## Power Iteration Summary

Eugeny Sosnovsky, for 22.05/22.211

October 26, 2012

This document discusses the Power Iteration algorithm, as it applies to finding  $k_{\infty}$ , the reactor eigenvalue for an infinite homogeneous problem.

Consider a group structure with G energy groups, and fully known group constants. With  $\Sigma_{tg}$  as the group g macroscopic total cross section, we define the total cross section matrix T:

$$\mathbf{T} = \begin{bmatrix} \Sigma_{t1} & & \\ & \ddots & \\ & & \Sigma_{tG} \end{bmatrix} . \tag{1}$$

Next, we define the scattering matrix S, with  $\Sigma_{sgg'}$  as the group g' to g scattering cross section:

$$\mathbf{S} = \begin{bmatrix} \Sigma_{s11} & \dots & \Sigma_{s1G} \\ \vdots & \ddots & \vdots \\ \Sigma_{sG1} & \dots & \Sigma_{sGG} \end{bmatrix}. \tag{2}$$

Let  $\overrightarrow{\chi}^{j_f}$  and  $\overrightarrow{\nu\Sigma}_f^{j_f}$  denote the group fission yield spectrum and group production cross section vectors for fissionable isotope  $j_f$ , respectively. Let the operator " $\otimes$ " denote the outer (Kronecker) product of two vectors, as follows:

$$\vec{\mathbf{a}} \otimes \vec{\mathbf{b}} = \vec{\mathbf{a}} \vec{\mathbf{b}}^{\mathrm{T}}. \tag{3}$$

The fission matrix  $\mathbf{F}$  is then defined as:

$$\mathbf{F} = \sum_{\text{all } j^f} \vec{\chi}^{j_f} \otimes \overrightarrow{\nu} \Sigma_f^{j_f}, \tag{4}$$

which with one fissionable isotope  $j_f$ , or with a isotope-independent group fission yield spectrum  $\vec{\chi}$  reduces to:

$$\mathbf{F} = \overrightarrow{\boldsymbol{\chi}} \otimes \overrightarrow{\boldsymbol{\nu}} \overrightarrow{\boldsymbol{\Sigma}}_{f} = \begin{bmatrix} \chi_{1} \nu \Sigma_{f1} & \dots & \chi_{1} \nu \Sigma_{fG} \\ \vdots & \ddots & \vdots \\ \chi_{G} \nu \Sigma_{f1} & \dots & \chi_{G} \nu \Sigma_{fG} \end{bmatrix}.$$
 (5)

The infinite homogeneous reactor's eigenvalue problem is the following:

$$\mathbf{T}\vec{\phi} = \mathbf{S}\vec{\phi} + \frac{1}{k_{\infty}}\mathbf{F}\vec{\phi}.\tag{6}$$

Equation (6) is a general eigenvalue problem. The fission matrix **F** has rank  $J_f$  (the number of fissionable isotopes in Eq. (4)), and so may only be nonsingular if  $J_f \geq G$ ; this is usually not the case, so we can assume F to be singular. This is not relevant for power iteration however, but may make a difference for some eigenvalue search methods.

The power iteration algorithm can be summarized as Algorithm 1:

## Algorithm 1 Power Iteration Algorithm

1: Set initial guesses  $k_{\infty}^{0}$  and  $\overrightarrow{\phi}^{0}$ 

- ▷ Can be arbitrary
- 2: Set iteration limit  $i_{max}$  and fractional eigenvalue tolerance  $\epsilon_k$
- 3: Set fractional group flux tolerance  $\epsilon_{\phi}$

▷ Often skipped

▶ Iteration counter

- 4:  $i \leftarrow 0$ 5:  $\overrightarrow{\mathbf{S}}^i \leftarrow \overrightarrow{\mathbf{F}} \overrightarrow{\boldsymbol{\phi}}^i$
- 6: repeat

7: Solve 
$$(\mathbf{T} - \mathbf{S})\vec{\phi}^{i+1} = \frac{1}{k_{\infty}^{i}}\vec{\mathbf{S}}^{i}$$
 for  $\vec{\phi}^{i+1}$ 

8: 
$$\vec{\mathbf{S}}^{i+1} \leftarrow \mathbf{F} \vec{\boldsymbol{\phi}}^{i+1}$$

8: 
$$\overrightarrow{\mathbf{S}}^{i+1} \leftarrow \overrightarrow{\mathbf{F}} \overrightarrow{\phi}^{i+1}$$
9:  $k_{\infty}^{i+1} \leftarrow \frac{\sum_{\text{all } g} S_g^{i+1}}{\frac{1}{k_{\infty}^i} \sum_{\text{all } g} S_g^i}$ 
10:  $i \leftarrow i+1$ 

10: 
$$i \leftarrow i + 1$$

11: **until** 
$$\left( \left| \frac{k_{\infty}^{i} - k_{\infty}^{i-1}}{k_{\infty}^{i}} \right| < \epsilon_{k} \text{ and } \max_{\text{all } g} \left| \frac{\phi_{g}^{i} - \phi_{g}^{i-1}}{\phi_{g}^{i}} \right| < \epsilon_{\phi} \right) \text{ or } i > i_{max} \qquad \triangleright \epsilon_{\phi} \text{ often skipped}$$

- Power iterations converged, algorithm successful 13:
- Normalize  $\vec{\phi}^i$  as required 14:
- 15: **else**
- 16: Iteration limit exceeded, algorithm failed
- 17: **end if**

Note, that the linear problem on line 7 in Algorithm 1 could easily be extended to a heterogeneous problem with a specified fission source. In such cases, the fission source ratio on line 9 is taken as the ratio of integrals over the entire reactor core. Such problems are referred to as the "fixed source problems," and are the basis of most eigenvalue search codes. If an iterative procedure is used to solve the fixed source problem (which is frequently the case with linear problems), it is referred to as the "inner iterations," while the power iterations on lines 6–11 are referred to as the "outer iterations."