#### **NE 795-014: Advanced Reactor Materials**

Fall 2023

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### Housekeeping

Reminder, test on Thursday on SFRs

# **MOLTEN SALTS**

#### What are molten salts?

- Salts are ionic compounds formed from a combination of electronegative and electropositive elements
- At elevated temperatures salts liquefy and are termed "molten salts"
- Halide salts are ionic compounds formed from the combination of a halogen (electronegative) and another electropositive element – commonly, but not exclusively, alkali metals or alkaline earths
- Examples: LiF, BeF2, MgCl2, NaCl (aka table salt), ZrF4, RbF, UF4, UCl3

**Alkaline** Alkali Halogens Metals **Farths** Be Beryllium 9.0122 Lithium Fluorine 6.94 18.998 Na Mg Sodium Magnesium 24.305 Chlorine 22.990 35.45 19 20 35 Ca Br Calcium 40.078 Potassium **Bromine** 39.098 79.904 37 38 53 **Rb** Sr Rubidium Strontium Iodine 85.468 87.62 126.90 55 56 85 Cs Ba At Caesium **Barium Astatine** 132.91 137.33 (210)88 117

Ra

Radium

Francium

(223)

Ts

Tennessine

#### **Molten Salt Reactors**

# Salt Cooled Reactor

Driven by some traditional fuel type in cladding, where the coolant is replaced with molten salt

# Static Salt Fueled Reactor

The fuel itself is a liquid actinide-salt mixture contained in a cladding, often coupled with a salt coolant

# Flowing Salt Fueled Reactor

The fuel itself is a liquid actinide-salt mixture which flows in a loops through the core and doubles as the coolant

### **Thermal and Fast**

- Often salt reactors are further subdivided into fast and thermal spectrum reactors
- List of examples of reactor systems with solid fuel, thermal liquid fuel, and fast liquid fuel

	Solid Fuel MSRs (all thermal spe	ctrum)	
Molten-Salt Reactor with Micro-Particle Fuel (MARS)	Kurchatov Institute, Russia	16	TRISO-coated LEU/FLiBe/Graphite pebble bed
Advanced High Temperature Reactor (AHTR)	ORNL, United States	3 400	Coated U particles in blocks or plates/FLiBe/Graphite
Small Advanced High Temperature Reactors (SmAHTR)	ORNL, United States	125	Coated U particles in blocks or plates/FLiBe/Graphite
Pebble Bed – Fluoride Salt-Cooled High Temperature Reactors (PB-FHR)	UC Berkeley, MIT and UW, United States	242	TRISO-coated LEU/FLiBe/Graphite pebble bed
Thorium Molten Salt Reactor, Solid Fuel (TMSR-SF)	SINAP, China	395	TRISO-coated U-Th/FLiBe/Graphite pebble bed
Indian High Temperature Reactor (IHTR)	BARC, India	600	TRISO-coated U-Th/FLiBe/Graphite pebble bed

	Thermal Spectrum Liquid Fuel	MSRs	
Thorium Molten Salt Reactor, Liquid Fuel (TMSR-LF)	Shanghai Institute of Applied Physics (SINAP), China	395	ThF <sub>4</sub> - <sup>233</sup> UF <sub>4</sub> / <sup>7</sup> LiF-BeF <sub>2</sub> /graphite
Integral Molten Salt Reactor (IMSR)	Terrestrial Energy, Canada and the United States	400	UF4/fluorides/graphite
ThorCon Reactor	ThorCon International, Singapore	557×2	UF4/NaF-BeF2/graphite
Liquid-Fluoride Thorium Reactor (LFTR)	Flibe Energy, United States	600	ThF <sub>4</sub> -233UF <sub>4</sub> /7LiF-BeF <sub>2</sub> /graphite
FUJI-U3	Japan	450	ThF4-233UF4/7LiF-BeF2/graphite
Advanced Molten-salt Break-even Inherently-safe Dual-mission Experimental and Test Reactor (AMBIDEXTER)	Ajou University, Korea	250	<sup>233</sup> UF <sub>4</sub> -ThF <sub>4</sub> /LiF-BeF <sub>2</sub>
Transatomic Power MSR (TAP)	Transatomic Power, United States	1 250	UF4/FLiNaK/SiC clad ZrH <sub>1.6</sub>
Compact Used fuel BurnEr (CUBE)	Seaborg Technologies, Denmark	250	SNF/fluorides/graphite
Process Heat Reactor	Thorenco, United States	50	UF4/NaF-BeF2,/Be rods
Stable Salt Thermal Reactor (SSR-U)	Moltex Energy, United Kingdom	300-2 500	UF4/fluorides/graphite
	Fast/Epithermal Spectrum Liquid F	uel MSRs	
Molten Salt Fast Reactor (MSFR)	SAMOFAR, France – EU – Switzerland	3 000	ThF4-UF47LiF-
Molten Salt Actinide Recycler and Transformer (MOSART)	Kurchatov Institute, Russia	2 400	TRUF <sub>3</sub> or ThF <sub>4</sub> -UF <sub>4</sub> / <sup>7</sup> LiF-BeF <sub>2</sub> or NaF <sup>7</sup> LiF-BeF <sub>2</sub>
U-Pu Fast Molten Salt Reactor (U-Pu FMSR)	VNIINM, Russia	3 200	UF4-PuF3/LiF-NaF-KF
Indian Molten Salt Breeder Reactor (IMSBR)	BARC, India	1 900	ThF4-UF4/LiF-
Stable Salt Fast Reactor (SSR-W)	Moltex Energy, United Kingdom	750-2 500	PuF <sub>3</sub> /Fluorides
Molten Chloride Fast spectrum Reactor (MCFR)	TerraPower, United States		U-Pu/Chlorides
	Elysium Industries, United States	100-5 000	U-Pu/Chlorides
Molten Chloride Salt Fast Reactor (MCSFR)	and Canada		

#### **Benefits of MSRs**

- MSRs operate at or close to atmospheric pressure, rather than the 75-150 times atmospheric pressure of typical light-water reactors (LWR)
- This reduces the large, expensive containment structures used for LWRs
- Radioactive fission gases can be naturally absorbed into the molten salt
- MSRs can have higher operating temperatures than a traditional LWR, providing higher electricity-generation efficiency, the possibility of grid-storage facilities, economical hydrogen production and, in some cases, process-heat opportunities.

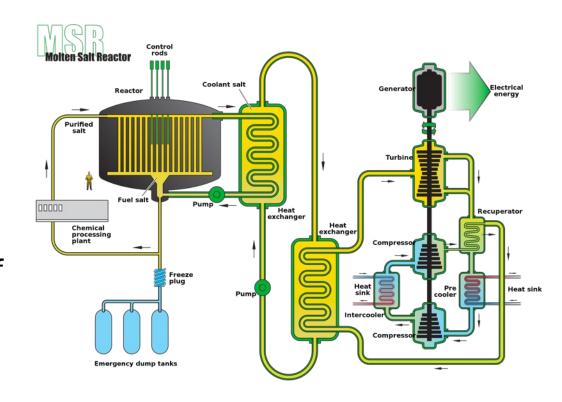
- MSRs have a strong negative reactivity feedback coefficient induced by the thermal expansion of the fluid
- Backup safety measure of dump tanks for rapid removal of fuel
- Possibility of online refueling and waste removal, plus benefits of reprocessing

**Table 1** Typical fuel salt inlet temperatures of selected MSR concepts

MSR concept	$T_{inlet}$	$T_{outlet}$	Reference
MSRE	908K	936K	2
MSBR	839K	977K	3
MSFR	903K	923K	4
MOSART	873K	988K	5
TMSR	873K	923K	

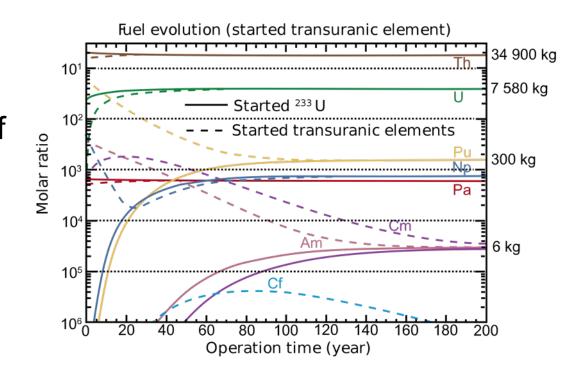
### Design/Nomenclature

- When people usually refer to an MSR, they mean a thermal, fluoride-fueled, flowing loop reactor
- This is due to historical reasons, as we will discuss
- Thus, we have, unless otherwise specified: a liquid fuel with fission species dissolved in the salt; the fuel being pumped through and out of the reactor to a primary HX; secondary loop is typically also a molten salt and goes to a heat exchanger; tertiary loop is your steam turbine loop



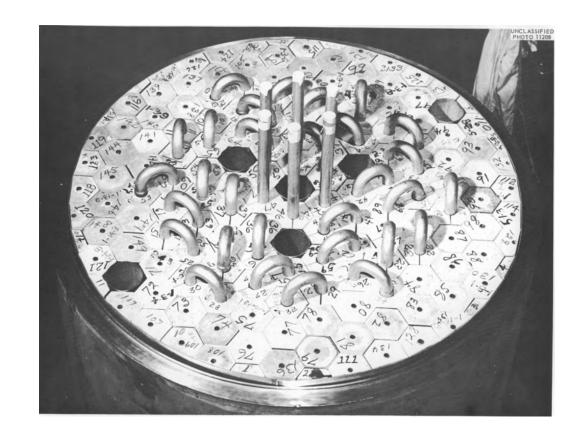
### **Challenges of MSRs**

- The corrosivity of hot salts can break down cladding components and leech alloying elements
- The changing chemical composition of the salt as it is transmuted by reactor radiation leads to changing properties
- The processing of liquid transuranic molten salt fuel from the current spent fuel and of the on-line reprocessing of MSR fuel need to be solved



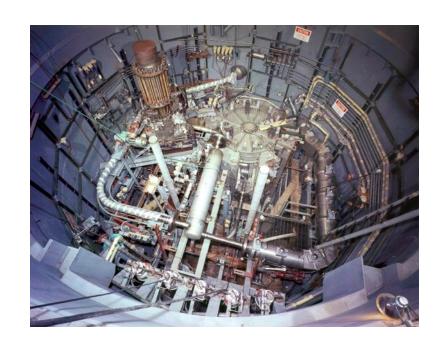
### **Beginning of MSRs**

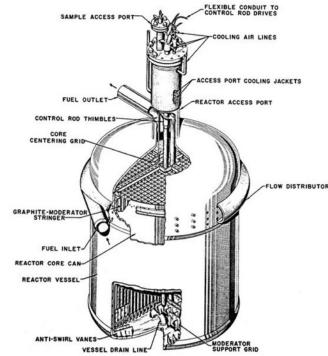
- Aircraft Reactor Experiment (1954) was a 2.5 MWth thermal-spectrum nuclear reactor experiment designed to attain a high power density and high output temperature for use as an engine in a nuclear-powered bomber aircraft
- It used the molten fluoride salt NaF-ZrF4-UF4 (53-41-6 mol%) as fuel, was moderated by a hexagonalconfiguration beryllium oxide (BeO), and had a peak temperature of 860C



### **MSRE**

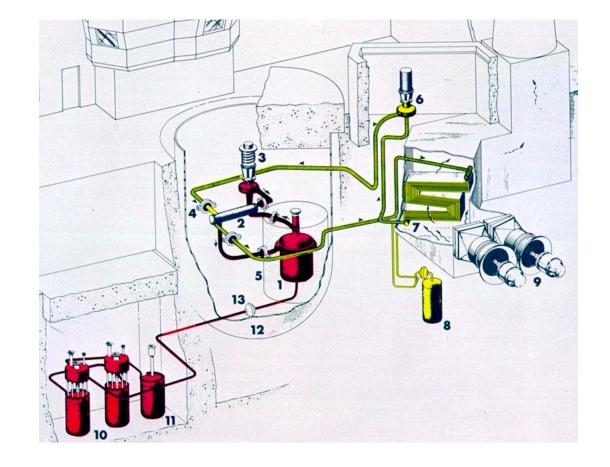
 The Molten-Salt Reactor Experiment (MSRE) was an experimental molten salt reactor at the Oak Ridge National Laboratory (ORNL); it went critical in 1965 and was operated until 1969





### **MSRE**

- MSRE was a circulating fuel type, utilizing LiF-BeF<sub>2</sub>-ZrF<sub>4</sub>-UF<sub>4</sub>
- The secondary coolant salt was LiF-BeF<sub>2</sub> (FLiBe), and the moderator was graphite
- Salt-facing component were made of Hastelloy-N, which is a low chromium, nickel molybdenum alloy



#### **Fuel and Coolant Salts**

- A fuel salt is a molten salt that contains fissile material
- Fuel salts consist of a mixture of some of all of the following:
  - Fissile material
  - Fertile material
  - Solvent (Lowers melting point, Decreases power density, Decreases viscosity)
  - Oxidation prevention material
  - Fission products

 Coolant salts are molten salts with advantageous heat transfer properties



### **Fuel Salt Requirements**

- Low neutron absorption of non-fissile species
- Radiolytic/thermal stability under in-core conditions
- Dissolve fissile materials
- High heat capacity, high boiling point, high thermal conductivity fluids
- Melting point below ~525C
- Low vapor pressure
- Relatively insensitive to fission products
- Both fluoride and chloride salts, under mildly reducing conditions, are reasonably compatible with high temperature structural alloys and graphite

Table 1	Thermodynamic	properties	of fluc	orides

Compound (solid state)	$-\Delta G_{fr1000}$ (kJ mol <sup>-1</sup> )	Compound (solid state)	$-\Delta G_{f,1000}$ (kJ mol $^{-1}$ )
LiF	522	AIF <sub>3</sub>	372
NaF	468	VF <sub>2</sub>	347
KF	460	TiF <sub>2</sub>	339
BeF <sub>2</sub>	447	CrF <sub>2</sub>	314
ThF <sub>4</sub>	422	FeF <sub>2</sub>	280
UF <sub>3</sub>	397	HF	276
ZrF <sub>4</sub>	393	NiF <sub>2</sub>	230
UF <sub>4</sub>	389	CF <sub>4</sub>	130

Elements or Isotopes Which may be Tolerable in High-Temperature Reactor Fuels

Material	Absorption Cross Section (barns at 2200 m/sec)	
Nitrogen-15	0.000024	
Oxygen	0.0002	
Deuterium	0.00057	
Carbon	0.0033	
Fluorine	0.009	
Beryllium	0.010	
Bismuth	0.032	
Lithium-7	0.033	
Boron-11	0.05	
Magnesium	0.063	
Silicon	0.13	
Lead	0.17	
Zirconium	0.18	
Phosphorus	0.21	
Aluminum	0.23	
Hydrogen	0.33	
Calcium	0.43	
Sulfur	0.49	
Sodium	0.53	
Chlorine-37	0.56	
Tin	0.6	
Cerium	0.7	
Rubidium	0.7	

### **Solution Species**

- Solute species serve to lower the melting point, and can dually serve as an oxygen getter
- Must have low neutron absorption cross section and be chemically compatible with cladding
- BeF2, LiF, ZrF, and NaF are commonly utilized due to the combination of these properties

Fluoride compounds in solid state	$\Delta G_f$ , 1000 $K^{a,b}$ (kJ/mol of F)	(kcal/mol of F)	Melting point (°C)	Thermal neutron cross-section (barns)
Diluents	PR CO	***		
CaF <sub>2</sub>	<b>- 523</b>	<b>- 125</b>	1418	0.43
LiF	<b>- 523</b>	<b>– 125</b>	910	0.18
BaF <sub>2</sub>	<b>- 519</b>	<b>- 124</b>	1368	1.17
SrF <sub>2</sub>	<b>- 515</b>	<b>- 123</b>	824	0.17
YF <sub>3</sub>	<b>- 473</b>	<b>– 113</b>	1290	0.23
$MgF_2$	<b>- 473</b>	<b>– 113</b>	848	0.033
NaF	<b>- 469</b>	<b>- 112</b>	727	0.032
KF	<b>- 456</b>	<b>– 109</b>	1387	1.27
BeF <sub>2</sub>	<b>- 435</b>	<b>- 104</b>	1263	0.063
ZrF <sub>4</sub>	<b>- 393</b>	<b>- 94</b>	995	0.53
AIF <sub>3</sub>	<b>– 377</b>	<b>- 90</b>	858	1.97
PbF <sub>2</sub>	<b>- 259</b>	-62	554	0.01
BiF <sub>3</sub>	<b>- 209</b>	<b>- 50</b>	649	0.032
Structural metals				
CrF <sub>2</sub>	- 310	- 74 <sup>a</sup>	1100	3.1
FeF <sub>2</sub>	<b>- 278</b>	$-67^{a}$	930	2.5
NiF <sub>2</sub>	<b>- 243</b>	- 58 <sup>a</sup>	1330	4.6

#### **Fuel Salt Classes**

- Thermal spectrum reprocessing optimized fluoride salts
  - FLiBe (<sup>7</sup>LiF-BeF<sub>2</sub>) and NaF-ZrF<sub>4</sub>
  - FLiBe produces tritium, while NaF-ZrF<sub>4</sub> has a higher vapor pressure
- Fast spectrum and thermal spectrum, once-through fuel cycle optimized fluoride salts
  - LiF-ThF<sub>4</sub>-UF<sub>4</sub>-(TRU)F<sub>3</sub>
  - Much higher fissile loading (actinide-rich eutectics)
  - Adequate fissile material content is a significant design challenge
- Chloride salts
  - Enables harder neutron spectrum and enhanced breeding
  - Isotopically separated chlorine preferable <sup>35</sup>Cl from <sup>37</sup>Cl, due to moderate capture cross section of <sup>35</sup>Cl

#### **Coolant Salts**

- Coolants have the same requirements as fuel salts, but without the consideration of fission product solubility, and a higher focus on chemical compatibility and thermal properties
- Some secondary systems also consider nitrate salts, such as NaNO3 and KNO3 or a mixture of the two, often referred to as solar nitrate; or fluoroborates such as NaF-NaBF3

**Table 3** Selected properties of the coolant salts

Property	LiF-BeF <sub>2</sub> (0.66-0.34)	NaF-NaBF <sub>4</sub> (0.08-0.92)	LiF-NaF-KF (0.465-0.115-0.42)
Melting point (K) $\rho$ (kg m <sup>-3</sup> ) $\eta$ (mPa s) $C_{\rho}$ (J K <sup>-1</sup> g <sup>-1</sup> ) $\lambda$ (W m <sup>-1</sup> K <sup>-1</sup> ) $\log_{10}(\rho(Pa))$	728 2146.3-0.4884 <i>T</i> (K) 1.81 exp (1912.2/ <i>T</i> (K)) 2.39 1.1 11.914-13003/ <i>T</i> (K)	$657\pm1$ 2446.3–0.711 $T$ (K) 0.0877 exp (2240/ $T$ (K)) 1.506 0.66–2.37 $\times$ 10 <sup>-4</sup> $T$ (K) 11.638–6550.6/ $T$ (K)	727 2579.3–0.6240 $T$ (K) 0.0248 exp (4477/ $T$ (K)) 1.88 0.36 $+$ 5.6 $\times$ 10 <sup>-4</sup> $T$ (K) 10.748–10789/ $T$ (K)

#### Fluorides or Chlorides

- Fluoride fuel salts have substantially more experimental data than chloride fuel salts, primarily due to history of MSRs in the thermal spectrum
- Fluorides have been operated in molten salt reactors, been tested in multiple in-pile loops, with many capsule tests
- Chloride salts have a history of laboratory measurements of physical properties, but no in-core testing of fuel salts
  - have extensive use in pyro-processing
- Chlorides typically have lower melting points, which is preferable

#### MSR Fuels

- The fuel in the MSR must fulfill several requirements with respect to its physical and chemical properties
- These include low melting point, high solubility of relevant species, low viscosity, high heat capacity, high thermal conductivity, low volatility/vapor pressure, neutron transparency

Selected properties of the fuel salts

 $\sim$ 1.5<sup>a</sup>

11.902-12 989/T (K)

Table 4

 $\lambda (W m^{-1} K^{-1})$ 

 $log_{10}(p(Pa))$ 

Property	LiF–ThF₄ (0.78–0.22)	LiF-BeF <sub>2</sub> -ThF <sub>4</sub> (0.717-0.16-0.123)	LiF-NaF-BeF <sub>2</sub> -PuF <sub>3</sub> (0.203-0.571-0.212-0.013)
Melting point (K)	841	771	775
$\rho$ (kg m <sup>-3</sup> )	5543.0-1.2500 T (K)	4124.3-0.8690 T (K)	2759.9-0.5730 T (K)
$\eta$ (mPas)	0.365exp(2735/T (K))	0.062exp(4636/T (K))	0.100exp(3724/T (K))
$C_p(J K^{-1} g^{-1})$	1.0	1.55	2.15

11.158–10790.5/T (K)

1.5<sup>a</sup>

 $0.402 + 0.5 \times 10^{-3}/T$  (K)

11.6509-12827/T (K)

### **Neutron Economy**

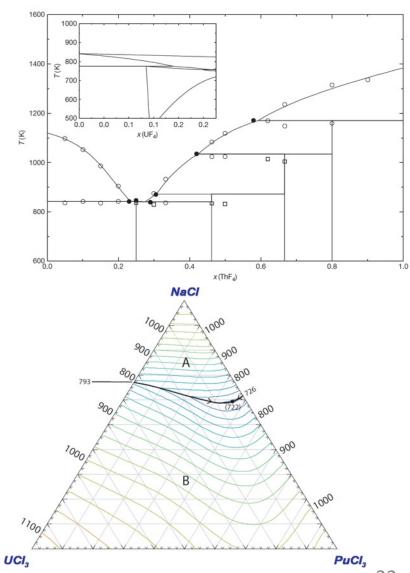
- Thermal spectrum molten salt fuels were identified as candidates for breeding (converting fertile into fissile), specifically in the thorium fuel cycle
- Li-Be-F salts were selected because of the low thermal neutron cross sections of  $^7$ Li ( $\sigma_{th}$  = 0.045 b) and Be ( $\sigma_{th}$  =0.0088 b)
- Natural lithium cannot be used as part of the nuclear fuel as it contains about 7.6% of  $^6$ Li (the remaining 92.4% is  $^7$ Li), which has a very high parasitic neutron capture cross-section ( $\sigma_{th}$  = 940 b)
- Therefore, enrichment of <sup>7</sup>Li is required before it can be used in the core
- Tritium production can become a problem without enrichment

### **Fast Spectrum MSRs**

- Current MSR designs move away from thermal graphite-moderated concepts, and favor non-moderated concepts that have a fast(er) neutron spectrum and can function to burn radioactive waste
- Fuel selection for the nonmoderated reactor concepts is more flexible, and elements other than <sup>7</sup>Li can be considered
- The neutron capture cross-section of the alkali halides and alkaliearth halides is generally lower in the fast spectrum than in the thermal spectrum
- Compounds like NaF, KF, RbF, or CaF2 can be considered as part of the fuel matrix
- There are also some fast MSR concepts which are based on chloride salts

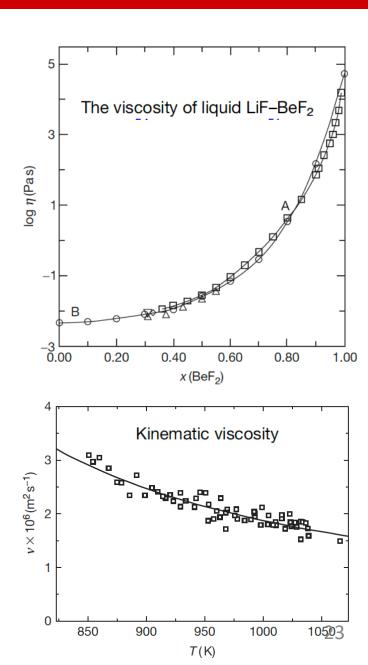
## Solubility

- The solubility of ThF4 in a matrix of LiF can be deduced from the binary phase diagram
- For example, the solubility of ThF4 in a melt of LiF for T 903K (inlet temperature of the MSFR) is between 20.0 and 32.3 mol%
- Compositions in this range are, thus, of interest as fuel for MSRs
- Similarly, eutectic composition and melting temperature are key factors that govern the utilization of specific salts
- Many fission products will form some kind of salt species, with varying degrees of solubility



## **Low Viscosity**

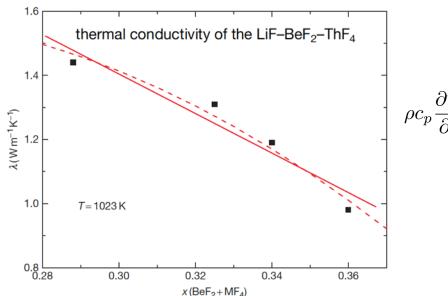
- The viscosity of a fluid is a measure of its resistance to deformation at a given rate; conceptualized as the internal frictional force that arises between adjacent layers of fluid that are in relative motion
- Lower viscosity results in less force to move the fluid at a given velocity
- It is sometimes more convenient to utilize the kinematic viscosity, defined as the ratio of the dynamic viscosity μ to the density of the fluid ρ
- The viscosity of fluoride systems shows significant non-ideal mixing behavior
- Thus, it is not possible to accurately estimate the viscosity of the fluoride salts from its constituents, and therefore more measurements are required



### **Heat Capacity and Thermal Conductivity**

- Heat Capacity is defined as the amount of heat to be supplied to an object to produce a unit change in its temperature
- High heat capacity enables a small amount of the refrigerant to transfer a large amount of heat very efficiently
- Data on experimental heat capacity and thermal conductivity of molten fluoride systems containing actinide fluorides are generally lacking

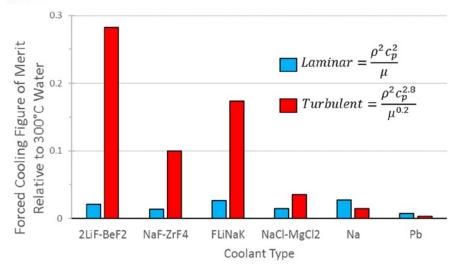
- The thermal conductivity of a material is a measure of its ability to conduct heat
- High thermal conductivity and thermal diffusivity will increase the rate of heat transfer through the fluid

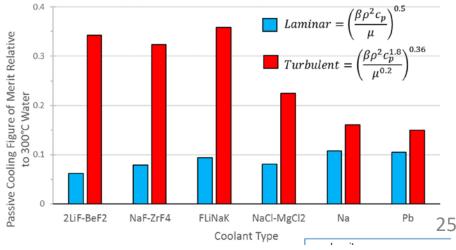


$$\alpha = \frac{k}{\rho c_p}.$$
 
$$\rho c_p \frac{\partial T}{\partial t} = \nabla \cdot (k \nabla T) + Q$$

#### **Heat Transfer**

- Large heat capacity and low viscosity are key properties allowing good heat transfer in molten salts
- In both forced flow and passive cooling conditions, molten salts outperform liquid metals in turbulent flow for heat transfer
- However, experimental data for heat transfer is still significantly scattered for the range of relevant MSR conditions

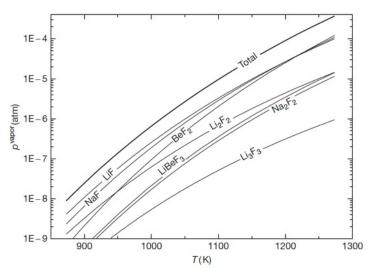




# Vapor Pressure (Volatility)

- Volatility is a material quality which describes how readily a substance vaporizes
- A substance with high volatility is more likely to exist as a vapor, while a substance with low volatility is more likely to be a liquid or solid
- Vapor pressure is a measurement of how readily a condensed phase forms a vapor at a given temperature

- High vapor pressures indicate a high volatility, while high boiling points indicate low volatility
- Salts should have high boiling and low freezing points, in addition to a low vapor pressure (low volatility)



# **Chemical Compatibility**

- For any high-temperature application, corrosion of the metallic container alloy is a primary concern
- The products of oxidation of metals by molten salts tend to be completely soluble in the corroding media
- Due to this solubility, passivation is precluded, and the corrosion rate depends on other factors, including oxidants, thermal gradients, salt flow rate, and galvanic coupling

- Examination of the free energies of formation for typical alloy components shows that chromium is the most active metal component
- Thus, any oxidative attachment to these alloys should be expected to show selective attack on the chromium
- Stainless steels, having more chromium than Ni-based alloys, are more susceptible to corrosion by fluoride melts

### Role of Oxygen Impurities

- The behavior of these systems can be significantly affected by the presence of the oxygen that might be resulting from contamination of the salt system
- Actinide oxides, such as UO2, can precipitate out for oxygen concentrations as low as 1E-4
- Metal oxides have much higher melting points and therefore appear as insoluble components at operating temperatures
- Oxygen also plays a key role in corrosion processes

 ZrF4 was implemented as an oxygen getter in the original MSRE fuels

$$UF_4 + 2H_2O \leftrightarrow UO_2 + 4HF$$
 $ZrF_4 + 2H_2O \leftrightarrow ZrO_2 + 4HF$ 
 $ZrF_4 + UO_2 \leftrightarrow ZrO_2 + UF_4$ 

AlCl<sub>3</sub> or CCl<sub>4</sub> can also be used as reactants to tie up oxygen, or strip O from UO2

### **Summary**

- Molten salt reactors: intro and history
- Fast and thermal; high temperatures; low pressures; flowing fuel salt with secondary salt loop
- Fluorides and chlorides salts for different applications
- Requirements for fuel salts: low melting point, good thermal conductivity, radiation/temperature stability, beneficial solution species, etc.
- Talked through the concepts of a number of the properties of salts
- Control of oxygen and redox conditions is critical to limit precipitation of the fuel and corrosion of the cladding