K=12.5 W/m-K

P. = 7.5 9/103

## 88/100

NucE 497 Fuel Performance Exam 1 covering modules 1 - 3

-3, 27/30

## Question 1:

U<sub>3</sub>Si<sub>5</sub> is a uranium silicide fuel being considered for use in light water reactors. It has a thermal conductivity of 12.5 W/(m K) and a density of Uranium metal of 7.5 g of U/cm<sup>3</sup>. Answer the following questions

What is the fissile isotope in U<sub>3</sub>Si<sub>5</sub>? What would be the enrichment of this isotope in the natural (unenriched) form of the fuel? (7 points)

-Uranium - 235 is the fissile isotope -0.740 ensippel

b) What enrichment would be required for U<sub>3</sub>Si<sub>5</sub> to have the same energy release rate of U<sub>3</sub>Si<sub>2</sub> enriched to 3% with a neutron flux of 3.2e13 n/(cm<sup>2</sup> s)? You can assume that U<sub>235</sub> has a negligible impact on the total molar mass of U

in the fuel (15 points) - U3512→340; Ø=3.2·1013 # 1/NU235 = BNap 1

4=0.03; Na =6.022·10<sup>23</sup>; P=11.31; Mu=0.03(235)+(1-0.03)238=237.91

L Q = EfNOT Win - to compare the a must be the

some, Et, OI Wan are the some so:

 $(N_{F})_{U_{35;}} = (N_{F})_{U_{35;1}} = (0.03)(6.0)(1.02.10^{23})(11.31) - \frac{1}{9} \cdot 6.026.10^{23} \cdot 7.5$  (50 me due to assumption)

0.03(11,31)=41(7.5)

 $g_{U} = 4.524$ ° for  $U_{3}S_{15}$ c) How would you rank  $U_{3}S_{15}$  as a potential fuel compared to  $U_{3}S_{12}$ ? Why? (8)

points)

I would rank Ussis as a wolse fuel soley based of the fact that it has lower density compared t U351, -owhich means less

-3, Thermal conductivity?

11-235 in the same unit of valume thus lose fission

Rf = D. 45cm tc = 0.06 cm : LHR = 250 W/cm ta=0.008cm Tac1 = 580K Question 2: 5 % Ke, 95% He; had = 2,5 W/cm2/ Consider a fuel rod with a pellet radius of 4.5 mm, an 80 micron gap, and a zircaloy cladding thickness of 0.6 mm. It is experiencing a linear heat rate of 250 W/cm with a coolant temperature of 580 K. The gap is filled with He and 5% Xe and the coolant conductance is 2.5 W/(cm<sup>2</sup> K). a) What is the surface temperature of the fuel rod? (15 points)  $\frac{LHR}{2\pi R_c hm_l} = \frac{250}{2\pi (0.45)(25)} + 580 = 35.37 + 580 = 1615.37 K = Teo$ · TIC = TCO + LHRTE = 250 (0.06) · TS = TZ( + ZTR+ Kgap) - Koop = [16.10-6(64656)] [0.70-6(64656)] [0.70-6(64656)] = 0.00 DZZ78,  $T_s = 646.56 + \frac{250 \cdot 0.08}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = 3 \pm 1.235 + 646.56 = \frac{957.795}{100.05 \cdot 0.2} = \frac{957.795}{10$ stress experienced by the pellet, given that uranium nitride has E = 246,7 GPa, v = 0.25, and  $\alpha = 7.5e-6 1/K?$  (10 points) F = 246. 7 CPA - mox 6 Hess is hoop stess: Toa = 0 + (1-3 32) · 0 = x = (To-Ts); To = 250 +957,785 = 1057, 25129 K X=7.5e-6 J 60 = 7.5.106.0 \$4.5.103 [99.522] = 61.38 MPa Too = U (1-3(1)2) = -61,38 MPa. 2 atatside c) Would you expect this stress to be higher or lower if the pellet was  $UO_2$ ? Why? (5 points) For UD, I would expect a hisher stress Kun= 7,5.106 because dislarger, the DTFuel is larger, and so ot dus = 11.10% d) What assumptions were made in your calculations for a) and b)? (5 points) -3, Primarily due to lower thermal conductivity and thus higher DT In colculations a: · Steady state · Oxissymmetric behavior (1,2) ·Tis constant in Z InportBI . static Gody · thermal conductivity is constant · growity is negligible ·Oxis symmetric isotrupic material response - And some assumptions

## Question 3:

Consider the stress state in a zircaloy fuel rod pressurized to 6 MPa with an average radius of 5.6 mm and a cladding thickness of 0.6 mm.

a) What assumptions are made in the thin walled cylinder approximation for the stress state? (5 points)

Thinwolled assumption is stress is can stant through wall of cylinder, because it is thin -2, isotropic and small strain

b) Calculate all three components of the stress using the thin walled cylinder

c) Quantify how accurate the thin walled cylinder approximation is for the cladding. Would the thin walled cylinder approximation be conservative if used to estimate if the cladding would fail? (10 points)

 $= 6 \left[ \frac{(5.95.6)^{2+1}}{(5.95.6)^{2+1}} \right] = 115 \text{ M/A} > 56 \text{ M/A} -4, \text{ Calculate stress at two radii and compare thin d)}$ d) Write the stress and strain tensors for the stress state in the thin walled

cylinder, with E = 70 GPa and v = 0.41. (10 points)