NE 795: Advanced Reactor Materials

Fall 2023 Dr. Benjamin Beeler

Housekeeping

- All should have received exam grades and presentation grades/comments
- Please come to me with any questions, concerns, comments, etc.
- Hopefully some presentation comments are helpful

Last Time

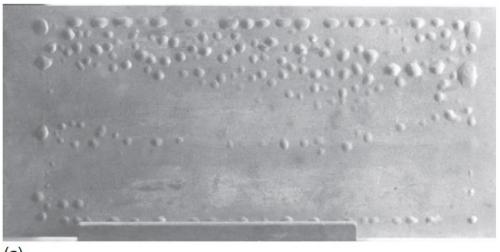
- Introduction to research reactor fuel
- Key features:
 - low temperatures, no gap or plenum, high burnup
- Primary type is dispersed particles of metallic compound/alloy in a fuel block embedded in AI matrix; key features often include solid/gaseous swelling, interaction layer growth, and amorphization
- Intro to both Alx and USi RR fuels
- USi-type fuels have a higher density than UAIx, but swell significantly more
- U3Si2 has more stable bubble morphology than U3Si, due to their amorphous behavior

Blister Testing

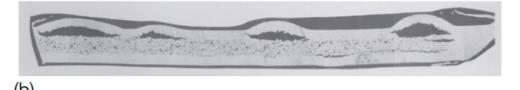
- A unique measure for sound fuel performance considered in research and test reactors is 'blister threshold temperature' testing with irradiated plates
- Because there is neither a gap nor a plenum, no fission gas release is possible outside of the fueled zone
- Fission gas and any gas included during fabrication remain in the fueled zone; in particular, fission gases are contained in pores or fission gas bubbles
- Gas pressure in large pores and fission gas bubbles, which may be insufficient to cause detrimental creep or yielding of fuel, could instead result in blistering of a fuel plate when the plate is heated to a certain temperature
- Two types of mechanisms can be considered for blistering: pore (or void) connection, and pressure rupture of fission gas bubbles

Blister Testing

- In the typical blister test, the sample plate is held at a specified temperature for 30–60 min during each annealing step
- The temperature at which blisters form is termed the 'blister temperature'
- Images of a U3Si2—Al dispersion fuel plate after a post-irradiation blister test at 450C



(a)



USi Blistering

- For typical fuel particle loadings, miniature scale plates of U3Si2 and U3Si were blistered in the range of 515–530C
- An increase in fuel loading dropped the blister temperature by about 75C
- When boron is added, the blister threshold temperature decreases by about 100C, similar to what is observed in UAIx fuels

- Boron is added as a burnable absorber in certain research reactor fuel designs as a means of flux/temperature balancing to avoid hot spots
- The blister threshold temperature for U-Si intermetallic dispersion fuels is less sensitive both to burnup and to fuel volume loading than UAIx dispersion fuels

USi Summary

- U3Si2 is presently considered the best qualified fuel in terms of uranium loading and performance for research and test reactors
- U3Si is unsuitable for a plate-type geometry because of unstable swelling, it is still considered for fuel rods
- The ILs in U3Si2/Al are free of porosity formation at reasonable burnups and the IL growth is considered reasonably slow

- Both U3Si and U3Si2 are amorphized under irradiation, but have inherent differences in fission gas bubble growth
- U3Si has excessive breakaway swelling, whereas U3Si2 has stable, albeit still large, swelling

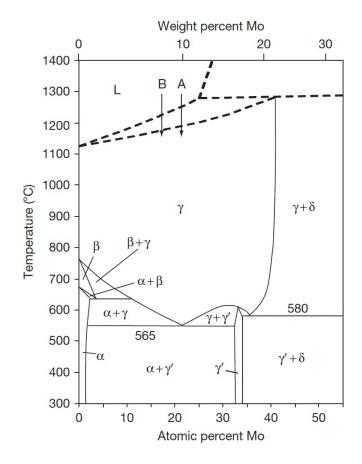
RERTR to U-Mo

- Failure to convert high-power research reactors using HEU to LEU U3Si2 resulted from the need for fuels of even higher uranium density
- The fuel development effort has shifted to uranium—molybdenum alloys with Mo content ranging 6— 10wt%, in both monolithic and dispersion fuel forms
- Since 1997, the U–Mo alloys have been irradiation-tested, driven by US leadership

- This program is now called the United States High Performance Research Reactor (USHPRR) Program
- Similar programs work in conjunction with the USHPRR in Argentina, Canada, France, and South Korea (used to also work with Russia)

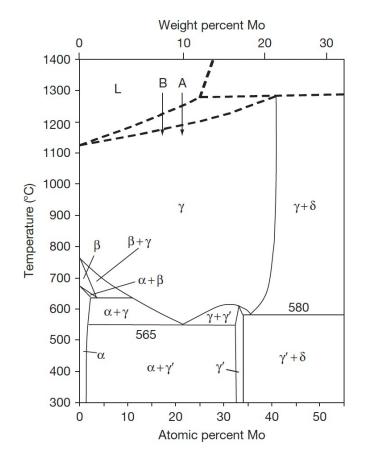
U-Mo

- It was recognized early on in the development of fast reactor fuels that molybdenum is one of the strongest gamma-stabilizers of the transition-metal elements, even stronger than Zr, and that it enables alloys with U to have relatively high U density
- A disadvantage of Mo as an alloying element is that it has higher neutron absorption cross sections than Si and Al, but not sufficiently significant to be problematic
- The solubility of Mo extends to 22 wt% (or 41 at.%) in the gamma-phase, but it is limited to a few percent in the alpha and beta-phases.



UMo Phases

- The gamma-phase undergoes a eutectoidal decomposition at 565C, transforming to the dual-phase mixture of the orthorhombic alpha-phase and the ordered tetragonal gamma'-phase which has the nominal stoichiometry of U2Mo
- This transformation is slow when the molybdenum content is more than about 6 wt%, so a gamma-phase metastable U–Mo alloy with 6–12 wt% Mo can be obtained by quenching the alloy melt into the gammaphase



Gamma Stability

- The radiation stability of U-Mo depends on its ability to retain the gamma phase
- At research reactor temperatures, the gamma phase wants to transform into the alpha/gamma' two phase system
- Radiation counteracts the driving thermodynamics by disordering the gamma' phase, retaining the gamma phase

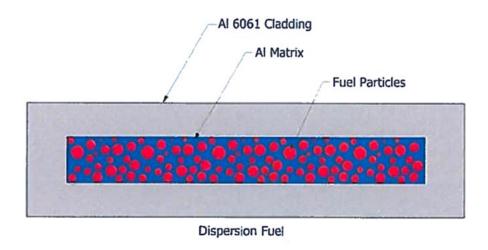
• The critical fission rate is the rate at which the minimum number of displacements that maintain the gamma-phase are in balance with the thermodynamic tendency to transform to the alpha/gamma'-phases

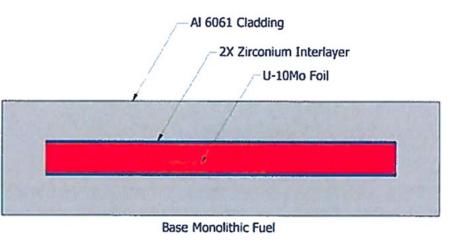
Temperature (K)	Critical fission rate $(m^{-3} s^{-1})$
644 658 672 686	8.8×10^{17} 2.2×10^{18} 4.8×10^{18} 9.2×10^{18}

Two plate designs

 Typical dispersion type fuel, U-Mo fuel particles dispersed in an Al matrix, Al cladding

 Monolithic fuel foil, Zr interdiffusion barrier, Al cladding





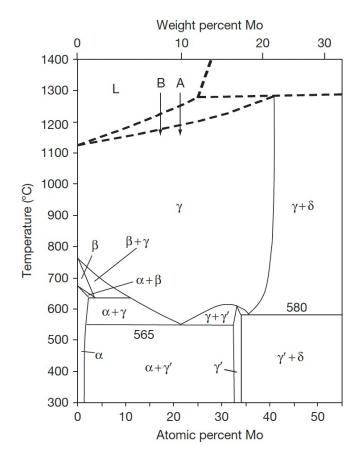
U-Mo Fabrication

- Dispersion fabrication is unique from monolithic fabrication
- U-Mo dispersion fuels are fabricated in a similar manner to U-Si dispersion fuels
- Both fragmentation and atomization processes can be applied
- The U-Mo alloy is quite ductile, which poses problems for the fragmentation process
- Alloys can be lightly oxidized to assist in fragmentation

- The fragmented powders have more equiaxially shaped grains and a more homogeneous distribution of grains than the atomized powder fuel because there is no thermal process involved during fabrication
- However, they are heavily coldworked and contain a high concentration of dislocations which can polygonize and serve as nucleation sites for gas bubbles

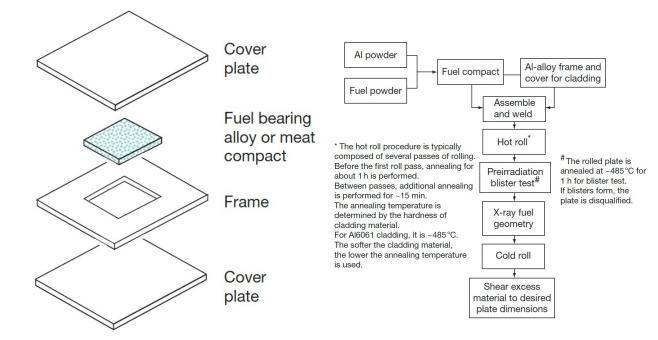
U-Mo Fabrication

- The microstructure of the atomized powder consists of a 'cellular' solidification structure which is commonly found in rapidly cooled alloys that have a pronounced solidus—liquidus gap
- Mo-rich phases solidify first upon cooling
- As the cooling progresses, the solid phase volume increases, while, simultaneously, the Mo content in the solid phase decreases
- If cooling is too rapid, Mo rich islands will form inside a network of a Mo lean matrix



U-Mo Fabrication

- The U–Mo fuel uses the same plate fabrication method as other U intermetallic fuels
- However, the hot rolling procedure can have significant effects on the performance of the fuel
- The thermal process changes particle characteristics and can enhance the interaction between the particles and the matrix



Fuel Swelling

 Fuel swelling by solid fission products is also applicable for U-Mo fuels

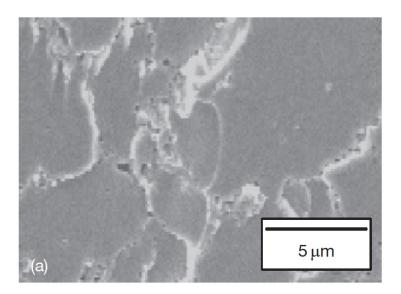
$$\left(\frac{\Delta V}{V_0}\right)_{\rm s} = 4.0 f_{\rm d}$$

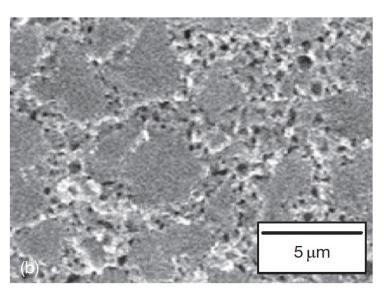
- The fuel swelling due to fission gases is unique in the UMo system
- U-Mo swelling, specifically, swelling by gas bubble growth, is known to have two distinct rates: slow at low burnup and much faster at high burnup

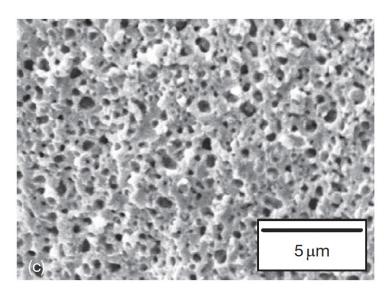
- The phenomenon underlying the transition is grain refinement or 'recrystallization' of the gamma-phase U-Mo
- After this transition, gas bubble agglomeration accelerates, resulting in faster swelling

Recrystallization + Swelling

- The evolution of fuel microstructure by fission gas bubble formation and growth is shown with three different burnups
- In an SEM, fission gas bubbles first appear along grain boundaries, with no large bubbles in the fuel





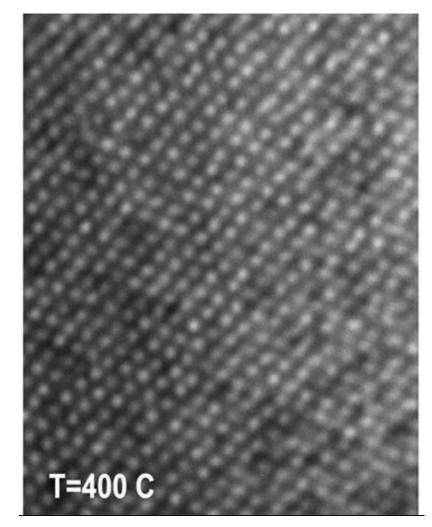


35% BU 65% BU 80% BU ₁

Fission Gas Superlattice

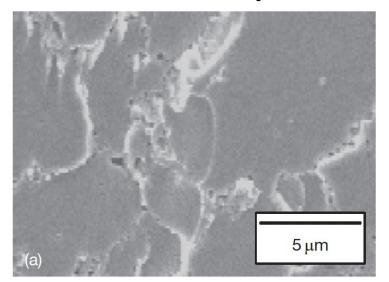
- TEM analysis has identified in the low BU regime a fission gas superlattice of 2nm bubbles
- As the bubbles are small, even though their number density is large, these bubbles are too small to produce much fuel volume increase
- The Young-Laplace equation denotes a force balance, and the ideal gas law gives an approximation of the pressure
- Thus, these small bubbles are highly pressurized and contain large amounts of gas

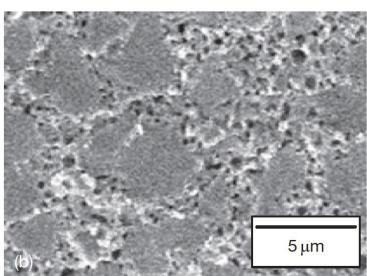
$$\Delta p = rac{2\gamma}{R}$$
 . $PV = nRT$

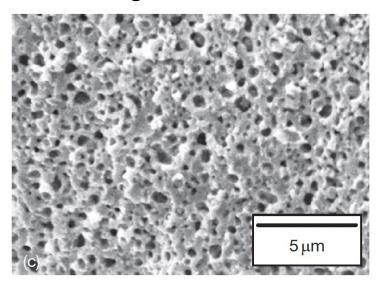


Recrystallization + Swelling

- As burnup increases (2.5–3.5E27 fissions/m3), the bubble population increases in the grain boundaries and additional bubbles progressively appear at newly formed grain boundaries as grain refinement proceeds
- At this stage, the average bubble size also increases with fission density as the number density increases, both of which increase the fuel swelling rate



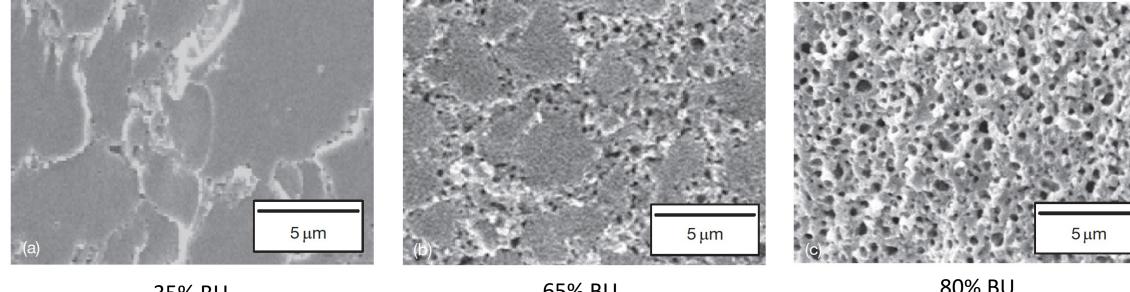




35% BU 65% BU 80% BU ₁

Recrystallization + Swelling

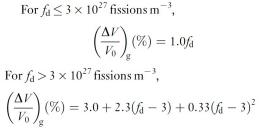
At higher burnup, large bubbles uniformly span the entire fuel cross section as the grain refinement is nearing completion

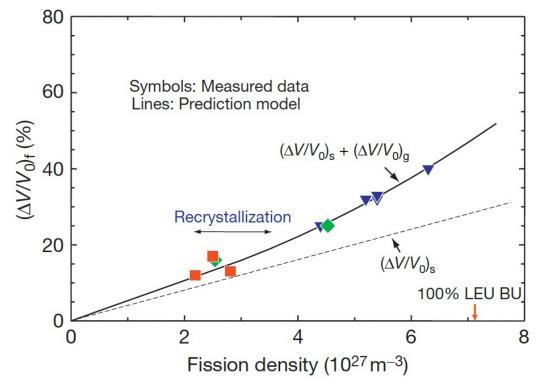


80% BU 65% BU 35% BU 20

U-Mo Gaseous Swelling

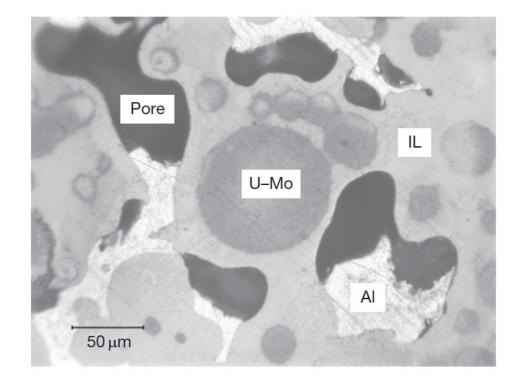
- Total swelling of U–Mo fuel is obtained from plate thickness changes before and after irradiation
- The gas bubble swelling is obtained by subtracting the solid fission product swelling
- The data can be fit to a linear function at low fission density, and a quadratic function at higher fission density





UMo-Al Interaction

- The interaction layer (IL) formation between U–Mo fuel particles and matrix AI poses potential fuel failure risks
- Pores tend to form in thick ILs
- Variable composition of the IL is possible because of its amorphous nature during irradiation
- Amorphization is usually accompanied by an increase in volume that facilitates atomic mobility, enhancing diffusion



UMo-Al Interaction

 The IL growth correlation follows a parabolic law, dependent upon the fission rate, temperature, and time

$$Y^2 = Af_{\rm r}^{0.5} t \, \exp\left(-\frac{q}{T}\right)$$

- The Al/(U+Mo) ratio of the interaction product in the (U–Mo)–Al dispersion from out-of-pile tests is in the range 3.5–7.6
- Based upon our knowledge of the U-Al system, the Al/U ratio is 2, 3 or 4

- Thus, Mo is facilitating the formation of higher AI content compounds
- As the Mo content increase, , the formation of (U,Mo)Al3, is suppressed, and the Al content in the IL increases
- The formation of high-Al content IL is unfavorable because: 1) its lower density, leading to more swelling; and 2) high Al content linked to IL pore development

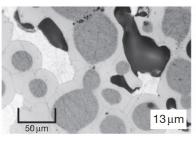
Alloying Additions

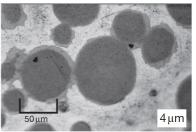
- A method to suppress the formation of UAI4 in the U-AI system depends on finding an element to suppress the peritectoid reaction UAI3 + AI ->UAI4
- There are a number of additives
 which have been identified to inhibit
 this reaction, most notably Ge, Si, Sn,
 and Zr
- In the US, Si was primarily investigated in UMo dispersion fuels to inhibit Al4 formation and suppress IL growth

- Si dramatically reduces the IL thickness
- The ILs of Si-added plates are commonly uneven, with different thicknesses on different particles
- This is due to the non-uniform concentration of Si
- At higher burnups, the IL continues to grow and the effect of Si is weakened

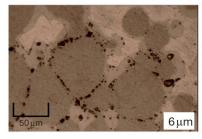
Addition of Si

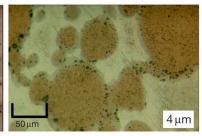
U-7Mo/Al-0.2Si R5R020 (C5) BU = 60% Time = 135 EFPD RERTR-6





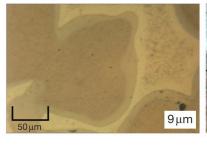
U-7Mo/Al-2Si R2R010 (C3) BU=57% Time=135 EFPD RERTR-6 U-7Mo/Al-2Si R2R078 (C3) BU = 78% Time = 98 EFPD RERTR-9A

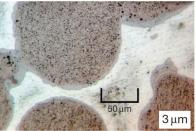




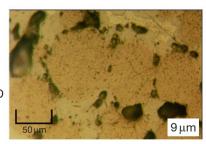
U-7Mo/AI-4.8Si R3R108 (C2) BU = 78% Time = 98 EFPD RERTR-9A

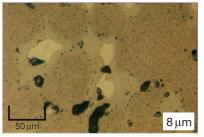
U-7Mo/AI-0.1Si R0R010 (B3) BU = 86% Time = 90 EFPD RERTR-7





U-7Mo/Al-2Si R2R040 (B2) BU = 86% Time = 90 EFPD RERTR-7 U-7Mo/Al-2Si R2R088 (B6) BU = 119% Time = 115 EFPD RERTR-9B





U-7Mo/Al-3.5Si R6R018 (B7) BU = 119% Time = 115 EFPD RERTR-9B

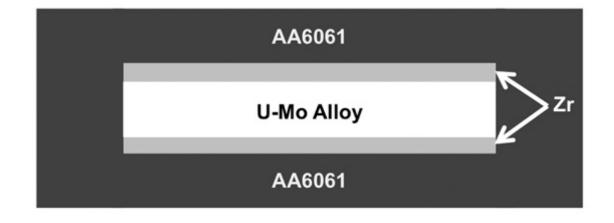
U-Mo Dispersion to Monolithic

- Higher power, high-burnup testing of aluminum matrix U-Mo dispersion fuel revealed a pattern of breakaway swelling behavior at intermediate burnup
- Breakaway swelling is defined by a rapid or unpredictable transition to high swelling behavior
- This swelling behavior was due to interaction of the fuel particles with the Al matrix, forming (U,Mo)Al compounds

- Large fission gas bubbles can form in these phases, weakening the fuel and exerting gas pressure, potentially resulting in fuel failure
- To remove this (U,Mo)Al phase, we can: 1) coat the particles; 2) modify chemistry (such as by adding Si); 3) substitute the Al matrix with another materials; 4) remove the matrix

Monolithic Fuels

- The U–Mo monolithic fuel, in which a U–Mo thin foil is sandwiched between a Zr liner and directly bonded to cladding, is currently under development
- This fuel design has the advantage of providing higher U density than the dispersion form while essentially eliminating the problem related to reaction products between the fuel and matrix
- Si additives were tried, but led to delamination of the fuel from cladding



This image is not to scale

Foil/Plate Fabrication

- Any new nuclear fuel first requires the development of conceptual fabrication technology to prepare samples for irradiation testing
- Monolithic fuel is fabricated, in general by (1) alloying and casting, (2) foil rolling, 3) bonding of the cladding, and (4) finishing and inspection for quality assurance
- Fabrication processes are still being refined

- Alloying is typically performed through vacuum induction melting (VIM) or arc melting of uranium and molybdenum metals at a temperature of 1300 1500C.
- The molten alloy is typically cast into a coated graphite book mold
- The casting is surface machined into a 'coupon' of the appropriate size for reduction into a foil though hot and cold rolling

Foil/Plate Fabrication

- Foil fabrication involves reducing the thickness of a cast alloy coupon to the desired fuel foil thickness while simultaneously bonding the Zr barrier layer to the surface of the U-Mo
- Hot rolling operations are conducted with the alloy temporarily encapsulated in a steel rolling can
- This allows the material to be heated to 650C in air without oxidizing the fuel alloy and the Zr

- After decanning, the resulting U-10Mo fuel foil, bonded to two thin (0.025 mm) layers of zirconium, sandwiched between two sheets of aluminum and placed in a steel can for HIP (Hot Isostatic Press) processing
- The HIP process is typically conducted at a temperature of 560C for a time of 90 minutes or longer
- The HIP can is cut open; the bonded plates are removed, and the plates are finished to final dimensions and inspected