Working title: Fuel cycle impacts of deploying High Assay Low Enriched Uranium (HALEU)-fueled reactors

#### I. Introduction

- A. Purpose of the work: investigate the effect of deploying HALEU-fueled advanced reactors on the nuclear fuel cycle in the US
- B. Scope:
  - 1. US facilities
  - 2. select advanced reactors: USNC MMR, X-energy Xe-100, NuScale VOYGR
  - 3. Front-end and back-end of the fuel cycle
- C. Motivations
  - 1. U.S. wants to develop a supply chain of HALEU
  - 2. HALEU has broad uses outside of reactors [1]
  - 3. Benefits of using HALEU in reactors
    - a. higher burnups
    - b. reduced LCOE
  - 4. Changing the fuel form affects fuel cycle dynamics

## D. Goals

- 1. understand how deploying HALEU reactors affects resource demand
- 2. understand which components of the fuel cycle are most sensitive to HALEU deployments
- 3. understand how implementing recycling with HALEU reactors affects the fuel cycle
- 4. understand how possible avenues to obtain fuel for HAELU reactors can affect neutronics
  - a. neutron flux
  - b. k-eff
  - c.  $\beta$ -eff
  - d. feedback coefficients,
  - e. lifetimes
  - f. uranium utilization in general
  - g. breeding ratio

## II. Lit Review

- A. The nuclear fuel cycle
  - 1. Once-through vs recycle [2]
  - 2. Enrichment facility/SWU calculations [2]
  - 3. Recycling processes [2]
    - a. overview of aqueous reprocessing
    - b. Known changes to LWR fuel cycle by recycling
- B. Fuel Cycle simulators
  - 1. Why we use them, their benefits
  - 2. why multiples have been created
  - 3. ideal functionalities and capcbilities [3, 4]
  - 4. uses of fuel cycle simulators
    - a. Department of Energy (DOE) Evaluation & screening [5]
    - (1) Differences in EG 01 and EG 02
    - b. EG29 analysis [6]
    - c. verification [7]
  - 5. Cyclus [8]

- a. basic fundamentals
- b. Cycamore [9]
- c. addresses many of the things brought up by [3]
- d. comparison to other codes [10]
- e. verification [11]
- C. sensitivity analysis and optimization
  - 1. Dakota [12]
- D. HALEU
  - 1. production methods
  - 2. fuel forms (ceramic vs metallic vs TRISO)
  - 3. expected demand [13]
  - 4. Reactors with HALEU
    - a. Effects of changing from 5% to 7% for PWR [14]
    - b. Effects on NuScale SMR [15]
    - c. Effects of impurities [16]

## III. Transition modeling methodology

- A. Reactors, inculuding serpent models
  - 1. USNC MMR [17]
  - 2. X-energy Xe-100 [18]
  - 3. NuScale VOYGR [19, 20]
- B. material flow in fuel cycles
  - 1. Once-through
    - a. once-through flow figure
    - b. scenario definitions
  - 2. recycle
    - a. recycle-flow figure
    - b. scenario definitions

## IV. Material requirements – Once through fuel cycles

## A. Results

- 1. Scenario 1 current fleet
  - a. reactor deployment compare to actual data??
  - b. uranium resources
  - c. SWU capacity
  - d. Waste
- 2. Reactor deployment
  - a. No growth scenarios
  - b. 1% growth scenarios
- 3. Uranium resources
  - a. No growth scenarios
  - b. 1% growth scenarios
- 4. SWU capacity
  - a. No growth scenarios
  - b. 1% growth scenarios
- 5. Waste

- a. No growth scenarios
- b. 1% growth scenarios

## V. Model fuel cycle with recycle

## A. Results

- 1. Reactor deployment
  - a. No growth scenarios
  - b. 1% growth scenarios
- 2. Uranium resources
  - a. No growth scenarios
  - b. 1% growth scenarios
- 3. SWU capacity
  - a. No growth scenarios
  - b. 1% growth scenarios
- 4. Waste
  - a. No growth scenarios
  - b. 1% growth scenarios

## VI. Sensitivity analysis and optimization

- A. Methdology
- B. Results

## VII. Effects on neutronics

- A. Methodology
- B. Models
- C. Results

## VIII. Conclusions

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