



UT-BATTELLE

Analytical and Numerical Modeling of the Detection Statistics from a Fissile Sample

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23 May, 2007

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 - Multiplicities and coincidences can give isotopic composition and mass of the sample.

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- Safeguards, nuclear materials management.
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 - Spontaneous fission.
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 - Change of multiplicities.
 - Multiplicities and coincidences can give isotopic composition and mass of the sample.
 - A full number distribution offers deeper insight than a few factorial moments.

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- Neutrons in a fissile sample can undergo different processes.

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- Neutrons in a fissile sample can undergo different processes.
 - Induce fission.
 - Capture.
 - (n, xn) -reactions.
 - Escape the sample.

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- These events will lead to a certain number distribution for a sample that will vary with mass and composition.

Probability Generating Functions

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Probability Generating Functions (PGFs)

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Probability Generating Functions

To find the probability distributions we used the mathematical tool Probability Generating Functions (PGFs)

$$h(z) = \sum_n p_1(n) z^n \quad \text{and} \quad H(z) = \sum_n P(n) z^n. \quad (1)$$

Starting with an initial neutron or a source event (spontaneous fission)

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with PGFs

$$q_s(z) = \sum_n p_s(n) z^n, \quad q_f(z) = \sum_n p_f(n) z^n. \quad (4)$$

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Factorial moments and probability distributions

The multiplicities (factorial moments) of a function f are obtained as derivatives of its PGF $g(z)$ evaluated at $z = 1$:

$$\langle n \rangle = \left. \frac{\partial g(z)}{\partial z} \right|_{z=1} = \sum_n n f(n) \quad (5)$$

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- Longer expressions, which on the other hand can be expressed recursively

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To find the terms expressed in a recursive manner we need to calculate initial terms such as $p_1(0)$

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$$p_1(0) = (1 - p)z + pq_f[h(z)] \Big|_{z=0} = p \sum_{n=0}^N p_f(n)[p_1(0)]^n. \quad (7)$$

8-th degree polynomial to be solved for $p_1(0)$, note the p -dependence for $p_1(0)$.

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Including absorption

The event of absorption can be included into the fission distribution:

$$\tilde{p}_f(n) = \frac{p' - p}{p'} \delta_{n,0} + \frac{p}{p'} p_f(n). \quad (8)$$

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The first master equation will then read as:

$$h(z) = (1 - p')z + p' \tilde{q}_f[h(z)]. \quad (9)$$

where $\tilde{q}_f(z)$ is the generating function of the $\tilde{p}_f(n)$ of Eq. (8).

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Including detection

- The process of detection can be added by considering the neutrons that have been emitted by the sample.

$$\varepsilon(z) = \epsilon z + (1 - \epsilon). \quad (10)$$

Here, ϵ is the detector efficiency for neutrons.

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$$h_d(z) = h[\varepsilon(z)] \quad , \quad H_d(z) = H[\varepsilon(z)]. \quad (11)$$

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- The derivatives needed for finding factorial moments as well as the statistics change in a simple way:

$$\frac{d^n h_d(z)}{dz^n} = \frac{d^n h(z)}{dz^n} \cdot (\epsilon)^n \quad , \quad \frac{d^n H_d(z)}{dz^n} = \frac{d^n H(z)}{dz^n} \cdot (\epsilon)^n \quad (12)$$

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- For the factorial moments the full change is

$$\tilde{\nu}_{d,n} = (\epsilon)^n \cdot \tilde{\nu}_n. \quad (13)$$

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The creation of photons is a more complicated process connected to neutrons. No self-multiplication.

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Higher multiplicities can be favourable from a detection view-point.

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Start with master equations describing the system

$$g(z) = (1 - p) + pr_f(z)q_f[g(z)] \quad (14)$$

and

$$G(z) = r_s(z)q_s[g(z)]. \quad (15)$$

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With PGFs for gammas produced in spontaneous and induced fission

$$g(z) = \sum_n f_1(n)z^n, \quad G(z) = \sum_n F(n)z^n. \quad (16)$$

Absorption and detection

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Photon absorption will be accounted for by the probability l_γ that describes the leakage probability for one single photon:

$$l(z) = l_\gamma z + (1 - l_\gamma). \quad (17)$$

Note that for photons the absorption of neutrons also play a fundamental role, while the opposite is not true.

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The next step in the simulation of the statistics obtained from measurements is to incorporate the process of detection:

$$\varepsilon_\gamma(z) = \epsilon_\gamma z + (1 - \epsilon_\gamma). \quad (18)$$

Using this equation we can obtain the detection statistics as:

$$g_d(z) = g[l\{\varepsilon(z)\}] \quad , \quad G_d(z) = G[l\{\varepsilon(z)\}]. \quad (19)$$

Photon-neutron correlation

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- Initial source photons are uncorrelated to source neutrons.

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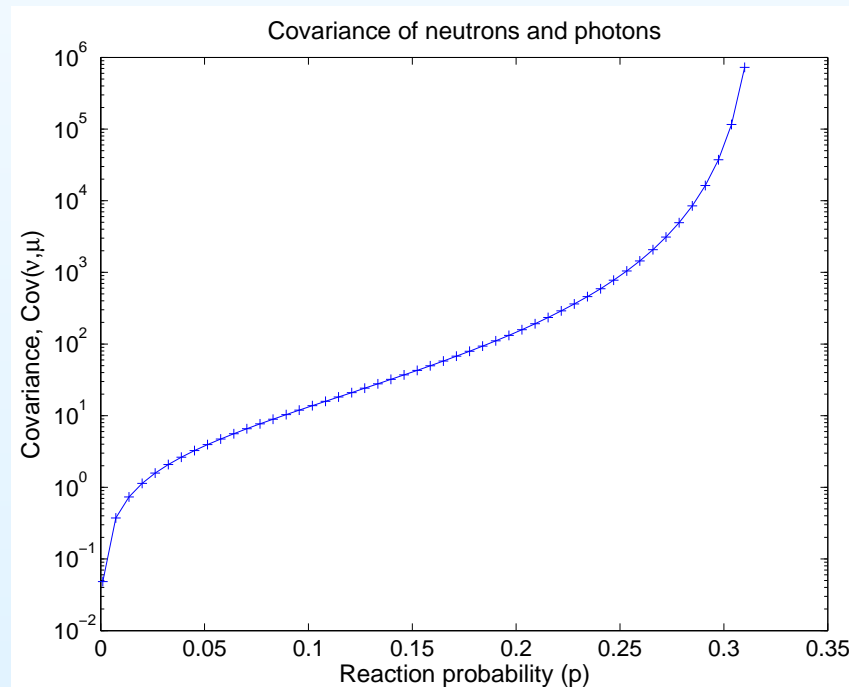
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- Initial source photons are uncorrelated to source neutrons.
- Multiplication of neutrons on the other hand is the reason for additional photon multiplication as well:

$$\begin{aligned} \text{Cov} \{ \tilde{\nu}, \tilde{\mu} \} = & \frac{p}{1-p} \left(\nu_{s,1} \nu_{r,1} + \nu_{s,2} - \nu_{s,1}^2 \right) \mu_{r,1} \mathbf{M}^2 \\ & + \left(\frac{p}{1-p} \right)^2 \left(\nu_{s,1} + \mu_{r,1} \nu_{r,2} \right) \mathbf{M}^3. \end{aligned} \quad (20)$$



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The symbolic code Mathematica have been used to do the derivations and find formulae for higher order terms which grow rapidly in size.

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$$\left\{ \begin{aligned} h_d'(z) &\rightarrow \frac{p'-1}{p'\nu_{d,s}(1)-1}, h_d''(z) \rightarrow -\frac{p'\nu_{d,s}(2)h_d'(z)^2}{p'\nu_{d,s}(1)-1}, \\ h_d^{(3)}(z) &\rightarrow \frac{-p'\nu_{d,s}(3)h_d'(z)^3 - 3p'\nu_{d,s}(2)h_d''(z)h_d'(z)}{p'\nu_{d,s}(1)-1}, \dots \end{aligned} \right\}$$

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As a final step the parameters are replaced with values to get numerical expressions.

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- Saves time and makes multiple evaluations easy.

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As a final step the parameters are replaced with values to get numerical expressions.

- Saves time and makes multiple evaluations easy.
- Comparisons with Monte Carlo simulations done with the MCNP-PoliMi code, from which we have taken the numerical value of p and the leakage and detection probabilities for neutrons and photons.

Effect of absorption

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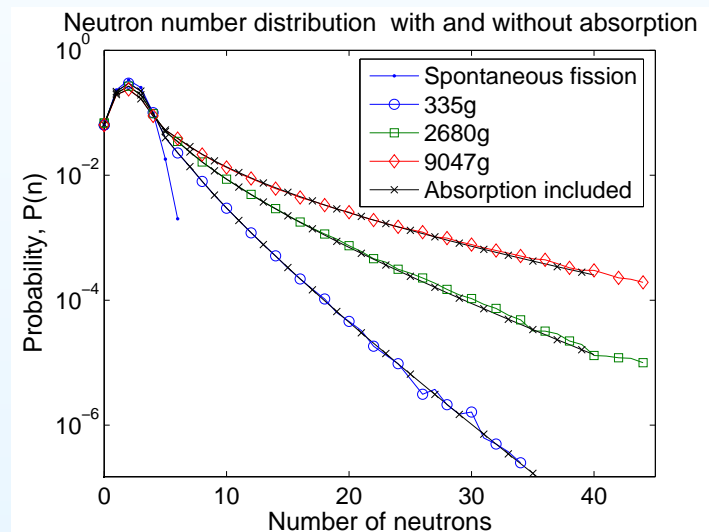
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- Dependence on mass shown in the parameter p , and also an increased probability of absorption. 20 wt% ^{240}Pu and 80 wt% ^{239}Pu .



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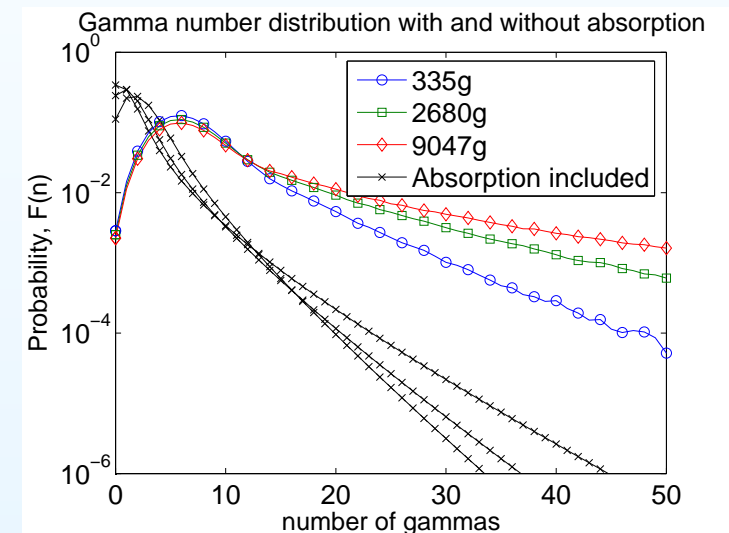
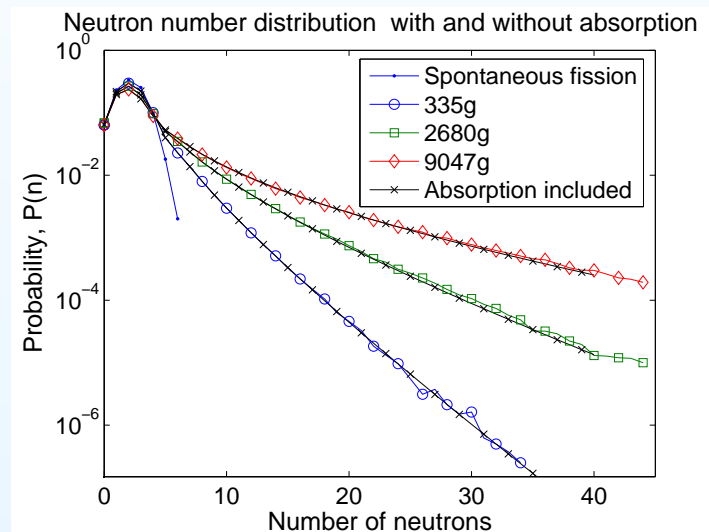
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- Change compared to non-multiplying case.
- Small effect of absorption on neutrons for such a heavy element.
- Large self-shielding for photons, which are still generated with high multiplicity, but few leak out.

Comparison with Monte Carlo

Number Distributions

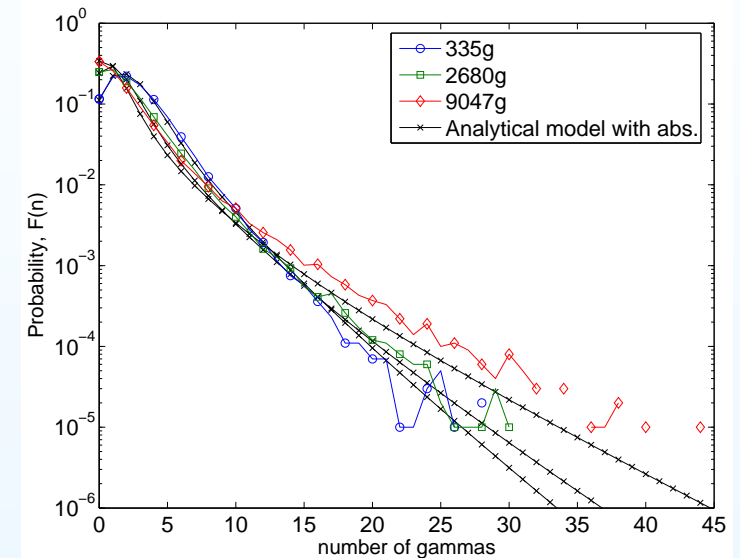
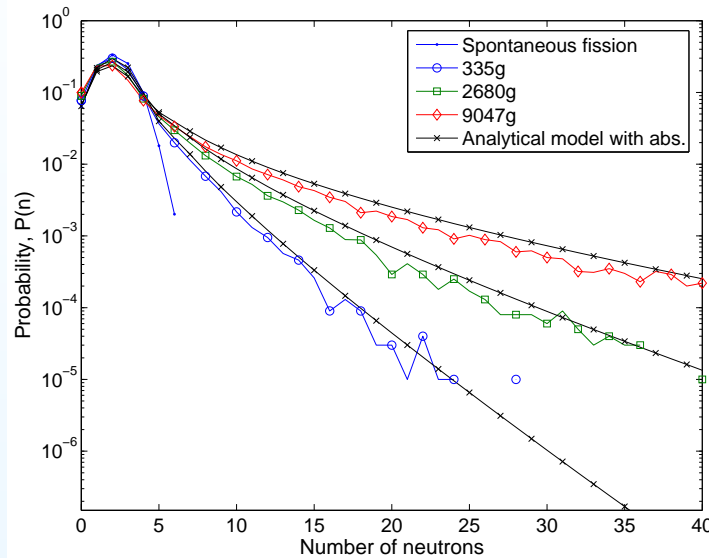
Neutron Distribution

Gamma Distribution

Results

- Tools
- Effect of absorption
- Comparison with Monte Carlo
- Detection
- Simulated scintillation detector results

Conclusions



- Good agreement with MCNP-PoliMi.
- For photons one have higher multiplicities per source event for *smaller* samples.

Detection

Number Distributions

Neutron Distribution

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Including detection into the model with a detection allows to simulate actual detection statistics:

Detection

[Number Distributions](#)

[Neutron Distribution](#)

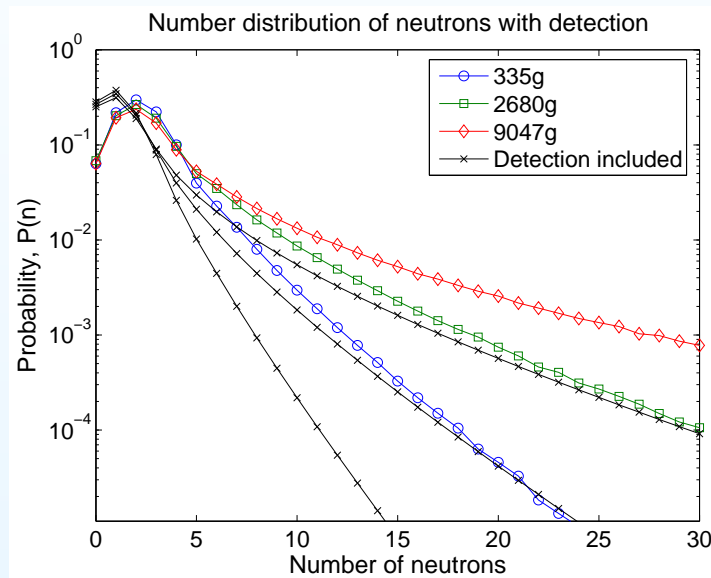
[Gamma Distribution](#)

[Results](#)

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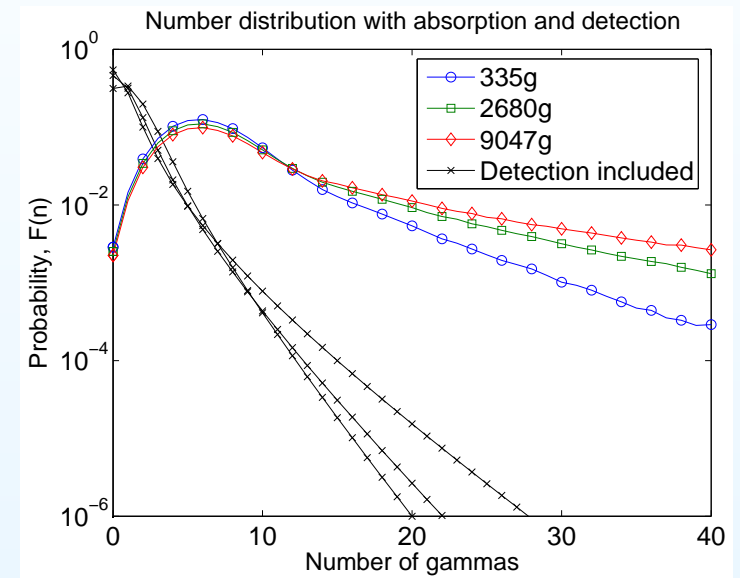
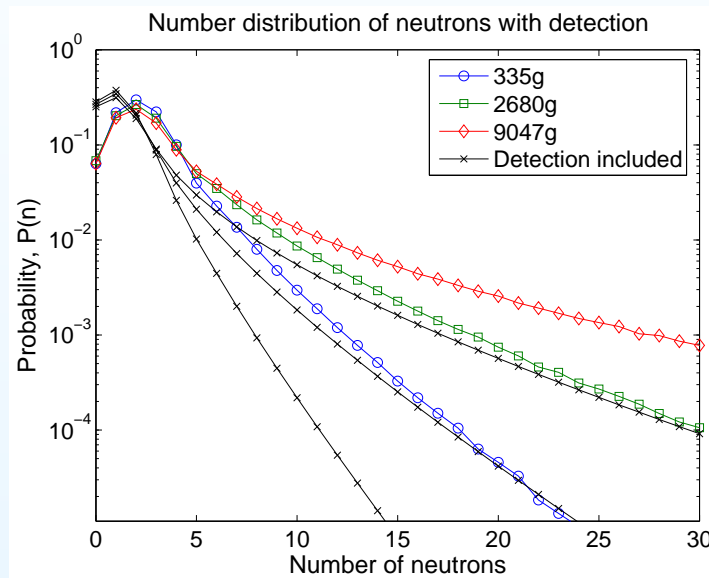
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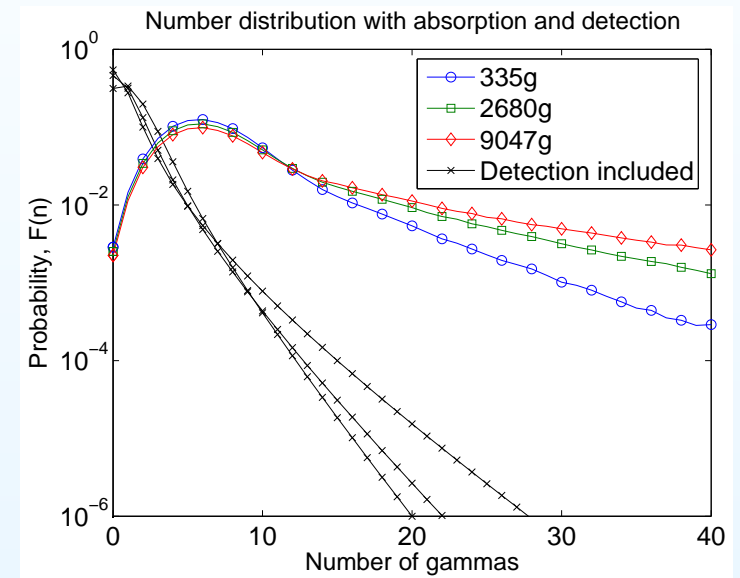
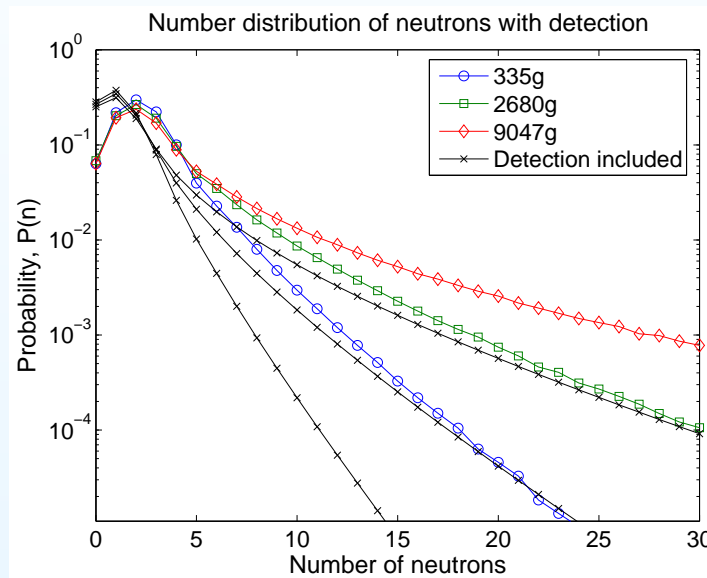
Gamma Distribution

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Including detection into the model with a detection allows to simulate actual detection statistics:



- Detection efficiencies of 50% were used here.
- The chance to detect many particles from the same source event decreases for both neutrons and photons.
- The photon multiplicities decrease so much that neutrons might be more favourable to observe even though they have lower source multiplicities.

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Simulated scintillation detector results

- Organic scintillators have the advantage to be able to detect both neutrons and photons, and to discriminate between them.
- MCNP-PoliMi was also used to simulate a scintillator detector setup with six detectors:

Simulated scintillation detector results

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Neutron Distribution

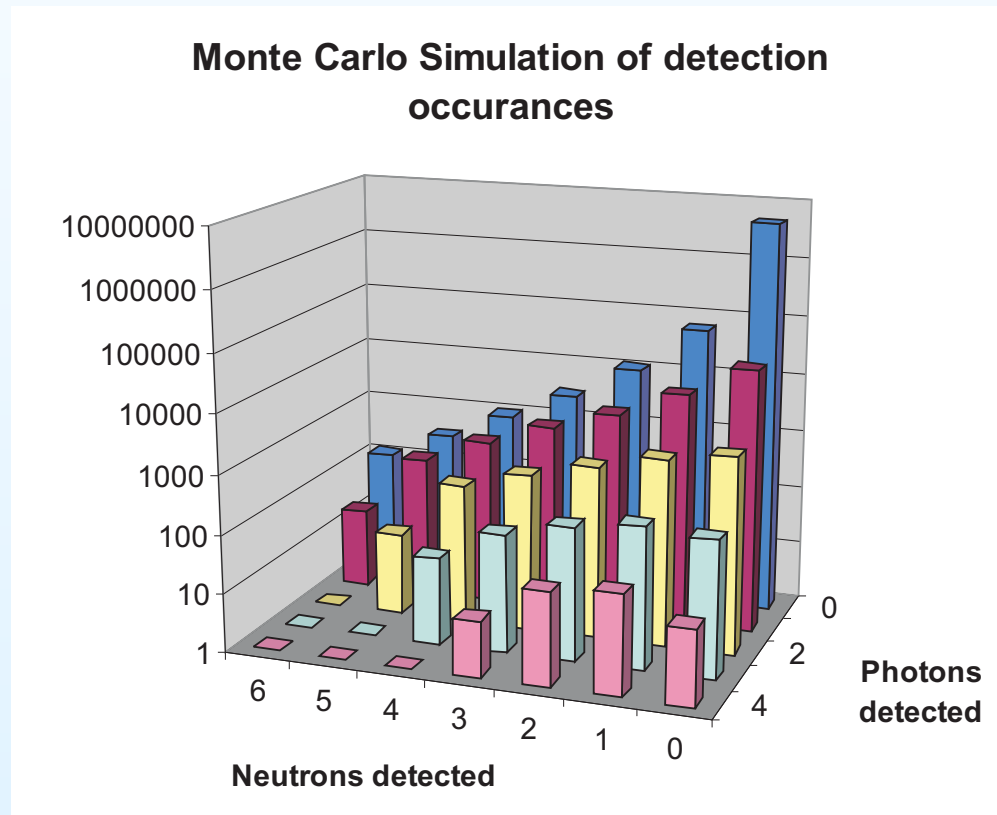
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- Master equations, earlier used for finding factorial moments, can be used to find probability distributions.

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- Symbolic computation makes it possible to find higher-order terms in a recursive manner.

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- Formal equivalence to factorial moments. We can now easily find factorial moments up to the same order as $P(n)$.

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- Extensions
 - Compare to experiments
 - Simulate realistic multiplicity counters