

Question(1)

10/12 - right process

Assumptions

- a) Constant thermal conductivity $k(T) = k$ ✓
 b) Assuming steady state ✓
 c) Assuming symmetry in y, z directions ✓

$$\frac{\partial}{\partial x} \left(k \frac{\partial T}{\partial x} \right) + Q = 0 \Rightarrow \frac{\partial}{\partial x} \left(k \frac{\partial T}{\partial x} \right) = -Q$$

$$k \frac{\partial T}{\partial x} = -Qx + C_1$$

from boundary condition

$$\left. \frac{\partial T}{\partial x} \right|_{x=0} = 0 \Rightarrow 0 = 0 + C_1$$

$$\therefore C_1 = 0 \quad \checkmark$$

$$k \frac{\partial T}{\partial x} = -Qx$$

$$\therefore T(x) = -\frac{Qx^2}{2k} + C_2$$

from boundary cond/ions: $T(X) = T_1$

$$\therefore T_1 = -\frac{QX^2}{2k} + C_2 \Rightarrow C_2 = T_1 + \frac{QX^2}{2k}$$

$$\therefore T(x) = -\frac{QX^2}{2k} + T_1 - \frac{Qx^2}{2k}$$

$$T(x) = T_1 + \frac{Q}{2k} (X^2 - x^2)$$

Temp. drop
across
fuel

$$T_0 - T_F = \frac{Q}{2k} X_F^2 \quad \left(\text{when } X_F: \text{fuel thickness} \right)$$

Question (2)

14/14

1st Without Coating

$$T_{CO} = 600 K$$

$$Q = \frac{250 W}{cm^3}$$

$$LHR = \pi R_f^2 Q$$

$$= \pi (0.6)^2 \times 250$$

$$= 90 \pi \frac{W}{cm}$$

$$T_{CI} = T_{CO} + \frac{LHR}{2\pi R_f} \frac{t_{clad}}{K_{clad}}$$

$$T_{CI} = 600 + \frac{90}{2 \times 0.6} \times \frac{0.05}{0.15}$$

$$T_{CI} = 600 + 25 = 625 K$$

Gap

$$T_F = T_{CI} + \frac{90 \pi}{2\pi R_f} \frac{t_g}{K_g}$$

$$T_F = 625 + \frac{90}{2 \times 0.6} \times \frac{0.005}{0.004} = 718.75 K$$

$$T_o = T_F + \frac{QR^2}{4K_f} = 718.75 + \frac{250 \times 0.6^2}{4 \times 0.05} = \boxed{1168.75 K}$$

Question (2)

With Coating

$$T_{\text{coating out}} = 600 \text{ K}$$

$$T_{\text{CO}} = T_{\text{coating}} + \frac{q_0 \pi}{2 \pi R_F} \frac{L_{\text{coat}}}{K_{\text{coat}}} = 600 + \frac{90}{2 \times 0.6} \times \frac{0.01}{0.015}$$
$$= 650 \text{ K}$$

$$T_{\text{CI}} = 650 + \frac{90}{2 \times 0.6} \times \frac{0.05}{0.15} = 675 \text{ K}$$

$$T_F = 675 + \frac{90}{2 \times 0.6} \times \frac{0.005}{0.004} = 768.75 \text{ K}$$

$$T_0 = 768.75 + \frac{250 \times 0.6^2}{4 \times 0.05} = 1218.75 \text{ K}$$

Question (3)

13/14

$$a) \phi = 5 \times 10^{12} \frac{n}{cm^2 \cdot s}$$

$$\text{Heat gen. rate} = \frac{200 \text{ MeV}}{\text{fission}} \times N_{U-235} \times \sigma_f \times \phi$$

238 \rightarrow 237.4

$$N_{UN} = \frac{P N_{AU}}{M} = \frac{12.3 \times 6.02 \times 10^{23}}{238 + 14} = 2.94 \times 10^{22} \frac{\text{f.u.}}{cm^3}$$

$$N_U = N_{UN} = 2.94 \times 10^{22} \frac{\text{atoms}}{cm^3}$$

Approximately: 19.5% of them are U-235

$$\therefore N_{U-235} = 2.94 \times 10^{22} \times \frac{19.5}{100} = 5.7 \times 10^{21} \frac{\text{atoms}}{cm^3}$$

$$\therefore \text{Heat gen. rate} = 200 \times 10^6 \times 1.6 \times 10^{-19} J \times 5.7 \times 10^{21} \frac{cm^{-3}}{cm^{-3}} \times 570 \times 10^{-24} \frac{cm^2}{cm^2} \times 5 \times 10^{12} \left(\frac{1}{cm \cdot s}\right)$$

$$\approx 520 \frac{W}{cm^3}$$

mass $\rightarrow 238(1-x) + 235x$

(b) if UO_2 instead of UN

$$N_{U-235} = 5.7 \times 10^{21} \frac{\text{atoms}}{cm^3}$$

$$\therefore N_U = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

$$N_{U-235} \approx N_U \times \text{enrichment}$$

$$\therefore N_{UO_2} = N_U = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

$$\therefore \frac{P_{UO_2} N_{AU}}{M_{UO_2}} = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

$$\frac{10.97 \times 6.02 \times 10^{23}}{238 + 16 \times 2} = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

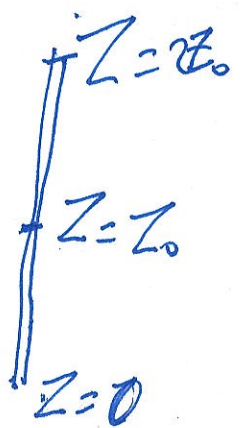
$$\therefore \text{enrichment} \approx 23.3\%$$

Question (4) $LHR^0 = 350 \frac{W}{cm}$, $\gamma = 1.3$, $Z_0 = \frac{3.5m}{2} = 1.75m$

(a) $LHR(z=1.4m) = LHR^0 \cos\left[\frac{\pi}{2\gamma}\left(\frac{z}{Z_0}-1\right)\right]$ ✓
 $= 350 \cos[1.2(0.8-1)]$ ✓
 $= 350 \times 0.97$ ✓
 $\approx \boxed{340 \frac{W}{cm}}$ ✓

(b) $C_p = 4200 \frac{J}{kg \cdot K}$, $\dot{m} = 0.22 \frac{kg}{s \cdot rod}$

$\Delta T_{\text{cod}} = \frac{1}{1.2} \frac{Z_0 LHR^0}{\dot{m} C_p} \left\{ \underset{\substack{\downarrow \\ 0.932}}{\sin(1.2)} + \underset{\substack{\downarrow \\ 0.932}}{\sin\left[1.2\left(\frac{z}{Z_0}-1\right)\right]} \right\}$ ✓
 $\left[T_{\text{out}} - T_{\text{in}} \right]$
 $= \frac{1}{1.2} \times \frac{1.75 \times 350 \times 10^2 \frac{W}{m}}{0.22 \times 4200 \frac{J}{kg \cdot K}} [0.932 + 0.932]$ ✓
 $\approx \boxed{103 K}$ ✓



Question (5)

$$dt = 0.5, \quad t_0 = 0, \quad y_0 = 4$$

$$\frac{dy}{dt} = t e^{-2t}$$

$$\cancel{y(t)} \quad y(t+dt) = y(t) + dt y'(t+dt) \quad \checkmark$$

$$\Rightarrow \boxed{y_0 = 4}$$

$$\Rightarrow y_{0.5} = 4 + 0.5 y'(0.5) = 4 + 0.5 \times [0.5 e^{-1}]$$
$$= \boxed{4.092}$$

$$\Rightarrow y_1 = 4.092 + 0.5 y'(1) = 4.092 + 0.5 [1 \cdot e^{-2}] = \boxed{4.16}$$

$$\Rightarrow y_{1.5} = 4.16 + 0.5 y'(1.5) = 4.16 + 0.5 [1.5 e^{-3}]$$
$$= \boxed{4.197}$$

Question (6)

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Fertile

Isotope that can be transformed into a fissile isotope by absorbing neutrons.

Fissile

Isotope that can undergo fission with neutrons of any energy (even thermal energies)

Fissionable

Isotope that can undergo fission only with neutrons of a minimum threshold energy

Question (7)

We don't use pure metallic U because:

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(1) ~~the~~ Phase changes

(2) swelling during thermal cycling

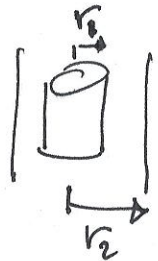
(3) Anisotropic thermal expansion & Anisotropic irradiation growth

Question (8)

Sluag density is the ratio between the fuel volume and the total internal volume of fuel element

$$\text{sluag density} = \frac{\pi r_1^2 h}{\pi r_2^2 h} = \frac{r_1^2}{r_2^2}$$

It is important to have a gap to accommodate fuel swelling & fission gas release



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Question (9)

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- ⇒ We need to enrich U to increase the percentage of U-235 isotope [which is the fissile isotope] ✓
- ⇒ Natural U has only 0.7% of U-235 ✓
- ⇒ We need U-235 bec. it undergoes fission with thermal neutrons
- ⇒ In enrichment process, UF_6 is utilized ✓
(Uranium hexafluoride) gaseous form

In Centrifuge

- opposite. heavy goes to outside
lighter gas with more U-235 goes
to the outside and can be extracted

Question (10)

① Finite ~~element~~ difference : simple and fast
(using discrete points) but can't be applied for heterogeneous structure

② Finite volume :

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~~③ Finite difference~~

③ Finite element :

Question (11)

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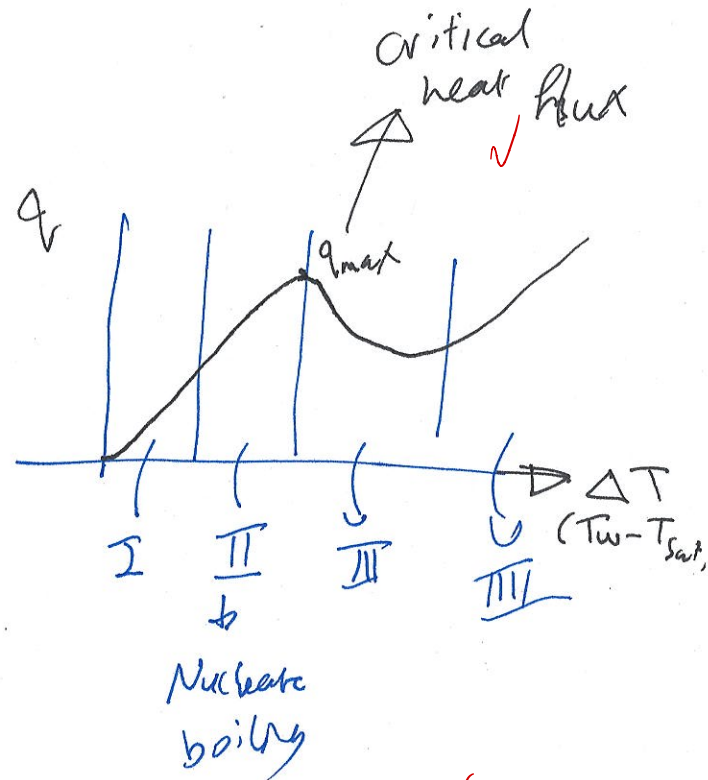
Departure from Nucleate boiling

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$$DNBR = \frac{\text{Critical heat flux}}{q(z)}$$

$$DNBR > 1$$

The margin allowed
is typically 1.15 → 1.3 ✓



Critical Heat Flux:

is the max heat flux that can be achieved before
a vapor film is created and then heat flux decreases ✓

Question (12)

TRISO :

UC or UCO ✓

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Carbide fuel

- py C layers

+ buffer

with SiC coating ✓

In

High temp. gas cooled reactor (HTGR)

