

# **NE 795: Advanced Reactor Materials**

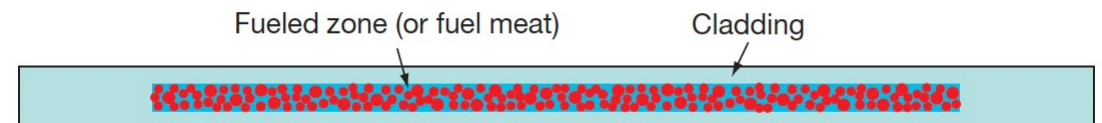
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# RESEARCH REACTORS

# Intermetallic Fuels

- Uranium intermetallic fuels such as U–Al, U–Si, and U–Mo are chiefly meant for research and test reactors in which neutron production, instead of power generation, is the main purpose
- The operation temperatures of these fuels are lower than those UO<sub>2</sub>
- In general, the U intermetallic fuels can achieve much higher fission densities than oxide fuels
- Currently available research reactor fuels are predominantly in a dispersion form that is composed of fuel particles dispersed in an inert matrix (often Al)
- The fueled zone in a dispersion fuel plate, that is, the fuel particles–matrix mixture zone, is metallurgically bonded to the Al cladding



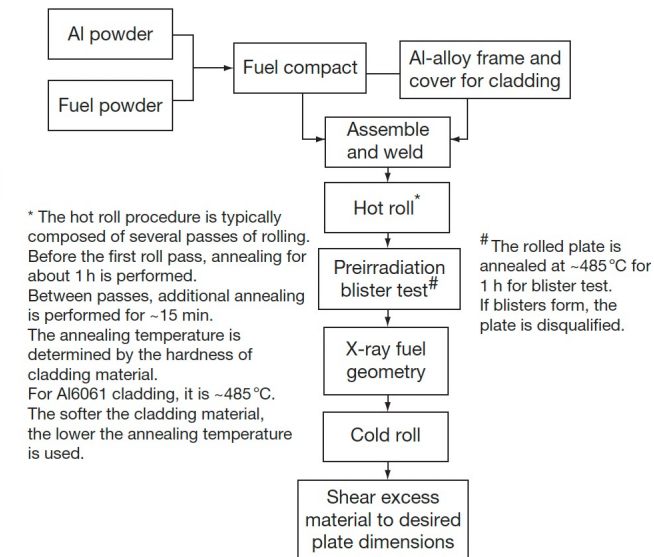
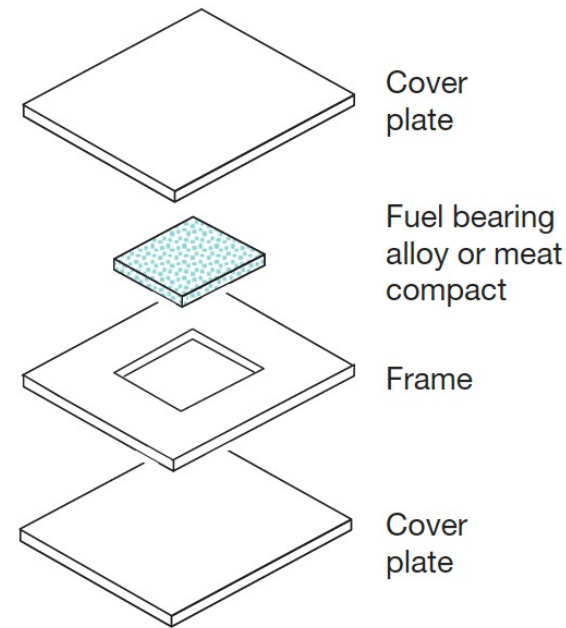
# Intermetallic Fuels

- U–Al, U–Si, and U–Mo fuels have been used in research reactors, with development from Al, to Si, to Mo driven by obtaining higher U densities
- Uranium metal is unsuitable, so intermetallics were developed to stabilize irradiation behavior
- The U-Al alloy was the first uranium intermetallic fuel chosen for research and test reactor purposes, largely because of compatibility with Al cladding

<i>Fuel</i>	<i>Melting point (°C)</i>	<i>Physical density (g cm<sup>-3</sup>)</i>	<i>Uranium loading (g cm<sup>-3</sup>)</i>
U	1133	19.1	19.1
U–7Mo	1145	18.4	17.1
U–10Mo	1150	18.2	16.4
U <sub>6</sub> Mn	726	17.8	17.1
U <sub>6</sub> Fe	815	17.7	17.0
U <sub>3</sub> Si <sup>a</sup>	930 <sup>b</sup>	15.6	15.0
U <sub>3</sub> Si <sub>2</sub> <sup>a</sup>	1665	12.2	11.3
USi	1580	10.96	9.8
UAl <sub>2</sub> <sup>a</sup>	1590	8.1	6.6
UAl <sub>3</sub> <sup>a</sup>	1350 <sup>b</sup>	6.8	5.0
UAl <sub>4</sub>	731 <sup>b</sup>	6.1	4.2
U <sub>0.9</sub> Al <sub>4</sub> <sup>a</sup>	641 <sup>b</sup>	5.7	3.7
UAl <sub>x</sub> <sup>c</sup>	NA	6.4	4.5
UC	2500	13.6	13.0
UN	2630	14.3	13.5
UO <sub>2</sub> <sup>a</sup>	2875	10.96	9.7
U <sub>3</sub> O <sub>8</sub> <sup>a</sup>	<sup>b</sup>	8.4	7.1
Al <sup>d</sup>	660	2.7	0

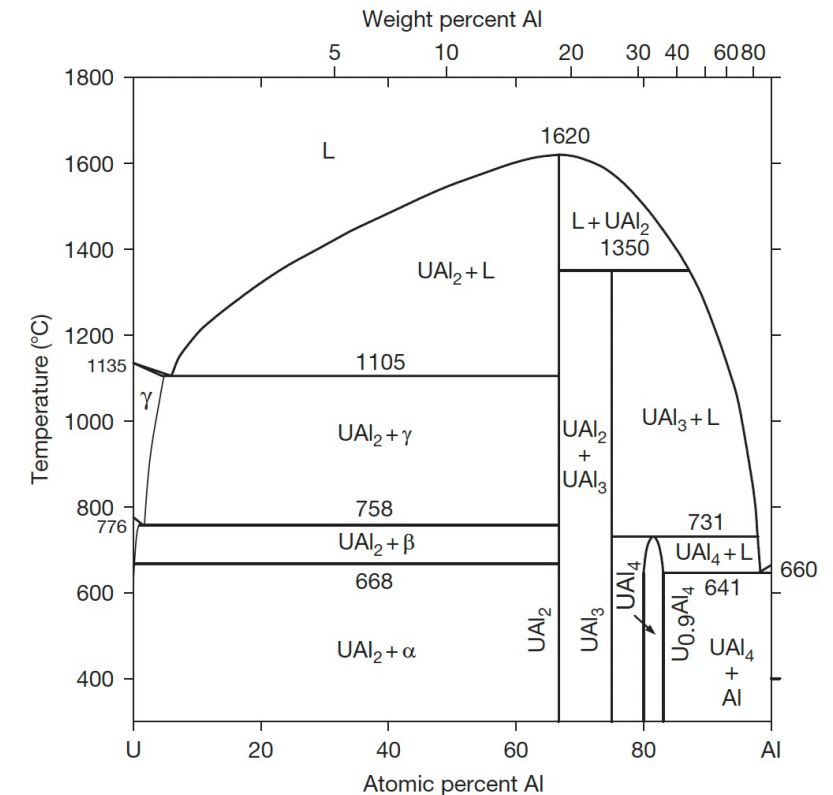
# U-Al Alloys

- U-Al was utilized as the fuel in the Materials Test Reactor (MTR; 1952-1970) and the Engineering Test Reactor (ETR: 1957-1981)
- Fabrication of U–Al alloys with high uranium contents poses difficulties during the rolling process, and uranium inhomogeneity increases proportionally with uranium content
- The application of monolithic U–Al alloy in higher power reactors was limited because of fabrication constraints and high fuel swelling



# U-Al Phases

- There are three intermetallic compounds in the U–Al system: UAl<sub>2</sub>, UAl<sub>3</sub>, and UAl<sub>4</sub>
- UAl<sub>2</sub> forms directly from the liquid, but UAl<sub>3</sub> and UAl<sub>4</sub> form by peritectoid reactions with aluminum
- UAl<sub>2</sub> is fcc, UAl<sub>3</sub> is L1<sub>2</sub> type, UAl<sub>4</sub> is bcc
- Densities range from 6.6 g/cc to 4.2 g/cc with decreasing U loading
- Thermal conductivity of dispersion fuels is largely governed by the matrix



**Table 1** The uranium weight percent, density and melting point of uranium aluminides

Compound	Wt% U	Density (g/cm <sup>3</sup> )	Uranium density (g/cm <sup>3</sup> )	Melting point (K)
UAl <sub>2</sub>	81.52	8.14	6.64	1893
UAl <sub>3</sub>	74.63	6.8	5.08	1623
UAl <sub>4</sub>	68.8	6.06–6.10	4.16	1004
U <sub>0.9</sub> Al <sub>4</sub>	64.2–66.3	5.6–5.7	3.648	

# U-Al Alloys

- The fuel form of U–Al alloy with a U density high enough to satisfy the need for high-power reactors is a mixture of UAl<sub>2</sub>, UAl<sub>3</sub>, and UAl<sub>4</sub>, known as UAl<sub>x</sub>
- UAl<sub>x</sub> has positive features that enable its superior performance in high-power reactors
- Fuel swelling can be reduced by accommodating fission product swelling in the powder dispersions, which include pores left during fabrication
- UAl<sub>x</sub> also has exceptional resistance to fission gas bubble formation
- In addition, fabrication with a uniform distribution of burnable absorbers is possible
- Typical powder lots used in the ATR contained phase fractions of 7.6 wt% UAl<sub>2</sub>, 78.6 wt% UAl<sub>3</sub>, and 13.8wt% UAl<sub>4</sub>
- These phase fractions can be modified based upon the fabrication process

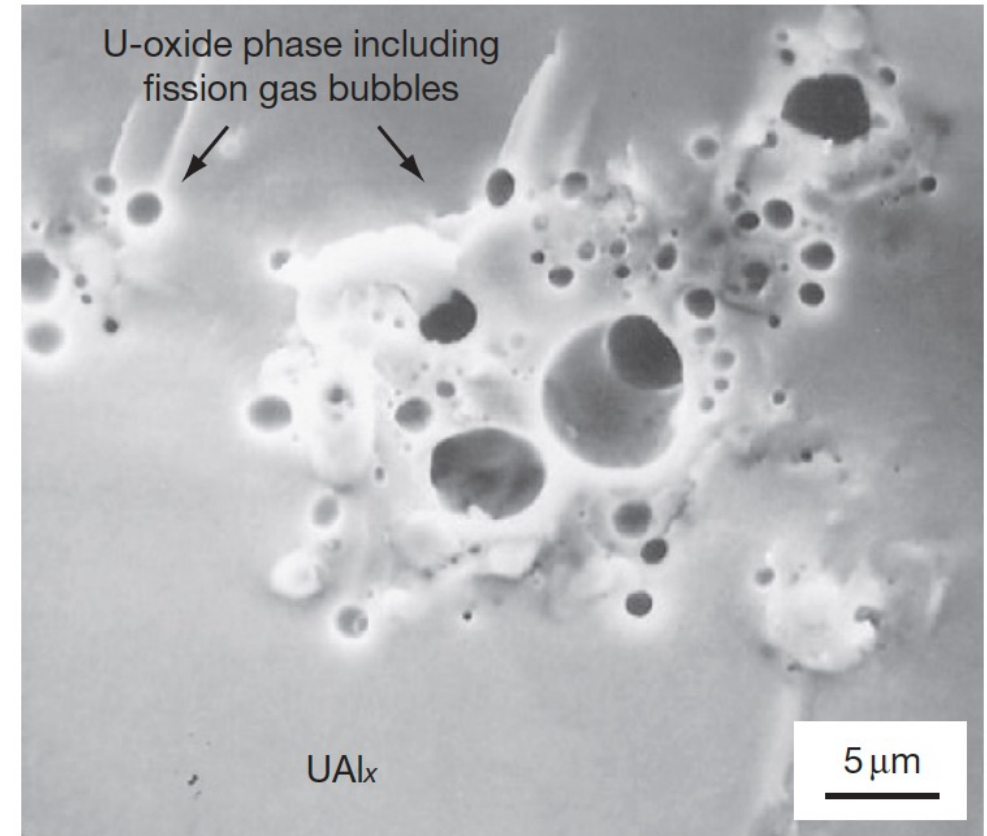
# Fuel Swelling

- Fuel swelling by fission products is divided into two distinct parts: solid and gaseous
  - Solid FP swelling is due to the difference between the volume of a uranium atom and solid fission products
  - Most fission gas atoms remain in the fuel, with solid FP swelling proportional only to burnup; independent of fabrication method, fuel type, temperature, etc.
  - Thus, solid swelling is applicable across other intermetallic fuels, with derivations from U-Zr being modified for UAl, USi, and UMo fuels
- $$\left(\frac{\Delta V}{V_0}\right)_s = 4.0f_d$$
- The solid FP swelling for UMo is given by the above equation, where  $f_d$  is fission density in  $10^{27}$  fissions/m<sup>3</sup>
  - 50% burnup is approximately  $4 \times 10^{27}$  f/m<sup>3</sup>



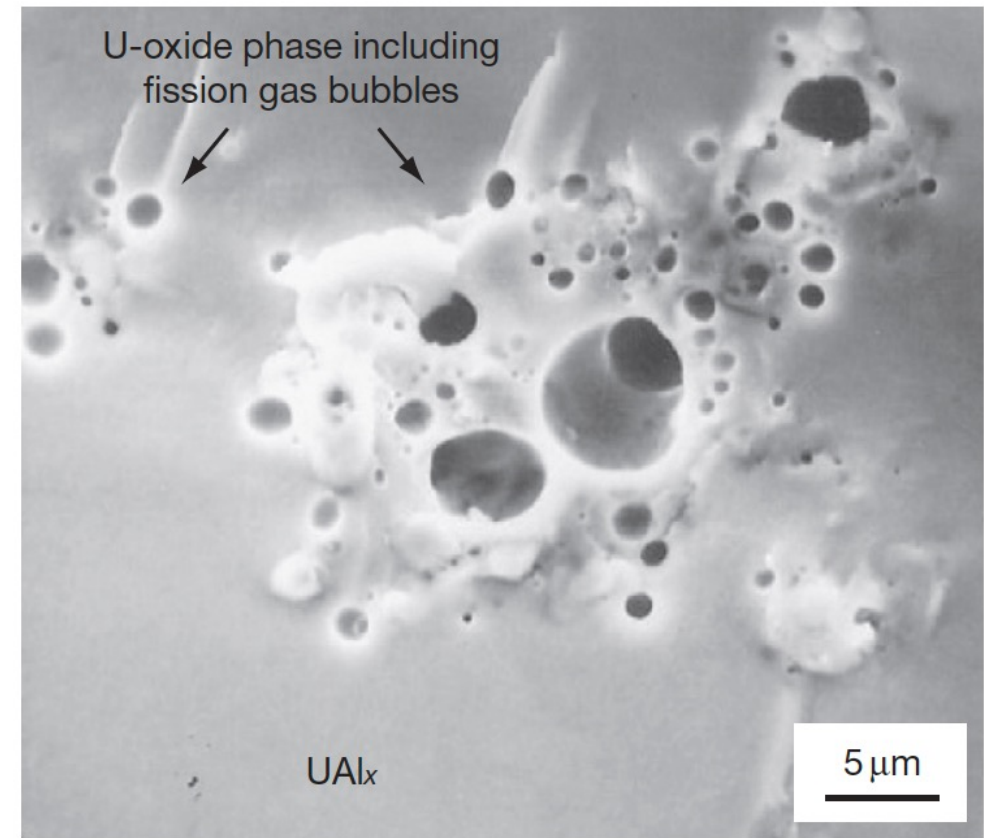
# Fuel Swelling

- Gaseous FP swelling is due to the formation of fission gas bubbles and is more difficult to quantify
- Historical examinations on  $UAl_x$  fuels showed no large fission gas bubbles in the fuel
- Thus, fission gas bubbles were sufficiently small to be beyond the scope of 1980s era SEM
- However, oxide inclusions showed large fission gas bubbles
- Oxides are present due to fabrication



# Fuel Swelling

- It is unclear whether the oxide clusters acted as reservoirs absorbing fission gas, or whether  $UAl_x$  helps retard bubble formation
- It is possible that nanoscale bubbles are forming, but are undetectable via SEM (I would argue this is necessary)
- The gas bubble swelling rate is estimated by subtraction of solid fission product swelling from the total swelling of the plate

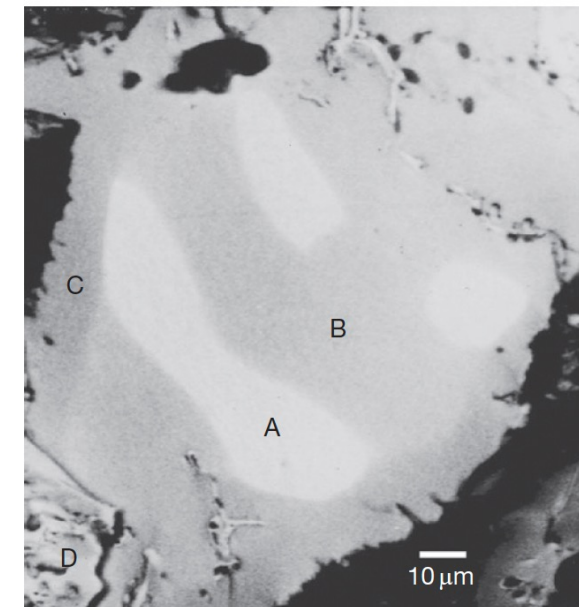
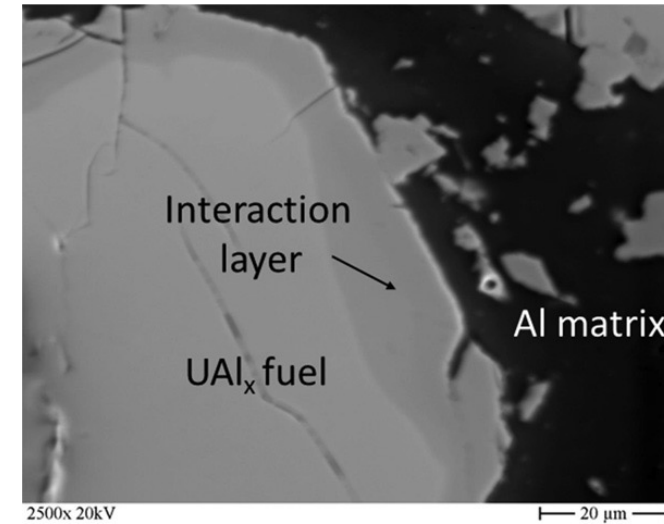


# Amorphization

- The performance of all U intermetallic fuels is closely related to whether they are crystalline or amorphous during irradiation
- The U intermetallic fuels tend to be amorphized by damage in the crystal structure caused by highly energetic fission fragments and low temperatures inhibiting recombination
- Amorphization of a crystalline material is accompanied by an increase in volume, which facilitates atomic mobility, enhancing diffusion
- Fission gas mobility is also high in amorphous materials and the fuel material is more readily deformed by the growing gas bubbles
- Thus, fission gas bubble growth in an amorphous material is faster
- The three U–Al intermetallics undergo amorphization depending on the fission rate and temperature
- The lower the irradiation temperature and the higher the fission rate, the more readily the fuel becomes amorphous

# UAl-Al Interaction

- UAl<sub>x</sub> and Al react during irradiation even at low temperatures due to irradiation-enhanced interdiffusion
- UAl<sub>2</sub> and UAl<sub>3</sub> react with matrix Al to generate UAl<sub>4</sub>, and since there are no higher content compounds, UAl<sub>4</sub> stays stable with Al
- In the image, A is UAl<sub>2</sub>, B is UAl<sub>3</sub>, C is UAl<sub>4</sub>, and D is U oxide
- Measured reaction data of UAl<sub>x</sub>-Al from in-pile tests are scarce

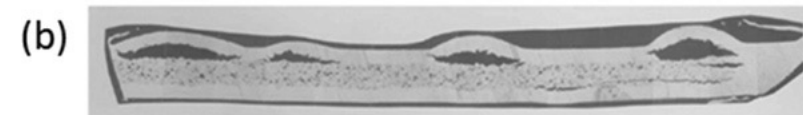
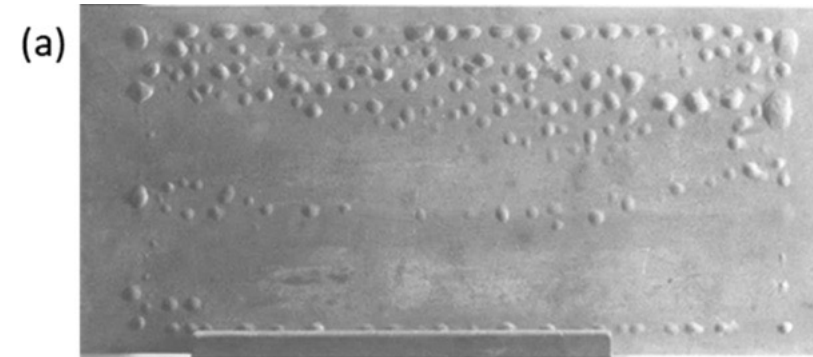
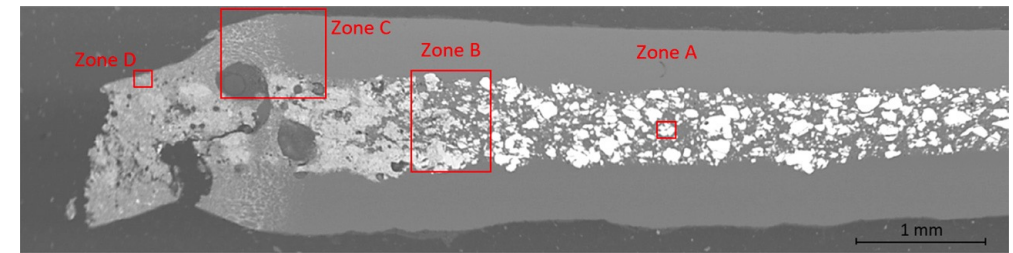
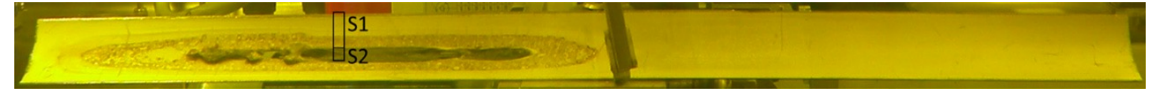


# Fission Gas Bubbles

- The fuel kernel and the interaction layers can both become amorphous
- Even in the amorphous phases, no large fission gas bubbles have been observed
- Relatively recent TEM work has showed that at 20% burnup, Xe is still in solid solution in the amorphized phases
- Conversion of UAl<sub>2</sub> to UAl<sub>3</sub> and UAl<sub>4</sub> results in a volume reduction of the fuel, which generates extra space for fission products to reside, increasing the effective solubility
- The high resistance of UAl<sub>x</sub> to large fission gas bubble formation and a higher as-fabricated porosity lead to a lower overall plate thickness increase in UAl<sub>x</sub>/Al compared to other fuel dispersion systems

# Off-normal behavior

- Most common accident scenario is a channel blockage, resulting in loss of coolant flow
- In the extreme, this can lead to fuel melting
- Higher temperatures at a minimum will lead to bubble formation, coalescence, and swelling
- Temperature increases can result in the blistering of the plates, a delamination of the cladding from the fuel, and potential burst of cladding and release of fission products





# U-Al Summary

- UAlx fuels are in dispersion form in an aluminum matrix
- The three uranium aluminides undergo amorphization depending on the fission rate and temperature
- UAl4 amorphizes most readily and UAl2 least readily
- UAlx–Al dispersions have lower fueled zone swelling than any other type fuel dispersions due to low fission gas bubble swelling
- UAlx fuels had limited utilization due to the requirement of very high U enrichment in relatively low U density alloys
- Additionally, UAl2 is highly pyrophoric, leading to difficulties in fabrication, significantly increasing costs

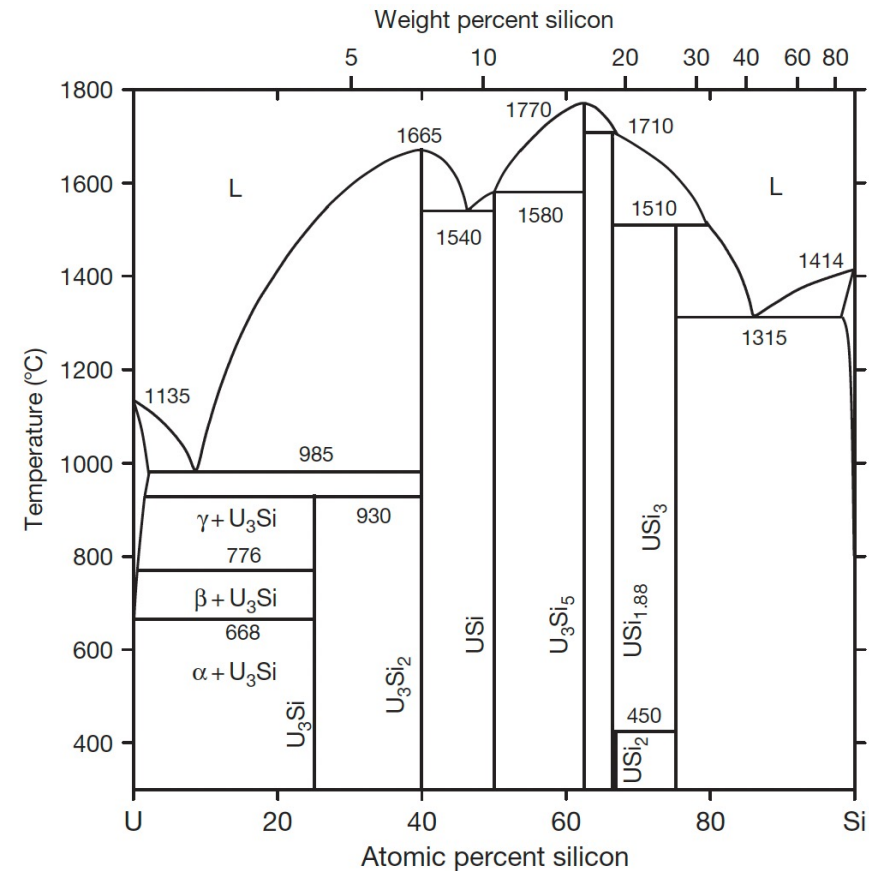
# RERTR

- The US DOE initiated the RERTR (Reduced Enrichment for Research and Test Reactor) program in 1978 to convert the world's research and test reactors using high-enrichment uranium (HEU) to those using low-enrichment uranium (LEU)
- An enrichment in  $^{235}\text{U}$  of 20 at.% is the threshold between HEU and LEU
- Reactors were/are using mainly  $\text{UAlx}$  and  $\text{U}_3\text{O}_8$  dispersion fuels
- To use a fuel with reduced enrichment while keeping the fuel phase volume the same in the fueled zone requires using a fuel with a higher uranium density to compensate for the reduced fissile fraction in LEU
- The fuel form developed to accomplish this is  $\text{U}_3\text{Si}_2$ , which allows the highest possible uranium loading among the qualified fuel types



# U-Si

- In the U–Si system,  $U_3Si$ ,  $U_3Si_2$ , and  $USi$  are the compounds of interest for candidate fuels chiefly because of their high uranium density: 15.3, 12.2, and 10.96 g/cc, respectively
- $U_3Si_2$  and  $USi$  form directly from liquid, but  $U_3Si$  forms only by a peritectoid reaction at 925C
- $U_3Si$  and  $U_3Si_2$  are of key interest, due to their higher U density
- $U_3Si_2$  is also of some interest in commercial LWR application

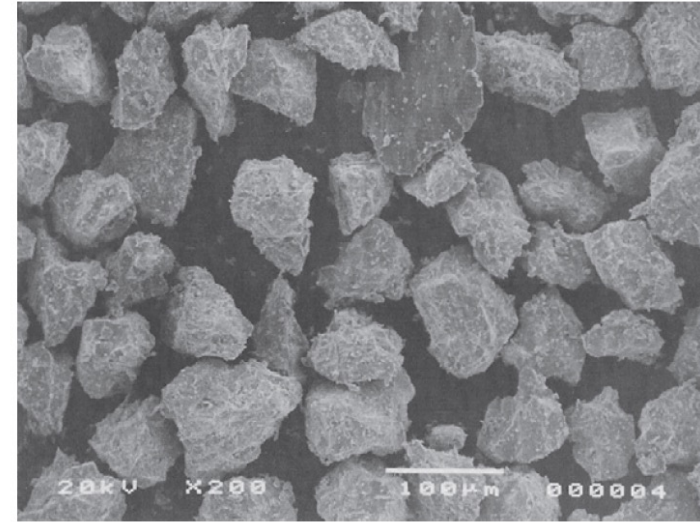


# USi Fabrication

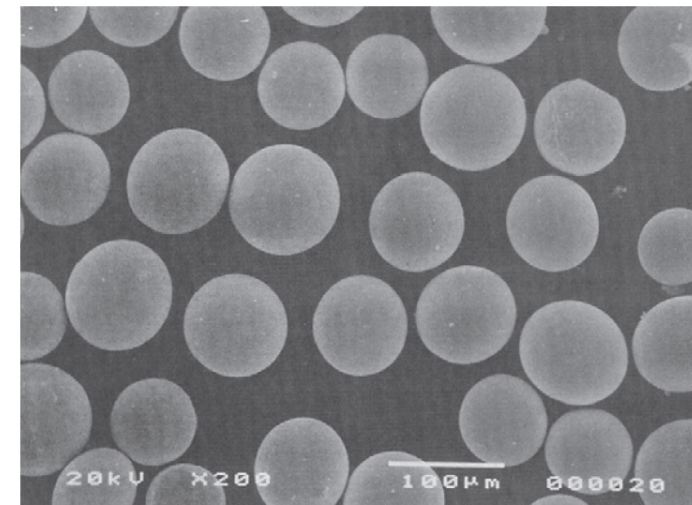
- In practice, it is almost impossible to fabricate the exact stoichiometric form of one of these U-Si compounds
- Typically, a higher content of Si is required to suppress the formation of solid solution U, or Si-lean U-Si compounds
- The secondary phases typically reside inhomogeneously in a fuel particle, which causes inhomogeneous size distributions of fission gas bubbles inside the fuel particles
- Alloy ingots of U–Si are made by mixing and melting of uranium and silicon with a desired Si/U ratio
- The ingots are sometimes annealed in an inert atmosphere to complete compound formation
- These ingots are then broken into smaller particles by a powder fabrication process
- $U_3Si$  is more ductile than  $U_3Si_2$ , and requires significantly more work to break into small particles

# USi Fabrication

- The fragmentation/comminution process results in jagged and irregular powders
- An atomization technology widely used in powder metallurgy is applied to fabricate spherical powders of  $U_3Si_2$  and  $U_3Si$ , involving liquid fuel droplets and centrifugal force
- Atomized powder has several advantages over comminuted powder: 1) surface-to-volume ratio is smaller, so reaction with matrix is less; 2) high homogeneity and fewer impurities; 3) lower residual stresses and defects



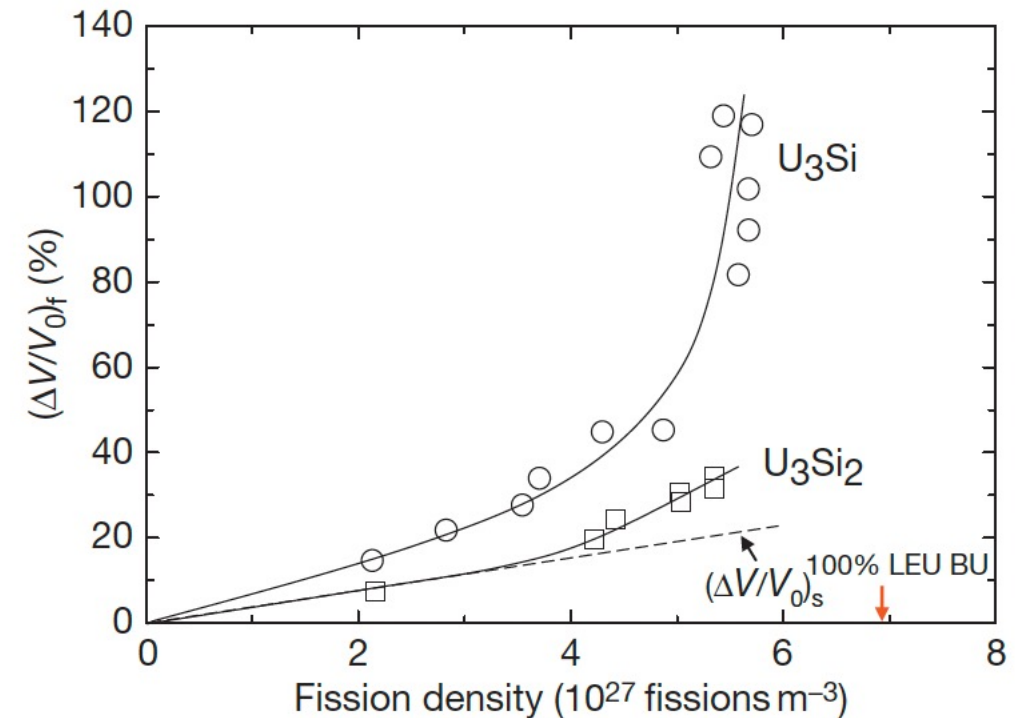
Comminution



Atomization

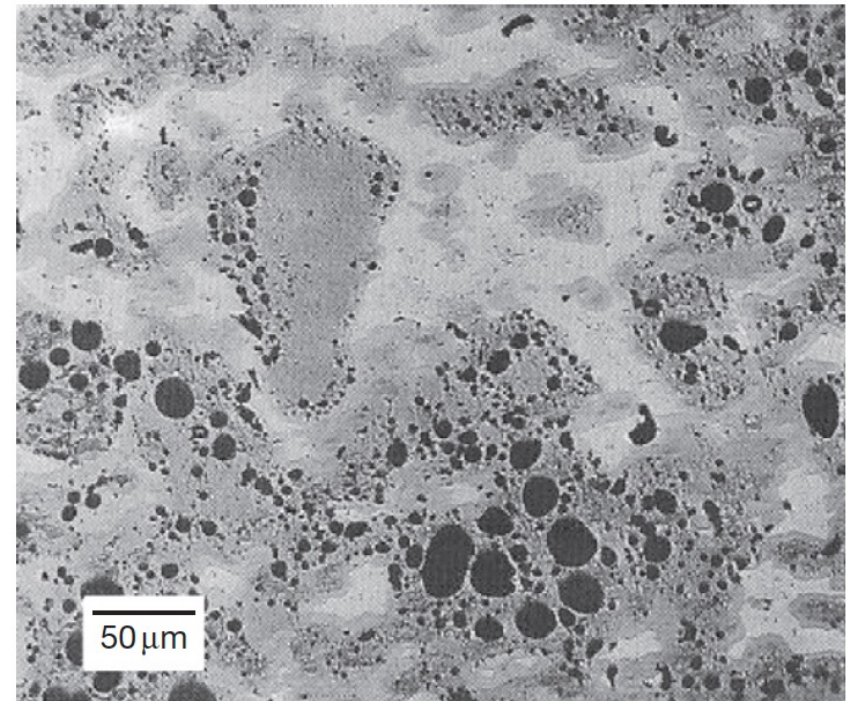
# USi Fuel Swelling

- Solid fission product swelling is treated as identical to that in UAlx fuels, but fission gas swelling is markedly different
- Large swelling for U-Si phases, more so for U<sub>3</sub>Si
- Fuel swelling kinetics of U–Si fuel particles is well documented in the literature
- Again, gaseous swelling is estimated by subtracting solid FP swelling from the total swelling



# Amorphous Swelling

- $\text{U}_3\text{Si}$  and  $\text{U}_3\text{Si}_2$  are known to become amorphous under irradiation at sufficiently low temperatures
- The primary damage in the crystal is due to highly energetic fission fragments
- In the amorphous fuel, fuel swelling depends on the viscosity of fuel
- Fission gas mobility is also high in amorphous material and the fuel material is more readily deformed by the growing gas bubbles

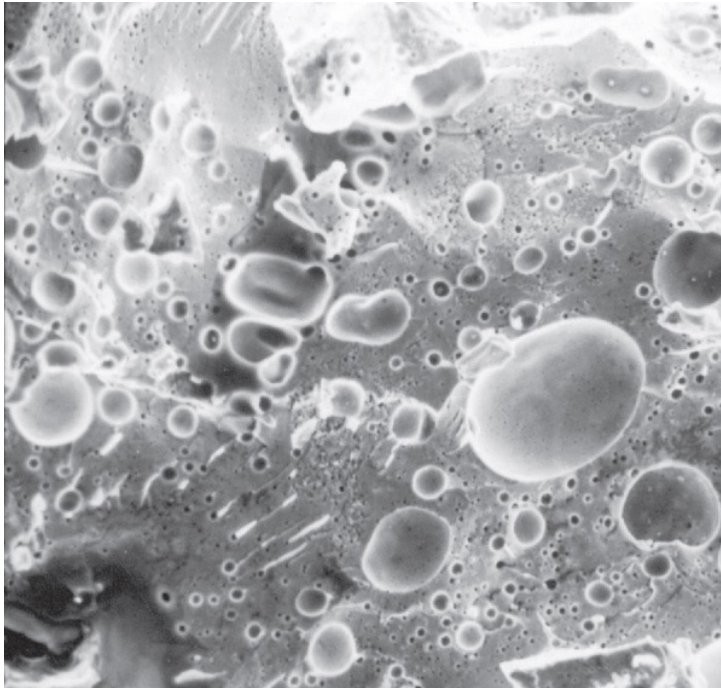


$\text{U}_3\text{Si}$  fission gas bubbles at  $4.5\text{E}27 \text{ f/m}^3$

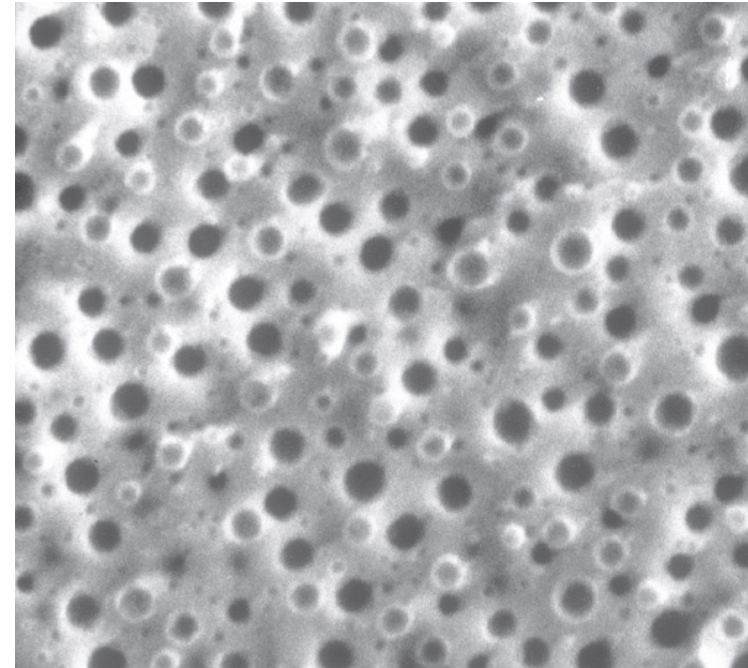


# USi Swelling

- Figures shows fuel microstructures and the fission gas bubble morphology of irradiated  $U_3Si$  and  $U_3Si_2$  at 100C to 15% and 19% burnup



$U_3Si$



$U_3Si_2$

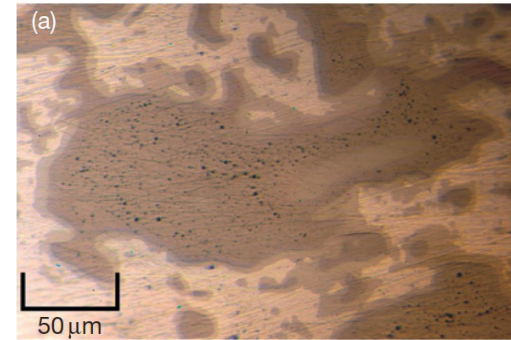
# USi Swelling

- Both U<sub>3</sub>Si and U<sub>3</sub>Si<sub>2</sub> are amorphous during research reactor irradiation
- Fission gas bubble growth in U<sub>3</sub>Si is high and unstable, while that of U<sub>3</sub>Si<sub>2</sub> is generally lower and stable
- An explanation is the correlation between free volume and viscosity, in that U<sub>3</sub>Si has larger free volume than U<sub>3</sub>Si<sub>2</sub>
- Thus U<sub>3</sub>Si viscosity is relatively lower
- The additional Si bonds in U<sub>3</sub>Si<sub>2</sub> have a large effect on the amount of free volume in the glassy state, and therefore also on the fluidity of the fuel, and thus the swelling behavior
- Amorphization is a prerequisite for this low-temperature high-swelling behavior

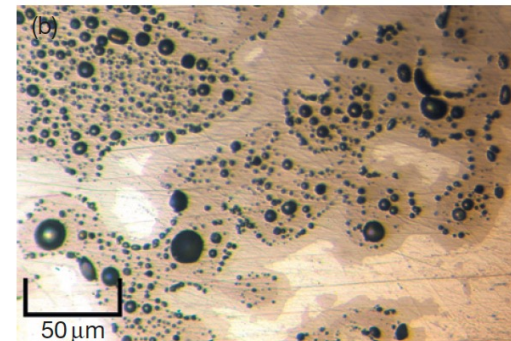
$$\eta = \eta_0 \exp\left(\frac{C}{\Delta V_R}\right)$$

# U<sub>3</sub>Si<sub>2</sub> Swelling w/ temperature

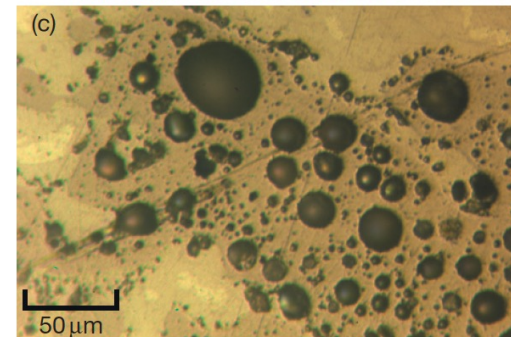
- The bubble morphology from higher temperature tests is available
- Bubble growth in U<sub>3</sub>Si<sub>2</sub> can be enhanced to the level of U<sub>3</sub>Si if the temperature is increased by about 60C (albeit at higher burnups)
- It appears that the low bubble growth advantage of U<sub>3</sub>Si<sub>2</sub> provided by the high Si/U ratio is negated if the temperature is increased



T=105 C and FD=3.2E27 f/m<sup>3</sup>



T=136 C and FD=5.4E27 f/m<sup>3</sup>

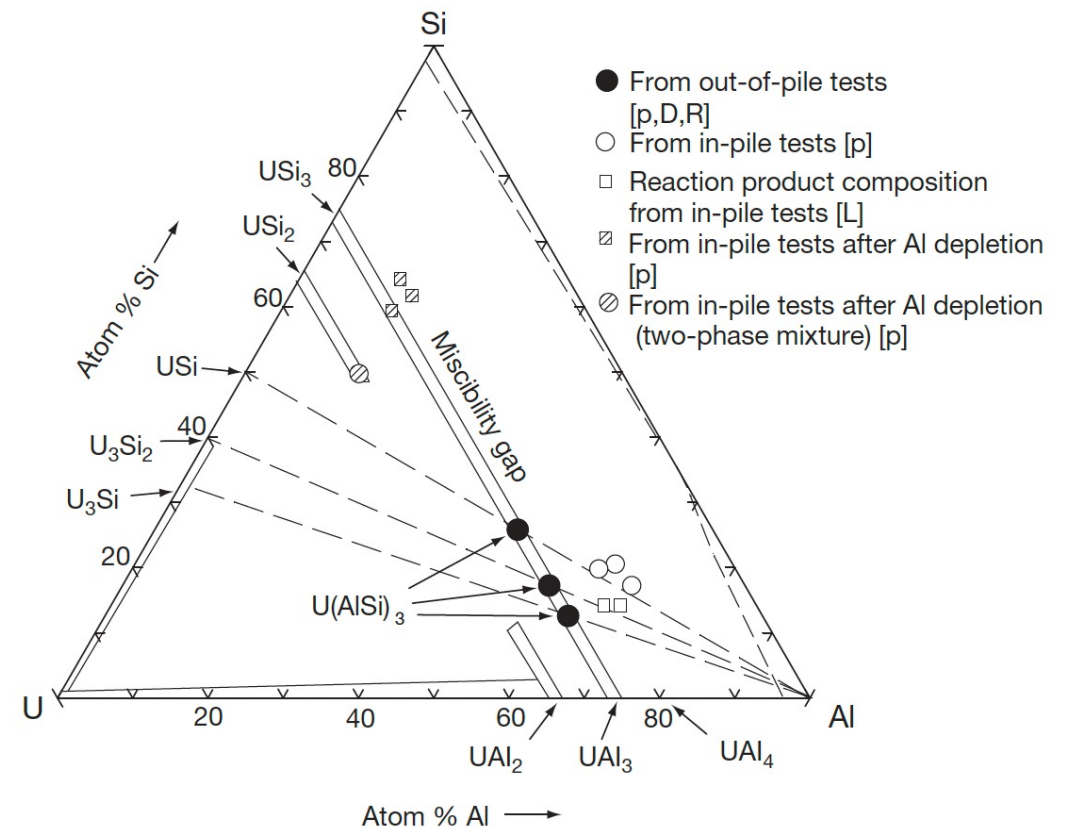


T=160 C and FD=6.1E27 f/m<sup>3</sup>



# USi interaction with Al

- $\text{U}_3\text{Si}$ ,  $\text{U}_3\text{Si}_2$ , and  $\text{USi}$  react with Al to form a single intermetallic compound,  $\text{U}(\text{AlSi})_3$
- The solubility of Al in the USi phases is very low
- $\text{U}(\text{AlSi})_3$  has a composition intermediate between  $\text{UAl}_3$  and  $\text{USi}_3$ , both of which are mutually soluble



# USi interaction with Al

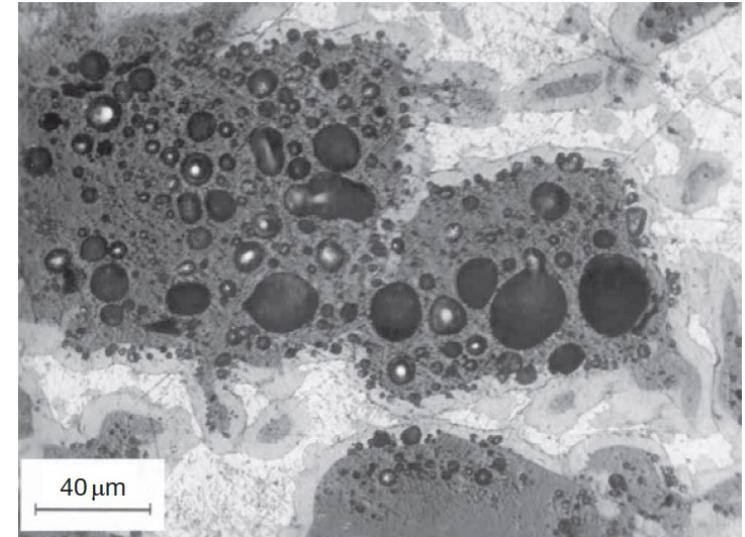
- Interaction layer growth is an interdiffusion controlled process, and can be investigated in out-of-pile experiments
- Out-of-pile tests are typically performed at high temperatures (600C) and have shown that interdiffusion is the rate-controlling process in IL growth of silicide–Al dispersion
- Fuel temperatures of typical in-pile tests are much lower (<200C) than the out-of-pile tests
- Simple extrapolations to the low temperature regime of the IL growth correlations for out-of-pile tests yield orders of magnitude smaller IL thickness values than observed
- This implies that thermally activated diffusion must be augmented by fission enhanced diffusion during irradiation

# USi interaction with Al

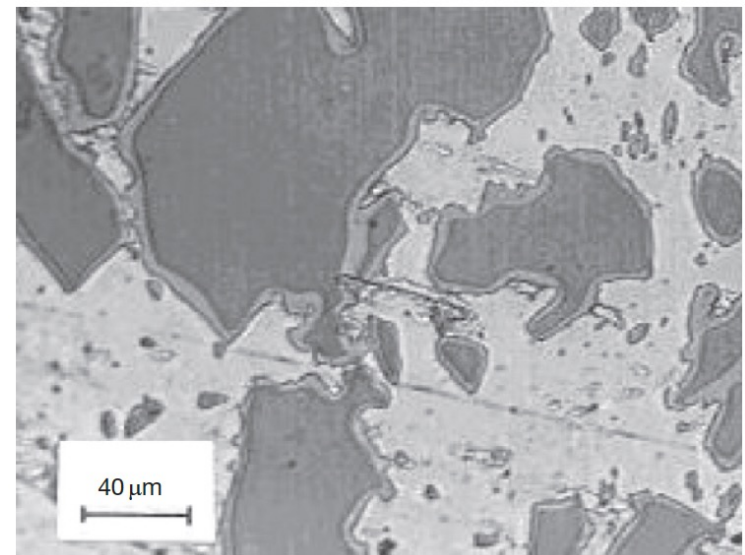
- The compositions in the interaction layer deviate from the exact stoichiometry of  $\text{U}(\text{AlSi})_3$
- This indicates that the reaction products become amorphous during irradiation, which has been observed in in-pile tests
- Since the IL is amorphous, U, Al, and Si atoms exist in a mixture without crystalline restriction of stoichiometry
- The formation of gas bubbles in the ILs is important because of its potential effects on the IL growth rate
- The gas bubbles in the IL reduce the effective diffusion area and thereby reduce the IL growth rate, limiting breakaway IL formation

# Micrographs of irradiated LEU $\text{U}_3\text{Si}-\text{Al}$ and $\text{U}_3\text{Si}_2-\text{Al}$

- The ILs of both fuels are generally uniform in thickness and free of visible fission gas bubbles
- The gas bubbles in the unreacted fuel serve as a boundary between the unreacted fuel and the ILs
- $\text{U}_3\text{Si}$  has dramatically higher swelling than  $\text{U}_3\text{Si}_2$
- Both samples have similar burnup and temperature



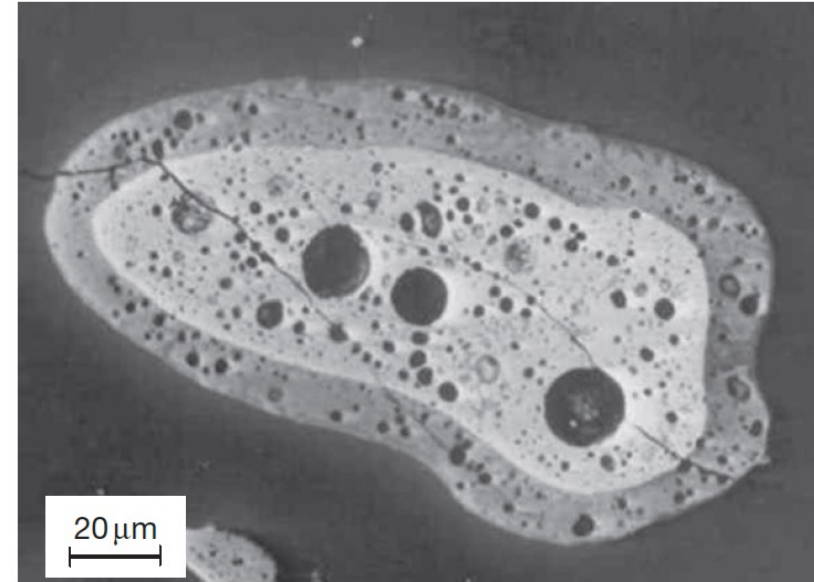
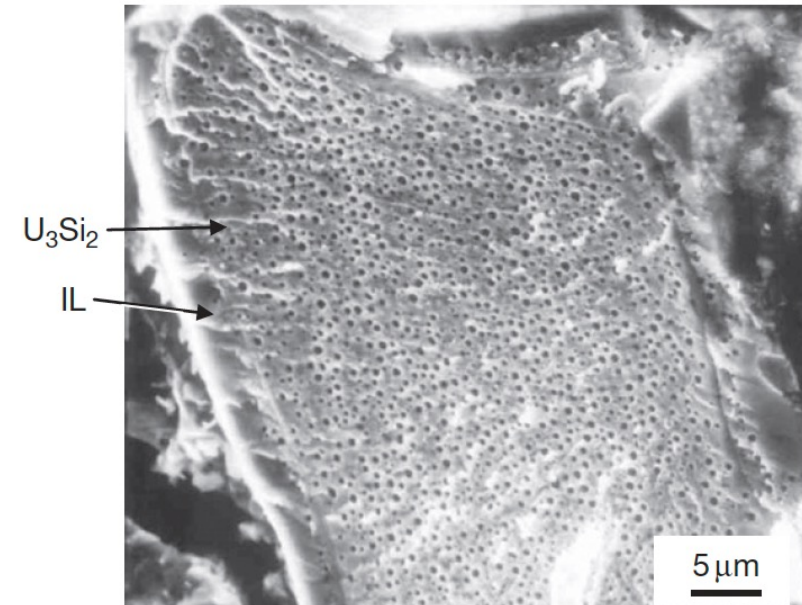
$\text{U}_3\text{Si}-\text{Al}$



$\text{U}_3\text{Si}_2-\text{Al}$

# Micrographs of irradiated HEU $\text{U}_3\text{Si}-\text{Al}$ and $\text{U}_3\text{Si}_2-\text{Al}$

- Gas bubbles are found in ILs of high-burnup HEU  $\text{USi}$  fuels
- These images are from ultra high burnup samples,  $\sim 4.5\text{X}$  the previous slide
- Thus, likely the formation of fission gas bubbles in the IL is dependent upon fission density
- Bubbles in the IL of  $\text{U}_3\text{Si}_2$  appear later than in  $\text{U}_3\text{Si}$

 $\text{U}_3\text{Si}-\text{Al}$  $\text{U}_3\text{Si}_2-\text{Al}$