



# NucE497: Reactor Fuel Performance

## Exam-2 review

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**PennState**  
College of Engineering

**MECHANICAL AND  
NUCLEAR ENGINEERING**

Exam Date:

April 20 at 12 am – April 21 at 11:59 pm

Total time:

TBD(Download+Solve+Convert/Upload)

If you create your PDF and then find something you would like to change, you are permitted to edit your PDF or add notes before submitting. I recommend you have two methods set up to convert your solutions to a PDF, as I will not accept "I couldn't make the PDF" as a valid excuse.



## Exam Format

- Exam includes both conceptual and workout problems.
- Conceptual questions include everything discussed in the class.
- Workout problems will be similar to the problems you did on the homework, or that we did in class.
- **Study thoroughly for the exam**

# Course content

- Module-1: Fuel basics
- Module-2: Heat transport
- Module-3: Mechanical Behavior
- Module-4: Materials issues in the fuel
- Module-5: Materials issues in the cladding
- Module-6: Accidents, used fuel, and fuel cycle

# You are responsible for the last three modules...

- Module-1: Fuel basics
- Module-2: Heat transport
- Module-3: Mechanical Behavior
- **Module-4: Materials issues in the fuel**
- **Module-5: Materials issues in the cladding**
- **Module-6: Accidents, used fuel, and fuel cycle**

## Module-4: Materials issues in the fuel

- Property evolution and intro to materials science
- Chemistry
- Grain growth
- Fission product and fission gas
- Densification, swelling and creep
- High burnup structure
- Fracture
- Thermal Conductivity

## Module-5: Materials issues in the cladding

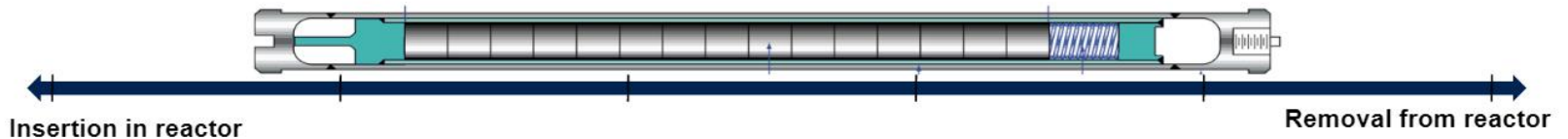
- Zirconium alloys and fabrication
- Cladding growth and creep
- Irradiation hardening
- Oxidation & Hydride formation
- Stress corrosion cracking

## Module-6: Accidents, used fuel, and fuel cycle

- RIA & LOCA
- Accident tolerant fuel
- Fuel cycle and used fuel disposition



# Module-4: Materials issues in the fuel



## Early life

- Thermal expansion
- Fracture
- Point defect and fission gas generation
- Fuel Densification

## Mid Life

- Point defect diffusion
- Point defect clustering
- Fission gas segregation to GB and voids
- Bubble nucleation

## Late life

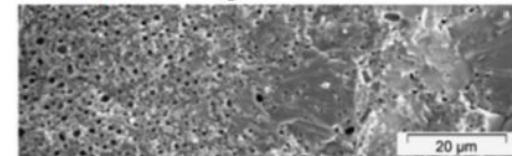
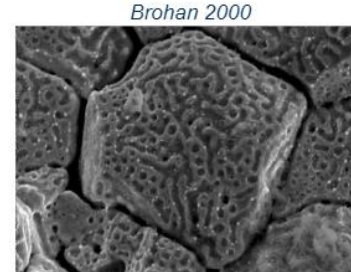
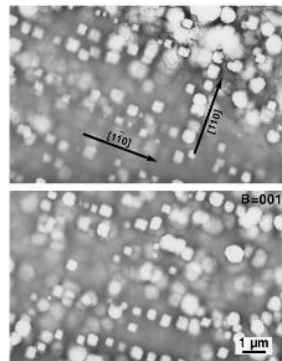
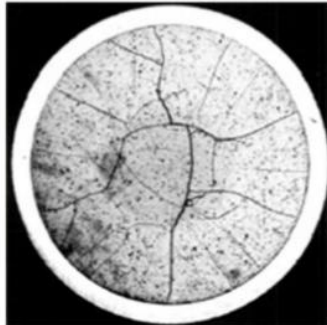
- Fission product swelling
- Bubble percolation and fission gas release
- Cladding creep
- Fuel creep

## Fuel failure

- Pellet/cladding interaction
- Cladding corrosion
- Cladding fracture

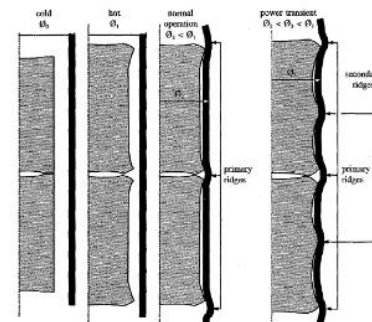
## Fuel Fabrication

- Sintered  $\text{UO}_2$
- Height 2 cm
- Diameter 1 cm
- $\sim 10 \mu\text{m}$  grain size
- Density > 95%

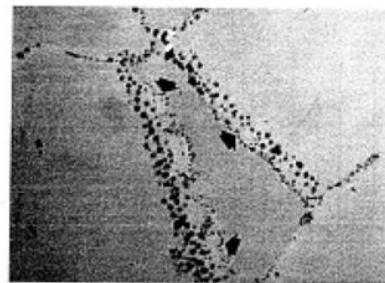
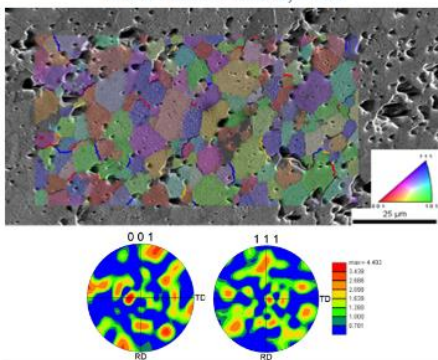


From Pedro Peralta, ASU

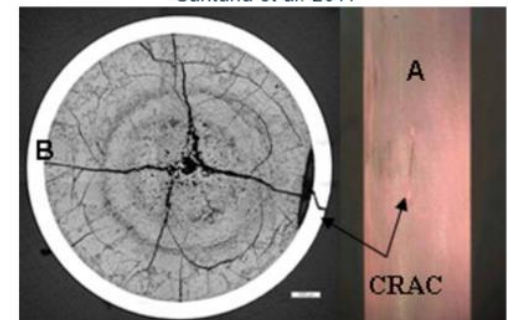
Zinkle and Singh 2000



Santana et al. 2011



Olander, p. 323 (1978)



# Example: Fission gas release

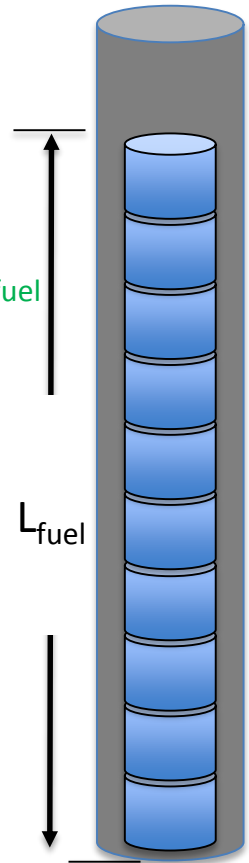
What is the total number of fission gas atoms produced within pellets after 2 years?(fission yield of Xe and Kr is  $y$ )

# of gas atoms produced in the fuel

$$N_{\text{gas}} = y \dot{F} t V_{\text{fuel}} \quad V_{\text{fuel}} [\text{cm}^3] = \pi R^2 L_{\text{fuel}}$$

$$\dot{F} [\text{fission}/\text{cm}^3\text{-s}] = q \sigma_f N \phi$$

$$t [\text{s}] = 2 \times 365 \times 24 \times 3600$$





## Example: Fission gas release (cont.)

What is the total number of fission gas atoms released to the gap+plenum after 2 years?

# of gas atoms released to the gap+plenum  $\xrightarrow{\quad}$   $N_{\text{released}} = f N_{\text{gas}}$

Fraction of gas atoms released

$$\text{in - pile} \Rightarrow f = 4 \sqrt{\frac{Dt}{\pi a^2}} - \frac{3}{2} \frac{Dt}{a^2}$$

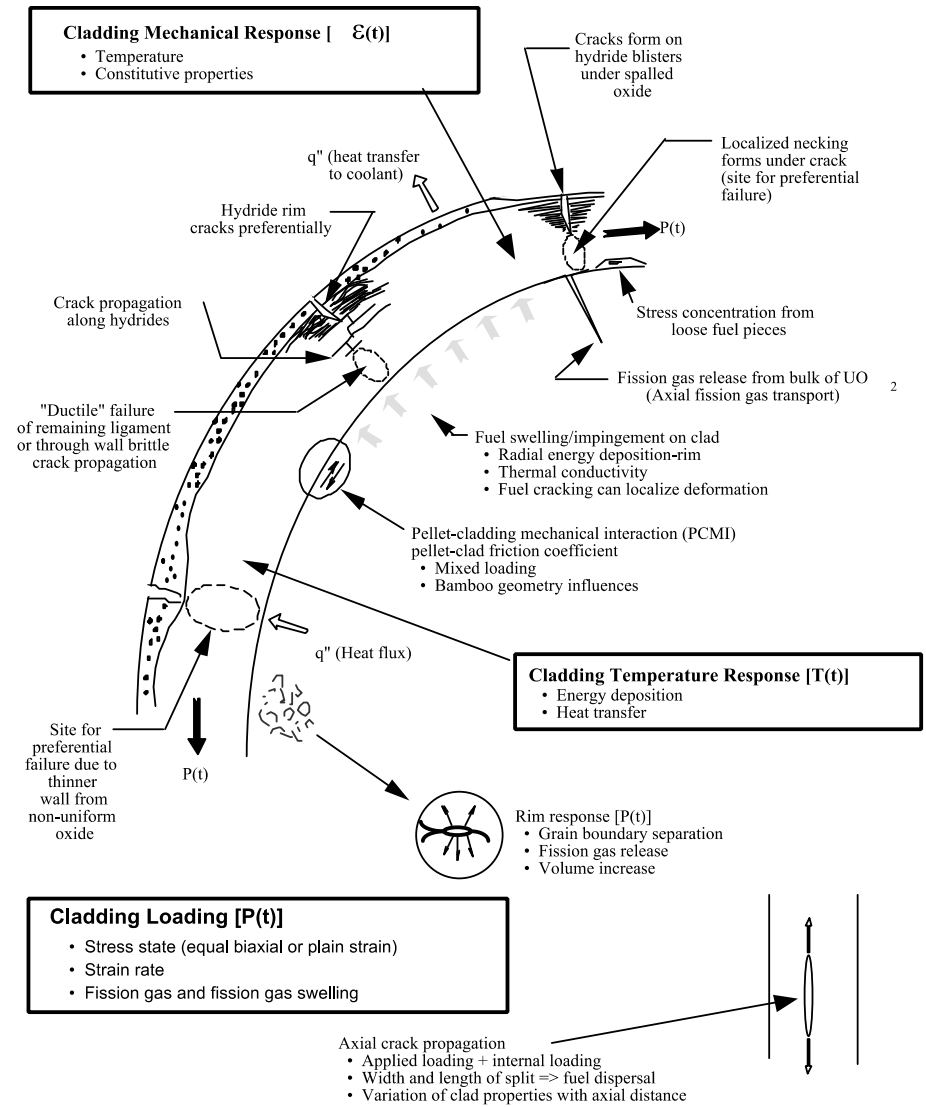
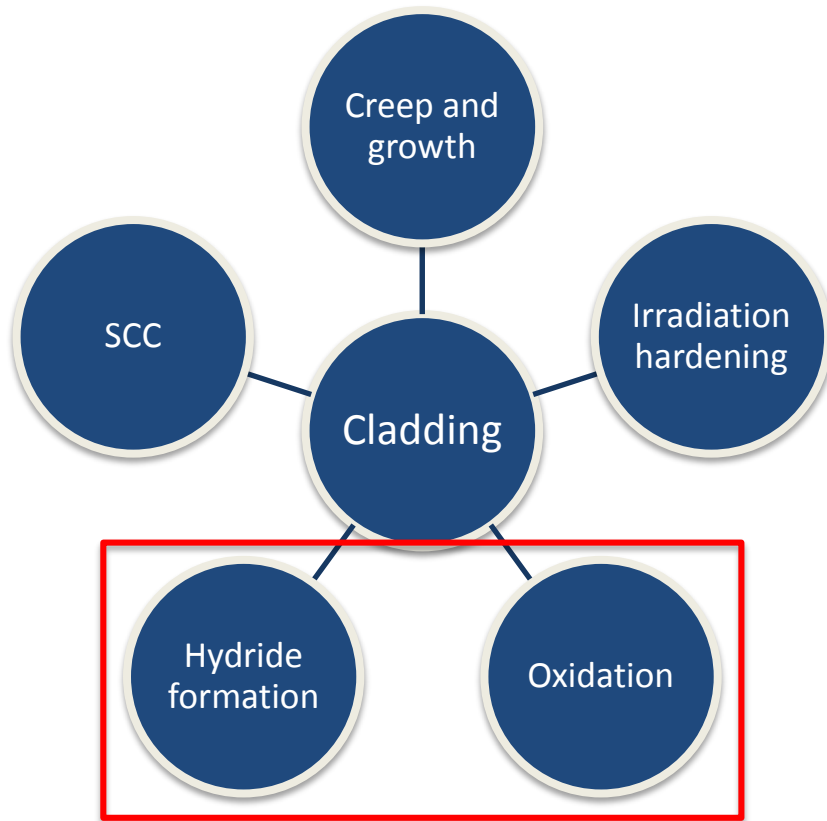
D: Diffusion coefficient

a : Grain size

- Constant
- Temperature dependent (Lec 25, Slide 21)

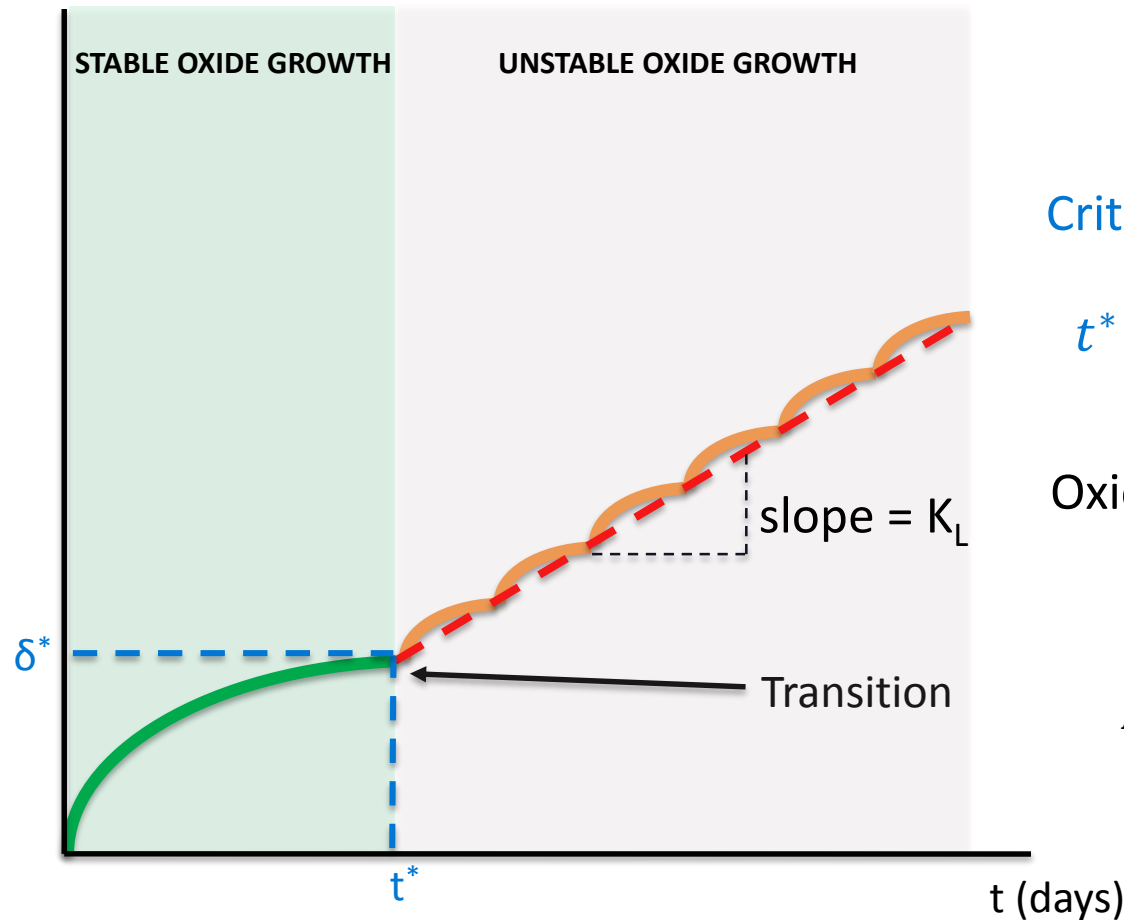
$$\text{out - of - pile} \Rightarrow f = 6 \sqrt{\frac{Dt}{\pi a^2}} - 3 \frac{Dt}{a^2}$$

# Module-5: Materials issues in the cladding



# Oxidation of Zr cladding

$\delta_{\text{oxide}} (\mu\text{m})$



Critical oxide thickness for transition:

$$\delta^* = 5.1 \exp \left[ \frac{-550}{T} \right]$$

Critical time for transition:

$$t^* = 6.62 \times 10^{-7} \exp \left[ \frac{11949}{T} \right]$$

Oxide thickness after transition is:

$$\delta [\mu\text{m}] = \delta^* + K_L (t - t^*)$$

$$K_L = 7.48 \times 10^6 \exp \left[ \frac{-12500}{T} \right]$$

Values are given for zirlo cladding

# Example-1: Oxidation of Zr cladding

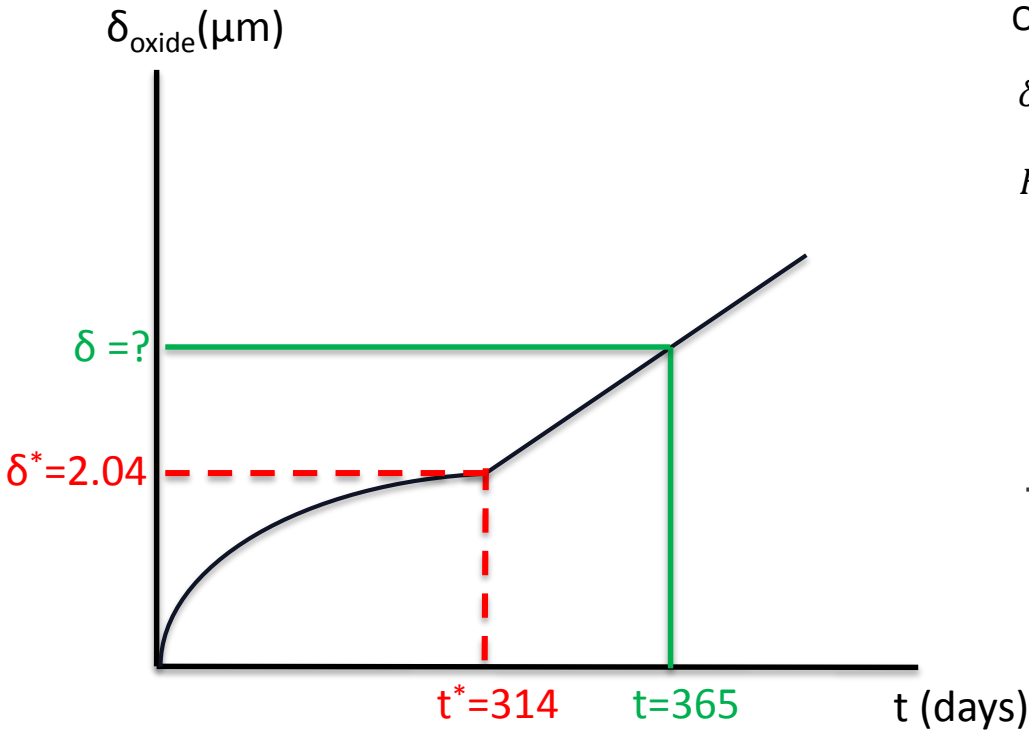
For zirlo cladding, what is the oxide thickness after 1 year of irradiation at 325°C?

Critical time for transition:

$$t^* = 6.62 \times 10^{-7} \exp \left[ \frac{11949}{598} \right] = 314 \text{ days}$$

Critical oxide thickness for transition:

$$\delta^* = 5.1 \exp \left[ \frac{-550}{598} \right] = 2.04 \text{ } \mu\text{m}$$



Oxide thickness after 1 year is:

$$\delta [\mu\text{m}] = \delta^* + K_L(t - t^*)$$

$$K_L = 7.48 \times 10^6 \exp \left[ \frac{-12500}{598} \right] = 6.25 \times 10^{-3}$$

$$\delta [\mu\text{m}] = 2.04 + 6.25 \times 10^{-3}(365 - 314)$$

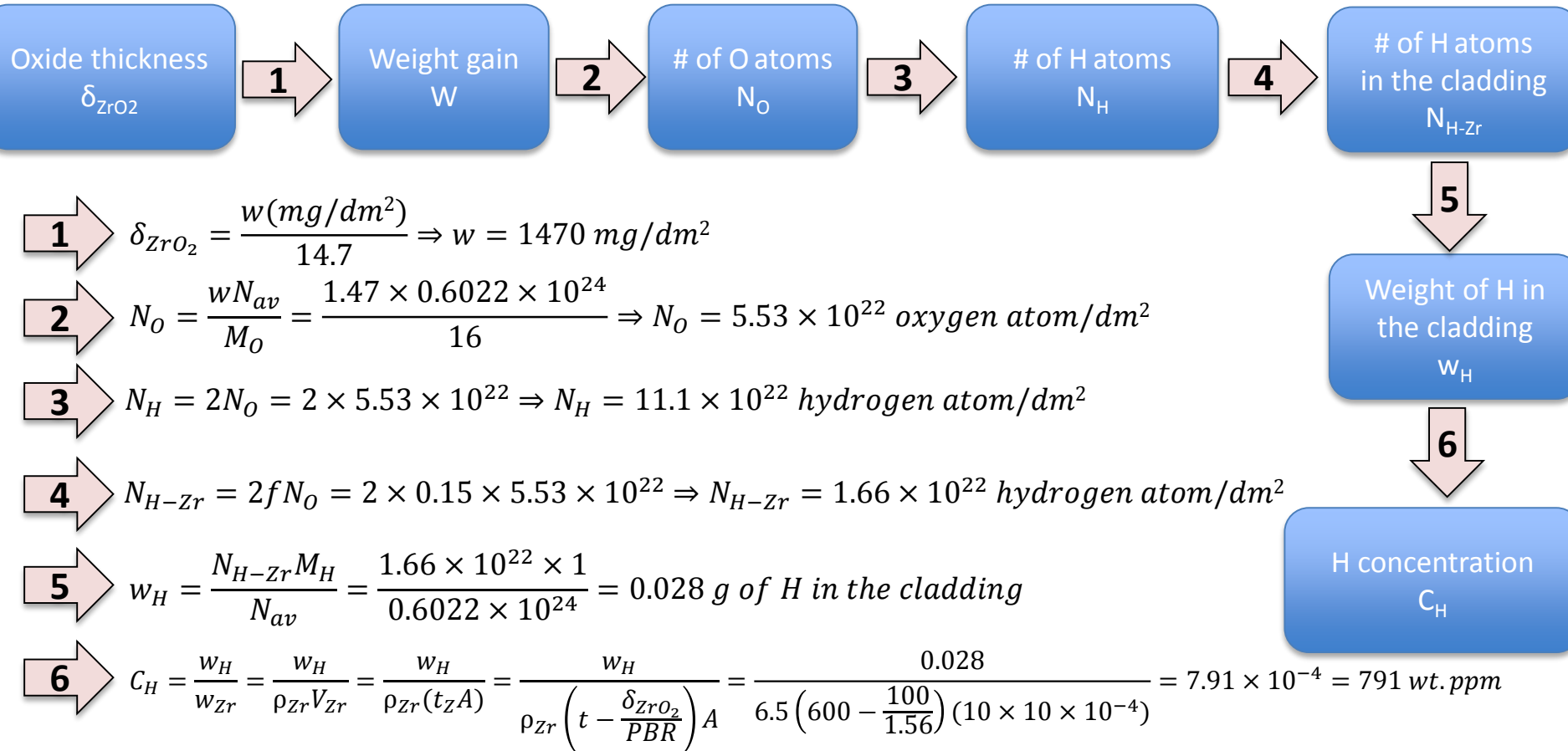
$$\delta [\mu\text{m}] = 2.34 \text{ } \mu\text{m}$$

Total weight gain ( $\text{mg}/\text{dm}^2$ ):

$$\delta [\mu\text{m}] = \frac{w \left[ \frac{\text{mg}}{\text{dm}^2} \right]}{14.7} \rightarrow w = 34.4 \text{ mg}/\text{dm}^2$$

## Example-2: Hydride formation in the cladding

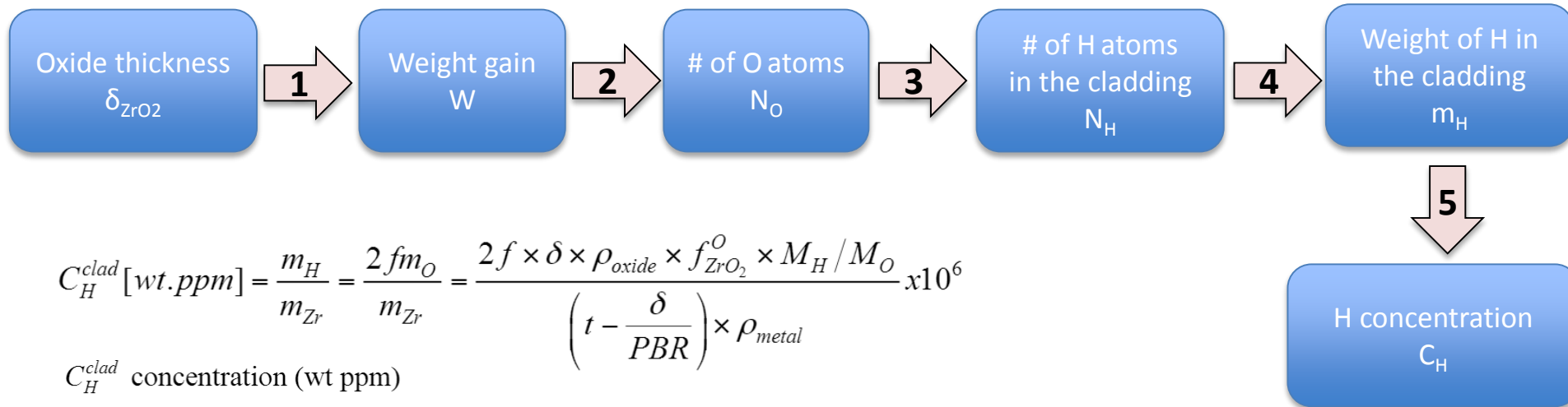
A cladding with an initial thickness of 600 microns undergoes corrosion to a total oxide thickness of 100 microns. What is the overall hydrogen content in wt. ppm if the hydrogen pickup fraction is 15%?





## Example-2: Hydride formation in the cladding

A cladding with an initial thickness of 600 microns undergoes corrosion to a total oxide thickness of 100 microns. What is the overall hydrogen content in wt. ppm if the hydrogen pickup fraction is 15%?



$$C_H^{clad} [wt.ppm] = \frac{m_H}{m_{Zr}} = \frac{2 f m_O}{m_{Zr}} = \frac{2 f \times \delta \times \rho_{oxide} \times f_{ZrO_2}^O \times M_H / M_O}{\left( t - \frac{\delta}{PBR} \right) \times \rho_{metal}} \times 10^6$$

$C_H^{clad}$  concentration (wt ppm)

$\rho_{oxide}$  oxide density

$\rho_{Zr}$  Zr metal density

$f_{ZrO_2}^O$  Fraction of oxygen in ZrO<sub>2</sub> mass

$PBR$  Pilling-Bedworth Ratio

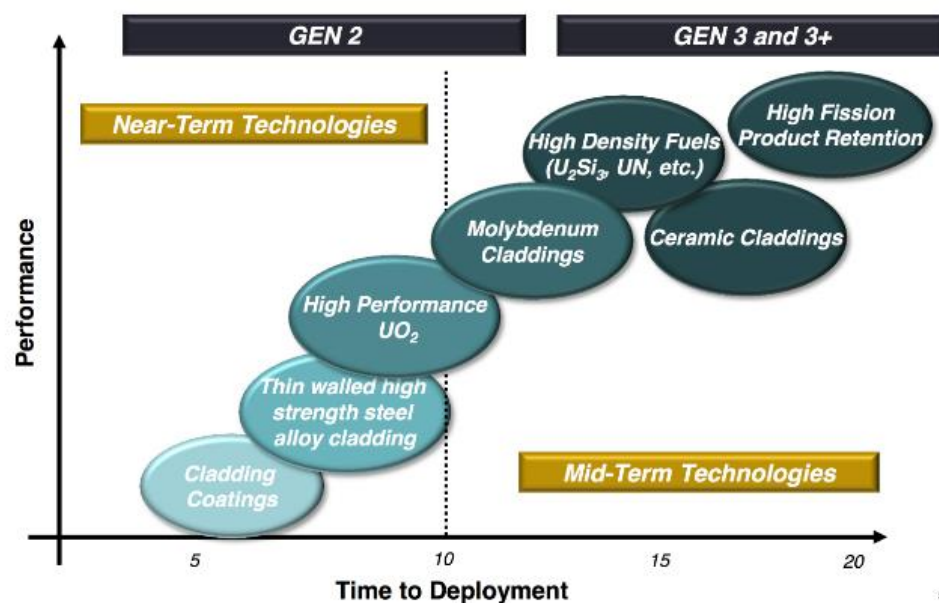
$M_H$  molecular mass of H

$M_O$  molecular mass of O

$t$  cladding thickness

## Module-6: Accidents, used fuel, and fuel cycle

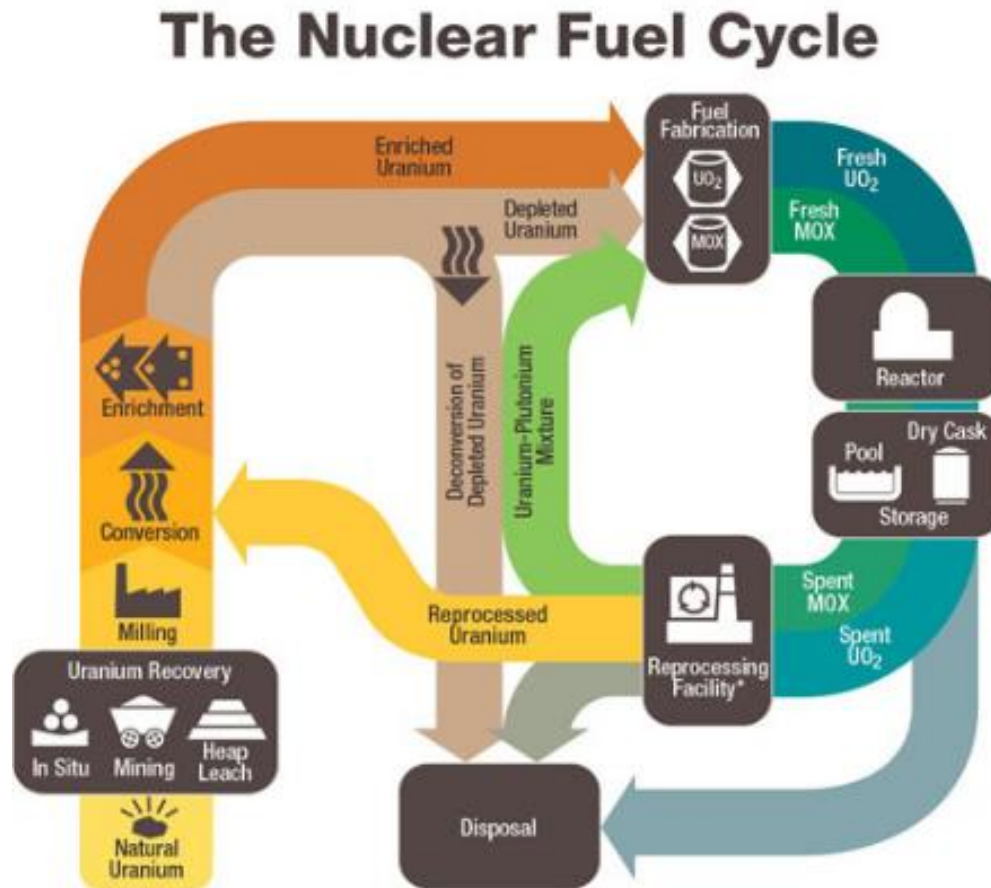
- RIA : Due to large and rapid insertion of reactivity
- LOCA : Due to loss of coolant
- Accident Tolerant Fuels:



S. Braag-Sitton, 2014

- The goal is to develop fuel that can tolerate loss of active cooling for a considerably longer period while maintaining or improving performance during normal operation

# Module-6: Accidents, used fuel, and fuel cycle



\* Reprocessing of spent nuclear fuel including MOX is not practiced in the U.S.  
Note: The NRC has no regulatory role in mining uranium.



