

### Question 1:

a) What is the fissile isotope in  $\text{U}_3\text{Si}_5$ ? What would be the enrichment of this isotope in the natural (unenriched) form of the fuel? (7 points)

b) What enrichment would be required for  $\text{U}_3\text{Si}_5$  to have the same energy release rate of  $\text{U}_3\text{Si}_2$  enriched to 3% with a neutron flux of  $3.2 \times 10^{13} \text{ n}/(\text{cm}^2 \text{ s})$ ? You can assume that  $\text{U}_{235}$  has a negligible impact on the total molar mass of U in the fuel (15 points)

c) How would you rank  $\text{U}_3\text{Si}_5$  as a potential fuel compared to  $\text{U}_3\text{Si}_2$ ? Why? (8 points)

### Question 2:

Consider a fuel rod with a pellet radius of 4.5 mm, an 80 micron gap, and a zircaloy cladding thickness of 0.6 mm. It is experiencing a linear heat rate of 250 W/cm with a coolant temperature of 580 K. The gap is filled with He and 5% Xe and the coolant conductance is 2.5 W/(cm<sup>2</sup> K).

- a) What is the surface temperature of the fuel rod? (15 points)
- b) Assume the pellet is made from Uranium Nitride. What is the maximum stress experienced by the pellet, given that uranium nitride has  $E = 246.7$  GPa,  $\nu = 0.25$ , and  $\alpha = 7.5 \times 10^{-6}$   $1/K$ ? (10 points)
- c) Would you expect this stress to be higher or lower if the pellet was  $UO_2$ ? Why? (5 points)
- d) What assumptions were made in your calculations for a) and b)? (5 points)

