

# Shehab Showsha

## Question(1)

- Assumptions
- a) Constant thermal conductivity  $k(T) = k$
  - b) Assuming steady state
  - c) Assuming symmetry in y, z directions

$$\frac{\partial}{\partial x} \left( k \frac{\partial T}{\partial x} \right) + Q = 0 \Rightarrow \frac{\partial}{\partial x} \left( k \frac{\partial T}{\partial x} \right) = -Q$$

$$k \frac{\partial T}{\partial x} = -Qx + C_1$$

from boundary condition  $\frac{\partial T}{\partial x} \Big|_{x=0} = 0 \Rightarrow 0 = 0 + C_1$

$$\therefore C_1 = 0$$

$$k \frac{\partial T}{\partial x} = -Qx$$

$$\therefore T(x) = -\frac{Qx^2}{2k} + C_2$$

from boundary cond/tns :  $T(X) = T_1$

$$\therefore T_1 = -\frac{QX^2}{2k} + C_2 \Rightarrow C_2 = T_1 - \frac{QX^2}{2k}$$

$$\therefore T(x) = -\frac{QX^2}{2k} + T_1 - \frac{Qx^2}{2k}$$

$$T(x) = T_1 - \frac{Q}{2k} (X^2 + x^2)$$

Temp. drop  
across  
fuel

$$T_0 - T_F = \frac{Q}{2k} X_F^2 \left( \text{when } X_F : \text{fuel thickness} \right)$$

Question (2)

1st Without Coating

$$T_{CO} = 600 K$$

$$Q = \frac{250 W}{cm^3}$$

$$LHR = \pi R_f^2 Q$$

$$= \pi (0.6)^2 \times 250$$

$$= 90 \pi \frac{W}{cm}$$

$$T_{CI} = T_{CO} + \frac{LHR}{2\pi R_f} \frac{t_{clad}}{K_{clad}}$$

$$T_{CI} = 600 + \frac{90}{2 \times 0.6} \times \frac{0.05}{0.15}$$

$$T_{CI} = 600 + 25 = 625 K$$

Gap

$$T_F = T_{CI} + \frac{q_0 \pi}{2\pi R_f} \frac{t_g}{K_g}$$

$$T_F = 625 + \frac{90}{2 \times 0.6} \times \frac{0.005}{0.004} = 718.75 K$$

$$T_o = T_F + \frac{QR^2}{4K_f} = 718.75 + \frac{250 \times 0.6^2}{4 \times 0.05} = \boxed{1168.75 K}$$

Question (2)

with Coating

$$T_{\text{coating out}} = 600 \text{ K}$$

$$T_{\text{CO}} = T_{\text{coating}} + \frac{q_0 \pi}{2 \pi R_F} \frac{L_{\text{coat}}}{K_{\text{coat}}} = 600 + \frac{90}{2 \times 0.6} \times \frac{0.01}{0.015} \\ = 650 \text{ K}$$

$$T_{\text{CI}} = ~~650~~ 650 + \frac{90}{2 \times 0.6} \times \frac{0.05}{0.15} = 675 \text{ K}$$

$$T_F = ~~675~~ 675 + \frac{90}{2 \times 0.6} \times \frac{0.005}{0.004} = 768.75 \text{ K}$$

$$T_0 = ~~768.75~~ 768.75 + \frac{250 \times 0.6^2}{4 \times 0.05} = \boxed{1218.75 \text{ K}}$$



### Question (3)

$$a) \phi = 5 \times 10^{12} \frac{n}{cm^2 \cdot s}$$

$$\text{Heat gen. rate} = \frac{200 \text{ MeV}}{\text{fission}} \times N_{U-235} \times \sigma_f \times \phi$$

$$N_{UN} = \frac{P N_{AU}}{M} = \frac{12.3 \times 6.02 \times 10^{23}}{238 + 14} = 2.94 \times 10^{22} \frac{\text{f.u.}}{cm^3}$$

$$N_U = N_{UN} = 2.94 \times 10^{22} \frac{\text{atoms}}{cm^3}$$

Approximately: 19.5% of them are U-235

$$\therefore N_{U-235} = 2.94 \times 10^{22} \times \frac{19.5}{100} = 5.7 \times 10^{21} \frac{\text{atom}}{cm^3}$$

$$\therefore \text{Heat gen. rate} = 200 \times 10^6 \times 1.6 \times 10^{-19} \text{ J} \times 5.7 \times 10^{21} \frac{cm^{-3}}{cm^3} \times 570 \times 10^{-24} \frac{cm^2}{cm^2} \times 5 \times 10^{12} \left(\frac{1}{cm \cdot s}\right)$$

$$\approx \boxed{520 \frac{W}{cm^3}}$$

(b) if  $UO_2$  instead of UN

$$N_{U-235} = 5.7 \times 10^{21} \frac{\text{atom}}{cm^3}, N_{U-235} \approx N_U \times \text{enrichment}$$

$$\therefore N_U = \frac{5.7 \times 10^{21}}{\text{enrichment}}, \therefore N_{UO_2} = N_U = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

$$\therefore \frac{P_{UO_2} N_{AU}}{M_{UO_2}} = \frac{5.7 \times 10^{21}}{\text{enrichment}} \Rightarrow \frac{10.97 \times 6.02 \times 10^{23}}{238 + 16 \times 2} = \frac{5.7 \times 10^{21}}{\text{enrichment}}$$

$$\therefore \text{enrichment} \approx 23.3\%$$

Question (4)

$$LHR^0 = 350 \frac{W}{cm}, \quad \gamma = 1.3, \quad Z_0 = \frac{3.5m}{2} = 1.75m$$

$$\begin{aligned} (a) \quad LHR(z=1.4m) &= LHR^0 \cos\left[\frac{\pi}{2\gamma}\left(\frac{z}{Z_0}-1\right)\right] \\ &= 350 \cos\left[1.2(0.8-1)\right] \\ &= 350 \times 0.97 \\ &\approx \boxed{340 \frac{W}{cm}} \end{aligned}$$

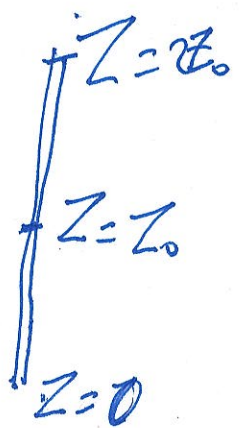
$$(b) \quad C_p = 4200 \frac{J}{kg \cdot K}, \quad \dot{m} = 0.22 \frac{kg}{s \cdot rod}$$

$$\Delta T_{\text{rod}} = \frac{1}{1.2} \frac{Z_0 LHR^0}{\dot{m} C_p} \left\{ \underset{\substack{\downarrow \\ 0.932}}{\sin(1.2)} + \sin\left[1.2 \left(\frac{z}{Z_0}-1\right)\right] \right\}$$

$\Delta T_{\text{rod}} \leftarrow [T_{\text{out}} - T_{\text{in}}]$

$$= \frac{1}{1.2} \times \frac{1.75 \times 350 \times 10^2 \frac{W}{m}}{0.22 \times 4200 \frac{J}{kg \cdot K}} [0.932 + 0.932]$$

$$\approx \boxed{103 K}$$



Question (5)  $dt = 0.5$ ,  $t_0 = 0$ ,  $y_0 = 4$

$$\frac{dy}{dt} = t e^{-2t}$$

$$\cancel{y(t)} \quad y(t+dt) = y(t) + dt y'(t+dt)$$

$$\Rightarrow \boxed{y_0 = 4}$$

$$\Rightarrow y_{0.5} = 4 + 0.5 y'(0.5) = 4 + 0.5 \times [0.5 e^{-1}]$$
$$= \boxed{4.092}$$

$$\Rightarrow y_1 = 4.092 + 0.5 y'(1) = 4.092 + 0.5 [1 \cdot e^{-2}] = \boxed{4.16}$$

$$\Rightarrow y_{1.5} = 4.16 + 0.5 y'(1.5) = 4.16 + 0.5 [1.5 e^{-3}]$$
$$= \boxed{4.197}$$



### Question (6)

Fertile Isotope that can be transformed into a fissile isotope by absorbing neutrons.

Fissile Isotope that can undergo fission with neutrons of any energy (even thermal energies)

Fissionable Isotope that can undergo fission only with neutrons of a minimum threshold energy

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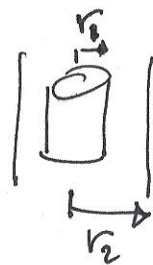
Question (7) We don't use pure metallic U because:

- (1) ~~the~~ Phase changes
  - (2) swelling during thermal cycling
  - (3) Anisotropic thermal expansion & Anisotropic irradiation growth
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Question (8) Swear density is the ratio between the fuel volume and the total internal volume of fuel element

$$\text{swear density} = \frac{\pi r_1^2 h}{\pi r_2^2 h} = \frac{r_1^2}{r_2^2}$$

It is important to have a gap to accommodate fuel swelling & fission gas release



### Question (9)

- ⇒ We need to enrich U to increase the percentage of U-235 isotope [which is the fissile isotope]
- ⇒ Natural U has only 0.7% of U-235
- ⇒ We need U-235 bec. it undergoes fission with thermal <sup>neutrons</sup>
- ⇒ In enrichment process,  $UF_6$  is utilized  
(Uranium hexafluoride) gaseous form

### In Centrifuge

Lighter gas with more U-235 goes  
to the outside and can be extracted

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### Question (10)

- ① Finite ~~element~~ difference : simple and fast  
(using discrete points) but can't be applied for heterogeneous structure
- ② Finite volume :
- ~~③ Finite difference :~~
- ③ Finite element :



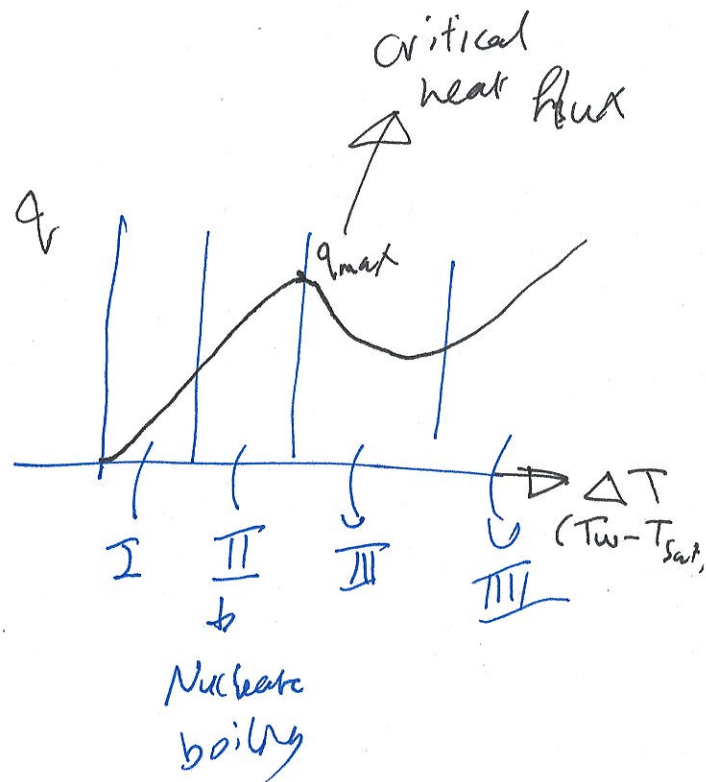
Question (11)

Departure from Nucleate boiling

$$DNBR = \frac{\text{Critical Heat Flux}}{q(z)}$$

$$DNBR > 1$$

The margin allowed  
is typically 1.15 to 1.3

Critical Heat Flux:

is the max heat flux that can be achieved before  
a vapor film is created and then heat flux decreases

Question (12)

TRISO : UC or UCO  
Carbide fuel

with SiC  
coating

In  
High temp. gas cooled reactor (HTGR)

