NE 533

Nuclear Frel Performance

5/1/2 2000

Exam }

a)
$$t^*(A): 6.63 \times 10^{-7} \exp\left(\frac{11949}{7}\right) = 133 \text{ Ay}$$

$$\delta^*(\mu m): 5.(\exp\left(\frac{-550}{7}\right) = 2.115 \text{ mm}$$

$$\delta(\mu m): \delta^* + K_L(t - t^*)$$

$$K_L f_{\mu m/J}): 7.48 \times 10^{-6} \exp\left(\frac{-1250}{7}\right) = 0.0154$$

$$\delta: 2.115 \times 0.0154 \left(\frac{400 - 137}{7}\right) = 6.23 \text{ mm}$$

$$= \frac{2(0.18)(0.33)(5.68)(0.16)}{(5.6.6)(5.68)(0.16)} \times (66)$$

$$f_{30}^{\circ} = \frac{32}{91 + 12} = 0.36$$

C+ = 64 w+ ppm

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a) Ein = Ein + Ea + Eser + EGEP
   f = 3.5x1013 f T=1200R Tref= 300K
                                         t=851y
   De.: 0.01 Bo: 5 mus/kgy e (40,): (0.97 }/a an: 11x10 /k
  1 = 4t Nu: 10.97 1/20 1 100 1 100 = 2.45 ×102 1/cc
 B = (3.5×1013)(85×24×3400)
= 0.0105 FIMA
  Bo = Smuy 450 = 0.0053 FIMA
 Etn = DT = (11x10-4) (1200-300) = 0.0099
  €0: B> P0 → €0: D60: -0.01
 Esea: 5.571 x10 + (5.571 x10) (10,97) (0.0105) = 0.0064
GGET: [1.94x10 ) PB (2805-7) exp (-120142 (2800-7)) exp (-17.8p B)
    GGA = (3.63 +10) (5.53 4 +10") (0.129)
      6650 : 2.59 ×10-5
 Gtot = 0.0099 - 0.01 + 0.0064 + 2-59 ×10-5
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(Etot = 0.0063)

3) total crep

$$E = (E_{GJ} \cdot E_{FJ})t$$
 $G_{m} = A_{00} MA_{0} T_{00} K CHA = 150 V/m E = 1.5 yr$
 $E_{SJ} = A_{0} (\frac{\sigma_{m}}{G})^{n} \exp(\frac{-Q}{RT})$
 $A_{0} = 3.14 \times 10^{37} J_{0} G_{0} G_{0}$

Etot: É ... t = (0.013/

- 4) soluble oxides, exide precipitates, metallic precipitates, volatile gases, noble gases
- Instead of utilizing burnup -bused empirical models, utilize the underlying microstructure, state variable, propurty relationships to returning behavior/evolution.

Does not depend a the experimental envelope of data. Allow, for more descriptor and predictive behavior of nuclear fuel performance.

- b) neutron transparent

 good competibility of coolant and fael

 cheep and ency to fabricate

 etc.
 - present under a temperature gratient in metallic fulls leading & different concentrations radially. Additionally, 2r diffuses up the temperature gratient & Soret diffusion
 - 8) Mox fuel contary Pu, lending to higher fission rates.

 Mox fuel is openhed at higher powers, lending to higher temperatures. Mox fuel is utilized in SFRI with a sodium wolant. Mox undergoes recenteration and four Zones with a central word. Mox is fuericated up a hypostoichio metric composition. Mox is openhed to very high barrap.

- 9) Sacrepaible material all for allogs we susceptible to SCC
 - corresine environment uslatile filling products are generated and diffuse to the feel/chal interface, participating in constion safficient stress PCMI and the internal pressure of the cladding exert a stress that is lower than by safficient time cladding in LURS operates for up to dyears, providing ample time for diffusion of FPS to face/clad interface and stress built up
 - 10) Temper-ture ramps or transients can increme the pressure of fission are bubbles, potentially lending to fracture. Phase-field modeling is tageting fracture stresses dependent upon the bubble pressures and morphologies to predict pulsui zation.
 - 11) RIA occurs very quickly and a bock occurs over a longer time. RIA can indec rapid (welling, pulvoi zutim, cludding, burit and fuel dispusit, LOCA is associated with large plastic deformations and break-way oxidation.

 Control rod ejection accident (CREA) is a hype of RIA.

- improved social-tion resistance (a)

- improved fact projection (b)

- improved Eladding projection (c)

- improved Gissum product retention (d)

FeCrAl cladding (a, 5)

FeCral cladding (a, 5)

(r-arped (b, d) etc.

Ussing (a, c)

Sic cladding (a, c)

13) pcm1, elading elogation /assembly bow, clading wear, oxidation, power to meet, intend cladding pressure, DNBR, exerctor limits