Project Involvements

☐ [MSR COUPLE] Development of a 3D Multi-physics Safety/Performance Platform

- OpenFOAM case development
- o OpenFOAM interface coupling methodology development

☐ [MSR] Development and Analysis of Fuel Salt Leakage Behavior in Discharge System Shell

- Project Development and Management (APM)
- Methodology development
- o Conceptualization and CFD analysis of fuel drain tank

☐ [SMR] Small and Micro Modular Reactors Technology Analyses

- Project Development and Management (APM)
- Literature and technology review
- Technical feasibility analysis
- o Commercial feasibility analysis

☐ [SMART] System-Integrated Modular Advanced Reactor Passive Containment Cooling System

- o Modeling of SMART CPRSS through GOTHIC code
- Literature survey for the design and arrangement of a SMART CPRSS sparger
 - Direct contact condensation and Chugging
- o Modeling of SISTA (CPRSS-related experimental facility) through MARS code
- CAP Verification
 - Simulations of condensation and boiling conceptual problems through MARS code
 - Simulations of a conceptual problem through MARS and SPACE codes
- CAP input preparation for Sub P/T Analysis
- o CPRSS hydraulics and heat structure calculation sheet preparation for CAP code

□ [PGSFR] Prototype Gen-IV Sodium-Cooled Fast Reactor Condition Analysis Project

- Simulations of Design Basis Events through MARS-LMR
 - Leakage of Intermediate Heat Transport System Pipe
 - Reactor Vessel Leak into the Containment Vessel

☐ [SMART DEC] Feasibility Study of the SPACE Code Application on DEC Accident Analysis of the SMART Nuclear Power Plant III

- SPACE code calculation sheet preparation
 - Passive Safety Injection System (PSIS)
 - Passive Residual Heat Removal System (PRHRS)

☐ [PCCS] Conceptual Design and Development of Passive Containment Cooling System for APR1400

- o Literature survey on condensation with non-condensable gases
 - Separate and Integral Test Facilities to validate the MARS condensation model
 - Available condensation models
 - Review of MARS condensation model
- o COPAIN test facility (on-plane condensation) simulations through MARS code
- o On-tube condensation simulations through MARS code
 - MARS trouble report to KINS: Volume average velocity deficiency
- o Multi-compartment and multi-dimension simulations (conceptual) through GOTHIC code

Erol Bicer, Ph.D.Nuclear Engineer



☐ [ATLAS] Determination of ATLAS Heat Loss and Improvement of MARS Input Model

- o ATLAS experimental facility heat loss quantification of RCS side
- o Integral and differential approaches
- Implementation of heat loss modeling to MARS input
- o Simulations of OECD and DSP problems with the new MARS input

☐ [NON-LOCA] SPACE Code Input Deck Preparation and Validation

- HANUL 1&2 hydraulics and heat structure calculation sheet preparation for SPACE code
 - Reactor Pressure Vessel, Steam Generators, Secondary Systems, Safety Systems
- HANUL5&6 hydraulics and heat structure calculation sheet preparation for SPACE code
 - Steam Generators, Steam Lines, Safety Systems
 - SPACE Code Validation
 - DSP01 ATLAS DVI Line Break SBLOCA Simulation
 - DSP02 ATLAS CL SBLOCA
 - Marviken Sensitivity Tests

☐ [PAFS] Passive Auxiliary Feedwater System Development

- o Calculation of pipe flow resistance coefficients
- o RELAP5 code input deck preparation for Hanul NPP Units 5&6 (OPR1000)
 - Full steady-state input model
 - Code-to-code verification with SPACE

☐ [ACHX] Air Cooling Heat Exchanger Experiment Simulations through MARS

- o Fin efficiency and fouling factor analyses
- o Heat transfer coefficient and loss coefficient calculations

☐ [IoT] A Review of Regulatory Criteria for using IoT Technologies in Radioactive Waste Management

- o Review of Radio Frequency Identification System and IoT technologies
- Review of IoT applications on radioactive waste management
- Review of global IoT standards and best practices

☐ [EUR] APR1000 PAFS Standard Design Development and EUR Certification Support

- o APR100 Design Review and Documentation
- o EUR Certification Documentation

Academic Contributions

Graduate Research

- □ Ph.D. Dissertation: **E. Bicer**, "Development of a Mean Bubble Size Approach to Improve Pool Scrubbing Swarm Region Modeling," Doctoral Dissertation, Seoul National University, 2024.
- Master's Thesis: E. Bicer, "A Comparative Study of DVI Line Break Accident between ATLAS Facility and APR1400," Master's Thesis, KEPCO International Nuclear Graduate School, 2015.



Journal Articles

- 1. **E. Bicer**, S. J. Hong, and H. K. Cho, "A Novel Correlation for Bubble Size Variation in the Swarm Region under Pool Scrubbing Conditions," *Progress in Nuclear Energy*, vol. 176, pp. 105366–105366, Aug. 2024.
- 2. **E. Bicer**, S. J. Hong, and H. K. Cho, "Investigation of Equivalent Spherical Bubble Diameter at High Inlet Velocity Pool Scrubbing Conditions," *Nuclear Engineering and Technology*, May 2024.

Proceedings

- 1. **E. Bicer**, S.-S. Jeon, Y.-J. Choo, S.-J. Hong, and Y. Cho, "Improving Swarm Region Modeling in Pool Scrubbing through a Mean Bubble Size Approach," in *The 13th Korea-Japan Symposium on Nuclear Thermal Hydraulics and Safety*, Seoul, South Korea: NTHAS-13, Oct. 2024.
- 2. **E. Bicer**, Y.-J. Choo, and S.-J. Hong, "Improvement of Swarm Rise Region Modelling in SPARC-90 Pool Scrubbing Code," in *Transactions of the Korean Nuclear Society Autumn Meeting*, Changwon, Korea: KNS, Oct. 2024.
- 3. **E. Bicer**, Y. J. Park, Y.-J. Choo, S. J. Hong, and Y. J. Cho, "The Effect of Bubble Diameter on Decontamination Factor in Pool Scrubbing Codes," in *Summer Conference of the Korean Society for Fluid Machinery*, Pyeongchang, South Korea: KSFM, Jul. 2024.
- 4. **E. Bicer**, Y. Park, Y. Choi, and S.-J. Hong, "Numerical Analysis of Melting/Solidification Problems using OpenFOAM," in *Transactions of the Korean Nuclear Society Spring Meeting*, Jeju, South Korea: KNS, May 2024.
- 5. **E. Bicer**, Y.-J. Choo, and S.-J. Hong, "Development of a Mean Bubble Size Correlation under Pool Scrubbing Conditions," in Transactions of the Korean Nuclear Society Spring Meeting, Jeju, South Korea: KNS, May 2024.
- 6. **E. Bicer**, S. Li, and Y. Liao, "Estimation of Turbulence Parameters in Pool Scrubbing Conditions," in *The 19th Multiphase Flow Conference*, Dresden, Germany: HZDR, Jun. 2023.
- 7. **E. Bicer**, S.-S. Jeon, Y.-J. Choo, and S.-J. Hong, "Influence of Inlet Turbulent Boundary Conditions on Bubble Diameter Calculation," in *Transactions of the Korean Nuclear Society Spring Meeting*, Jeju, South Korea: KNS, May 2023.
- 8. **E. Bicer**, S.-J. Hong, and S.-S. Jeon, "Bubble Diameter Evaluation in Pool Scrubbing Geometries," in *Proceedings of ICAPP*, Gyeongju, South Korea: ICAPP, Apr. 2023.
- 9. **E. Bicer**, H. Cho, and H. Joon, "CFD Simulation of High Inlet Velocity Air Flow into a Large Tank at Pool Scrubbing Conditions," in *Transactions of the Korean Nuclear Society Autumn Meeting*, Changwon, South Korea: KNS, Oct. 2021.
- 10.S.-J. Hong, S.-S. Jeon, Y.-J. Choo, J.-H. Lee, J.-H. Bae, **E. Bicer**, B.-D. Chung, Y.-I. Kim, "CAP Code Version-up to 3.0 and Its Application to Pressure and Temperature Analysis," in *Transactions of the Korean Nuclear Society Autumn Meeting*, Changwon, South Korea: KNS, Oct. 2020.
- 11.S.-S. Jeon, **E. Bicer**, J.-H. Bae, S.-J. Hong, B.-U. Bae, and K.-H. Kang, "System Code Analysis of ATLAS Facility Including Heat Loss Evaluation," in *Transactions of the Korean Nuclear Society Spring Meeting*, Jeju, South Korea: KNS, Apr. 2019.
- 12. **E. Bicer**, Y.-J. Choo, and S.-J. Hong, "Capability of MARS-KS on Predicting Wall Condensation in the Presence of Non-Condensable Gases," in *Transactions of the Korean Nuclear Society Autumn Meeting*, Gyeongju, South Korea: KNS, Oct. 2016.
- 13.E. Bicer, Y.-J. Choo, S.-S. Jeon, S.-S. Kim, Y.-H. Kim, and S.-J. Hong, "Prediction of Wall Condensation in the Presence of Non-Condensable Gases through Various Thermal-Hydraulic Codes," in 25th International Conference Nuclear Energy for New Europe, Portorož, Slovenia: NENE-2016, Sep. 2016.
- 14.**E. Bicer**, A. L. Tatu, and T. Kim, "Scaling Analysis for DVI Line Break Accident of APR 1400 Based on ATLAS Experiment," in *International Topical Meeting on Nuclear Reactor Thermal Hydraulics*, Chicago, USA: NURET-16, Aug. 2015.

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