
PROBLEM SET 3 SOLUTIONS
22.211 Reactor Physics I

Due: 10 March 2012

Bryan Herman

Problem: Add U-238 resonance absorption model to your slowing down MC code with the following:

- Model absorption resonances from 0 to 1 keV (ignore resonance scattering)
- Use 0.1 barns for U-238 capture cross section above 1 keV
- Explicitly include SLBW model for the lowest 14 s-wave resonances
- Generate simple “statistical model” for resonance up to 1 keV
- Assume coolant temperature is always 300 K and U-238 temperature is specified
- Follow neutrons down to 1e-5 eV cutoff
- Add pure $1/v$ absorber in at 2 barns/Hydrogen atom

Self-shielding: Below are the spectrum plots and effective resonance integrals. All results generated with 1 million histories.

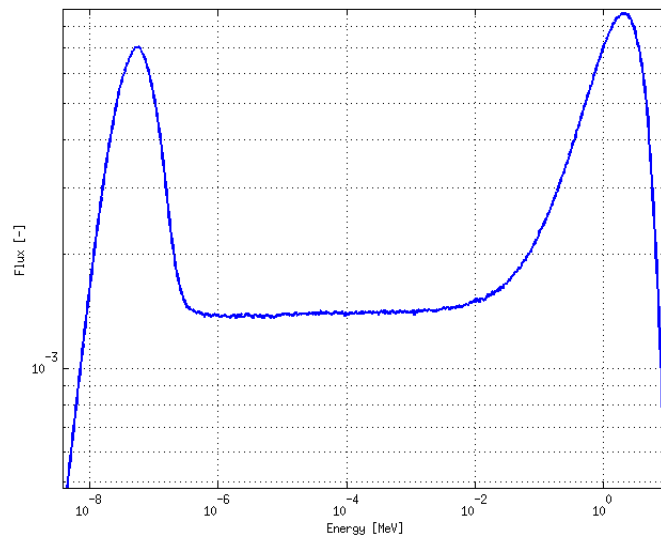


Figure 1: Flux Spectrum - U/H 1e-4%

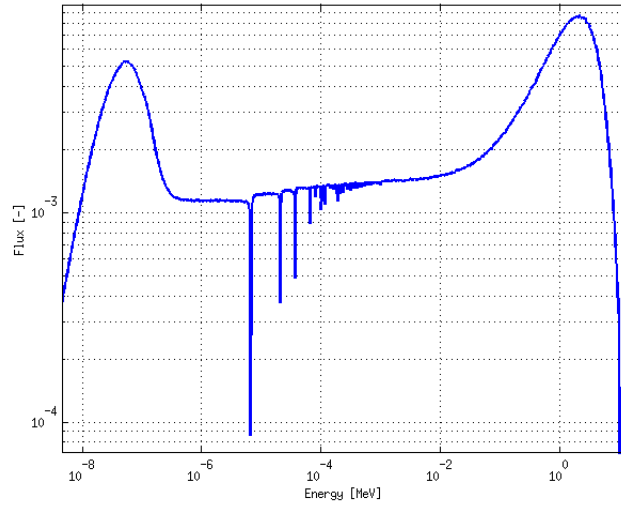


Figure 2: Flux Spectrum - U/H 10%

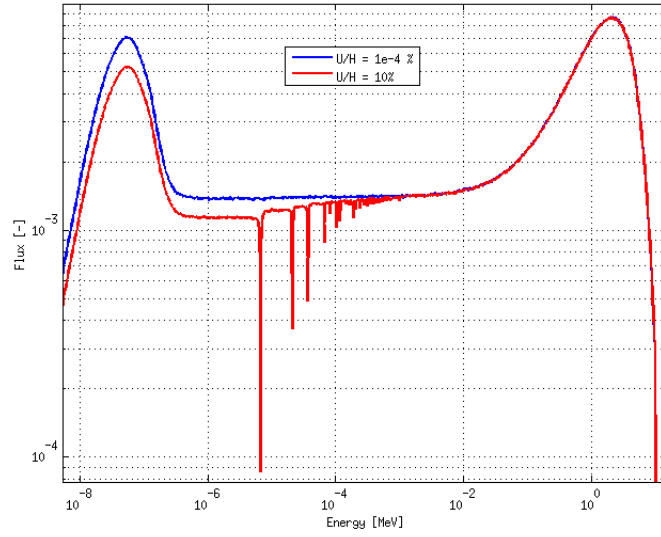


Figure 3: Flux Spectrum - U/H Comparison

Table 1: Effective Resonance Integrals - U-238 @ 300K

Energy Range [eV]	U/H = 1×10^{-6}	U/H = 0.10
0 - 1	21.24	20.90
1 - 6	1.47	1.46
6 - 10	127.00	13.83
10 - 25	66.57	6.93
25 - 50	41.07	5.24
50 - 100	12.24	2.87
100 - 1000	23.28	10.47

Doppler: Below are effective resonance integral results. All results generated with 1 million histories.

Table 2: Effective Resonance Integrals - U-238 @ 300K

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	21.21	20.90	18.42
1 - 6	1.47	1.46	1.41
6 - 10	49.26	13.83	4.06
10 - 25	25.36	6.93	2.16
25 - 50	17.95	5.24	1.63
50 - 100	8.02	2.87	0.82
100 - 1000	19.28	10.47	3.27

Table 3: Effective Resonance Integrals - U-238 @ 600K

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	21.21	20.90	18.41
1 - 6	1.47	1.46	1.41
6 - 10	55.42	15.08	4.14
10 - 25	28.78	7.60	2.21
25 - 50	20.06	5.86	1.66
50 - 100	8.73	3.31	0.90
100 - 1000	20.01	11.60	3.77

Table 4: Effective Resonance Integrals - U-238 @ 900K

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	21.21	20.90	18.41
1 - 6	1.47	1.47	1.42
6 - 10	59.67	16.16	4.22
10 - 25	31.08	8.19	2.25
25 - 50	31.52	6.06	1.69
50 - 100	9.14	3.62	0.96
100 - 1000	20.47	12.31	4.14

Table 5: Effective Resonance Integrals - U-238 @ 1200K

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	21.21	20.90	18.42
1 - 6	1.48	1.47	1.42
6 - 10	63.03	17.14	4.31
10 - 25	32.91	8.72	2.30
25 - 50	22.59	6.41	1.73
50 - 100	9.44	3.86	1.01
100 - 1000	20.74	12.84	4.46

Below are a set of 3 tables comparing the effective resonance integrals of 600K, 900K and 1200K with 300K. This will allow us to see the doppler effect on different resonances.

Table 6: Percent Difference of Effective Resonance Integrals - U-238 @ 600K (wrt to 300K)

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	0	0	-0.05
1 - 6	0	0	0
6 - 10	12.51	9.04	1.97
10 - 25	13.49	9.67	2.31
25 - 50	11.75	11.83	1.84
50 - 100	8.85	15.33	9.75
100 - 1000	3.78	10.79	15.29

Table 7: Percent Difference of Effective Resonance Integrals - U-238 @ 900K (wrt to 300K)

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	0	0	-0.05
1 - 6	0	0.68	0.71
6 - 10	21.13	16.85	3.94
10 - 25	22.56	18.18	4.17
25 - 50	75.60	15.65	3.68
50 - 100	13.97	26.13	17.07
100 - 1000	6.17	17.57	26.61

Table 8: Percent Difference of Effective Resonance Integrals - U-238 @ 1200K (wrt to 300K)

Energy Range [eV]	U/H = 1%	U/H = 10%	U/H = 100%
0 - 1	0	0	0
1 - 6	0.68	0.68	0.71
6 - 10	27.95	23.93	6.16
10 - 25	29.77	25.83	6.48
25 - 50	25.85	22.33	6.14
50 - 100	17.71	34.49	23.17
100 - 1000	7.57	22.64	36.39

A Sample Input File for Slowing Down Code

```
<?xml version="1.0"?>
<input>

<!-- Settings Information -->
  <settings>
    <histories> 1000000 </histories>
    <seed> 5 </seed>
    <source_type> 1 </source_type>
    <source_path> /home/bherman/Documents/Spring2012/211/SlowMC/lib/fission.h5 </source_path>
  </settings>

  <material>

    <nuclide N="1" A="1" thermal="true" >
      <path> /home/bherman/Documents/Spring2012/211/SlowMC/lib/H_1.h5 </path>
    </nuclide>

    <nuclide N="1" A="1" thermal="false" >
      <path> /home/bherman/Documents/Spring2012/211/SlowMC/lib/v_abs.h5 </path>
    </nuclide>

    <nuclide N="1" A="238" thermal="false" >
      <path> /home/bherman/Documents/Spring2012/211/SlowMC/lib/U_238_900.h5 </path>
    </nuclide>

  </material>

  <tallies>
    <tally type="micro_capture" isotope="3" >
      <Ebins> 1e-11 1e-6 6e-6 10e-6 25e-6 50e-6 100e-6 1000e-6 </Ebins>
    </tally>
    <tally type="flux" >
      <Ebins> 1e-11 1e-6 6e-6 10e-6 25e-6 50e-6 100e-6 1000e-6 </Ebins>
    </tally>
  </tallies>

</input>
```

B Main Codes

B.1 SLBW Code

```
% Single Level Breit Wigner xs generator for U-238
tic
% Inputs for user
isoname = 'U_238'; % isotope name
```

```

n_res = 14;           % number of resonances to read from file
T = 1200;             % temperature of resonances
sig_pot = 11.2934;    % potential cross section of isotope
A = 238;              % isotope atomic weight
maxE = 1000;          % max energy in eV

% load isotope file
load('U_238_res.txt');

% constants
k = 8.6173324e-5;

% extract resonance information from loaded dataa
Gg = U_238_res(1:n_res,4);
Gn = U_238_res(1:n_res,3);
E0 = U_238_res(1:n_res,1);

% create energy vector
dE = 0.001;
E = 10.^(log10(1e-5):dE:log10(20e6))';
sizeE = length(E);

% set psi-chi vectors
psi = zeros(sizeE,1);
chi = zeros(sizeE,1);

% initialize xs
xs = zeros(sizeE,2);
xs(:,2) = sig_pot;

% begin loop around resonances
for j = 1:n_res

    % psi-chi parameters
    G = Gg(j) + Gn(j);
    r = 2603911/E0(j)*((A+1)/A);
    q = sqrt(r*sig_pot);
    xi = G*sqrt(A/(4*k*T*E0(j)));
    x = 2*(E-E0(j))/G;

    % compute psi-chi functions
    y = ((x+1i)/2*xi);
    psichi = pi*xi/(2*sqrt(pi))*W(y); % compute complex value
    psi = real(psichi);
    chi = 2*imag(psichi);

    % compute xs
    xs(:,1) = (Gn(j)/G)*(Gg(j)/G)*sqrt(E0(j)./E).*(r*psi) + xs(:,1);
    xs(:,2) = (Gn(j)/G)*(Gn(j)/G)*(r*psi + q*chi) + xs(:,2);

    % display resonance info
    fprintf('Completed Resonance: %d\n',E0(j));

end

```



```

% append pseudo resonances (25eV spacing)
Elast = E0(length(E0));
clear E0;
E0 = Elast + 25;
Gg = 0.023;

% begin loop around building pseudo resonances
while E0 < maxE

    % compute neutron width
    Gn = 0.050*sqrt(E0/Elast);

    % psi-chi parameters
    G = Gg + Gn;
    r = 2603911/E0*((A+1)/A);
    q = sqrt(r*sig_pot);
    xi = G*sqrt(A/(4*k*T*E0));
    x = 2*(E-E0)/G;

    % compute psi-chi functions
    y = ((x+1i)/2*xi);
    psichi = pi*xi/(2*sqrt(pi))*W(y); % compute complex value
    psi = real(psichi);
    chi = imag(psichi);

    % compute xs
    xs(:,1) = (Gn/G)*(Gg/G)*sqrt(E0./E).*(r*psi) + xs(:,1);
    xs(:,2) = (Gn/G)*(Gn/G)*(r*psi + q*chi) + xs(:,2);

    % get next E0
    E0 = E0 + 25;

    % display resonance info
    fprintf('Completed Resonance: %d\n',E0);

end
toc

% change units on E
E = E/1e6;

% zero out xs over 1 keV
xs(:,1) = xs(:,1).*(E <= 1e-3);

% put 0.1 barns after
xs(:,1) = xs(:,1) + (E > 1e-3)*0.1;

% get size
sizeE = length(xs(:,1));

% get capture
xs_capt = xs(:,1);

```

```

% set scattering to 0
xs_scatt = zeros(sizeE,1);

% filename
hdfile = horzcat(isoname, '_', num2str(T), '.h5');

% write out hdf5 file
delete(hdfile);
h5create(hdfile, '/vecsize', 1);
h5write(hdfile, '/vecsize', sizeE);
h5create(hdfile, '/xs_scatt', sizeE);
h5write(hdfile, '/xs_scatt', xs_scatt);
h5create(hdfile, '/xs_capt', sizeE);
h5write(hdfile, '/xs_capt', xs_capt);
h5create(hdfile, '/E_width', 1);
h5write(hdfile, '/E_width', dE);

```

B.2 Thermal Scattering Kernel Generation Code

```

function [thermalcdf, Erat] = free_gas(A, T, sizeN, kTfactor)
% free gas thermal scattering kernel generation

% constants
k = 8.6173324e-5;
eta = (A+1)/(2*sqrt(A));
rho = (A-1)/(2*sqrt(A));
alpha = ((A-1)/(A+1))^2;

% create output space
p = zeros(sizeN, length(kTfactor));
thermalcdf = zeros(sizeN, length(kTfactor));

% begin loop around temperatures
for i = 1:length(kTfactor)

    % create energy space
    kT = k*T;
    E = kT*kTfactor(i);
    Ep = linspace(0, 5*E, sizeN);

    % compute probability
    p(:, i) = ((eta^2/2)*(erf(eta*sqrt(Ep/kT) - rho*sqrt(E/kT)) - ...
        erf(eta*sqrt(Ep/kT) + rho*sqrt(E/kT)) + ...
        exp((E-Ep)/kT).*(erf(eta*sqrt(E/kT) - rho*sqrt(Ep/kT)) + ...
        erf(eta*sqrt(E/kT) + rho*sqrt(Ep/kT))))).*(Ep >= E) + ...
        ((eta^2/2)*(erf(eta*sqrt(Ep/kT) - rho*sqrt(E/kT)) + ...
        erf(eta*sqrt(Ep/kT) + rho*sqrt(E/kT)) + ...
        exp((E-Ep)/kT).*(erf(eta*sqrt(E/kT) - rho*sqrt(Ep/kT)) - ...
        erf(eta*sqrt(E/kT) + rho*sqrt(Ep/kT))))).*(Ep < E);

    % multiply by alpha

```

```

p(:,i) = p(:,i)*(1-alpha);

% relative energy ratio
Erat = Ep/E;

% loop around energy ratio and integrate
for j = 2:length(Erat)

    % perform integral
    thermalcdf(j,i) = trapz(Erat(1:j),p(1:j,i));

end

% renormalize cdf
thermalcdf(:,i) = thermalcdf(:,i)/thermalcdf(length(thermalcdf),i);

% plot pdfs and cdfs
rn1 = rand(1,1);
rn2 = rand(1,1);
rn3 = rand(1,1);
figure(3)
xlabel('Scattered Energy Ratio (out/in)')
ylabel('Probability')
hold on
pdfplot = plot(Erat,p(:,i), 'Color',[rn1,rn2,rn3], 'LineWidth',2.0);
if i == 1
    legend(horzcat('E = ',num2str(kTfactor(i)), ' kT'))
else
    [LEGH,OBJH,OUTH,OUTM] = legend;
    legend([OUTH;pdfplot],OUTM{:},horzcat('E = ',num2str(kTfactor(i)), ' kT'));
end

figure(2)
xlabel('Scattered Energy Ratio (out/in)')
ylabel('Cumulative Probability')
hold on
cdfplot = plot(Erat,thermalcdf(:,i), 'Color',[rn1,rn2,rn3], 'LineWidth',2.0);
if i == 1
    legend(horzcat('E = ',num2str(kTfactor(i)), ' kT'))
else
    [LEGH,OBJH,OUTH,OUTM] = legend;
    legend([OUTH;cdfplot],OUTM{:},horzcat('E = ',num2str(kTfactor(i)), ' kT'));
end

end

%% Re-adjust cdf for sampling optimization
Enew = zeros(sizeN,length(kTfactor));
cdfnew = linspace(0,1,sizeN)';
thermalcdftemp = thermalcdf;

% correct thermal cdf for precision unique issues
for i = 1:size(thermalcdftemp,2)
    for j = 1:size(thermalcdftemp,1)-1

```

```

        if abs(thermalcdftemp(j,i) - thermalcdftemp(j+1,i)) < 1e-8
            thermalcdf(j+1,i) = 1.01*thermalcdf(j,i);
        end
    end
end

% create new energy vector
for i = 1:size(thermalcdf,2)
    Enew(:,i) = interp1(thermalcdf(:,i),Erat,cdfnew,'linear');
end

% bank to saved variables
thermalcdf = cdfnew;
Erat = Enew;
width = thermalcdf(2) - thermalcdf(1);

```

B.3 Slowing Down

```

program main
=====
!> @mainpage SlowMC: Slowing Down Monte Carlo
!>
!> @section Overview
!>
!> This program solves the slowing down neutron transport equation in either
!> infinite medium or effective two-region collision probability theory. It
!> models parts of the same physics performed by the NJOY data processing code.
!> This code is for strictly academic purposes and allows the user to see the
!> relative impact of physics in the generation of multigroup cross sections
!> and on flux spectra. This code currently uses the following external
!> libraries:
!> - HDF5 v1.8.#
!>
!> The package HDF5 can be downloaded from http://www.hdfgroup.org/HDF5/
!>
!> @section Compiling
!>
!> Compiling is as easy as running the Makefile with:
!>
!> @verbatim
!>     make xml-fortran
!>     make slowmc
!> @endverbatim
!>
!> @section Running
!>
!> To run SlowMC, execute the following:

```

```

!>
!> @verbatim
!>   slowmc
!> @endverbatim
!>
!=====

implicit none

! initialize problem
call initialize()

! run problem
call run_problem()

! finalize problem
call finalize()

! terminate program
stop

contains

!=====
! INITIALIZE
!> @brief high level routine for intializing problem
!=====

subroutine initialize()

  use hdf5
  use global,    only: seed,allocate_problem,mat,tal,emax,emin,time_init
  use input,     only: read_input
  use materials, only: compute_macroxs
  use output,    only: print_heading
  use timing,    only: timer_start,timer_stop

  ! local variables
  integer :: error ! hdf5 error
  real(8) :: rn    ! initial random number

  ! begin timer
  call timer_start(time_init)

  ! initialize the fortran hdf5 interface
  call h5open_f(error)

```

```

! print heading information
call print_heading()

! read input
call read_input()

! initialize random number generator
rn = rand(seed)

! precompute macroscopic cross section of materials
call compute_macroxs(mat)

! end timer
call timer_stop(time_init)

end subroutine initialize
=====
! RUN_PROBLEM
!> @brief main routine for executing the transport calculation
=====

subroutine run_problem()

  use global,      only: nhistories,mat,neut,eidx,emin,add_to_tallies,      &
  & bank_tallies,time_run
  use particle,    only: init_particle
  use physics,     only: sample_source,perform_physics,get_eidx
  use timing,      only: timer_start,timer_stop

  ! local variables
  integer :: i ! iteration counter

  ! begin timer
  call timer_start(time_run)

  ! begin loop over histories
  do i = 1,nhistories

    ! initialize history
    call init_particle(neut)

    ! sample source energy
    call sample_source()

    ! begin transport of neutron
    do while (neut%alive)

```

```

! call index routine for first tally
eidx = get_eidx(neut%E)

! record collision temp tally
call add_to_tallies()

! perform physics
call perform_physics()

! check for energy cutoff
if (neut%E < emin) neut%alive = .FALSE.

end do

! neutron is dead if out of transport loop (ecut or absorb) —> bank tally
call bank_tallies()

! print update to user
if (mod(i,nhistories/10) == 0) then
  write(*,'(/A,1X,I0,1X,A)') 'Simulated',i,'neutrons...'
end if

end do

! end timer
call timer_stop(time_run)

end subroutine run_problem

```

```

! FINALIZE
!> @brief routine that finalizes the problem

```

```

subroutine finalize()

  use global, only: finalize_tallies,deallocate_problem
  use hdf5
  use output, only: write_output

  ! local variables
  integer :: error ! hdf5 error

  ! calculate statistics on tallies
  call finalize_tallies()

```

```

! write output
call write_output()

! deallocate problem
call deallocate_problem()

! close the fortran interface
call h5close_f(error)

end subroutine finalize

end program main

```

C Module Files

C.1 Global

```

=====
! MODULE: global
!
!> @author Bryan Herman
!>
!> @brief Contains all of the global variables
=====

module global

  use materials, only: material_type
  use particle,   only: particle_type
  use tally,      only: tally_type
  use timing,     only: Timer

  implicit none
  save

  ! version information
  integer :: VERSION_MAJOR   = 0
  integer :: VERSION_MINOR   = 1
  integer :: VERSION_RELEASE = 1

  ! list all types
  type(material_type)          :: mat

```



```

type(particle_type)          :: neut
type(tally_type), allocatable :: tal(:)

! list history input information
integer :: nhistories
integer :: seed
integer :: source_type

! list global vars that are set during run
integer :: eidix      ! energy index for cross sections
integer :: n_tallies ! number of tallies

! set max and min energy
real(8) :: emin = 1e-11_8
real(8) :: emax = 20.0_8

! kT value base on 300K
real(8) :: kT = 8.6173324e-5_8*300*1.0e-6_8

! timers
type(Timer) :: time_init
type(Timer) :: time_run

contains

=====
! ALLOCATE_PROBLEM
!> @brief allocates global variables for calculation
=====

subroutine allocate_problem(n)

! formal variables
integer :: n ! size of tallies

! allocate tallies
if (.not.allocated(tal)) allocate(tal(n))

end subroutine allocate_problem

=====
! DEALLOCATE_PROBLEM
!> @brief deallocates global variables
=====

subroutine deallocate_problem()

```

```

use materials, only: deallocate_material
use tally,      only: deallocate_tally

! local variables
integer :: i ! loop counter

! deallocate material
call deallocate_material(mat)

! deallocate within tallies
do i = 1,n_tallies

    ! deallocate tally
    call deallocate_tally(tal(i))

end do

! deallocate tally variable
if (allocated(tal)) deallocate(tal)

end subroutine deallocate_problem

```

```

! ADD_TO_TALLIES
!> @brief routine that adds temporary value to tallies

```

```

subroutine add_to_tallies()

use tally, only: add_to_tally

! local variables
integer :: i ! loop counter
real(8) :: fact = 1.0_8 ! multiplier factor
real(8) :: totxs ! total macroscopic xs of material

! compute macroscopic cross section
totxs = sum(mat%totalxs(eidx,:))

! begin loop over tallies
do i = 1,n_tallies

    ! set multiplier
    select case (tal(i)%react_type)

        ! flux only
        case (0)

```

```

        fact = 1.0_8

! absorption
case(1)
    fact = sum(mat%absorxs(eidx,:))

! scattering
case(2)
    fact = sum(mat%scattxs(eidx,:))

! micro capture
case(3)
    fact = mat%isotopes(tal(i)%isotope)%xs_capt(eidx)
case DEFAULT
    fact = 1.0_8

end select

! call routine to add tally
call add_to_tally(tal(i),fact,totxs,neut%E)

end do

end subroutine add_to_tallies

```

```

! BANK_TALLIES
!> @brief routine that record temporary history information in tallies

```

```

subroutine bank_tallies()

    use tally, only: bank_tally

! local variables
integer :: i ! loop counter

! begin loop over tallies
do i = 1,n_tallies

    ! call routine to bank tally
    call bank_tally(tal(i))

end do

end subroutine bank_tallies

```

```

!=====
! FINALIZE_TALLIES
!> @brief routine that calls another routine to compute tally statistics
!=====

subroutine finalize_tallies()

  use tally, only: calculate_statistics

  ! local variables
  integer :: i ! loop counter

  ! begin loop over tallies
  do i = 1,n_tallies

    ! call routine to compute statistics
    call calculate_statistics(tal(i),nhistories)

  end do

end subroutine finalize_tallies

end module global

```

C.2 Input

```

!=====
! MODULE: input
!
!> @author Bryan Herman
!>
!> @brief Handles reading in the input xml file and intializing global vars
!=====

module input

  implicit none
  private
  public read_input

contains

!=====
! READ_INPUT
!> @brief Reads the input xml file and sets global variables

```

```

subroutine read_input

  use global,          only: nhistories,seed,source_type,mat,emin,emax,    &
&                      allocate_problem,tal,n_tallies
  use materials,       only: setup_material,load_source,load_isotope
  use tally,           only: set_user_tally,set_spectrum_tally
  use xml_data_input_t

  ! local variables
  logical              :: file_exists  ! see if file exists
  character(255)       :: filename     ! filename to open
  real(8)              :: N            ! temp number dens
  real(8)              :: A            ! temp atomic weight
  character(255)       :: path         ! path to isotope file
  logical              :: thermal      ! contains thermal lib
  integer              :: i            ! iteration counter
  integer              :: react_type   ! reaction type
  integer              :: isotope=0    ! isotope for micro mult
  real(8), allocatable :: Ebins(:)     ! tally energy bins

  ! check for input file
  filename = "input.xml"
  inquire(FILE=trim(filename), EXIST=file_exists)
  if (.not. file_exists) then
    write(*,*) 'Cannot read input file!'
    stop
  else

    ! tell user
    write(*,'(A/)') "Reading INPUT XML file..."

  end if

  ! read in input file
  call read_xml_file_input_t(trim(filename))

  ! read in settings
  nhistories = settings_%histories
  seed = settings_%seed
  source_type = settings_%source_type
  mat%nisotopes = size(material_(1)%nuclides)

  ! load the source
  call load_source(mat,source_type,settings_%source_path)

```

```

! set up the material object
call setup_material(mat,emin,emax)

! begin loop over isotope materials
do i = 1,mat%n_isotopes

    ! extract info
    N = material_(1)%nuclides(i)%N
    A = material_(1)%nuclides(i)%A
    path = material_(1)%nuclides(i)%path
    thermal = material_(1)%nuclides(i)%thermal

    ! load the isotope into memory
    call load_isotope(mat,N,A,path,thermal)

end do

! get size of tallies
if (.not.associated(tallies_%tally)) then
    n_tallies = 1
else
    n_tallies = size(tallies_%tally) + 1
end if

! allocate problem
call allocate_problem(n_tallies)

! begin loop over tallies
do i = 1,n_tallies-1

    ! set reaction type
    select case(trim(tallies_%tally(i)%type))
        case('flux')
            react_type = 0
        case('absorption')
            react_type = 1
        case('scattering')
            react_type = 2
        case('micro_capture')
            react_type = 3
            isotope = tallies_%tally(i)%isotope
        case DEFAULT
            react_type = 0
    end select

    ! preallocate Ebins
    if(.not. allocated(Ebins)) allocate(Ebins(size(tallies_%tally(i)%Ebins)))

```

```

! set Ebins
Ebins = tallies_%tally(i)%Ebins

! set up user-defined tallies
call set_user_tally(tal(i),Ebins,size(Ebins),react_type,isotope)

! deallocate Ebins
if(allocated(Ebins)) deallocate(Ebins)

end do

! set up spectrum tally
call set_spectrum_tally(tal(n_tallies),emax,emin)

end subroutine read_input
end module input

```

C.3 Materials

```

=====
! MODULE: materials
!
!> @author Bryan Herman
!>
!> @brief Contains information about the isotopics of problem
=====

module materials

  implicit none
  private
  public :: setup_material,load_source,load_isotope,compute_macroxs,      &
  &          deallocate_material

  type :: source_type

    real(8), allocatable :: E(:)      ! energy range for fission source
    real(8)               :: cdf_width ! width of cdf bins from 0 to 1

  end type source_type

  type :: thermal_type

```

```

integer          :: kTsize      ! size of kT vector
integer          :: cdfsize     ! size of cdf
real(8), allocatable :: kTvec(:) ! vector of kT values
real(8), allocatable :: Erat(:, :) ! energy
real(8)          :: cdf_width  ! width of cdf interval from 0 to 1

end type thermal_type

type :: iso_type

real(8)          :: N          ! number density
real(8)          :: A          ! atomic weight
real(8)          :: alpha      !  $(A-1)^2/(A+1)^2$ 
real(8), allocatable :: xs_capt(:) ! capture micro xs
real(8), allocatable :: xs_scatt(:) ! scattering micro xs

logical          :: thermal     ! thermal scatterer
type(thermal_type) :: thermal_lib ! thermal library

end type iso_type

type, public :: material_type

type(source_type)      :: source      ! the source of neutrons
type(iso_type), allocatable :: isotopes(:) ! 1-D array of isotopes in mat
integer                :: nisotopes   ! number of isotopes in mat
integer                :: curr_iso    ! the current isotope
integer                :: npts        ! number of points in energy
real(8)                :: E_width     ! width of energy interval
real(8)                :: E_min       ! min energy
real(8)                :: E_max       ! max energy
real(8), allocatable   :: totalxs(:, :) ! array of macroscopic tot xs
real(8), allocatable   :: scattxs(:, :) ! array of macroscopic scat xs
real(8), allocatable   :: absorxs(:, :) ! array of macroscopic abs xs

end type material_type

contains

=====
! SET_UP_MATERIALS
!> @brief routine that initializes the materials
!=====

subroutine setup_material(this, emin, emax)

    ! formal variables

```



```

type(material_type) :: this      ! a material
real(8)              :: emin      ! minimum energy to consider
real(8)              :: emax      ! maximum energy to consider

! allocate isotopes array
if (.not. allocated(this%isotopes)) allocate(this%isotopes(this%nisotopes))

! set up current isotope index
this%curr_iso = 1

! set energy bounds
this%E_min = emin
this%E_max = emax

end subroutine setup_material

```

```

! LOAD_ISOTOPE
!> @brief routine that loads isotope properties, xs, etc. into memory
!

```

```

subroutine load_isotope(this,N,A,path,thermal)

use hdf5

! formal variables
type(material_type),target :: this      ! a material
real(8)                  :: N           ! number density
real(8)                  :: A           ! atomic weight
character(len=255)       :: path        ! path to isotope
logical                  :: thermal     ! contains a thermal lib

! local variables
integer                  :: error        ! hdf5 error
integer(HID_T)           :: hdf5_file   ! hdf5 file id
integer(HID_T)           :: dataset_id  ! hdf5 dataset id
integer(HSIZE_T), dimension(1) :: dim1    ! dimension of hdf5 var
integer(HSIZE_T), dimension(2) :: dim2    ! dimension of hdf5 var
integer                  :: vecsize     ! vector size
type(thermal_type), pointer :: therm

! display to user
write(*,*) 'Loading isotope:',this%curr_iso

! set parameters
this%isotopes(this%curr_iso)%N = N
this%isotopes(this%curr_iso)%A = A

```

```

this%isotopes(this%curr_iso)%alpha = ((A-1)/(A+1))*2
this%isotopes(this%curr_iso)%thermal = thermal

! open up hdf5 file
call h5fopen_f(trim(path),H5F_ACC_RDWR_F,hdf5_file,error)

! read size of vector
call h5dopen_f(hdf5_file,"/vecsize",dataset_id,error)
dim1 = (/1/)
call h5dread_f(dataset_id,H5T_NATIVE_INTEGER,vecsize,dim1,error)
call h5dclose_f(dataset_id,error)

! allocate all xs vectors
if (.not.allocated(this%isotopes(this%curr_iso)%xs_scat)) &
&      allocate(this%isotopes(this%curr_iso)%xs_scat(vecsize))
if (.not.allocated(this%isotopes(this%curr_iso)%xs_capt)) &
&      allocate(this%isotopes(this%curr_iso)%xs_capt(vecsize))

! keep the size
this%npts = vecsize

! zero out xs vectors
this%isotopes(this%curr_iso)%xs_scat = 0.0_8
this%isotopes(this%curr_iso)%xs_capt = 0.0_8

! read in xs
call h5dopen_f(hdf5_file,"/xs_scat",dataset_id,error)
dim1 = (/vecsize/)
call h5dread_f(dataset_id,H5T_NATIVE_DOUBLE, &
&      this%isotopes(this%curr_iso)%xs_scat,dim1,error)
call h5dclose_f(dataset_id,error)
call h5dopen_f(hdf5_file,"/xs_capt",dataset_id,error)
dim1 = (/vecsize/)
call h5dread_f(dataset_id,H5T_NATIVE_DOUBLE, &
&      this%isotopes(this%curr_iso)%xs_capt,dim1,error)
call h5dclose_f(dataset_id,error)

! get energy interval width
call h5dopen_f(hdf5_file,"/E_width",dataset_id,error)
dim1 = (/1/)
call h5dread_f(dataset_id,H5T_NATIVE_DOUBLE,this%E_width,dim1,error)
call h5dclose_f(dataset_id,error)

! check for thermal scattering kernel and load that
if (this%isotopes(this%curr_iso)%thermal) then

! set pointer

```

```

therm => this%isotopes(this%curr_iso)%thermal_lib

! load sizes
call h5dopen_f(hdf5_file, "/kTsize", dataset_id, error)
dim1 = (/1/)
call h5dread_f(dataset_id, H5T_NATIVE_INTEGER, therm%kTsize, dim1, error)
call h5dclose_f(dataset_id, error)
call h5dopen_f(hdf5_file, "/cdfsize", dataset_id, error)
dim1 = (/1/)
call h5dread_f(dataset_id, H5T_NATIVE_INTEGER, therm%cdfsize, dim1, error)
call h5dclose_f(dataset_id, error)

! read in cdf width
call h5dopen_f(hdf5_file, "/cdf_width", dataset_id, error)
dim1 = (/1/)
call h5dread_f(dataset_id, H5T_NATIVE_DOUBLE, therm%cdf_width, dim1, error)
call h5dclose_f(dataset_id, error)

! preallocate vectors
if(.not.allocated(therm%kTvec)) allocate(therm%kTvec(therm%kTsize))
if(.not.allocated(therm%Eratt)) allocate(therm%Eratt(therm%cdfsize, therm%←
    kTsize))

! read in vectors
call h5dopen_f(hdf5_file, "/kT", dataset_id, error)
dim1 = (/therm%kTsize/)
call h5dread_f(dataset_id, H5T_NATIVE_DOUBLE, therm%kTvec, dim1, error)
call h5dclose_f(dataset_id, error)
call h5dopen_f(hdf5_file, "/Eratt", dataset_id, error)
dim2 = (/therm%cdfsize, therm%kTsize/)
call h5dread_f(dataset_id, H5T_NATIVE_DOUBLE, therm%Eratt, dim2, error)
call h5dclose_f(dataset_id, error)

end if

! close hdf5 file
call h5fclose_f(hdf5_file, error)

! increment isotope counter
this%curr_iso = this%curr_iso + 1

end subroutine load_isotope

```

```

! LOAD_SOURCE
!> @brief routine to load fission source into memory

```

```

subroutine load_source(this,source_type,source_path)

  use hdf5

  ! formal variables
  type(material_type) :: this      ! a material
  integer              :: source_type ! 0 – fixed, 1 – fission
  character(len=255)   :: source_path ! path to source file

  ! local variables
  integer              :: error      ! hdf5 error
  integer(HID_T)       :: hdf5_file ! hdf5 file id
  integer(HID_T)       :: dataset_id ! hdf5 dataset id
  integer(HSIZE_T), dimension(1) :: dim1 ! dimension of hdf5 var
  integer              :: vecsize   ! vector size for fission

  ! check for fission source
  if (source_type == 1) then

    ! open the fission source file
    call h5fopen_f(trim(source_path),H5F_ACC_RDWR_F,hdf5_file,error)

    ! open dataset and read in vector size
    call h5dopen_f(hdf5_file,"/vecsize",dataset_id,error)
    dim1 = (/1/)
    call h5dread_f(dataset_id,H5T_NATIVE_INTEGER,vecsize,dim1,error)
    call h5dclose_f(dataset_id,error)

    ! open dataset and read in width of cdf interval
    call h5dopen_f(hdf5_file,"/cdf_width",dataset_id,error)
    dim1 = (/1/)
    call h5dread_f(dataset_id,H5T_NATIVE_DOUBLE,this%source%cdf_width,dim1, &
    & error)
    call h5dclose_f(dataset_id,error)

    ! preallocate vectors in source object
    if(.not.allocated(this%source%E)) allocate(this%source%E(vecsize))

    ! open dataset and read in energy vector
    call h5dopen_f(hdf5_file,"/E",dataset_id,error)
    dim1 = (/vecsize/)
    call h5dread_f(dataset_id,H5T_NATIVE_DOUBLE,this%source%E,dim1,error)
    call h5dclose_f(dataset_id,error)

    ! close the file

```

```

        call h5fclose_f(hdf5_file,error)

    end if

end subroutine load_source

=====
! COMPUTE_MACROXS
!> @brief routine to pre-compute macroscopic cross sections
=====

subroutine compute_macroxs(this)

    ! formal variables
    type(material_type),target :: this ! a material

    ! local variables
    integer :: i ! loop counter
    type(iso_type), pointer :: iso ! pointer to current isotope

    ! allocate xs arrays
    if (.not.allocated(this%totalxs)) &
    & allocate(this%totalxs(this%npts,this%nisotopes))
    if (.not.allocated(this%scattxs)) &
    & allocate(this%scattxs(this%npts,this%nisotopes))
    if (.not.allocated(this%absorxs)) &
    & allocate(this%absorxs(this%npts,this%nisotopes))

    ! zero out total xs
    this%totalxs = 0.0_8

    ! begin loop over isotopes
    do i = 1,this%nisotopes

        ! set pointer to isotope
        iso => this%isotopes(i)

        ! multiply microscopic cross section by number density and append
        this%scattxs(:,i) = iso%N*(iso%xs_scat)
        this%absorxs(:,i) = iso%N*(iso%xs_capt)
        this%totalxs(:,i) = iso%N*(iso%xs_capt + iso%xs_scat)

    end do

end subroutine compute_macroxs

=====

```

```

! DEALLOCATE_MATERIAL
!> @brief routine to deallocate a material
!
=====

subroutine deallocate_material(this)

! formal variables
type(material_type) :: this ! a material

! local variables
integer :: i ! loop counter

! deallocate source information
if (allocated(this%source%E)) deallocate(this%source%E)

! begin loop over isotopes for deallocation
do i = 1,this%nisotopes

! deallocate thermal library
if (allocated(this%isotopes(i)%thermal_lib%kTvec)) deallocate &
& (this%isotopes(i)%thermal_lib%kTvec)
if (allocated(this%isotopes(i)%thermal_lib%Erat)) deallocate &
& (this%isotopes(i)%thermal_lib%Erat)

! deallocate xs
if (allocated(this%isotopes(i)%xs_scat)) deallocate &
& (this%isotopes(i)%xs_scat)
if (allocated(this%isotopes(i)%xs_capt)) deallocate &
& (this%isotopes(i)%xs_capt)

end do

! deallocate isotopes
if (allocated(this%isotopes)) deallocate(this%isotopes)

! deallocate macro xs
if (allocated(this%totalxs)) deallocate(this%totalxs)
if (allocated(this%scattxs)) deallocate(this%scattxs)
if (allocated(this%absorxs)) deallocate(this%absorxs)

end subroutine deallocate_material

end module materials

```

C.4 Output

```

!=====
! MODULE: output
!
!> @author Bryan Herman
!>
!> @brief Contains routines for outputting major info to user
!=====

module output

  implicit none
  private
  public :: print_heading, write_output

contains

!=====
! PRINT_HEADING
!> @brief prints the code heading and run information
!=====

  subroutine print_heading()

    use global, only: VERSION_MAJOR, VERSION_MINOR, VERSION_RELEASE

    ! local variables
    character(len=10) :: today_date
    character(len=8)  :: today_time

    ! write header
    write(*, FMT='(/9(A/))') &
& ' .d8888b. 888      888b      d888 .d8888b. ' , &
& 'd88P Y88b 888      8888b      d8888 d88P Y88b ' , &
& 'd88P Y88b 888      8888b      d8888 d88P Y88b ' , &
& 'd88P Y88b 888      8888b      d8888 d88P Y88b ' , &
& ' "Y888b. 888 .d88b. 888 888 888 888Y88888P888 888 ' , &
& '      "Y88b. 888 d88""88b 888 888 888 888 Y888P 888 888 ' , &
& '      "888 888 888 888 888 888 888 888 Y8P 888 888 888 ' , &
& 'Y88b d88P 888 Y88..88P Y88b 888 d88P 888 " 888 Y88b d88P ' , &
& ' "Y8888P" 888 "Y88P" "Y8888888P" 888      888 "Y8888P" '

    ! Write version information
    write(*, FMT=*) &
& '      Developed At: Massachusetts Institute of Technology' &
    write(*, FMT='(6X,"Version:",7X,I1,".",I1,".",I1)') &
&      VERSION_MAJOR, VERSION_MINOR, VERSION_RELEASE
  end subroutine print_heading

```

```

! Write the date and time
call get_today(today_date, today_time)
write(*, FMT='(6X,"Date/Time:",5X,A,1X,A)' )
&      trim(today_date), trim(today_time)

! write out divider
write(*,FMT='(A/)' ) '-----'

end subroutine print_heading

```

```

! WRITE_OUTPUT
!> @brief routine that writes timing info and hdf5 file

```

```

subroutine write_output()

  use global, only: time_init,time_run,tal,n_tallies
  use hdf5

  ! local variables
  integer          :: i           ! loop counter
  integer          :: error       ! hdf5 error
  integer(HID_T)   :: hdf5file    ! hdf5 file
  integer(HID_T)   :: dataspace_id ! dataspace identifier
  integer(HID_T)   :: dataset_id  ! dataset identifier
  integer(HID_T)   :: group_id    ! group id
  integer(HSIZE_T), dimension(1) :: dim1      ! vector for hdf5 dims
  character(11)    :: talnum      ! tally number

  ! write results header
  write(*,'(A/,A/,A/,)' ) "Results","-----"

  ! write timing information
  write(*,100) "Initialization time",time_init%elapsed
  write(*,100) "Transport time",time_run%elapsed
  write(*,*)

  ! format for write statements
100 format (1X,A,T35,"=",ES11.4," seconds")

  ! open up output hdf5 file
  call h5fcreate_f("output.h5",H5F_ACC_TRUNC_F,hdf5file,error)

  ! begin loop around tallies to write out
  do i = 1,n_tallies

```



```

! get tally number
write (talnum, '(I11)') i
talnum = adjustl(talnum)

! open up a group
call h5gcreate_f(hdfile,"tally_"//trim(talnum),group_id,error)

! write mean
dim1 = (/size(tal(i)%mean)/)
call h5screate_simple_f(1,dim1,dataspace_id,error)
call h5dcreate_f(hdfile,"tally_"//trim(talnum)//"/mean",H5T_NATIVE_DOUBLE&
&
,dataspace_id,dataset_id,error)
call h5dwrite_f(dataset_id,H5T_NATIVE_DOUBLE,tal(i)%mean,dim1,error)
call h5sclose_f(dataspace_id,error)
call h5dclose_f(dataset_id,error)

! write standard deviation
dim1 = (/size(tal(i)%std)/)
call h5screate_simple_f(1,dim1,dataspace_id,error)
call h5dcreate_f(hdfile,"tally_"//trim(talnum)//"/std",H5T_NATIVE_DOUBLE &
&
,dataspace_id,dataset_id,error)
call h5dwrite_f(dataset_id,H5T_NATIVE_DOUBLE,tal(i)%std,dim1,error)
call h5sclose_f(dataspace_id,error)
call h5dclose_f(dataset_id,error)

! only write energy edges if a user tally
if (.not.tal(i)%flux_tally) then

! write tally data to file
dim1 = (/size(tal(i)%E)/)
call h5screate_simple_f(1,dim1,dataspace_id,error)
call h5dcreate_f(hdfile,"tally_"//trim(talnum)//"/E",H5T_NATIVE_DOUBLE,&
&
,dataspace_id,dataset_id,error)
call h5dwrite_f(dataset_id,H5T_NATIVE_DOUBLE,tal(i)%E,dim1,error)
call h5sclose_f(dataspace_id,error)
call h5dclose_f(dataset_id,error)

end if

! close the group
call h5gclose_f(group_id,error)

end do

! close the file
call h5fclose_f(hdfile,error)

```

```

end subroutine write_output

!=====
! GET_TODAY
!> @brief calculates information about date/time of run
!=====

subroutine get_today(today_date, today_time)

    character(10), intent(out) :: today_date
    character(8),  intent(out) :: today_time

    integer          :: val(8)
    character(8)     :: date_
    character(10)    :: time_
    character(5)     :: zone

    call date_and_time(date_, time_, zone, val)
    ! val(1) = year  (YYYY)
    ! val(2) = month (MM)
    ! val(3) = day   (DD)
    ! val(4) = timezone
    ! val(5) = hours  (HH)
    ! val(6) = minutes (MM)
    ! val(7) = seconds (SS)
    ! val(8) = milliseconds

    if (val(2) < 10) then
        if (val(3) < 10) then
            today_date = date_(6:6) // "/" // date_(8:8) // "/" // date_(1:4)
        else
            today_date = date_(6:6) // "/" // date_(7:8) // "/" // date_(1:4)
        end if
    else
        if (val(3) < 10) then
            today_date = date_(5:6) // "/" // date_(8:8) // "/" // date_(1:4)
        else
            today_date = date_(5:6) // "/" // date_(7:8) // "/" // date_(1:4)
        end if
    end if
    today_time = time_(1:2) // ":" // time_(3:4) // ":" // time_(5:6)

end subroutine get_today

end module output

```

C.5 Particle

```
!=====!  
! MODULE: particle  
!  
!> @author Bryan Herman  
!>  
!> @brief Contains information about the particle that is transporting  
!=====!  
  
module particle  
  
    implicit none  
    private  
    public :: init_particle  
  
    type, public :: particle_type  
  
        real(8) :: E      ! particle's energy  
        logical :: alive  ! am i alive?  
  
    end type particle_type  
  
contains  
  
!=====!  
! INIT_PARTICLE  
!> @brief routine to initialize a particle  
!=====!  
  
subroutine init_particle(this)  
  
    ! formal variables  
    type(particle_type) :: this ! a particle  
  
    ! initialize  
    this%E = 0.0_8  
    this%alive = .true.  
  
end subroutine init_particle  
  
end module particle
```

C.6 Physics

```

!=====
! MODULE: physics
!
!> @author Bryan Herman
!>
!> @brief Contains routines to model the physics of the problem
!=====

module physics

    implicit none
    private
    public :: sample_source,perform_physics,get_eidx

contains

!=====
! SAMPLE_SOURCE
!> @brief routine to sample source from cdf
!=====

    subroutine sample_source()

        use global, only: mat,neut

        ! local variables
        integer :: idx ! index for sampling
        real(8) :: rn ! sampled random number

        ! sample a random number
        rn = rand(0)

        ! compute index in cdf
        idx = ceiling(rn / mat%source%cdf_width) + 1

        ! bounds checker
        if (idx > size(mat%source%E)) then
            write(*,*) 'Bounds error on source samplings'
            write(*,*) 'Random number:',rn
            write(*,*) 'Index Location:',idx
            stop
        end if

        ! extract that E and set it to neutron
        neut%E = mat%source%E(idx)

    end subroutine sample_source

```

```

!=====
! PERFORM_PHYSICS
!> @brief high level routine to perform transport physics
!=====

subroutine perform_physics()

  use global, only: neut

  integer :: isoidx    ! isotope index
  integer :: reactid   ! reaction id

  ! sample isotope
  isoidx = sample_isotope()

  ! sample reaction in isotope
  reactid = sample_reaction(isoidx)

  ! perform reaction
  if (reactid == 1) then ! absorption
    neut%alive = .FALSE.
  else if (reactid == 2) then ! scattering
    call elastic_scattering(isoidx)
  else
    write(*,*) "Something is wrong after isotope sampling"
    stop
  end if

end subroutine perform_physics

!=====
! GET_EIDX
!> @brief function to compute the index in unionized energy grid
!=====

function get_eidx(E) result(eidx)

  use global, only: mat

  ! formal variables
  real(8)          :: E      ! neutron's energy
  integer          :: eidx   ! the energy index

  ! compute index
  eidx = ceiling((log10(E) - log10(mat%E_min))/mat%E_width) + 1

```

```

! check bounds
if (eidx == 0 .or. eidx >= mat%npts) then
    write(*,*) 'Energy index out of bounds!'
    write(*,*) 'Energy:', E
    write(*,*) 'Width:', mat%E_width
    stop
end if

end function get_eidx

```

```

! SAMPLE_ISOTOPE
!> @brief function to sample interaction isotope
!

```

```

function sample_isotope() result(isoidx)

    use global, only: mat, eidx

    ! formal variables
    integer :: isoidx ! the index of the isotope sampled

    ! local variables
    real(8), allocatable :: pmf(:) ! probability mass function
    real(8), allocatable :: cdf(:) ! cumulative distribution function
    real(8) :: rn ! sampled random number
    integer :: i ! iteration counter

    ! allocate pmf and cdf
    if(.not. allocated(pmf)) allocate(pmf(mat%nisotopes+1))
    if(.not. allocated(cdf)) allocate(cdf(mat%nisotopes+1))

    ! set both to zero
    pmf = 0.0_8
    cdf = 0.0_8

    ! create pmf at that energy index
    pmf(2:size(pmf)) = mat%totalxs(eidx,:) / sum(mat%totalxs(eidx,:))

    ! create cdf from pmf
    do i = 1, size(pmf)
        cdf(i) = sum(pmf(1:i))
    end do

    ! sample random number
    rn = rand(0)

```

```

! do linear table search on cdf to find which isotope
do i = 1,size(cdf)
  if (rn <= cdf(i)) then
    isoidx = i - 1
    exit
  end if
end do

! check iso
if (isoidx == 0) then
  isoidx = 1
end if

! deallocate pmf and cdf
if (allocated(pmf)) deallocate(pmf)
if (allocated(cdf)) deallocate(cdf)

end function sample_isotope

```

```

! SAMPLE_REACTION
!> @brief function to sample reaction type

```

```

function sample_reaction(isoidx) result(reactid)

  use global, only: mat,eidx

  ! formal variables
  integer :: isoidx    ! the sampled isotope index
  integer :: reactid   ! the id of the reaction type

  ! local variables
  real(8) :: pmf(3)    ! probability mass function
  real(8) :: cdf(3)    ! cumulative distribution function
  real(8) :: rn        ! sampled random number
  integer :: i         ! iteration counter

  ! set up pmf
  pmf = (/0.0_8,mat%isotopes(isoidx)%xs_capt(eidx),
&
&                                     mat%isotopes(isoidx)%xs_scatt(eidx)/)

  ! normalize pmf
  pmf = pmf / sum(pmf)

  ! compute cdf
  do i = 1,3

```

```

        cdf(i) = sum(pmf(1:i))
    end do

    ! sample random number
    rn = rand(0)

    ! perform linear table search
    do i = 1,3
        if (rn < cdf(i)) then
            reactid = i - 1
            exit
        end if
    end do

end function sample_reaction

```

```

! ELASTIC_SCATTERING
!> @brief routine to perform thermal/asymptotic elastic scattering physics
!

```

```

subroutine elastic_scattering(isoidx)

    use global, only: neut,mat,kT

    ! formal variables
    integer :: isoidx ! isotope sampled index

    ! local variables
    integer :: i      ! iteration counter
    integer :: idx    ! index in cdf vector
    integer :: kTidx  ! index in kT vector
    real(8) :: rn     ! sampled random number
    real(8) :: EkT    ! energy / kT
    real(8) :: Eint   ! interpolated E value
    real(8), allocatable :: Evec(:)

    ! sample random number
    rn = rand(0)

    ! check for thermal scattering
    if (neut%E < 4e-6_8 .and. mat%isotopes(isoidx)%thermal) then

        ! get index in cdf
        idx = ceiling(rn/mat%isotopes(isoidx)%thermal_lib%cdf_width)

        ! check index

```



```

if (idx == 0) idx = 1

! preallocate energy vector
if (.not.allocated(Evec))
& allocate(Evec(size(mat%isotopes(isoidx)%thermal_lib%kTvec)))

! set possible energy ratios vector
Evec = mat%isotopes(isoidx)%thermal_lib%Erat(idx,:)

! get energy in kT units
EkT = neut%E/kT

! find index in kT space
do i = 1,size(mat%isotopes(isoidx)%thermal_lib%kTvec)
  if (EkT < mat%isotopes(isoidx)%thermal_lib%kTvec(i)) then
    kTidx = i
    exit
  end if
end do

! interpolate on energy value
if (kTidx == 1) then
  neut%E = Evec(kTidx)
else
  ! perform linear interpolation on kT value
  Eint = Evec(kTidx-1) + (EkT -
&      mat%isotopes(isoidx)%thermal_lib%kTvec(kTidx-1))*((Evec(kTidx) &
&      - Evec(kTidx-1))/(mat%isotopes(isoidx)%thermal_lib%kTvec(kTidx) &
&      - mat%isotopes(isoidx)%thermal_lib%kTvec(kTidx-1)))

  ! multiply by incoming energy
  neut%E = neut%E*Eint

end if

! deallocate energy vector
if(allocated(Evec)) deallocate(Evec)

else

  ! perform asymptotic elastic scattering
  neut%E = neut%E - neut%E*(1-mat%isotopes(isoidx)%alpha)*rn;

end if

end subroutine elastic_scattering

```

```
end module physics
```

C.7 Tally

```
!=====!  
! MODULE: tally  
!  
!> @author Bryan Herman  
!>  
!> @brief Contains information about tallying quantities  
!=====!  
  
module tally  
  
  implicit none  
  private  
  public :: set_spectrum_tally, add_to_tally, bank_tally, deallocate_tally, &  
  &          set_user_tally, calculate_statistics  
  
  type, public :: tally_type  
  
    real(8), allocatable :: E(:)      ! user defined energy structure  
    real(8), allocatable :: val(:)    ! the temporary value  
    real(8), allocatable :: sum(:)    ! the sum for the mean and var  
    real(8), allocatable :: sum_sq(:) ! the sum for the variable  
    real(8), allocatable :: mean(:)   ! mean of tallies  
    real(8), allocatable :: std(:)    ! standard deviation of tallies  
    logical :: flux_tally = .false.   ! is this the flux tally  
    integer :: nbins                  ! number of tally regions  
    real(8) :: width                  ! the uniform width  
    real(8) :: emax                   ! max e  
    real(8) :: emin                   ! min e  
    integer :: react_type              ! reaction type id  
    integer :: isotope                 ! isotope number  
  
  end type tally_type  
  
contains  
  
!=====!  
! SET_USER_TALLY  
!> @brief routine to initialize user-defined tallies  
!=====!  
  
  subroutine set_user_tally(this, Ebins, n, react_type, isotope)
```

```

! formal variables
type(tally_type) :: this      ! a tally
integer          :: n         ! size of Ebins
integer          :: react_type ! reaction type
integer          :: isotope   ! isotope for multiplier
real(8)          :: Ebins(n)  ! vector of energy bins

! preallocate user-defined energy structure
if (.not.allocated(this%E)) allocate(this%E(n))

! set energy structure
this%E = Ebins

! set reaction type
this%react_type = react_type

! set isotope
this%isotope = isotope

! preallocate vectors
if(.not.allocated(this%val)) allocate(this%val(n-1))
if(.not.allocated(this%sum)) allocate(this%sum(n-1))
if(.not.allocated(this%sum_sq)) allocate(this%sum_sq(n-1))

! zero out tallies
this%val = 0.0_8
this%sum = 0.0_8
this%sum_sq = 0.0_8

end subroutine set_user_tally
=====
! SET_SPECTRUM_TALLY
!> @brief routine to initialize all tallies
=====

subroutine set_spectrum_tally(this,emax,emin)

! formal variables
type(tally_type) :: this      ! a tally
real(8)          :: emax      ! max e
real(8)          :: emin      ! min e

! set up automatic flux tally
this%flux_tally = .true.
this%nbins = 1000

```

```

this%emax = emax
this%emin = emin
this%width = (log10(emax) - log10(emin))/dble(this%nbins)

! preallocate vectors
if(.not.allocated(this%val)) allocate(this%val(1000))
if(.not.allocated(this%sum)) allocate(this%sum(1000))
if(.not.allocated(this%sum_sq)) allocate(this%sum_sq(1000))

! zero out tallies
this%val = 0.0_8
this%sum = 0.0_8
this%sum_sq = 0.0_8

end subroutine set_spectrum_tally

```

```

! ADD_TO_TALLY
!> @brief routine to add quantities during transport of a particle
!

```

```

subroutine add_to_tally(this,fact,totxs,E)

! formal variables
type(tally_type) :: this      ! a tally
real(8)          :: fact      ! multiplier for tally
real(8)          :: totxs     ! totalxs
real(8)          :: E         ! neutron energy

! local variables
integer :: i      ! iteration counter
integer :: idx=0  ! index in tally grid

! use uniform grid sampling if flux tally
if (this%flux_tally) then

    ! calculate index
    idx = ceiling((log10(E) - log10(this%emin))/this%width)

else

    ! check for output bounds
    if (E < minval(this%E) .or. E > maxval(this%E)) return

    ! begin loop around energy vector to get index
    do i = 1,size(this%E)
        if (E < this%E(i)) then

```

```

        idx = i - 1
        exit
    end if
end do

end if

! add to tally
if (idx /= 0) this%val(idx) = this%val(idx) + fact/totxs

end subroutine add_to_tally

```

```

! BANK_TALLY
!> @brief routine to bank a histories tallies

```

```

subroutine bank_tally(this)

! formal variables
type(tally_type) :: this ! a tally

! record to sums
this%sum      = this%sum      + this%val
this%sum_sq   = this%sum_sq   + this%val**2

! zero out temp value
this%val = 0.0_8

end subroutine bank_tally

```

```

! CALCULATE_STATISTICS
!> @brief routine to compute mean and standard deviation of tallies

```

```

subroutine calculate_statistics(this,n)

! formal variables
type(tally_type) :: this ! a tally
integer          :: n      ! number of histories run

! preallocate mean and stdev
if (.not.allocated(this%mean)) allocate(this%mean(size(this%sum)))
if (.not.allocated(this%std))  allocate(this%std(size(this%sum)))

! compute mean

```

```

    this%mean =  this%sum / dble(n)

    ! compute standard deviation
    this%std = sqrt((this%sum_sq/dble(n) - this%mean**2)/dble(n))

end subroutine calculate_statistics

=====
! DEALLOCATE_TALLY
!> @brief routine to deallocate tally types
!=====

subroutine deallocate_tally(this)

    ! formal variables
    type(tally_type) :: this ! a tally

    ! deallocate all
    if (allocated(this%E)) deallocate(this%E)
    if (allocated(this%val)) deallocate(this%val)
    if (allocated(this%sum)) deallocate(this%sum)
    if (allocated(this%sum_sq)) deallocate(this%sum_sq)

end subroutine deallocate_tally

end module tally

```