### **NE 255**

# **Numerical Simulations in Radiation Transport**

Geometry, Collisions, and Scoring

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## MAJOR COMPONENTS OF MC ALGORITHM

- **PDFs**: the physical/mathematical system must be described by a set of pdfs.
- Random number generator: a source of random #s uniformly distributed on the unit interval.
- **Sampling rule**: prescription for sampling the pdf (given having random #s)
- *Scoring*: the outcomes must be accumulated/<u>tallied</u> for quantities of interest
- **Error estimation**: an estimate of the statistical error (<u>variance</u>) of the solution
- Variance Reduction: methods for reducing the variance and computation time simultaneously
- Parallelization: efficient use of computers

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### **OUTLINE**

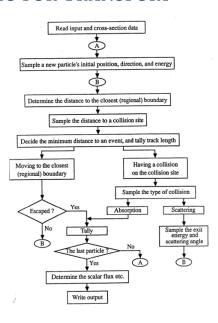
- Determining next event location
  - Sampling flight path
  - Distance to boundary
  - Next event selection
- 2 Collision Physics
  - Sampling target nuclide
  - Sampling reaction type
  - Sampling exit direction
  - Sampling exiting particles
- 3 Scoring

Notes derived from Jasmina Vujic and Paul Wilson

# LEARNING OBJECTIVES

- Understand basic tracking of particles through a geometry
  - Understand the steps necessary for tracking particles
  - Understand the use of mean free path
  - Sample the distance to the next physics event
  - · Determine next event
- 2 Understand what sampling needs to happen after a collision
- 3 Understand how to translate interactions into a score

#### MONTE CARLO FOR TRANSPORT



### POSSIBLE FUTURES FOR A PARTICLE

After we've gotten to Circle B, we have a neutral particle:

- At point  $(x_p, y_p, z_p)$
- Moving in direction (u, v, w)
- With energy E

What are possible next events?

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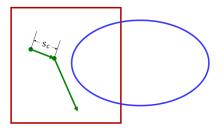


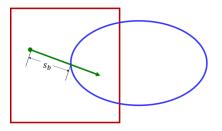
Figure 1: Collision

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**Figure 1:** fig/Surface Crossing

### SAMPLING DISTANCE TO COLLISION

#### Collisions are probabilistic

- Note that  $\Sigma_t$ , the total macroscopic cross section, will be a function of space if we have multiple materials
- Along a particular path, the probability of a collision at distance s from the start:

$$p_c(s)ds = \sum_t (s)e^{-\sum_t (s)s}ds$$

$$P_c(s) = \int_0^s \sum_t (s)e^{-\sum_t (s)s'}ds' = -e^{-\sum_t (s)s'}|_0^s = 1 - e^{-\sum_t (s)s}$$

• The cross section,  $\Sigma_t(s)$ , is piecewise constant, but changing

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### SAMPLING DISTANCE TO COLLISION

• Variable transformation: measure distance in units of *mean free path*:

$$n = \Sigma_t(s)s$$
,  $dn = \Sigma_t(s)ds$ 

We'll start with the PDF and integrate to get the CDF

$$p_c(n)dn = e^{-n}dn$$
  
 $P_c(n) = \int_0^n e^{-n'}dn' = -e^{-n'}|_0^n = 1 - e^{-n}$ 

• Importantly, this is now independent of the material

### SAMPLING DISTANCE TO COLLISION

Randomly sample to determine number of mean free paths until next collision,  $n_c$ 

- $g(n_c)dn_c = e^{-n_c}dn_c$
- $G(n_c)dn_c = 1 e^{-n_c}$
- Directly invert to get:  $n_c = -\ln(1-\xi)$  [note  $(1-\xi)$  is equivalent to  $\xi$ ]
- In the absence of material boundaries ( $\Sigma_t \neq f(s)$ ), the distance to a collision,  $s_c$ , is

$$s_c = \frac{n_c}{\Sigma_t}$$

### CALCULATING DISTANCE TO BOUNDARY

- Usually have more than one material
- Distance to boundary is deterministic
- Algebra to determine distance between point and surface,  $s_h$
- Convert it to units of mean free path for the current cell's material,

$$n_b = s_b \Sigma_t$$

### **GEOMETRY REPRESENTATIONS**

- Combinatorial Surfaces
  - Define surfaces
  - Boolean operations combine surfaces to create cells
- Combinatorial Solids
  - Choose solid objects
  - Boolean operations combine objects to create regions
- B-Rep (Vertex-Edge)
  - Each object is a single set of vertices and edges connecting them

We're skipping how to find  $s_b$ , just know that we can find it using the internal geometry representation

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#### $n_b > n_c$ :

- Boundary is further away than collision
- Collision occurs

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- Using physics models and/or cross-sections
  - Sample target nuclide
  - Sample reaction type
  - Sample new direction
  - Sample new energy
  - Sample exiting particles
- Some of these may depend on one another

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- Some of these may depend on one another
- Repeat
  - Sample new  $n_c$  following collision
  - Calculate new  $n_b$  in new direction

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### **OPTION B: CELL BOUNDARY**

#### $n_b < n_c$ :

- Boundary is closer than collision
- Boundary crossing occurs

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#### $n_b < n_c$ :

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- Boundary crossing occurs
- Move particle along ray
  - Update  $n_c = n_c n_b$
- **DO NOT SAMPLE** for new *n<sub>c</sub>*

### **OPTION B: CELL BOUNDARY**

#### $n_b < n_c$ :

- Boundary is closer than collision
- Boundary crossing occurs
- Move particle along ray
  - Update  $n_c = n_c n_b$
- **DO NOT SAMPLE** for new *n<sub>c</sub>*
- Calculate new  $n_b$  in new cell
  - New set of boundaries
  - New value of  $\Sigma_t$

### So You HAD A COLLISION?

• Sample **target nuclide** for a mixture with *J* nuclides

$$\Sigma_t = \sum_{j=1}^J N_j \sigma_{t,j}$$

• Discrete PDF to determine which nuclide is hit

$$p_j = \frac{\Sigma_{t,j}}{\Sigma_t}$$

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• Discrete PDF to determine which nuclide is hit

$$p_j = \frac{\Sigma_{t,j}}{\Sigma_t}$$

• Sample reaction type for an nuclide with R types of reactions

$$\Sigma_{t,j} = \sum_{x=1}^{R} \Sigma_{x,j}$$

• Discrete PDF to determine which reaction occurs

$$p_x = \frac{\Sigma_{x,j}}{\Sigma_{t,j}}$$

### **OUTCOME OF REACTION**

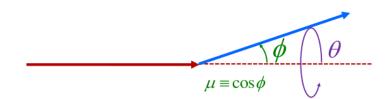
- Particle maybe absorbed
- Particle may continue its history in a *different direction* and/or with a *different energy*
- Energy-angle distributions are tabulated in different formats
  - Scattering laws have analytic forms with parameters in data tables (Direct inversion or rejection sampling)
  - Tabulated data that describes a piecewise analytic interpolation (Hybrid sampling; we skipped this)

#### USING A SCATTERING ANGLE

Scattering angles are defined relative to the original direction (considered as the z-axis)

- Polar angle,  $\phi$ , determined by sampling from data
- Azimuthal angle,  $\theta$ , determined by sampling isotropically
- The new direction is  $(\sin(\theta)\cos(\phi),\sin(\theta)\sin(\phi),\cos(\phi))$

$$= \left(\sqrt{1-\mu^2}\cos(\theta), \sqrt{1-\mu^2}\sin(\theta), \mu\right)$$



### SUMMARY OF PART I

We've developed a general sense of using MC for neutron transport

• Basic Algorithm

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We've developed a general sense of using MC for neutron transport

- Basic Algorithm
- We can determine if particles have collisions or cross boundaries
- *After a collisions* we need to determine many things associated with the collisions (target, reaction, direction, energy)
- Repeat analysis for collisions/crossing until particle **terminates**