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Thorium Fuel Cycle in Nuclear Reactors

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Abstract

This project explores the potential of thorium-based nuclear fuels in Pressurized Water Reactors (PWRs), focusing on the thorium fuel cycle and its various implementations. The research investigates the benefits and challenges associated with thorium fuel, including its higher conversion ratios, improved thermal properties, and intrinsic proliferation resistance. Detailed simulations using OpenMC analyze the behavior of thorium oxide (ThOX) with ^{233}U at different concentrations. The results demonstrate that while thorium-based fuels can achieve breeding, maintaining criticality presents challenges. The project also examines the impact of fuel composition and concentration on reactor performance and safety. The findings suggest that optimizing these parameters is crucial for enhancing the performance and safety of thorium-based fuels. The research concludes with a discussion on the future prospects of thorium in the nuclear industry and the need for further research and development to fully realize its potential as a sustainable nuclear fuel.

1. Introduction

At the time, there were concerns that the global supply of ^{235}U would be insufficient to meet future energy demands. Weinberg and his colleagues recognized that molten salt reactors (MSRs) could be used to breed ^{233}U from thorium [1]. Between 1965 and 1969, ORNL successfully operated an MSR experiment for 15 months, achieving an impressive 87% operational uptime, even utilizing ^{233}U as fuel [1]. Despite this success, the MSR program was eventually discontinued as other nuclear concepts received greater political support [2]. Thorium is not a fissile material but rather a fertile material, so the idea of the thorium fuel cycle is to breed ^{233}U .

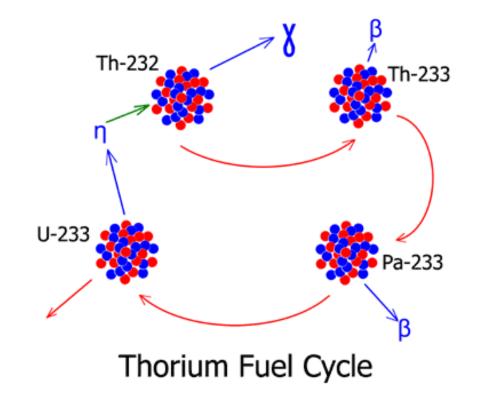


Fig. 1: ^{233}U breeding from ^{232}Th . Image from [3]

2. Simulations

The simulation uses OpenMC, a Monte Carlo neutron transport simulation library developed by the Computational Reactor Physics Group (CRPG) at MIT. The objective was to analyze the behavior of thorium-based nuclear fuels in a PWR as a reference for other reactor types. In this project, a Pressurized Water Reactor (PWR) simulation was conducted to evaluate different thorium fuel cycles.



Fig. 2: OpenMC Logo [4].

References

- [1] Dolan, T. J. Molten Salt Reactors and Thorium Energy (Woodhead Publishing, 2017), 1 edn.
- [2] International Atomic Energy Agency. Thorium fuel cycle potential benefits and challenges. Tech. Rep. IAEA-TECDOC-1450, Vienna, Austria (2005).
- [3] Donev, J. et al. Energy education (2024). URL https://energyeducation.ca. Accessed: 2024.
- [4] OpenMC Documentation. Openmc (2023). Accessed: 2023-10-25.
- [5] Lau, C. W., Demazière, C., Nylén, H. & Sandberg, U. Improvement of lwr thermal margins by introducing thorium. *Progress in Nuclear Energy* **61**, 48–56 (2012).

3. Results

Simulations were conducted using uranium oxide fuel with thorium concentrations of 10% and 50%. The results indicated that increasing the thorium concentration improved the conversion ratio, consistent with findings in [5]. However, higher thorium concentrations also presented challenges in maintaining criticality. The results suggest that while higher thorium concentrations can enhance fuel utilization, careful optimization of fuel composition and reactor parameters is essential to ensure stable and efficient reactor operation. **Fig.** 3 shows the isotopes concentration in the fuel as a function of time for a 10% thorium concentration. Additional, **Fig.** 4 shows the reactivity as a function of time for the same thorium concentration.

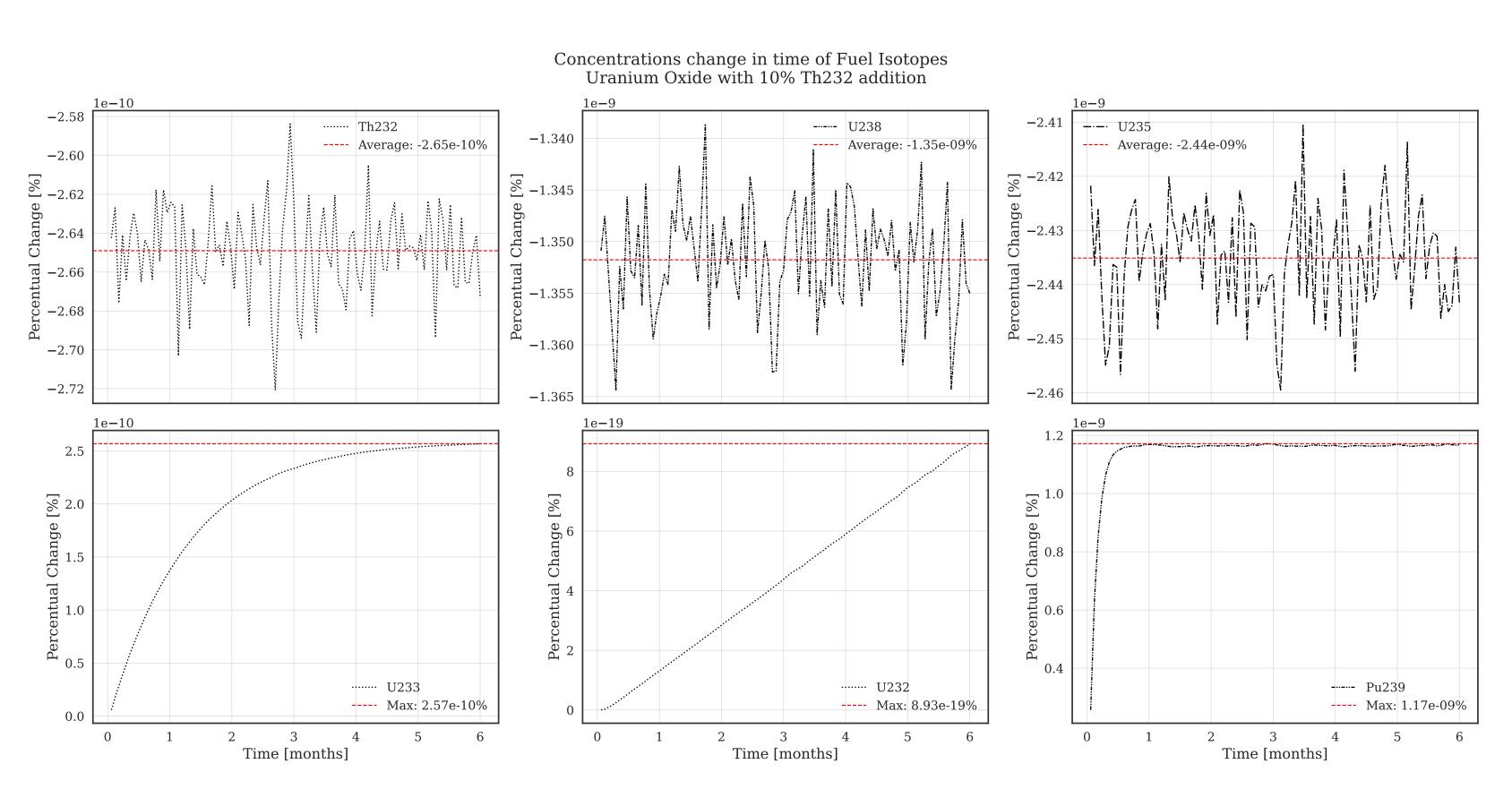


Fig. 3: Isotopes concentration in the fuel as function time for a 10% thorium concentration.

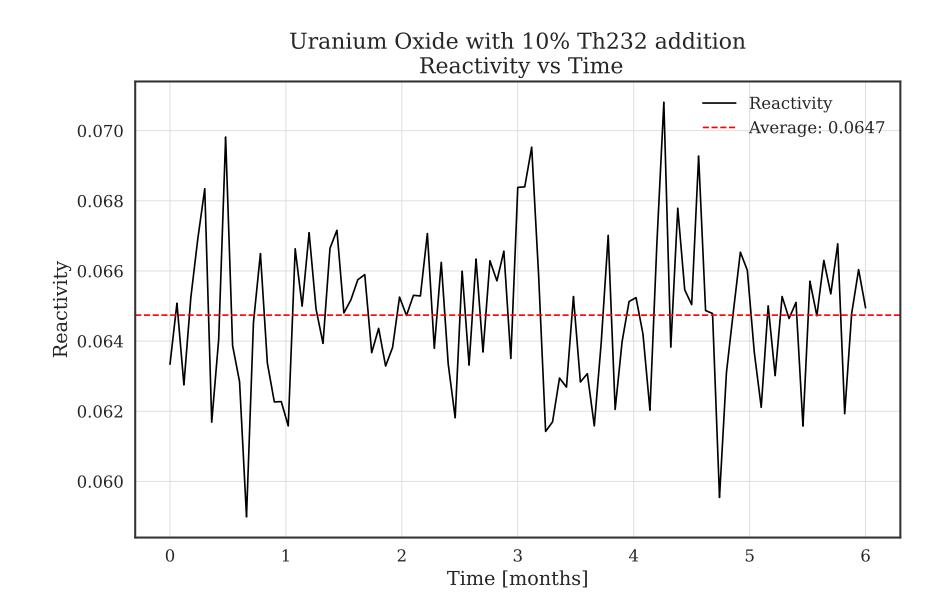


Fig. 4: Reactivity as function of time.

Additional simulations explored thorium-based fuel with thorium oxide and 10% plutonium as the seed isotope. Results demonstrate that the fuel assembly achieves a critical state and attains a breeding ratio for the production of ^{233}U , showcasing the potential for sustainable fuel generation.

4. Conclusions

The study demonstrates that thorium-based fuels, particularly ThOX with plutonium, can achieve breeding but exhibit higher reactivity, necessitating optimized fuel composition and core configurations. Thorium, as a fertile material, cannot sustain a chain reaction independently and requires external support for activation. Despite its potential benefits, the implementation of thorium fuel cycles faces significant challenges, including historical political inaction and current economic hurdles in reprocessing irradiated thorium fuel. These factors have hindered its widespread adoption in modern reactors. Further research and development are essential to overcome these challenges and fully realize the potential of thorium as a sustainable nuclear fuel.