11(8)
5701.49(20)	0.716(18)	548.70(5)	1.14(8)
5795.02(24)	0.513(14)	556.67(4)	2.61(15)
6058.03(22)	0.663(19)	577.45(8)	1.10(10)
Cadmium			
558.32(3)	1860(30)	608.34(4)	1.97(11)
576.04(3)	107.0(17)	634.03(9)	0.94(7)
651.19(3)	358(5)	693.24(5)	1.02(7)
725.19(3)	107.0(13)	819.00(6)	1.43(10)
805.85(3)	134.0(18)	847.50(6)	1.21(8)
1209.65(4)	122.0(19)	5892.38(15)	1.17(9)
1364.30(4)	123.0(21)	Tin	
1399.54(4)	97.7(15)	158.65(6)	0.0145(3)
Indium			
60.97(4)	8.6(5)	463.31(6)	0.0128(3)
85.66(4)	11.1(6)	703.87(7)	0.0078(3)
96.11(4)	13.8(7)	733.91(6)	0.00925(21)
126.49(4)	2.05(11)	813.26(7)	0.0071(3)
141.17(7)	1.61(24)	818.71(6)	0.0127(4)
155.40(5)	1.38(9)	925.90(6)	0.0097(3)
162.50(4)	15.8(8)	931.81(6)	0.0111(3)
171.16(4)	1.92(10)	972.59(6)	0.0158(5)
173.87(4)	2.30(14)	1171.28(6)	0.0879(13)
186.32(4)	14.9(8)	1229.64(6)	0.0673(13)
202.58(5)	1.50(9)	1293.53(6)	0.1340(21)
235.21(4)	2.75(15)	1356.70(7)	0.0075(3)
273.05(4)	18.3(9)	2112.17(7)	0.0152(5)
285.00(4)	2.54(14)	2225.15(18)	0.0082(5)
291.00(4)	1.42(8)	Antimony	
295.58(4)	1.55(9)	87.83(4)	0.212(6)
298.72(4)	4.78(25)	88.96(9)	0.0220(25)
321.24(5)	1.28(8)	101.69(5)	0.0215(11)
335.47(4)	4.59(24)	103.79(5)	0.0578(18)
337.84(5)	1.39(8)	105.95(4)	0.161(4)
375.89(4)	1.47(9)	115.04(4)	0.271(9)
385.06(4)	6.8(4)	121.64(4)	0.360(8)
422.23(5)	0.97(6)	124.17(5)	0.0310(14)
433.80(4)	3.62(20)	133.95(4)	0.0608(19)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
138.12(5)	0.0286(12)	631.81(4)	0.0581(16)
141.54(5)	0.0577(18)	746.85(9)	0.034(3)
143.35(5)	0.0331(14)	775.58(9)	0.020(3)
148.39(4)	0.257(6)	824.31(9)	0.040(3)
155.27(5)	0.091(3)	921.04(4)	0.076(4)
166.56(5)	0.0699(23)	5563.4(3)	0.0200(24)
167.73(6)	0.0512(20)	5868.89(22)	0.035(3)
173.91(6)	0.0192(11)	5885.08(20)	0.055(4)
194.20(4)	0.0534(18)	6009.1(3)	0.020(3)
201.70(4)	0.091(3)	6048.81(25)	0.0184(25)
204.68(5)	0.0355(15)	6082.94(22)	0.0182(23)
232.23(4)	0.0356(12)	6363.5(3)	0.024(3)
233.28(4)	0.0996(24)	6379.82(22)	0.043(4)
246.42(4)	0.0589(16)	6467.8(4)	0.022(3)
252.89(4)	0.0474(14)	6523.87(18)	0.075(3)
255.54(7)	0.027(3)	6728.38(23)	0.045(4)
256.37(8)	0.021(3)	Tellurium	
265.51(6)	0.0299(16)	602.723(12)	2.37(24)
272.36(7)	0.0225(14)	645.823(14)	0.26(3)
274.22(8)	0.0388(18)	722.729(15)	0.52(5)
275.72(8)	0.0306(16)	1488.89(3)	0.120(12)
282.73(4)	0.274(7)	2746.94(5)	0.138(14)
286.60(5)	0.0375(17)	Iodine	
288.21(7)	0.0267(18)	124.27(4)	0.183(8)
313.97(5)	0.0318(18)	133.59(4)	1.42(6)
322.19(5)	0.0390(20)	142.12(4)	0.156(7)
330.91(6)	0.058(3)	147.10(4)	0.109(5)
332.15(5)	0.101(3)	152.99(4)	0.214(9)
335.09(5)	0.0284(14)	156.49(4)	0.118(5)
351.57(5)	0.0345(15)	160.71(4)	0.192(8)
378.14(5)	0.0500(18)	193.54(4)	0.127(5)
384.55(4)	0.0702(22)	224.15(4)	0.095(4)
419.95(7)	0.071(8)	248.73(4)	0.149(6)
485.34(6)	0.0218(15)	268.32(4)	0.082(4)
491.21(5)	0.0354(16)	301.89(4)	0.229(9)
513.88(8)	0.0359(21)	344.76(4)	0.102(5)
542.35(8)	0.0270(20)	374.27(5)	0.091(5)
546.01(6)	0.0315(20)	385.46(4)	0.087(4)
555.18(12)	0.024(4)	420.85(5)	0.144(11)
564.26(5)	0.0532(25)	Xenon	
598.66(5)	0.058(3)	483.77(9)	0.51(7)
603.49(12)	0.020(3)	536.29(9)	1.71(24)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
586.23(10)	0.48(7)	523.47(17)	0.151(23)
600.22(9)	0.54(8)	525.08(9)	0.39(3)
630.40(9)	1.38(19)	529.15(7)	0.519(23)
667.87(9)	6.9(10)	539.16(7)	0.360(11)
772.76(9)	1.9(3)	554.51(7)	0.206(9)
1028.88(8)	0.40(6)	557.57(11)	0.142(12)
1318.00(8)	1.03(14)	570.42(7)	0.221(12)
6467.02(13)	1.33(19)	645.53(9)	0.248(13)
Caesium			
59.85(7)	0.443(14)	662.98(9)	0.155(9)
113.60(7)	0.777(15)	708.20(7)	0.220(11)
116.21(7)	2.83(4)	911.24(12)	0.177(14)
118.04(8)	0.230(7)	966.47(10)	0.168(13)
120.42(7)	0.414(10)	1077.67(9)	0.209(12)
130.05(7)	1.410(21)	5493.52(23)	0.230(19)
174.06(7)	0.420(11)	5505.46(20)	0.333(22)
176.21(7)	2.47(4)	5572.00(25)	0.249(20)
186.67(7)	0.282(9)	5637.41(23)	0.277(21)
198.11(7)	1.100(19)	5748.9(3)	0.146(15)
205.43(7)	1.560(25)	6052.3(3)	0.240(20)
211.15(7)	0.223(10)	6175.64(22)	0.252(16)
218.18(7)	0.309(9)	6189.11(24)	0.191(14)
219.57(7)	0.344(9)	6697.91(24)	0.224(17)
234.15(7)	1.070(23)	Barium	
245.66(7)	0.740(15)	283.67(5)	0.0403(10)
256.44(7)	0.235(8)	454.78(5)	0.0858(22)
260.99(7)	0.401(11)	462.80(5)	0.0656(17)
268.82(7)	0.199(6)	627.30(5)	0.293(6)
293.15(8)	0.185(9)	732.32(5)	0.0239(7)
295.24(8)	0.231(10)	818.47(5)	0.212(4)
307.07(7)	1.45(3)	1009.61(5)	0.0167(5)
309.52(7)	0.237(9)	1047.74(5)	0.0319(10)
316.87(8)	0.149(10)	1435.65(6)	0.308(8)
356.06(7)	0.445(12)	1444.71(6)	0.0799(21)
367.54(8)	0.173(8)	1550.86(7)	0.0228(8)
377.05(7)	0.310(9)	1898.47(8)	0.0285(11)
386.73(7)	0.163(9)	2594.00(10)	0.0185(8)
442.66(8)	0.316(12)	2639.09(11)	0.0170(8)
450.27(8)	0.99(5)	3641.22(13)	0.0560(16)
502.86(8)	0.256(13)	4095.77(15)	0.154(4)
510.81(9)	1.54(3)	4723.12(18)	0.0262(11)
518.91(7)	0.349(18)	5730.58(22)	0.0612(20)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Lanthanum			
63.26(3)	0.176(6)	3265.07(13)	0.049(5)
155.65(3)	0.192(5)	3281.12(14)	0.048(5)
162.74(3)	0.490(13)	3424.65(11)	0.070(4)
209.29(4)	0.0434(19)	3442.03(16)	0.040(3)
218.30(3)	0.781(21)	3476.53(16)	0.048(4)
235.82(3)	0.111(3)	3606.05(14)	0.054(4)
237.747(24)	0.320(6)	3609.85(16)	0.047(3)
255.49(3)	0.0409(15)	3665.23(8)	0.132(6)
272.420(22)	0.502(8)	3679.24(8)	0.137(6)
280.01(3)	0.0644(25)	3727.27(11)	0.069(4)
283.69(4)	0.0411(25)	3737.46(25)	0.042(4)
288.333(23)	0.729(12)	3900.56(14)	0.053(4)
422.742(23)	0.371(7)	4389.17(9)	0.256(9)
426.51(5)	0.044(3)	4415.77(10)	0.240(9)
478.11(5)	0.0408(22)	4502.26(11)	0.159(7)
495.66(3)	0.081(3)	4558.45(14)	0.047(3)
538.93(5)	0.0455(25)	4842.33(9)	0.656(17)
549.02(3)	0.098(3)	4888.37(12)	0.146(7)
553.19(6)	0.061(4)	5097.40(10)	0.680(18)
567.413(23)	0.335(7)	5125.96(15)	0.110(7)
Cerium			
595.07(3)	0.103(3)	475.09(6)	0.082(7)
602.02(4)	0.0524(25)	662.03(5)	0.233(18)
623.60(4)	0.0518(23)	737.43(7)	0.026(3)
640.62(6)	0.054(3)	765.97(5)	0.0145(12)
658.30(3)	0.103(3)	1107.66(5)	0.040(3)
667.67(4)	0.058(3)	1153.97(5)	0.0146(12)
708.22(4)	0.134(4)	4290.99(8)	0.053(4)
710.07(8)	0.067(3)	4336.46(8)	0.0251(20)
722.52(3)	0.212(5)	4765.96(9)	0.109(9)
782.86(8)	0.040(3)	Praseodymium	
868.11(6)	0.056(3)	60.18(5)	0.134(14)
991.83(7)	0.049(3)	64.56(5)	0.137(6)
1020.36(7)	0.054(3)	68.67(5)	0.116(6)
2757.44(9)	0.050(5)	85.16(5)	0.207(11)
2862.97(9)	0.066(4)	126.92(4)	0.307(15)
2924.52(12)	0.040(3)	140.98(3)	0.479(10)
2988.29(19)	0.045(4)	176.95(3)	1.06(4)
3016.74(9)	0.065(3)	182.87(3)	0.377(14)
3035.23(11)	0.046(3)	460.24(5)	0.057(3)
3082.71(7)	0.135(5)	508.89(6)	0.104(10)
3188.94(15)	0.045(4)	528.23(3)	0.0579(19)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
546.47(3)	0.148(4)	Europium	
560.48(4)	0.150(7)	52.39(9)	55(3)
570.15(4)	0.112(5)	56.73(16)	16(6)
573.88(5)	0.084(5)	59.79(14)	10(3)
619.35(3)	0.152(4)	63.43(23)	12(5)
633.19(4)	0.113(4)	65.1(3)	16(8)
645.651(25)	0.311(7)	68.23(9)	69(20)
729.24(3)	0.0712(23)	71.24(12)	45(14)
746.94(3)	0.146(4)	73.21(9)	106(22)
893.36(5)	0.053(3)	74.86(12)	43(12)
956.89(7)	0.091(7)	77.40(8)	187(13)
991.87(6)	0.138(10)	79.78(22)	12(6)
1006.30(5)	0.153(8)	82.51(13)	7(5)
1102.83(7)	0.056(3)	85.28(13)	9(5)
1150.98(4)	0.141(5)	87.13(11)	29(3)
3602.56(16)	0.054(3)	88.31(12)	42(5)
3650.12(16)	0.061(3)	89.97(8)	1430(30)
3653.98(14)	0.060(4)	91.20(10)	20(10)
3790.15(11)	0.140(6)	95.25(11)	8(3)
4496.29(16)	0.098(6)	100.86(23)	24(5)
4691.91(14)	0.291(10)	103.34(13)	48(5)
4722.39(22)	0.083(4)	106.57(14)	42(6)
4800.96(16)	0.140(8)	109.63(13)	22(9)
5095.9(4)	0.208(8)	111.0(3)	22(6)
5137.43(22)	0.098(4)	113.1(3)	15(5)
5140.60(17)	0.269(11)	117.54(10)	14.7(22)
5665.98(18)	0.379(15)	119.71(13)	11.9(25)
5842.92(18)	0.147(6)	121.71(11)	17.7(25)
Neodymium		124.01(16)	25(3)
453.920(20)	3.00(9)	125.19(16)	25(3)
618.044(16)	13.4(3)	129.06(12)	14.7(16)
696.487(20)	33.2(17)	130.93(15)	15.0(16)
742.088(18)	3.07(8)	132.71(10)	20.7(13)
814.128(20)	5.05(13)	135.42(9)	27.8(14)
864.356(22)	5.08(13)	137.89(20)	7(3)
1413.16(3)	1.85(6)	140.19(9)	21(4)
6502.32(14)	3.18(11)	143.54(8)	43(3)
Samarium		148.80(22)	13(4)
334.02(5)	4790(60)	150.59(19)	7(3)
712.25(5)	268(4)	154.14(9)	22(3)
737.48(5)	598(8)	157.22(7)	7.5(22)
		158.31(21)	9.3(16)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
160.29(16)	9.3(17)	295.41(10)	13.4(5)
163.89(14)	13.1(24)	297.40(12)	7.0(4)
167.01(13)	18.9(19)	299.83(8)	24.0(6)
169.28(9)	54.8(22)	304.22(9)	7.3(6)
171.95(9)	40(3)	309.71(8)	11.5(9)
176.6(3)	6(3)	313.97(24)	4.5(10)
179.83(13)	20(3)	316.18(12)	10.8(9)
182.38(11)	23(3)	318.95(11)	11.7(9)
187.37(8)	31.2(14)	321.61(12)	9.8(8)
190.96(11)	19.7(14)	326.15(21)	12(4)
193.11(13)	28.3(20)	330.82(11)	9.0(8)
194.73(25)	11.7(20)	334.45(10)	11.1(10)
197.10(16)	14.1(14)	337.58(23)	4.1(9)
199.12(10)	25.5(15)	340.01(17)	5.5(9)
203.63(10)	18.4(14)	344.53(10)	7.1(14)
206.53(8)	58.7(20)	348.73(12)	7.5(13)
208.51(18)	16.1(21)	353.10(18)	4.4(4)
209.93(25)	8.5(24)	354.81(12)	8.7(14)
214.57(17)	13(3)	358.27(11)	7.6(15)
221.30(8)	73(3)	360.06(17)	5.1(4)
225.11(21)	11.2(23)	364.82(10)	7.8(5)
228.7(4)	5.6(22)	366.57(9)	8.8(7)
233.22(14)	15.9(23)	369.39(15)	5.9(8)
239.25(23)	12.4(25)	370.82(12)	8.3(5)
243.1(3)	12.2(20)	376.75(9)	8.4(5)
244.88(24)	26.3(22)	378.98(10)	6.5(4)
246.5(3)	15(3)	381.56(10)	5.3(5)
253.52(10)	11(3)	388.00(16)	4.3(6)
256.20(9)	12.0(25)	390.61(12)	8.7(7)
260.66(9)	15.9(18)	392.96(12)	7.5(6)
265.0(5)	3.8(5)	396.92(11)	7.5(6)
266.96(14)	8.0(11)	400.52(19)	4.2(6)
270.84(10)	6.5(11)	404.34(14)	9.6(9)
273.65(8)	17.3(12)	411.61(17)	5.3(7)
276.14(9)	10.9(11)	414.24(11)	9.1(8)
279.91(14)	6.9(5)	423.32(10)	13.1(10)
281.78(9)	20.4(8)	427.02(13)	8.0(9)
283.53(24)	5.9(4)	433.04(10)	10.3(11)
285.10(9)	23.2(18)	438.1(3)	5.3(9)
287.29(10)	11.5(8)	440.83(24)	6.2(9)
288.82(11)	9.3(6)	444.6(3)	4.7(10)
293.68(14)	6.0(4)	449.85(20)	5.4(11)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
472.38(12)	5.3(9)	68.25(24)	0.035(14)
526.49(11)	4.3(4)	74.89(8)	1.78(18)
5379.7(4)	9.2(19)	76.77(12)	0.089(12)
5500.68(18)	7.0(4)	79.28(8)	0.43(6)
5595.20(20)	5.3(4)	84.21(14)	0.050(10)
5816.5(8)	3.7(12)	87.46(9)	0.160(19)
6069.29(18)	8.2(7)	89.04(9)	0.21(3)
6229.7(7)	4.1(8)	93.06(8)	0.218(25)
Gadolinium			
79.71(6)	4040(110)	97.36(8)	0.50(6)
89.17(6)	1380(40)	101.16(15)	0.023(5)
182.12(6)	7300(400)	103.80(9)	0.089(10)
199.42(6)	2000(600)	108.69(14)	0.026(5)
255.80(6)	373(30)	112.26(9)	0.089(10)
277.73(6)	495(12)	117.76(12)	0.028(5)
780.15(6)	1020(23)	131.00(9)	0.064(8)
870.85(6)	434(11)	135.44(8)	0.39(4)
897.66(5)	1080(50)	139.03(15)	0.052(6)
897.66(5)	1200(50)	141.06(11)	0.107(12)
915.11(6)	392(11)	150.45(7)	0.144(15)
944.70(10)	3080(70)	153.52(7)	0.44(5)
962.18(5)	1980(50)	158.85(7)	0.111(12)
977.22(5)	1420(30)	163.02(7)	0.105(11)
1003.97(7)	391(30)	176.79(10)	0.070(9)
1097.03(5)	660(16)	184.37(13)	0.11(3)
1107.51(6)	1840(40)	193.32(7)	0.37(4)
1116.52(5)	418(10)	209.61(8)	0.055(6)
1119.23(5)	1180(30)	212.38(12)	0.032(4)
1141.36(7)	474(30)	214.61(11)	0.036(5)
1184.32(7)	1160(120)	220.96(12)	0.022(4)
1186.75(5)	1550(190)	228.09(9)	0.032(4)
1186.75(5)	1600(190)	234.38(18)	0.026(5)
1259.91(5)	420(11)	235.88(14)	0.032(6)
1263.73(5)	644(16)	238.81(18)	0.023(5)
1323.48(5)	641(17)	241.64(20)	0.035(8)
5903.39(13)	453(14)	243.03(8)	0.219(24)
6750.05(14)	963(30)	247.98(7)	0.30(3)
Terbium			
59.48(8)	0.48(6)	255.39(12)	0.112(16)
61.59(25)	0.052(15)	257.81(14)	0.045(7)
63.74(8)	1.46(16)	262.32(22)	0.022(6)
65.94(15)	0.090(17)	264.75(14)	0.031(7)
65.94(15)	0.090(17)	270.57(8)	0.102(12)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
275.49(8)	0.124(14)	459.70(9)	0.085(12)
277.64(9)	0.093(11)	464.28(7)	0.192(21)
278.75(7)	0.083(11)	491.51(23)	0.024(6)
282.86(12)	0.049(8)	497.07(15)	0.041(9)
284.10(9)	0.087(11)	519.73(19)	0.059(13)
288.07(7)	0.126(14)	521.32(23)	0.046(12)
290.41(9)	0.052(7)	525.65(8)	0.22(3)
295.87(9)	0.062(8)	529.24(6)	0.022(8)
302.75(8)	0.086(10)	532.71(8)	0.129(16)
308.04(9)	0.056(8)	541.57(8)	0.121(15)
310.46(8)	0.177(21)	545.14(11)	0.064(10)
315.81(8)	0.118(14)	585.69(13)	0.054(8)
317.42(8)	0.121(15)	600.02(7)	0.155(18)
319.75(8)	0.132(15)	611.47(18)	0.034(9)
339.35(7)	0.35(4)	625.64(16)	0.027(7)
341.01(9)	0.069(9)	634.67(11)	0.037(7)
345.29(8)	0.128(16)	5184.6(6)	0.023(9)
348.61(13)	0.053(10)	5228.0(5)	0.052(12)
350.99(10)	0.176(22)	5238.6(7)	0.026(10)
352.37(10)	0.160(21)	5245.4(6)	0.061(13)
356.22(11)	0.117(17)	5288.8(5)	0.027(7)
357.64(8)	0.26(3)	5460.9(5)	0.029(7)
359.90(16)	0.048(9)	5524.3(4)	0.051(13)
361.61(10)	0.095(12)	5608.1(6)	0.042(9)
363.69(9)	0.120(15)	5661.3(5)	0.037(7)
369.90(8)	0.057(7)	5684.4(6)	0.027(7)
372.86(9)	0.070(8)	5754.6(4)	0.031(8)
374.51(8)	0.099(11)	5776.2(3)	0.120(17)
376.11(7)	0.154(16)	5784.1(4)	0.041(9)
378.60(8)	0.161(19)	5842.1(11)	0.054(10)
379.8(3)	0.024(8)	5860.8(10)	0.036(8)
399.42(11)	0.074(11)	5891.2(3)	0.137(19)
404.69(10)	0.127(17)	5896.0(6)	0.023(7)
414.66(16)	0.092(22)	5953.5(3)	0.103(13)
420.55(8)	0.092(12)	5993.8(3)	0.114(15)
426.89(7)	0.147(17)	6138.4(3)	0.110(15)
437.21(11)	0.077(16)	6218.5(3)	0.190(22)
441.73(13)	0.077(12)	6240.8(3)	0.072(10)
447.20(17)	0.10(3)	6268.7(4)	0.029(6)
451.44(15)	0.21(3)	6311.9(7)	0.028(6)
453.14(22)	0.033(12)		
455.4(3)	0.029(12)		

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Dysprosium			
50.44(7)	33.9(15)	5557.15(17)	28.7(14)
80.64(7)	12.0(4)	5607.73(18)	35.9(16)
108.23(7)	15.6(5)	Holmium	
184.34(7)	146(15)	69.79(4)	1.09(6)
185.19(9)	33.8(9)	116.84(4)	8.1(4)
260.11(7)	8.3(3)	136.67(4)	14.5(7)
282.89(7)	7.8(3)	149.32(4)	2.25(12)
349.14(8)	14.7(6)	180.96(5)	0.94(5)
351.20(8)	10.9(5)	221.18(4)	2.05(11)
386.08(7)	34.8(10)	239.13(4)	2.25(12)
389.83(8)	7.3(4)	289.04(4)	1.16(6)
392.66(7)	11.3(5)	290.61(4)	0.96(5)
411.71(7)	35.1(10)	304.63(4)	1.34(7)
415.03(7)	30.8(9)	333.61(4)	1.04(6)
421.10(10)	11.8(11)	371.74(4)	1.56(8)
447.96(7)	17.4(5)	401.57(4)	1.07(9)
465.46(7)	38.0(10)	410.45(4)	1.23(7)
470.25(8)	9.3(6)	425.90(4)	2.88(15)
477.10(7)	15.8(5)	455.53(4)	0.78(4)
496.96(7)	44.9(11)	489.45(4)	1.15(6)
499.43(9)	13.0(10)	542.74(4)	1.94(13)
500.62(9)	10.3(5)	543.69(4)	1.00(5)
Erbium			
509.06(9)	9.5(6)	99.07(3)	3.73(14)
510.81(14)	8.5(7)	184.301(25)	56(5)
515.33(7)	9.7(5)	198.267(24)	29.9(16)
538.65(7)	69.2(19)	284.71(3)	13.7(12)
570.05(9)	9.7(5)	447.556(24)	3.07(11)
584.00(7)	25.7(7)	631.709(19)	7.9(3)
807.46(7)	12.1(5)	730.649(19)	11.6(4)
882.27(6)	18.3(6)	741.372(20)	6.72(24)
888.13(7)	10.4(5)	816.003(23)	42.5(15)
911.99(7)	16.0(7)	821.20(3)	6.2(3)
979.98(9)	8.5(4)	830.01(4)	4.12(19)
994.64(7)	9.2(4)	853.505(20)	7.5(3)
2947.66(19)	10.8(7)	914.952(20)	6.99(24)
3012.35(13)	7.8(5)	1277.57(8)	2.82(16)
3035.56(12)	10.9(6)	Thulium	
3114.14(15)	7.4(6)	66.06(10)	0.51(10)
3443.43(14)	10.6(16)	68.54(6)	1.75(23)
4123.88(15)	13.1(9)	75.23(9)	0.94(8)
5144.00(22)	15.7(10)	87.44(5)	1.29(3)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
105.11(6)	0.780(23)	603.91(5)	1.40(5)
114.50(5)	3.19(6)	611.80(8)	0.83(4)
129.99(5)	0.940(25)	632.37(6)	0.74(3)
144.43(5)	5.96(11)	637.75(4)	1.25(4)
149.66(5)	7.11(12)	640.56(8)	0.70(3)
165.69(5)	3.29(6)	650.21(6)	1.45(5)
180.92(5)	3.85(14)	658.85(5)	1.56(5)
198.46(5)	0.96(3)	703.71(5)	1.32(4)
204.41(5)	8.72(19)	710.70(7)	0.60(3)
219.65(5)	3.64(6)	719.12(8)	1.01(3)
231.71(6)	0.60(3)	720.61(8)	0.57(3)
235.12(5)	1.18(4)	724.48(5)	0.68(3)
237.19(5)	5.52(10)	815.56(5)	0.76(3)
242.58(5)	1.28(4)	854.23(5)	1.41(4)
310.97(5)	2.50(5)	1178.65(9)	0.56(4)
352.91(6)	0.547(23)	4732.63(22)	0.58(5)
384.04(5)	1.95(5)	5158.2(4)	0.47(5)
400.21(5)	0.717(19)	5737.50(20)	1.42(7)
411.46(5)	2.37(5)	5908.3(3)	0.49(4)
424.61(5)	0.556(25)	5943.14(20)	1.51(7)
442.06(8)	0.51(4)	6001.51(22)	0.99(10)
446.31(5)	1.62(4)	6387.49(22)	1.48(7)
455.96(6)	1.16(4)	6442.19(23)	0.47(3)
457.23(11)	0.557(25)	Ytterbium	
468.62(7)	0.45(4)	180.23(5)	0.52(5)
472.94(8)	0.60(5)	363.33(3)	0.89(9)
496.52(5)	0.80(3)	428.28(3)	0.59(6)
499.32(5)	0.88(3)	435.88(3)	0.53(5)
505.00(6)	0.90(3)	477.23(3)	0.71(7)
506.61(6)	0.84(3)	514.87(3)	9.0(9)
510.43(11)	0.61(3)	534.83(3)	0.49(5)
512.01(5)	1.96(5)	639.73(3)	1.45(15)
523.32(7)	0.48(3)	5284.9(5)	1.49(15)
532.39(6)	0.59(3)	Lutetium	
535.78(5)	1.18(4)	71.46(7)	3.96(16)
537.97(6)	1.00(4)	93.97(8)	0.71(4)
562.39(5)	0.85(3)	111.65(7)	1.02(5)
565.22(5)	1.58(4)	112.83(7)	1.16(5)
569.25(5)	1.02(3)	119.70(7)	1.12(5)
585.09(6)	0.60(4)	121.54(7)	5.20(17)
589.13(10)	0.58(10)	138.57(6)	6.76(25)
590.18(7)	1.27(10)	144.65(7)	1.34(8)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
145.84(9)	1.51(9)	1167.02(6)	3.95(10)
147.15(6)	4.96(19)	1174.77(8)	4.8(7)
150.34(6)	13.7(4)	1175.65(11)	2.6(5)
162.44(6)	5.29(17)	1205.93(13)	1.47(23)
168.61(7)	0.95(5)	1207.11(7)	3.9(3)
171.80(6)	1.73(6)	1229.19(6)	4.26(11)
185.49(6)	3.40(12)	1269.27(6)	2.26(7)
188.01(6)	1.40(6)	1329.72(6)	2.09(7)
192.00(6)	2.09(8)	1333.66(6)	1.73(7)
201.58(7)	0.79(6)	1340.41(6)	2.40(8)
207.77(7)	1.02(5)	1420.57(7)	1.83(7)
225.34(6)	1.73(6)	5723.90(15)	2.52(11)
235.83(6)	0.82(4)	Tantalum	
259.35(6)	1.89(8)	97.77(7)	12.6(6)
263.29(9)	0.72(9)	133.89(6)	57(6)
264.28(9)	0.77(9)	146.80(6)	12.7(4)
268.75(5)	3.64(13)	156.12(6)	21.1(5)
284.54(6)	0.75(4)	173.22(6)	109.0(23)
301.10(6)	0.74(4)	190.34(6)	16.5(6)
310.13(5)	1.49(6)	270.48(6)	235(5)
318.98(5)	3.83(13)	297.19(6)	56.4(15)
335.81(5)	1.32(6)	360.60(6)	16.0(6)
347.96(6)	0.85(4)	402.70(5)	106.0(21)
367.38(5)	2.22(8)	511.85(9)	14.9(8)
413.66(5)	0.94(4)	5964.90(14)	12.5(7)
457.94(4)	8.3(3)	Tungsten	
761.64(4)	2.63(10)	111.11(9)	0.162(4)
838.99(7)	0.90(5)	127.46(9)	0.129(5)
1080.25(6)	0.69(4)	145.74(9)	0.970(21)
1088.06(5)	0.84(4)	162.21(9)	0.187(5)
Hafnium		201.42(9)	0.319(8)
63.16(6)	5.26(14)	204.80(9)	0.148(4)
213.43(6)	29.4(6)	226.13(10)	0.113(17)
214.38(7)	20.6(4)	252.93(9)	0.101(3)
215.37(8)	2.82(16)	273.02(9)	0.272(7)
303.98(6)	4.29(9)	289.93(9)	0.0603(22)
325.55(6)	6.89(15)	313.14(9)	0.054(4)
1066.04(6)	1.96(5)	423.92(9)	0.0497(22)
1077.71(6)	2.40(6)	473.85(10)	0.055(5)
1081.35(6)	2.82(7)	499.96(9)	0.0491(23)
1102.72(6)	2.96(8)	531.19(9)	0.052(3)
1143.66(6)	1.84(6)	557.11(9)	0.125(5)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
577.25(8)	0.191(5)	Rhenium	
611.23(9)	0.066(3)	74.76(5)	1.29(8)
616.14(9)	0.059(3)	87.20(4)	0.84(4)
657.42(13)	0.083(5)	92.33(5)	1.14(6)
694.27(9)	0.073(3)	99.36(7)	0.230(24)
745.76(10)	0.053(3)	103.16(5)	0.43(3)
782.13(9)	0.143(6)	105.82(4)	1.91(8)
788.69(11)	0.070(5)	107.40(7)	0.352(25)
791.86(9)	0.113(6)	111.50(4)	1.80(7)
816.24(9)	0.104(4)	114.85(6)	0.43(5)
840.03(8)	0.143(5)	122.53(5)	0.74(4)
866.24(9)	0.068(3)	127.67(7)	0.43(4)
888.17(9)	0.079(4)	130.83(7)	0.43(3)
891.42(9)	0.136(5)	139.32(12)	0.43(5)
894.52(9)	0.078(4)	141.52(5)	1.46(8)
903.16(9)	0.113(4)	144.03(6)	1.85(9)
908.82(9)	0.092(4)	145.45(16)	0.44(5)
979.58(9)	0.104(4)	147.36(11)	0.47(5)
1026.17(8)	0.164(6)	149.28(11)	0.44(5)
1070.98(10)	0.053(3)	151.38(6)	1.15(7)
1082.03(10)	0.061(4)	156.59(10)	0.73(8)
1274.51(9)	0.130(5)	167.30(4)	1.46(6)
3469.42(13)	0.103(6)	174.21(5)	0.382(24)
3492.76(17)	0.051(4)	176.34(8)	0.31(3)
3534.66(16)	0.063(5)	177.70(13)	0.26(3)
3561.02(14)	0.060(4)	181.92(5)	0.388(25)
3739.00(16)	0.069(4)	188.82(5)	1.11(5)
3847.35(17)	0.051(4)	190.05(12)	0.284(24)
4014.64(16)	0.055(4)	193.29(5)	0.43(3)
4118.85(16)	0.059(4)	199.44(5)	0.91(4)
4249.36(18)	0.115(6)	205.18(13)	0.37(8)
4384.34(21)	0.057(5)	207.92(4)	4.44(21)
4574.19(18)	0.104(9)	210.59(7)	1.50(10)
4626.40(15)	0.124(7)	214.62(5)	2.53(14)
4650.6(3)	0.052(5)	216.76(22)	0.30(7)
4684.37(14)	0.150(7)	219.34(8)	0.67(9)
5164.24(14)	0.226(9)	223.09(17)	0.24(6)
6144.21(18)	0.186(12)	227.04(5)	1.78(12)
6190.60(17)	0.513(18)	232.07(13)	0.36(7)
7412.02(24)	0.072(4)	236.59(5)	1.45(10)
		251.45(6)	1.80(23)
		252.12(11)	0.58(16)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
254.94(4)	1.15(5)	407.45(3)	0.134(5)
257.15(6)	1.52(22)	478.11(3)	0.523(14)
261.13(4)	0.67(3)	527.60(3)	0.300(10)
262.71(6)	0.267(17)	537.75(4)	0.121(6)
274.30(8)	0.80(6)	558.02(3)	0.84(3)
275.51(11)	0.51(4)	569.38(3)	0.694(25)
284.88(8)	0.41(4)	605.34(3)	0.113(4)
290.66(6)	3.5(4)	633.12(3)	0.585(16)
300.03(6)	0.70(5)	634.99(4)	0.405(12)
307.60(9)	0.34(3)	829.34(4)	0.167(6)
316.43(4)	2.21(10)	5146.63(14)	0.409(20)
318.82(16)	0.25(3)	5277.11(22)	0.116(15)
358.19(8)	0.236(19)	5683.87(21)	0.167(13)
360.24(5)	0.449(25)	Iridium	
362.82(5)	0.46(3)	58.83(6)	5.3(3)
378.35(4)	0.54(3)	63.19(5)	70(3)
390.80(4)	1.15(5)	64.81(5)	121(4)
518.34(19)	0.24(6)	66.62(9)	3.22(23)
607.24(18)	0.25(3)	71.54(20)	0.6(3)
608.72(17)	0.25(3)	73.35(5)	42.7(15)
680.49(10)	0.34(3)	77.79(5)	4.8(4)
795.02(12)	0.31(3)	84.21(5)	7.7(4)
4663.71(23)	0.24(3)	86.75(7)	0.65(13)
4860.7(3)	0.37(4)	88.64(5)	3.67(24)
5007.0(3)	0.27(4)	90.65(5)	1.25(15)
5027.89(23)	0.29(5)	95.37(6)	0.9(3)
5073.41(24)	0.43(5)	107.94(5)	2.62(12)
5137.4(4)	0.39(4)	110.65(7)	1.18(8)
5871.62(21)	0.299(23)	112.12(6)	1.69(10)
5910.21(21)	0.60(4)	118.38(8)	0.89(13)
Osmium			
73.43(4)	0.174(8)	124.41(8)	1.12(13)
155.18(3)	1.19(3)	126.88(5)	1.86(10)
175.80(4)	0.189(8)	136.20(5)	11.5(4)
186.85(3)	2.08(5)	138.43(10)	1.29(10)
235.24(3)	0.184(6)	140.01(10)	0.95(9)
272.87(3)	0.242(6)	144.79(6)	3.95(19)
275.34(3)	0.173(5)	148.85(6)	2.33(14)
323.02(4)	0.242(9)	151.51(6)	2.89(20)
361.19(3)	0.466(15)	156.38(6)	2.76(12)
371.35(3)	0.574(14)	162.52(13)	0.63(13)
397.50(5)	0.115(5)	165.41(18)	1.7(7)
		169.25(5)	3.05(13)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
177.00(18)	0.6(4)	351.59(5)	10.9(4)
178.91(8)	2.1(5)	365.02(13)	1.15(10)
183.35(14)	1.0(4)	371.34(6)	2.11(12)
184.67(16)	0.92(22)	417.99(5)	3.45(15)
193.59(8)	1.31(24)	432.55(5)	1.85(7)
197.12(21)	0.73(19)	459.46(7)	1.44(9)
199.02(10)	1.07(18)	461.97(10)	0.78(7)
201.48(9)	1.36(17)	486.87(10)	0.93(13)
203.83(8)	1.67(12)	4531.38(22)	0.61(5)
206.19(6)	3.70(18)	4867.01(17)	0.68(4)
208.07(16)	0.70(9)	4980.43(17)	0.82(4)
210.74(10)	2.1(4)	4985.92(18)	0.58(3)
211.49(5)	0.6(3)	5020.66(19)	0.66(6)
215.37(15)	0.74(9)	5028.44(18)	0.67(6)
216.75(5)	5.57(24)	5129.20(16)	0.90(5)
222.36(10)	0.83(16)	5147.51(15)	1.29(6)
226.23(14)	4.0(4)	5166.97(16)	0.96(6)
231.64(8)	0.95(13)	5219.77(21)	0.72(5)
241.70(15)	0.65(13)	5283.60(17)	0.85(6)
245.60(8)	1.05(10)	5304.48(18)	0.73(5)
248.07(18)	0.9(3)	5327.56(21)	0.71(5)
250.63(8)	0.87(10)	5357.49(17)	1.03(6)
254.29(9)	1.08(11)	5431.36(17)	0.78(4)
259.11(8)	1.29(18)	5458.96(22)	0.60(5)
262.01(6)	3.05(18)	5467.0(3)	0.59(7)
263.90(11)	1.39(13)	5487.39(22)	0.58(4)
267.35(9)	0.93(21)	5517.18(19)	0.76(4)
270.79(12)	0.86(20)	5534.73(17)	1.39(6)
273.23(17)	0.72(17)	5564.68(17)	1.71(8)
274.88(16)	0.74(16)	5570.03(22)	0.67(4)
278.33(7)	1.95(16)	5595.77(17)	0.72(4)
284.29(7)	1.95(15)	5612.60(17)	1.06(5)
294.16(13)	1.12(17)	5667.81(16)	2.68(10)
297.51(23)	0.65(17)	5689.23(16)	1.73(7)
300.05(7)	1.07(12)	5728.93(17)	1.15(5)
302.91(7)	1.20(11)	5782.85(18)	1.34(6)
308.23(9)	1.45(11)	5866.76(19)	0.79(5)
310.04(19)	0.61(10)	5954.4(3)	0.74(4)
315.94(9)	2.4(4)	5958.09(23)	1.79(8)
333.79(6)	1.53(10)	5962.25(23)	0.75(4)
337.48(7)	0.96(9)	6082.02(18)	2.62(11)
340.48(12)	0.72(9)		

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Platinum			
326.20(4)	0.511(10)	511.50(8)	1.26(9)
332.84(4)	2.580(25)	515.92(5)	0.57(3)
355.54(4)	6.17(6)	529.30(4)	2.80(17)
521.02(4)	0.336(10)	540.27(4)	0.60(4)
5254.41(19)	0.397(11)	543.97(4)	0.54(3)
Gold			
55.11(3)	2.90(12)	548.91(4)	0.85(5)
74.94(4)	0.390(18)	565.72(6)	0.43(5)
97.24(3)	4.51(6)	571.62(5)	0.61(7)
101.93(3)	0.953(17)	625.35(5)	0.45(3)
146.44(4)	0.43(3)	640.55(5)	0.59(4)
158.44(3)	1.250(14)	672.72(3)	0.635(17)
168.36(3)	3.53(4)	702.22(3)	0.565(7)
170.17(3)	1.510(17)	835.81(5)	0.758(23)
180.83(5)	0.53(4)	4799.83(5)	0.996(23)
188.17(5)	0.51(4)	4852.60(9)	0.406(18)
192.55(4)	4.6(3)	4898.11(9)	0.411(17)
204.15(4)	0.513(8)	4905.79(9)	0.423(17)
215.01(3)	7.77(8)	4957.67(6)	0.95(3)
219.42(5)	0.42(3)	4998.64(8)	0.530(20)
247.63(3)	5.56(6)	5086.25(7)	0.607(16)
261.36(3)	6.3(3)	5102.64(5)	1.110(23)
271.35(9)	0.42(6)	5140.69(8)	0.395(14)
291.77(4)	1.48(3)	5148.64(9)	0.500(15)
307.73(4)	0.607(21)	5226.41(8)	0.450(18)
311.95(4)	0.627(25)	5279.40(7)	0.524(16)
328.49(3)	2.09(4)	5354.86(7)	0.401(13)
343.62(3)	1.080(20)	Mercury	
346.86(5)	0.58(5)	367.96(3)	251(5)
350.79(4)	1.30(7)	661.39(3)	29.5(6)
355.53(4)	0.460(21)	1202.25(7)	15.9(4)
371.05(4)	0.572(18)	1205.67(7)	17.8(6)
381.22(3)	4.22(6)	1225.51(4)	16.3(4)
418.90(3)	1.060(21)	1262.96(4)	28.5(6)
439.77(8)	1.49(23)	1273.52(4)	14.0(4)
440.66(13)	0.69(15)	1407.94(4)	12.6(3)
444.35(4)	0.83(3)	1570.32(4)	39.1(9)
449.54(4)	0.646(24)	1693.31(4)	74.4(21)
456.23(5)	0.57(3)	2002.03(5)	32.2(12)
458.15(5)	0.59(3)	2639.67(5)	15.3(4)
498.53(5)	0.457(25)	3185.77(6)	15.0(5)
		3288.75(6)	17.6(5)
		4675.64(9)	17.2(5)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
4739.44(8)	39.8(10)	1110.37(8)	0.0413(12)
4759.06(9)	16.4(5)	1121.29(7)	0.0600(17)
4842.44(9)	26.5(8)	1155.43(7)	0.0605(17)
5050.06(9)	26.5(8)	1234.69(7)	0.0746(25)
5388.48(10)	23.1(6)	1478.77(8)	0.0544(22)
5658.17(10)	36.4(9)	1741.01(8)	0.0548(25)
5967.00(10)	82.7(20)	1756.27(12)	0.027(3)
6457.78(12)	30.5(10)	4115.08(17)	0.0222(17)
Thallium			
139.94(9)	0.400(7)	4225.47(17)	0.045(3)
154.01(9)	0.0926(17)	4309.00(24)	0.0210(22)
198.33(8)	0.0408(10)	4343.56(12)	0.034(3)
265.86(9)	0.0210(7)	4402.60(15)	0.0208(15)
292.26(8)	0.0983(20)	4495.74(13)	0.043(4)
304.86(9)	0.0225(12)	4540.62(15)	0.0413(25)
310.31(9)	0.0245(12)	4600.95(16)	0.0292(22)
318.88(8)	0.325(6)	4687.58(12)	0.098(4)
325.85(8)	0.0301(10)	4705.83(14)	0.058(3)
330.09(9)	0.0267(10)	4752.24(11)	0.148(5)
330.09(9)	0.0267(10)	4841.40(15)	0.090(4)
331.76(9)	0.0371(10)	4913.57(11)	0.164(5)
347.96(8)	0.361(10)	4980.97(20)	0.036(3)
383.99(8)	0.0341(12)	5014.61(15)	0.058(3)
395.62(8)	0.0862(20)	5130.50(23)	0.058(4)
424.81(8)	0.1200(25)	5180.38(12)	0.141(5)
471.90(8)	0.116(3)	5261.48(13)	0.084(4)
488.11(8)	0.096(4)	5279.86(12)	0.207(6)
563.21(8)	0.0356(15)	5404.41(12)	0.147(5)
591.13(9)	0.0225(10)	5451.07(14)	0.079(3)
624.46(8)	0.0413(10)	5533.35(13)	0.131(5)
626.54(8)	0.0388(10)	5603.28(13)	0.282(10)
629.12(8)	0.0388(10)	5641.57(12)	0.316(7)
678.01(8)	0.0361(15)	5917.48(16)	0.084(4)
732.09(9)	0.064(3)	6025.21(24)	0.0222(25)
737.12(8)	0.118(5)	6118.79(23)	0.0232(20)
764.13(9)	0.0316(12)	6166.61(14)	0.166(6)
818.14(8)	0.0279(10)	6183.05(15)	0.081(4)
873.16(8)	0.168(4)	6222.57(16)	0.065(4)
931.39(8)	0.0257(12)	6336.11(22)	0.0245(22)
949.88(8)	0.0479(15)	6514.57(15)	0.129(5)
1013.27(9)	0.0217(12)		
1093.02(8)	0.0353(12)		

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Lead			
6737.53(14)	0.00691(19)	539.66(10)	0.061(3)
7367.83(12)	0.137(3)	548.23(11)	0.042(10)
Bismuth			
162.34(4)	0.00162(21)	556.93(11)	0.040(10)
319.83(4)	0.0115(14)	561.25(11)	0.033(8)
673.99(4)	0.0026(4)	566.63(10)	0.19(5)
774.95(5)	0.00141(20)	578.02(9)	0.105(5)
808.79(4)	0.00119(16)	583.27(9)	0.279(11)
900.21(6)	0.00102(14)	586.02(10)	0.045(3)
1337.07(5)	0.00156(21)	593.23(10)	0.043(3)
2505.31(8)	0.0021(3)	605.41(10)	0.054(4)
2598.28(9)	0.00166(24)	612.45(9)	0.018(3)
2624.22(8)	0.00154(21)	659.56(16)	0.0173(20)
2828.27(8)	0.00179(24)	665.11(10)	0.084(4)
3080.67(10)	0.00145(20)	681.81(9)	0.079(4)
3356.53(11)	0.00167(24)	705.17(11)	0.050(4)
3396.18(11)	0.00170(24)	714.23(10)	0.052(3)
3632.83(12)	0.00136(20)	752.05(16)	0.0142(19)
4054.32(10)	0.0137(18)	797.79(9)	0.0416(20)
4101.62(11)	0.0089(12)	808.53(11)	0.0212(14)
4165.44(14)	0.00173(24)	814.75(10)	0.0196(13)
4170.96(11)	0.0171(22)	834.83(14)	0.047(5)
4256.42(13)	0.0024(3)	860.61(13)	0.0268(15)
		872.13(11)	0.059(5)
		968.78(9)	0.132(6)
Thorium			
77.09(15)	0.09(3)	1013.84(11)	0.037(3)
211.86(11)	0.0191(17)	1034.27(11)	0.0165(14)
229.08(11)	0.0163(13)	1100.98(11)	0.0211(16)
256.25(11)	0.093(17)	2703.55(24)	0.014(5)
277.48(11)	0.0312(25)	2719.67(18)	0.016(3)
281.40(11)	0.0170(14)	2824.9(3)	0.0144(22)
311.91(10)	0.0187(10)	3148.23(10)	0.0208(14)
316.64(10)	0.0397(18)	3196.66(12)	0.0171(13)
319.08(10)	0.082(3)	3287.94(14)	0.0165(14)
327.80(10)	0.0269(16)	3341.90(13)	0.0168(13)
329.88(11)	0.0221(17)	3398.09(13)	0.0191(14)
331.37(11)	0.0291(19)	3436.17(12)	0.0211(15)
335.92(10)	0.089(4)	3448.42(10)	0.0233(16)
354.27(10)	0.0408(20)	3473.00(8)	0.057(3)
472.30(10)	0.165(8)	3509.43(14)	0.0170(14)
522.73(10)	0.102(5)	3530.96(13)	0.0397(24)
531.58(10)	0.0404(23)	3946.42(10)	0.0268(15)

APPENDIX I. BUDAPEST GAMMA RAY DATA

TABLE I.1. ISOTOPIC GAMMA RAY ENERGY AND THERMAL NEUTRON RADIATIVE CROSS-SECTIONS
MEASURED WITH THE THERMAL NEUTRON BEAM OF THE BUDAPEST REACTOR (cont.)

E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)	E_γ (keV)	$\sigma_\gamma^Z(E_\gamma)$ (b)
Uranium			
521.89(5)	0.072(3)	909.06(6)	0.026(4)
551.808(22)	0.207(5)	943.14(7)	0.082(10)

Appendix II

REFERENCES FROM THE ENSDF THERMAL NEUTRON CAPTURE GAMMA RAY DATABASE

The ENSDF database contains for each thermal neutron capture data set one to three primary references that indicate the main literature sources. Additional references are included in the data set and can be found in the original ENSDF formatted files on the accompanying CD-ROM. Each reference is assigned an eight digit key

number specifying the publication year, first two initials of the first author's last name, and an arbitrary sequence code. Reference key numbers for all of the primary ENSDF references used in this database are summarized in Table II.1. The complete citations for each reference follow Table II.1. The contents of this appendix are unedited.

TABLE II.1. REFERENCE KEY NUMBERS FROM THE PRIMARY ENSDF REFERENCES USED IN THE DATABASE

Isotope	NSR reference key number(s)
¹ H	1994Ki27,1982Va13,1980Is02
² H	1982Ju01,1980Al31
⁶ Li	1985Ko47
⁷ Li	1991Ly01
⁹ Be	1983Ke11,1974JuZW
¹⁰ B	1986Ko19
¹² C	1982Mu14
¹³ C	1982Mu14
¹⁴ N	1997Ju02,1994Ra17,1990Is05
¹⁶ O	1977Mc05
¹⁷ O	1978LoZW,1978LoZT
¹⁹ F	1996Ra04
²⁰ Ne	1986Pr05
²¹ Ne	1986Pr05
²² Ne	1986Pr05
²³ Na	1983Hu11,1983Ti02
²⁴ Mg	1992Wa06,1991MiZQ
²⁵ Mg	1992Wa06,1991Ki04
²⁶ Mg	1992Wa06
²⁷ Al	1982Sc14
²⁸ Si	1992Ra19,1990Is02
²⁹ Si	1992Ra19,1990Is02
³⁰ Si	1992Ra19,1990Is02
³¹ P	1989Mi16,1985Ke11
³² S	1985Ra15
³³ S	1985Ra15
³⁴ S	1985Ra15
³⁶ S	1984Ra09,1997Be42
³⁵ Cl	1982Kr12,1985Ke04,1996Co16

Isotope	NSR reference key number(s)
³⁷ Cl	1973Sp06
³⁶ Ar	1970Ha56
⁴⁰ Ar	1970Ha56
³⁹ K	1984Vo01
⁴⁰ K	1984Kr05
⁴¹ K	1985Kr06
⁴⁰ Ca	1967Gr16,1970Cr04
⁴² Ca	1969Gr08
⁴³ Ca	1972Wh02
⁴⁴ Ca	1968Gr11
⁴⁶ Ca	1970Cr04
⁴⁸ Ca	1970Cr04,1969ArZT
⁴⁵ Sc	1982Ti02
⁴⁶ Ti	1972Kn07
⁴⁷ Ti	1989Co01,1984Ru06
⁴⁸ Ti	1992Ku17,1983Ru08
⁴⁹ Ti	1984Ru06,1971Te01
⁵⁰ Ti	1971Ar39
⁵⁰ V	1991Mi08,1978Ro03,1973HaWJ
⁵¹ V	1991Mi08
⁵⁰ Cr	1974KoYY,1972Ko15,1972Lo26
⁵² Cr	1980Ko01,1972Ko15
⁵³ Cr	1989Ho15,1988Li30,1994Co09
⁵⁴ Cr	1972Wh05
⁵⁵ Mn	1980De20,1975Co05,1974Bo19
⁵⁴ Fe	1972Ko15,1967Ar14,1990Ku26
⁵⁶ Fe	1980Ve05,1978Ve06,1969Ko05
⁵⁷ Fe	1969Fa05,1973Ko27
⁵⁸ Fe	1983VeZZ,1980Ve05,1978Ve06

APPENDIX II. REFERENCES FROM ENSDF DATABASE

TABLE II.1. REFERENCE KEY NUMBERS FROM THE PRIMARY ENSDF REFERENCES USED IN THE DATABASE (cont.)

Isotope	NSR reference key number(s)	Isotope	NSR reference key number(s)
⁵⁹Co	1984Ko29	⁹⁷Mo	1971He10
⁵⁸Ni	1993Ha05,1977Is01,1972St06	⁹⁸Mo	1973De39
⁶⁰Ni	1993Ha05	¹⁰⁰Mo	1990Se17
⁶¹Ni	1970Fa06,1975Wi06	⁹⁹Tc	1979Pi08
⁶²Ni	1977Is01,1970GaZQ,1972Ko15	⁹⁹Ru	1988Co18,1988CoZU,1991Is05
⁶⁴Ni	1977Is01	¹⁰⁰Ru	1982Ba69
⁶³Cu	1983De28	¹⁰¹Ru	1991Is05
⁶⁵Cu	1983De29	¹⁰²Ru	1979SeZT
⁶⁴Zn	1972Bo75	¹⁰⁴Ru	1978Gu14,1974Hr01
⁶⁶Zn	1971Kn06,1975DeYM,1970Ba21	¹⁰³Rh	1981Ke03
⁶⁷Zn	1971Ot01	¹⁰²Pd	1970Bo29
⁶⁸Zn	1972Bo75	¹⁰⁴Pd	1970Bo29
⁶⁹Ga	1967Ba79,1970Li04,1971Ve03	¹⁰⁵Pd	1987Co03,1970Or05
⁷¹Ga	1970Li04,1971Ve03	¹⁰⁸Pd	1980Ca02
⁷⁰Ge	1991Is01,1972Gr34,1972We10	¹⁰⁷Ag	1985Ma54
⁷²Ge	1972Gr34,1972Ha74,1972We10	¹⁰⁹Ag	1979Bo41
⁷³Ge	1985HoZQ,1991Is01	¹¹⁰Cd	1987BaYW,1991NeZX
⁷⁴Ge	1972Gr34,1972Ha74,1991Is01	¹¹¹Cd	1993De01
⁷⁶Ge	1972Gr34,1972Ha74	¹¹³Cd	1984Mh01,1979Br25,1968Gr32
⁷⁵As	1990Ho10	¹¹³In	1975Ra07
⁷⁴Se	1984To11,1982ToZS,1981En07	¹¹⁵In	1976Al06,1972Ra39,1973Sc23
⁷⁶Se	1982ToZS,1985To10	¹¹⁵Sn	1991Ra01
⁷⁷Se	1987Su05,1981En07,1979BrZE	¹²¹Sb	1972Sh02,1978Al09,1977Va11
⁷⁸Se	1979BrZE,1970Ba54,1981En07	¹²³Sb	1973ShZZ,1980Al22
⁸⁰Se	1971Ra07	¹²²Te	192000Bo24
⁷⁹Br	1978Do06,1977DoZP	¹²³Te	1995Ge06,1983Ro13,1969Bu05
⁸¹Br	1978Do06	¹²⁴Te	1999Ho01,1998Ho16,1997BoZW
⁸³Kr	1987Ha21,1972Ma42	¹²⁸Te	1981Ho12,1999Bo31
⁸⁶Kr	1977Je03	¹³⁰Te	1980Ho29,1977RuZR
⁸⁵Rb	1969Da15,1969Ra10,1968Ir02	¹²⁷I	1991Sa07
⁸⁶Sr	1986Wi16	¹²⁹Xe	1988Ha28,1971Gr28
⁸⁷Sr	1987Wi15	¹³¹Xe	1988Ha28,1971Gr28
⁸⁸Sr	1989Wi05	¹³⁶Xe	1977Pr07
⁸⁹Y	1993Mi04	¹³³Cs	1987Bo24
⁹⁰Zr	1978LoZX	¹³⁴Ba	1993Bo01
⁹¹Zr	1979HeZT,1972FaZW	¹³⁵Ba	1990Is07,1983BrZK,1969Ge07
⁹²Zr	1977Ba33	¹³⁶Ba	1995Bo03
⁹⁴Zr	1977Ba33,1976BaYM	¹³⁷Ba	1995Bo05
⁹³Nb	1985Bo48,1968Ju01	¹³⁸Ba	1969Mo13
⁹²Mo	1991Is05	¹³⁹La	1970Ju04,1988BoZH,1990Is09
⁹⁴Mo	1973Ba57	¹³⁶Ce	1981KoZW
⁹⁵Mo	1970He27	¹³⁸Ce	1969Gr31
⁹⁶Mo	1973De39	¹⁴⁰Ce	1970Ge03

APPENDIX II. REFERENCES FROM ENSDF DATABASE

TABLE II.1. REFERENCE KEY NUMBERS FROM THE PRIMARY ENSDF REFERENCES USED IN THE DATABASE (cont.)

Isotope	NSR reference key number(s)	Isotope	NSR reference key number(s)
¹⁴² Ce	1976Ge02	¹⁷⁴ Yb	1971Al27,1971Br17
¹⁴¹ Pr	1985AlZN,1981Ke11,1968Ke08	¹⁷⁶ Yb	1972Al19,1973PrZI,1990Bo49
¹⁴² Nd	1976Mi19,1993Bo29	¹⁷⁵ Lu	1991Kl02
¹⁴³ Nd	1983Sn04	¹⁷⁶ Lu	1965Ma18,1975Ge11,1971Ma45
¹⁴⁴ Nd	1975Hi03	¹⁷⁴ Hf	1971Al01
¹⁴⁵ Nd	1983Sn01,1976Bu14	¹⁷⁶ Hf	1967Pr08,1967Na07
¹⁴⁶ Nd	1975Ro16,1976Ro03	¹⁷⁷ Hf	1986Ha22,1987Bo52
¹⁴⁸ Nd	1976Pi04	¹⁷⁸ Hf	1989Ri03,1976Be23
¹⁵⁰ Nd	1975SmZT,1976Pi13,1985BuZU	¹⁷⁹ Hf	1974Bu22,1990Bo52,1986RoZM
¹⁴⁴ Sm	1978WaZM	¹⁸⁰ Hf	1971Al22,1967Pr08
¹⁴⁷ Sm	1971Gr37,1993Ju01	¹⁸⁰ Ta	1973LaZY
¹⁴⁸ Sm	1982Ba15	¹⁸¹ Ta	1979Va10,1971He13,1974An12
¹⁴⁹ Sm	1966Sm03,1963Gr18,1969Re11	¹⁸² W	1997Pr02
¹⁵⁰ Sm	1986Va08	¹⁸³ W	1974Gr11,1975Bu01
¹⁵² Sm	1963Gr18,1969Sm04,1971Be41	¹⁸⁴ W	1973PrYV
¹⁵⁴ Sm	1982Sc03	¹⁸⁶ W	1973PrZI,1969BoZN,1989BoYT
¹⁵¹ Eu	1978Vo05	¹⁸⁵ Re	1969La11,1973Gl06
¹⁵³ Eu	1987Ba52,1978PrZY,1984Ro06	¹⁸⁷ Re	1972Sh13,1968Su01,1978Sc10
¹⁵² Gd	1996SpZZ	¹⁸⁴ Os	1974PrZY,1974Pr15
¹⁵⁴ Gd	1986Sc25	¹⁸⁶ Os	1974Pr15,1974NeZY
¹⁵⁵ Gd	1982Ba28	¹⁸⁷ Os	1983Fe06
¹⁵⁶ Gd	1993Ko01,1986GrZR,1971Gr42	¹⁸⁸ Os	1992Br17,1976Be50
¹⁵⁷ Gd	1978Gr14,1970Bo29,1994GrZZ	¹⁸⁹ Os	1979Ca02
¹⁵⁸ Gd	1971Gr42	¹⁹⁰ Os	1991Bo35
¹⁶⁰ Gd	1971Gr42	¹⁹² Os	1978Be22,1979Wa04
¹⁵⁹ Tb	1974Ke01,1989Du03	¹⁹¹ Ir	1991Ke10
¹⁶⁰ Dy	1977Be03	¹⁹³ Ir	1998Ba85,1998Ba42,1987CoZW
¹⁶¹ Dy	1995Be02,1967Ba34	¹⁹⁴ Pt	1987Ca03,1982Wa20
¹⁶² Dy	1989Sc31,1967Sc05,1986Bo43	¹⁹⁵ Pt	1979Ci04
¹⁶³ Dy	1964Sc25	¹⁹⁶ Pt	1978Ya07
¹⁶⁴ Dy	1965Sc09,1983Is04	¹⁹⁷ Au	1996Ma70,1996Ma75,1993Pe04
¹⁶⁵ Ho	1967Mo05,1984Ke15	¹⁹⁹ Hg	1970Or05,1971Ma10,1974Br02
¹⁶⁶ Er	1965Ko13,1970Mi01	²⁰¹ Hg	1975Br02
¹⁶⁷ Er	1991Da12,1991DaZT,1996Gi09	²⁰³ Tl	1974Co21,1975RaYX
¹⁶⁸ Er	1970Mu15	²⁰⁴ Pb	1967Ju02,1983Hu13
¹⁷⁰ Er	1971Al01,1984MuZY	²⁰⁶ Pb	1983Hu13
¹⁶⁹ Tm	1994HoZZ,1989Du03,1968Lo09	²⁰⁷ Pb	1998Be19,1983Ma55
¹⁶⁸ Yb	1969Bo16,1972Wi12,1973GrZV	²⁰⁹ Bi	1989Sh20,1983Ts01
¹⁷⁰ Yb	1972Wa10	²³² Th	1974Ke13,1979Je01
¹⁷¹ Yb	1985Ge02,1975Gr32,1988Su01	²³⁴ U	1972Ri08,1979Al03
¹⁷² Yb	1971Al01	²³⁵ U	1975OtZX,1973Gr20,1970Ka22
¹⁷³ Yb	1987Ge01,1981Gr01	²³⁸ U	1978Bo12,1972Bo46,1984Ch05

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DEFINITIONS

The definitions given below may not necessarily conform to definitions adopted elsewhere for international use.

at. wt. Atomic weight.

E_γ . The energy of a gamma ray emitted in the decay process from neutron capture.

g_w . The Westcott g factor; defined by Eq. (2.12).

\hat{g} . The effective g factor; defined by Eq. (2.20).

k_0 . The prompt k_0 factor; defined by Eq. (2.1).

$k_0(x)$ or $k_0(E_\gamma)$. The prompt k_0 factor of the specific gamma ray (of energy E_γ) from element x relative to the hydrogen 2223 keV gamma ray.

N_γ . Number of gamma rays.

$P(E_\gamma)$. The absolute emission probability of a gamma ray of energy E_γ (gammas per capture).

v . Neutron speed.

v_0 . A neutron speed of 2200 m/s.

θ . The natural abundance of the capturing isotope involved in the subsequent emission of the prompt gamma ray of interest.

$\sigma_\gamma(v)$. The nuclear capture cross-section for a neutron of speed v .

σ_0 or $\sigma_\gamma \equiv \sigma_\gamma(v_0)$. The thermal neutron capture cross-section or the nuclear capture cross-section for a neutron of speed v_0 .

σ_γ^Z or σ_0^Z . The thermal neutron capture cross-section for the element $Z = \sum_i^{\text{all isotopes}} (\theta\sigma_\gamma)_i$

$\sigma_\gamma(E_\gamma)$. The nuclear partial capture cross-section, $\sigma_\gamma(E_\gamma) = P(E_\gamma)\sigma_0$.

$\sigma_\gamma^Z(E_\gamma)$. The elemental partial capture cross-section, $\sigma_\gamma^Z(E_\gamma) = \theta P(E_\gamma)\sigma_0 = \theta\sigma_\gamma(E_\gamma)$; Eq. (2.2).

$\hat{\sigma}$. The effective capture cross-section; defined by Eq. (2.3).

$\langle\sigma\rangle$. The effective capture cross-section; defined by Eq. (2.5).

ABBREVIATIONS FOR PROMPT GAMMA ACTIVATION ANALYSIS

No single abbreviation has been universally agreed in the analytical use of gamma rays from the capture of slow neutrons. The technique has most often been called PGAA or PGNAA during the course of this CRP. The following list has been collected from the literature:

CGA:	Capture gamma ray analysis
NCGA:	Neutron capture gamma ray analysis
PCGRA:	Prompt capture gamma ray analysis
PGA:	Prompt gamma analysis
PGAA:	Prompt gamma activation analysis
PGNA:	Prompt gamma neutron analysis
PGNAA:	Prompt gamma ray neutron activation analysis
PNAA:	Prompt neutron activation analysis
PNCAA:	Prompt neutron capture activation analysis
RNC:	Radiative neutron capture
TCGS:	Thermal neutron capture gamma ray spectroscopy

Additional abbreviations have been used when cold neutrons are employed:

CNPGAA:	Cold neutron prompt gamma activation analysis
CPGAA:	Cold prompt gamma activation analysis
PGCNA:	Prompt gamma cold neutron activation analysis
TNPGAA:	Thermal neutron prompt gamma activation analysis

Another abbreviation of note is:

INAA:	Delayed instrumental neutron activation analysis
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LIST OF PARTICIPANTS

Choi, H.D.	Department of Nuclear Engineering, Seoul National University, Shinrim-dong, Gwanak-ku, Seoul 151-742, Republic of Korea
Firestone, R.B.	Isotopes Project, MS 88R0192, Lawrence Berkeley National Laboratory, University of California, 1 Cyclotron Road, Berkeley, CA 94720, United States of America
Frankle, S.C.	MS F663, P.O. Box 1663, Los Alamos National Laboratory, Los Alamos, NM 87545, United States of America
Goswami, A.	Nuclear Chemistry Section, Radiochemistry Division, Bhabha Atomic Research Centre, Trombay, Mumbai 400 085, India
Lindstrom, R.M.	Analytical Chemistry Division, Stop 8395, National Institute for Standards and Technology, 100 Bureau Drive, Gaithersburg, MD 20899-8395, United States of America
Lone, M.A.	Office of the Chief Engineer, Station E4, Atomic Energy of Canada Ltd, Chalk River Laboratories, Ontario K0J 1J0, Canada
Molnár, G.L.	Nuclear Research Department, Chemical Research Centre, Institute of Isotope and Surface Chemistry, Hungarian Academy of Sciences, P.O. Box 77, 1525 Budapest, Hungary
Mughabghab, S.F.	Building 197D, Energy Technology Division, Brookhaven National Laboratory, P.O. Box 5000, Upton, NY 11973-5000, United States of America
Nguyen Canh Hai	Department of Nuclear Physics and Techniques, Nuclear Research Institute, 1 Nguyen Tu Luc, Dalat, Vietnam
Reddy, A.V.R.	Nuclear Chemistry Section, Radiochemistry Division, Bhabha Atomic Research Centre, Trombay, Mumbai 400 085, India
Révay, Z.	Nuclear Research Department, Chemical Research Centre, Institute of Isotope and Surface Chemistry, Hungarian Academy of Sciences, P.O. Box 77, 1525 Budapest, Hungary
Zhou, Chunmei	China Nuclear Data Centre, China Institute of Atomic Energy, P.O. Box 275 (41), 102413-Beijing, China

Prompt gamma activation analysis (PGAA) from neutron capture is particularly valuable as a non-destructive nuclear method. This database improves the quality and quantity of the data for reliable applications of PGAA in fields such as materials science, geology, mining, archaeology, environmental science, food analysis and medicine. The database provides, for all the natural elements, tables that include the following data: isotopic composition, thermal radiative cross-sections (total and partial), Westcott g factors, energies of gamma rays (prompt and delayed), decay modes, half-lives and branching ratios. The CD-ROM included with this report contains the complete database, the retrieval system and important electronic documents related to the database.