
PROBLEM SET 1 SOLUTIONS
22.211 Reactor Physics I

Due: 22 February 2012

Bryan Herman

Problem 1. Generate plots of scatter neutron energy vs. generation for C-12.

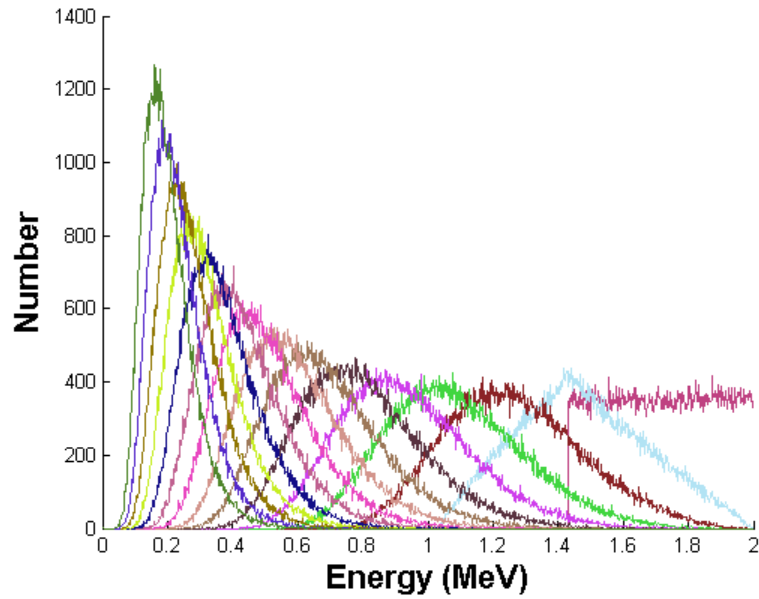


Figure 1: C-12 Scattered Energy by Generation - 15 generations 100,000 neutrons

Problem 2. Generate plots of scatter neutron energy vs. generation for H-1.

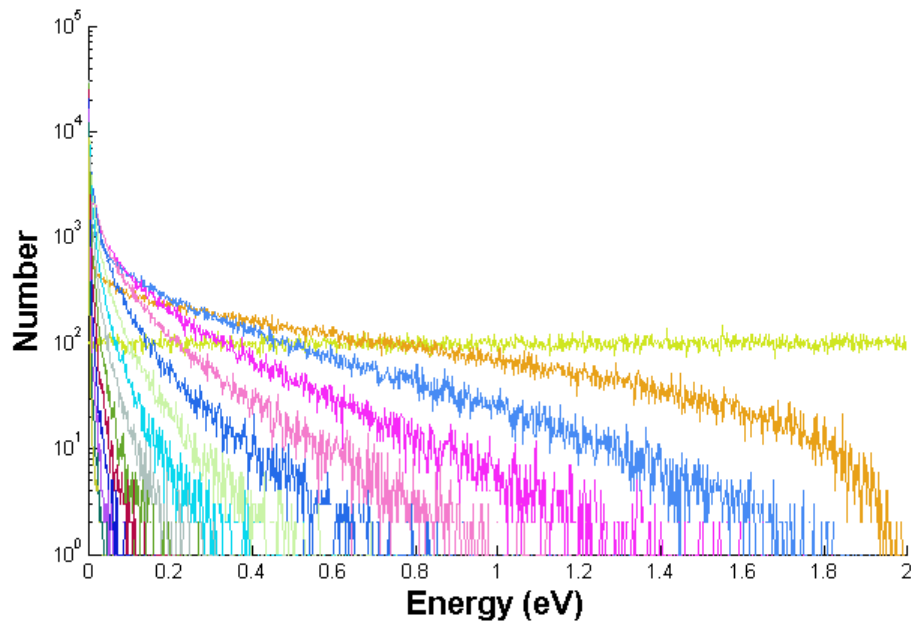


Figure 2: H-1 Scattered Energy by Generation - 15 generations 100,000 neutrons

Problem 3. Generate plots of scatter neutrons flux vs. energy for C-12.

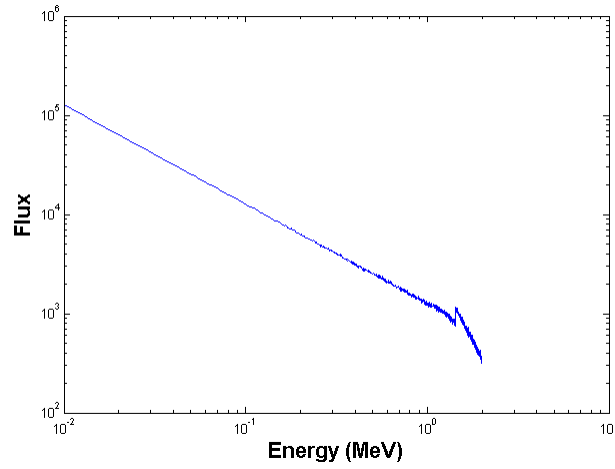


Figure 3: C-12 Flux Spectrum (energy bins) 100,000 neutrons

Problem 4. Generate plots of scatter neutrons flux vs. lethargy for C-12.

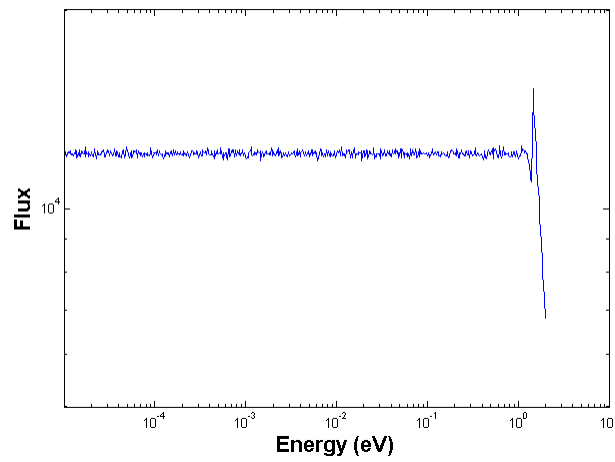


Figure 4: C-12 Flux Spectrum (lethargy bins) 100,000 neutrons

Problem 5. Explain the “Placzek Transients” in flux vs. lethargy for C-12 slowing down.

The Placzek transient is a discontinuity of the flux, and derivatives of the flux, at integer multiples of αE . The transient results from the fact that after the first generation, every neutron must have $E \geq \alpha E$. Neutrons which have an energy $E \geq \alpha E$ can only consist of neutrons in the 2nd, 3rd, or latter generations, and so forth for each multiple of αE . Thus, the discontinuity at αE arises from the fact that to the left and right of each multiple of αE our neutron flux is accumulated from a monotonically decreasing number of neutron generations.

Problem 6. Generate cdf plot for fission neutron emission energy.

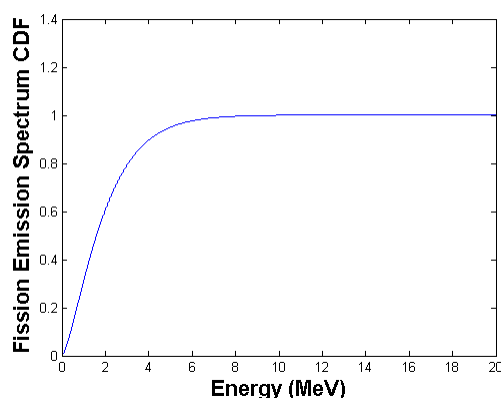


Figure 5: Cumulative Distribution Function for Fission Emission Spectrum

Problem 7. Generate neutron emission plot from 10,000 random fissions.

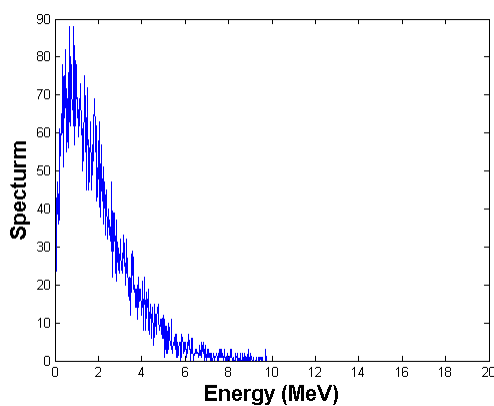


Figure 6: Fission Emission Spectrum - 10,000 neutrons

The flux spectrum for a constant cross section from a fission source is shown below. Notice the slight bump at 1 MeV. This is seen since it is the most probable energy of a neutron emitted from fission. If you don't see this slight bump, you may not be tallying the flux spectrum after you sampled the fission source energy, BUT before you sampled a new energy from scattering.

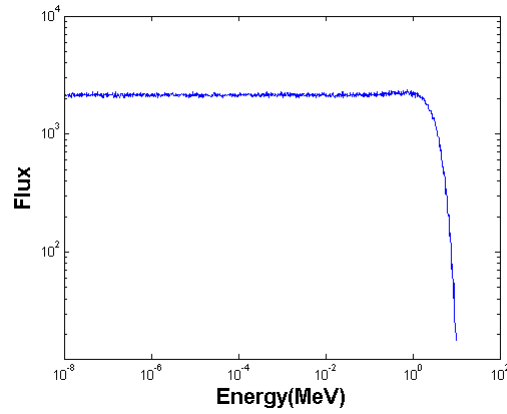


Figure 7: H-1 (constant xs) Flux Spectrum (lethargy bins)

After incorporating the true Hydrogen-1 cross section from ENDF, the shape of the curve changes as shown below.

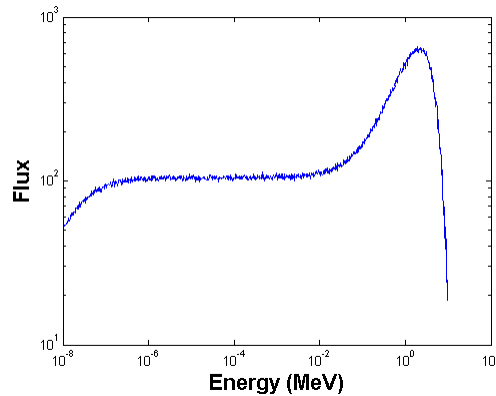


Figure 8: H-1 (ENDF xs) Flux Spectrum (lethargy bins)

A Main Codes

A.1 Slowing Down by Generation

```
% Bryan Herman
% Slowing Down Code
% HW1 22.211
%
%% Slowing down part (by generation)
% set input information
n_histories = 100000;
n_generations = 15;
Eo = 2.0;
A = 1;
seed = 5;
n_bins = 1000;
source = 'const';

% initialize objects
neut(1:n_histories) = particle_class();
tal = tally_class(n_bins,Eo);
pdf = pdf_class(seed);
mat = material_class(A);

% set materials (may put this in class later)
mat = mat.load_isotope('H_1');

% begin loop
for i = 1:n_generations

    % display information
    fprintf('\nExecuting Generation %d\n',i);
    fprintf('=====\n\n');

    for j = 1:n_histories

        % sample source energy if fission spectrum is on
        if i == 1
            pdf = pdf.sample_source_energy(source);
            neut(j) = neut(j).set_energy(pdf.E);
        end

        % sample new energy
        pdf = pdf.sample_collision_energy(neut(j).E,mat.alpha);

        % set particle to that new energy
        neut(j) = neut(j).set_energy(pdf.E);

        % bank sampled energy (only seed 1 isotope for now)
        tal = tal.bank_tally(neut(j).E,mat.isotopes{1});

        if mod(j,1000) == 0
```

```

        fprintf('Histories: %d ...\n',j);
    end
end

% plot that generation
tal.plot_tally(1);

% reset tallies
tal = tal.reset;

end

% plot flux
tal.plot_flux(2);

```

A.2 Fission Emission Spectrum

A.3 Slowing Down by Particle

```

% Bryan Herman
% Slowing Down Code
% HW1 22.211
%
%% Slowing down part (by particle)

% set input information
n_histories = 100000;
Eo = 20.0;
A = 1;
seed = 5;
n_bins = 1000;
source = 'fission';

% initialize objects
neut(1:n_histories) = particle_class();
tal = tally_class(n_bins,Eo);
pdf = pdf_class(seed);
mat = material_class(A);

% set materials (may put this in class later)
mat = mat.load_isotope('H_1');

% begin loop around histories
for j = 1:n_histories

    % sample source energy
    pdf = pdf.sample_source_energy(source);
    neut(j) = neut(j).set_energy(pdf.E);

    while neut(j).alive == 1

```

```

    % bank sampled energy (only seed 1 isotope for now)
    tal = tal.bank_tally(neut(j).E, mat.isotopes{1});

    % sample new energy
    pdf = pdf.sample_collision_energy(neut(j).E, mat.alpha);

    % set particle to that new energy
    neut(j) = neut(j).set_energy(pdf.E);

    if(neut(j).E < 1e-8)

        % kill particle
        neut(j) = neut(j).kill;

        % reset tallies and bank flux
        tal = tal.reset;

    end

end

% display calculation progress
if mod(j,1000) == 0
    fprintf('Histories: %d ...\n', j);
end

end

% plot flux
tal.plot_flux(1);

```

B Class Files

B.1 Particle Class

```

classdef particle_class
    %PARTICLE_CLASS defines a particle type for slowing down
    %   this class defines a particle along with its methods that will
    %   be used in the Monte Carlo slowing down code

    properties (SetAccess = private, GetAccess = public)
        E
        alive
    end

    methods

        % Constructor
        function obj=particle_class()
            obj.alive = 1;
        end
    end
end

```



```

end

% kill particle
function obj=kill(obj)
    obj.alive = 0;
end

% sets particle energy
function obj = set_energy(obj,E)
    obj.E = E;
end

end

end

```

B.2 PDF Class

```

classdef pdf_class

    % PDF_CLASS contains information about the distribution function
    % contains all of the random number sampling and pdfs needed by MC

    properties (SetAccess = private, GetAccess = public)

        rng      % random number generator object
        E        % new sampled energy
        egrid    % energy grid for chi pdf
        chicdf   % cdf for chi

    end

    methods

        % constructor
        function obj = pdf_class(seed)

            % initialize random number generator with seed
            obj.rng = RandStream('mcg16807','Seed',seed);

            % initialize watt fission spectrum cdf
            obj.egrid = linspace(0,20,10000);

            % create cdf
            obj = obj.watt_fission_cdf();

        end

        % sample energy
        function obj = sample_collision_energy(obj,E,alpha)

```

```

        obj.E = E - E*(1-alpha)*obj.rng.rand;

    end

    % sample source energy
    function obj = sample_source_energy(obj,opt)

        if strcmp(opt,'const')

            % assume fixed source at 2.0 MeV
            obj.E = 2.0;

        elseif strcmp(opt,'fission');

            % sample random number
            rn1 = obj.rng.rand;

            % find bin location
            idx = find(obj.chicdf.*(obj.chicdf >= rn1),1,'first')-1;
            obj.E = obj.egrid(idx);

        else
            error('FATAL==>Source cant be sampled.')
        end

    end

end

end

methods (Access = private)

    % pdf for watt fission spectrum
    function chi = watt_fission(obj,E)

        chi = 0.453*exp(-1.036*E).*sinh(sqrt(2.29*E));

    end

    % construct cdf
    function obj = watt_fission_cdf(obj)

        % allocate
        obj.chicdf = zeros(length(obj.egrid),1);

        % create cdf
        for i = 2:length(obj.egrid)

            % numerically integrate
            obj.chicdf(i) = trapz(obj.egrid(1:i),obj.watt_fission(obj.egrid(1:i)));

        end

    end

end

```

```
end  
  
end
```

B.3 Material Class

```
classdef material_class  
    %MATERIAL_CLASS Summary of this class goes here  
    % Detailed explanation goes here  
  
    properties (SetAccess = private, GetAccess = public)  
        A  
        alpha  
        n_isotopes  
        isotopes  
        iso_list  
    end  
  
    methods  
  
        % constructor  
        function obj = material_class(A)  
  
            % set vars  
            obj.A = A;  
            obj.alpha = ((A-1)/(A+1))^2;  
            obj.n_isotopes = 0;  
  
        end  
  
        % import xsdata file  
        function obj = load_isotope(obj,name)  
  
            % increment number of isotopes  
            obj.n_isotopes = obj.n_isotopes + 1;  
  
            % call constructor of xsdata  
            obj.isotopes{obj.n_isotopes} = cross_section_class(name);  
  
            % append to list  
            obj.iso_list{obj.n_isotopes} = name;  
  
        end  
  
    end  
  
end
```

B.4 Cross Section Class

```

classdef cross_section_class
    %CROSS_SECTION keeps track of ENDF xs data sets

    properties
        egrid
        xs
        name
    end

    methods

        % constructor
        function obj = cross_section_class(name)

            % get name
            obj.name = name;

            % load xs data file expecting E and xs
            disp(strcat('Loading xsdata file: ',name, '.m'));
            load(name);

            % set energy grid and xs
            obj.egrid = E;
            obj.xs = xs;

        end
    end
end

```

B.5 Tally Class

```

classdef tally_class
    %TALLY_CLASS defines the object for tallying MC quantites
    % defines the object for tallying MC quantites

    properties(SetAccess = private,GetAccess = public)

        nbins    % number of energy grid bins
        egrid    % energy grid
        lgrid    % lethargy grid
        counts    % count
        lcounts  % letharg bin counts
        eave     % array of average energy in egrid bins
        leave    % arrage of average energy in lethargy bins
        de       % energy spacing
        le       % lethargy spacing
        flux     % flux accumulation
        lflux    % flux in lethargy bins
    end
end

```

```

end

methods

% constructor
function obj = tally_class(nbins,Eo)

    % initialize values
    obj.nbins = nbins;
    obj.egrid = linspace(0,Eo,nbins+1);
    obj.eave = linspace((Eo/(nbins*2)),Eo-(Eo/(nbins*2)),nbins);
    obj.counts = zeros(1,nbins);
    obj.flux = zeros(1,nbins);
    obj.de = obj.egrid(2) - obj.egrid(1);

    obj.lgrid = logspace(-8,log10(Eo),nbins+1);
    obj.leave = 0.5*(obj.lgrid(1:nbins) + obj.lgrid(2:nbins+1));
    obj.lcounts = zeros(1,nbins);
    obj.lflux = zeros(1,nbins);
    obj.le = log(max(obj.lgrid)/obj.lgrid(1)) - log(max(obj.lgrid)/obj.lgrid(2));

end

% bank tally
function obj = bank_tally(obj,E,iso)

    % get micro cross section
    xs = interp1(iso.egrid,iso.xs,E,'linear','extrap');

    % find out bin index
    idx = find(obj.egrid.*(obj.egrid >= E),1,'first')-1;

    if idx > length(obj.egrid) || idx == 0
        error('FATAL ==> index out of bounds');
    end
    % bank a count
    obj.counts(idx) = obj.counts(idx) + 1/xs;

    % find out lethargy bin index
    idx = find(obj.lgrid.*(obj.lgrid >= E),1,'first')-1;

    % bank a count (disregard below the cutoff energy)
    if (idx > 0)
        obj.lcounts(idx) = obj.lcounts(idx) + 1/xs;
    end

end

% plot tally
function plot_tally(obj,n)

    % get random rgb code
    r = rand(1);
    g = rand(1);

```

```

        b = rand(1);

        figure(n);
        hold on;
        plot(obj.eave,obj.counts,'Color',[r,g,b]);

    end

    % plot flux
    function plot_flux(obj,n)

        figure(n)
        loglog(obj.eave,obj.flux);

        figure(n+1)
        loglog(obj.leave,obj.lflux);

    end

    % clear tally
    function obj = reset(obj)

        % bank in flux first
        obj.flux = obj.flux + obj.counts;
        obj.lflux = obj.lflux + obj.lcounts;

        % now reset
        obj.counts(:) = 0;
        obj.lcounts(:) = 0;

    end

end

methods(Access = private)

    % determine the index on the energy grid
    function idx = get_energy_idx(obj,E)

        idx = ceil(E/obj.de);

    end

    % determine the index on the lethargy grid
    function idx = get_lethargy_idx(obj,E)

        % this is complex since we are putting on a log energy grid
        l = log(max(obj.lgrid)/E);
        idx = length(obj.lgrid) - floor(l/obj.le) - 1;
        if idx <= 0
            idx = 1;
        end

    end

end

```

```
end
end
```