## 86/100

NucE 497 Fuel Performance Exam 1 covering modules 1 - 3

-3, 27/30Question 1:

U<sub>3</sub>Si<sub>5</sub> is a uranium silicide fuel being considered for use in light water reactors. It has a thermal conductivity of 12.5 W/(m K) and a density of Uranium metal of 7.5 g of U/cm<sup>3</sup>. Answer the following questions

a) What is the fissile isotope in U<sub>3</sub>Si<sub>5</sub>? What would be the enrichment of this isotope in the natural (unenriched) form of the fuel? (7 points)

U235, Natural Wranium is 0.7% U235 fissile isotope

b) What enrichment would be required for U<sub>3</sub>Si<sub>5</sub> to have the same energy release rate of U<sub>3</sub>Si<sub>2</sub> enriched to 3% with a neutron flux of 3.2e13 n/(cm<sup>2</sup> s)? You can assume that U235 has a negligible impact on the total molar mass of U in the fuel (15 points)

Pu for U3Siz is = 11.319/cm3 [Mu = ws (Mas) + (1-wss) (Mse) = 0.03 (235) + (1-0.03) (238) = 237,913/mm Q= Ef Nf 0f 0f (3×10" fission) (0.03) (11.31) (0.022×1023 (55×10-22 cm²) (3.2×103 h) Q=453.47 7/m3 Assume: 0735= 5,5 × 10-22 cm2

Assume Mu 2 237.919/mol => rearrange Q Equation: (W/0) = Q (MW) [FEX(PUSSS) (Of 35) (Q+W) (NA)]  $= \frac{(453.47)(237.91)}{(3E-11)(7.5)(5.5E-22)(3.2E+13)(6.022E+23)} = 0.045 \Rightarrow 4.5\%$ enrich

> (c)) How would you rank U3Si5 as a potential fuel compared to U3Si2? Why? (8 points)

Not as feasible because it requires higher enrichment for same energy output because it has less pur .

higher enrichment not preferred due to cost of fabrication (enrichment) + proliferation (weapons) concern.

Question 2:

Consider a fuel rod with a pellet radius of 4.5 mm, an 80 micron gap. and a zircaloy cladding thickness of 0.6 mm. It is experiencing a linear heat rate of 250 W/cm with a coolant temperature of 580 K. The gap is filled with He and 5% Xe and the coolant conductance is  $2.5 \text{ W/(cm}^2 \text{ K)}$ .

a) What is the surface temperature of the fuel rod? (15 points)

SEE ATTACHED P. O. For WORK 957.91K

b) Assume the pellet is made from Uranium Nitride. What is the maximum stress experienced by the pellet, given that uranium nitride has E = 246.7 GPa, v = 0.25, and  $\alpha = 7.5e-6 1/K$ ? (10 points)

SEE ATTACHED P. D. Por Work
-0.0613GPa

c) Would you expect this stress to be higher or lower if the pellet was UO<sub>2</sub>? Why? (5 points) WOZ X = 1,10 E-5 K-1 -2, Lower thermal conductivity and thus higher DT is a much · O\* for 402 would be larger VS. UN X = 7.5 E-6 K-1 larger effect · > DAD = - 0\* (at r=0) =) this would be a HIGHER COMPRESSIVE STRESS (MORE NECHATIVE) for => DOD MORE NECT > more compressive UDz has a LARGER Thermal expansion well. d) What assumptions were made in your calculations for a) and b)? (5 points) · axisymmetric body (smull strains)
· aylinder w / thermal expansion but FOR b: no pressure realitable, Isotropic material response For a: Assume Zirc clad Kchd=0.17 W/LMK, UN Repuel = 0.2 W/Kmk \* Steady State, Axisymmetric behavior,

Thermal conductivity independent of Temp.

Temp constant in Z direction (axial),

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Question 2!
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P. UZ

Rf= 0.45cm tgap= 0.008 cm tolad= 0.06cm

Assume Zirc cladding:

Kelad=0.17 VcmK

LHR = 250 W/cm

TC001 = 580 K

hcool= 2.5 W/cm2 K

Chap: 5% Xe

OUTER CLAD TEMP: To = LHR + TCOOL = 250 201615,37 K

INNER CLAD TEMP! TC1 = LHIZ. + Clad + TC0 = 250 (0.06) 27 (0.45) (0.17) + 615.37 = 646.58 K

GIAP Properties' KHE = (16E-6) (TC179) = 0.0027 Wank

Kxe = (0.7E-6)(Tc10.79) = 1.16 E-4 W/cmk

Kgap = KHE Kxe = 0.00 227 W/cmk 4=0.05

hgap = Vgap = 0.284 W/cm2K

FUEL SURF TEMP: TES = LHR ZAMRENGAD +Te1 = 250 + 646.58=957.91K

3) UN Max stress @ center of pellet + noop stress E = 246.7 GPa far UN: Kfuel = 0.2 ycmr V=0,25

Q=7.5E-6 K-1

 $\Rightarrow \mathcal{N} = \frac{r}{R_0} \Rightarrow r = 0 \text{ for may stress } \Rightarrow \mathcal{N} = 0$ 

 $T^* = \frac{KF(T_{CL} - T_{FS})}{4(1-U)} = \frac{(7.5E-6 / K)(246.7 GPa)(1057.38 - 957.91)K}{4(1-0.25)}$ 

TCL THR +TAS = 250 + 957.91 = 1057.38K 0 = 0.061348

= 0.0613 CIPA = -61.3 MPa

Question 3: -7,28/35

Consider the stress state in a zircaloy fuel rod pressurized to 6 MPa with an average radius of 5.6 mm and a cladding thickness of 0.6 mm.

a) What assumptions are made in the thin walled cylinder approximation for the stress state? (5 points)

· Static Body · Gravity is Neulinide · Axisy mmetric

-3, constant stress across radius

· Isotropic Material Response

b) Calculate all three components of the stress using the thin walled cylinder approximation. (10 points)

SEE P.Q3 for Work

0= 56MPa 0= 28MPa OF = -3 MPa

c) Quantify how accurate the thin walled cylinder approximation is for the cladding. Would the thin walled cylinder approximation be conservative if used to estimate if the cladding would fail? (10 points)

SEE P. Q3 for WORK

using thick walled approx, max stress (moop@inner radius)

000 = 53.16 MPa

Orr = - 6 MPa

522 = 23,58MPA

-4, Calculate stress and multiple radii to check approximation

=) Since the thin wall approx estimates lureur 500 it is more conservative.

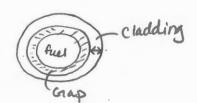
d) Write the stress and strain tensors for the stress state in the thin walled cylinder, with E = 70 GPa and v = 0.41. (10 points)

SEE P. Q3

Question 3

P. QS

auci. radius: R=5.6mm toud: Su = 0.6mm



c) USING Thick Walled Approx:

Ro=Ronzer 5.6mm, Ri=Rinner = Ro-S=5mm, S=D.6mm

$$\begin{array}{c} \mathcal{E}_{zz} = 8.95 \, \text{E} - 5 \\ = \begin{array}{c} \mathcal{E}_{rr} & 0 & 0 \\ 0 & \mathcal{E}_{zz} \end{array} \end{array}$$

L values from d Assume no shear