ÉCOLE POLYTECHNIQUE
FÉDÉRALE DE LAUSANNE

Plan de recherche / *Research Plan*
Bayesian Uncertainty Quantification of
Physical Models in Thermal-Hydraulics System Codes

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1   Introduction (cadre général) / *Introduction*:

During the early days of reactor safety analysis, prediction using model of power plant behavior during transient, off-normal or accident conditions, is approached with high-degree of conservatism. Conservatism calls for the most penalizing modeling assumptions to ensure conservative results, which is far below their regulatory limits. This approach, though less realistic, is justified by limited modeling capabilities as well as limited knowledge of physical processes. However, it was later found that there are conditions where conservative assumptions do not necessarily lead to conservative (or even physical) predictions.

As an example of this contradiction, consider the following situation in the analysis of Loss-of-Coolant-Accident (LOCA) as described in [1]. Assuming less interfacial shear between liquid and gas phase of water reducing mist flow during LOCA is conservative in the sense that less heat will be transferred to the coolant flow in the upper region of the core. This penalizes the fuel temperature prediction. But this assumption will also imply that time to refill the core is shorter as there are more liquid retained in the cooling system. Furthermore, with less shear, there is less resistance in injecting emergency coolant into the core (condition known as counter-current flow limitation). Both of these are clearly non-conservative and contradicting the first conservative assumption.

Therefore, a more accurate prediction of two-phase flow transient behavior was deemed necessary for safety analysis of nuclear power plant under accident conditions. As opposed to conservative analysis, this approach was coined *best-estimate* analysis. Such an analysis calls for physically sound thermal-hydraulics models with realistic modeling assumptions which at the same time are also backed by experimental data. With the idea of having more realistic prediction in mind, a best-estimate thermal-hydraulics (TH) system code was developed. The code is designed to be a comprehensive tool capable simulating realistically wide range of transients foreseen in nuclear power plant operation. It was developed using the current best understanding of flow processes expected to happen during the transients.

1.1   Models and Codes

As mentioned, thermal-hydraulics system codes was designed to simulate system behavior of a power plant during off-normal or accident conditions. The typical structure of system code is illustrated in Fig. 1. As can be seen, system code constitutes of several building blocks that can be used to modeled and simulate wide range of system and conditions. The core of system code is the balance equations describing the dynamic state of the fluid. To close the system[1], it has to be complemented by 2 additional set of constitutive equations. First, the equation of state describing the thermodynamic relation between state variables of a given fluid. And second, the closure relationships describing the interaction between phases and each phase to the boundary wall in terms of mass, momentum, and energy.

In a typical usage, a user will specify the geometry (the configuration of the plant), as well as boundary and initial conditions for the problem of interest. Based on this information, the code will then solves a set of balance equations describing the fluid flow. As a result, the code will make prediction on the evolution of the fluid state, namely, pressure, enthalpy, and fluid velocity in each part of the modeled plant.

The state-of-the-art model widely implemented in TH system codes to describe the dynamics of fluid flow in nuclear power plant is based on the two-fluid model [2]. This model separately treats the transport phenomena of the two phases of fluid (gas and liquid) resulting in 6 balance equations (mass, momentum, and energy for each phase). The model can capture phenomena where thermal

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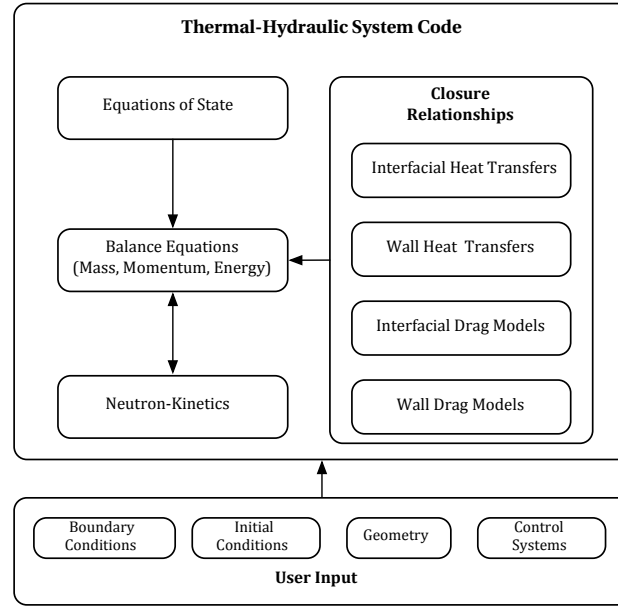[1]*i.e.*, to have the same number of equations as the number of unknowns

Figure 1: Structure of a system code

and mechanical non-equilibrium conditions exist between the two phases, conditions which gives more realistic picture in wide range of nuclear power plant transients.

The validity of the two-fluid model widely implemented in thermal-hydraulics system codes relies on the proper modeling of the transfer terms between phases and each phase to the boundary wall (see Fig. 1). The transfer terms include interfacial drag, wall drag, interfacial heat transfer, and wall heat transfer. In principle, based on different two-phase flow pattern, a different phase distribution as well as different interfacial structure can be observed. As such, each of those transfer terms takes different form with different parameters depending on the pattern of the two-phase flow and they represent different physical processes take place at each pattern.

These physical models, or the so-called *closure laws or relationships* mentioned previously, close the set of balance equations for mass, momentum, and energy of the two phases. They are to be derived experimentally, but proved to be a difficult effort [3, 4] due to various reasons ranging from lack of knowledge to limitation in measurement instruments. Simplifying assumptions and at times extrapolations have to be made simply because of limited available data. Ultimately, the set of models and correlations implemented in the codes is a major of source of uncertainty[2].

1.2 Statistical Uncertainty Analysis

As explained, best-estimate analysis uses more realistic modeling assumptions for analyzing transient behavior of nuclear power plant. It attempts as realistically as possible to describe the behavior of physical processes occur in power plant transient. And yet, even the best available knowledge for some two-phase flow processes in nuclear power plant during transient is still limited. Data support for some processes is limited or understanding of complex phenomena might not yet adequate. Simplifying assumptions, approximations and expert judgments to some degree are unavoidable and still required to have a complete analysis. Hence, best-estimate analysis has to be complemented with uncertainty analysis. The ultimate goal of uncertainty analysis is to associate code prediction with its uncertainty.

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[2]Uncertainty here is defined as a state of limited knowledge, that is of *epistemic* nature

These combined quantities are then compared with certain regulatory safety limits (*e.g.,* Peak Cladding Temperature, PCT) to check whether the limits still fall outside the uncertainty band of the code prediction.

Before continuing our discussion on uncertainty analysis of code predictions, it will be worthwhile to define some terminologies to avoid latter confusion. We will refer to the following figure to describe system code in more generic way (*i.e.,* an I/O model). They were adapted from [20] to fit our discussion on system code and its physical model.
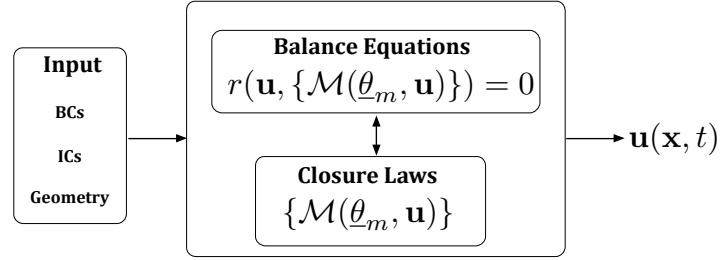

Figure 2: Simplified illustration of system code as an Input/Output model

From above figure, it can be seen that *input* defines specific problem of interest. It includes specifications for geometrical configuration, choice of material and fluid involves, as well as initial and boundary conditions. It may also includes specific numerical solution settings. As the input are often parametrized, we will refer to them as *input parameters*, $\underline{\theta}_i$. Defining the input (as far as user is concerned) completely defines the problem and the code will solve balance equation which output the dynamic state of $\mathbf{u}(\mathbf{x}, t)$ (fluid pressure, temperature, etc.). The balance equations are closed with additional set of closure laws $\{\mathcal{M}(\underline{\theta}_m, \mathbf{u})\}$. These closure laws in turn are parametrized with a set of parameters $\underline{\theta}_m$ and also a function of the state variables $\mathbf{u}$. We will refer to $\underline{\theta}_m$ as the *physical model parameters*.

There are several known sources of uncertainty that render $\mathbf{u}(\mathbf{x}, t)$ prediction uncertain. The following are the sources of primary interest in the current research:

1. Initial and boundary conditions are not perfectly known for a given problem. Additionally, other constituents of input (material properties) might not also not known exactly.

2. The balance equations, originally in partial differential equations, are discretized and solved with finite precision.

3. The closure laws are approximation. They derived from limited experimental data, and in some cases, in different experimental conditions as compared to the purpose of the model.

4. The value model parameters $\underline{\theta}_m$ are often not known *a priori* and requires calibration to fit observed data.

In statistical uncertainty analysis, the input and physical model parameters are modeled as random variable ($\mathbf{\Theta}_i$ and $\mathbf{\Theta}_m$, respectively) with probability distributions. Then using Monte Carlo technique, samples are generated from their respective distribution. These samples are used to run the code multiple times resulting in multiple results. Finally, statistics of the code output (which is now also a random variable, $\mathcal{U}(\mathbf{x}, t)$) are summarized to obtain uncertainty measure of the prediction. In other words, the uncertainties in the input and physical model are *propagated* forward into the code to quantify the uncertainty of the predictions as shown in Fig. 3.
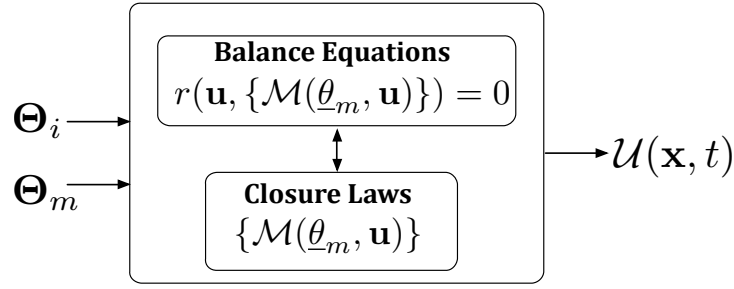
Figure 3: Uncertainty quantification (propagation)

1.3   State of the Research

The importance of characterizing the uncertainty in the physical models is acknowledged by the Working Group on the Analysis and Management of Accidents (WGAMA) of the OECD/NEA. This led to the PREMIUM (Post-BEMUSE Reflood Models Input Uncertainty Methods) project whose main goal is to report the state-of-the-art of the available methodologies to quantify the uncertainty in the physical models. The following will briefly describe the PREMIUM project which provides a context for this research as well as some of the available methodologies proposed by different organizations.

1.3.1   OECD/NEA PREMIUM Benchmark

The PREMIUM benchmark is an activity launched by the OECD/NEA in 2012 with the aim to advance the methods for quantifying the physical models uncertainties in thermal-hydraulics system codes. Unlike the previous project BEMUSE (Best-Estimate Methods - Uncertainty and Sensitivity Evaluation), the emphasis in the current project is placed on the model uncertainty and its quantification will be based on a given set of data.

The steps taken for the benchmark consist mainly of three parts: identification of important parameters, uncertainty quantification of model uncertainty based on a given data set, and propagation of the obtained model uncertainty and compare with another set of data. The model considered are focused on the reflood models and the available data is taken from series of reflood experiments at FEBA and PERICLES facilities.

The term "input" defined in the benchmark requires further clarification. The input here is not necessarily coming from usual user specification which is strictly speaking involves mainly system geometry, initial condition, boundary condition specifications. In the case of reflood models, most codes are expected to calculate reflood based on the built-in conditions along with built-in parameters associated with each constituent models. That is, reflood models are part of internal machinery of the codes. However, the parameters associated with reflood model do become input for the purpose of uncertainty propagation analysis as there are uncertainties associated with these models and parameters. The purpose of the benchmark is to obtain this uncertainties in form of probability distribution to be sampled and propagated.

1.3.2   GRS Methodology

GRS methodology was developed by GRS[3] in Germany [5] and for more recent review [6]. It is based on the statistical technique to propagate the uncertainties in the input and to obtain estimate of reliability measure of the output. The considered sources of uncertainties are comprehensive, ranging from initial

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[3]Gesellschaft für Anlagen- und Reaktorscherheit

conditions, boundary conditions, code models (if an alternative model is available), to the setting for numerical solutions.

The uncertainty analysis of the output of interest (*e.g.,* PCT) is carried out by propagating the sampled uncertain input from their distribution. This simply means running the code multiple times with different input values. The required minimum of simulation runs is determined by the Wilks' formula [7] which depends on the desired tolerance level of the output [6]. For example, based on the formula, 59 code runs are required to obtain the 95% confidence level that the output of interest will not exceed the 95th (percentile) of the corresponding output distribution. The number of runs does not depends on number of uncertain input parameters and full characterization of output distribution is not required.

The uncertainty of the input is treated in probabilistic framework with probability is interpreted as state of knowledge. This is a Bayesian view of uncertainty. The probability characterization involves elicitation process which derived from expert judgments [5].

1.3.3   CIRCÉ

CIRCÉ (Calcul des Incertitudes Relatives aux Corrélation Élementaires) is a tool/methodology developed by CEA[4] in France. It treats the problem of quantifying parametric uncertainty in system code as an inverse problem. The likelihood of observing the data using code with a set of intermediate parameters is maximized. The posterior of this intermediate parameters are assumed to be independent normal distribution. The relationship between important parameter and code code quantity of interest is assumed to be linear and only the first (local) derivative is required for the analysis.

1.3.4   FFTBM Methodology

FFTBM (Fast Fourier Transform Based Method) is a methodology developed by University of Pisa as part of their integrated uncertainty methodology UMAE (Uncertainty Method Based on the Accuracy Extrapolation) [8, 9]. The UMAE method is based on the idea that uncertainty of code results for full scale reactor transient simlation can be extrapolated from the accuracy of code results made on integral test facilities (where reference data is available). This differs from most of the other methods in a sense that the propagation of uncertainty is directly carried out from test data into full scale facility.

In FFTBM, the distributions of the important parameters are assumed to be uniformed and independent, but the supports are inferred from the available data. Fast Fourier Transform is applied to quantify the accuracy in the code results and the available data for the quantity of interest (usually in the form of time series). The Average Amplitude (AA), the ratio between the transformed code results and data, describes the discrepancy between the two. In the subsequent analysis, sensitivity analysis is first carried out to determine the important parameters that the AA is sensitive to. Then by defining criterion for maximum allowed deviation of AA from the base case, the range of each parameters can be inferred.

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[4]Commissariat à l'énergie atomique et aux énergies alternatives

2   Objectifs / *Objectives*:

2.1   Statement of the Problem

The development of closure laws for reflooding described in [3] showed the difficulties and the amount of assumptions used. In principle, system code development is an effort to consolidate correlations and mechanistic models, to create phenomenological-based code that can provide the best-estimate results. This consolidated effort results in a code that can simulate wide range of transients foreseen in nuclear power plant operation in a best-estimate way. Alas, to come up with a consistent set of closure laws is a great challenge for code developers.

The closure laws required to close the two-fluid model pose particularly difficult challenges [4]. For instance, to have correlation of heat transfer between the wall and the fluid, temperature data from each of the constituents are needed (the wall, the liquid phase, and the gas phase). But measuring temperature of the individual phases in arbitrary interfacial topology has its own technical difficulties that no such data exists or available to be implemented in the closure laws. Additionally, the experiments to obtain hydrodynamic closure laws (*e.g.,* interfacial friction factor, wall friction factor, etc.) were carried out in adiabatic condition. This excludes the coupling to any heat transfer phenomena between the phases and with the wall in such correlation.

Furthermore, during development of TRACE code, programming considerations also came into the picture. For robustness, simplification is often required and continuity is enforced. Transitionary regime between two flow regimes where experimental data does not exist is modeled to be the averaged of the two bounding regimes. In the end, the phenomenological models implemented in TRACE are specific to TRACE. Different code development, which used different assumptions and experimental database, comes up with other set of closure laws with their own parametrization (see for instance [10] for CATHARE development). Several authors have expressed their concerns about the uncertainty stemming from the closure laws ([4, 11, 12]).

As an example of the point given above, consider that in TRACE code, after some derivations, the interfacial drag coefficient closure law in the inverted slug flow regime $C_{i,IS}$ is given by,

$$C_{i,IS} = \Psi_{SET} \times \frac{1}{24} \frac{\rho_g}{\mathrm{La}} \frac{(1-\alpha)}{\alpha^{1.8}} \quad ; \Psi_{SET} = 0.75 \tag{1}$$

There are several remarks about the closure law given above. First, the second term in the right-hand side was derived from experimental data but not originally in the form above. In the inverted slug regime, saturated liquid core breaks up into ligaments. These ligaments are *assumed* to take form as prolate ellipsoid. The drag coefficient of distorted droplet from experimental data is then *assumed.* Then to take into account of multi-particle effect, the coefficient is multiplied by void fraction raised to the power of 1.8 (this was taken from experimental data of inertial regime). Lastly, the first term, $\Psi_{SET} = 0.75$ was put *to match* the experimental data from FLECHT-SEASET reflood facility. This first term, although clearly non-physical, is an important tuning parameter of the model nonetheless. Its uncertainty should be considered in uncertainty analysis, especially when reflood is expected to occur. However, no statement of the uncertainty is given. Several other such terms exist in TRACE code [13].

From the above discussion, it is clear that models in thermal-hydraulics system code, like any other models, are flawed. Various experimental programs were carried out to gain better understanding of an important phenomena, and to validate (and, as noted above, to calibrate) the models. Series of the experiments, carried out in Separate Effect Test Facilities (SETFs) were aimed to reproduce limited part of the transient or components during limited time periods following a postulated scenario. For example in the case of reflooding, over the past three decades, several facilities existed and data were accumulated (FEBA, PERICLES, etc.). But, there has not been an orchestrated effort to incorporate the

accumulated data into the calibration process, in systematic way, while acknowledging the uncertainties of other sources in the process.

2.2   Objective

The overall objective of the research is to quantify the uncertainty of physical models implemented in TRACE in the form of closure laws. These models are parametrized either with physical parameters or tuning parameters. Usual practice by the code developer is to calibrate them against some an experimental without any statement of their uncertainties. To conform with the framework of uncertainty propagation already better developed, especially for *forecasting* purpose, where no data is available whatsoever, uncertainty of these parameters are required in the form of probability distribution. Additionally, this quantification process also need to take into account the accumulated data that has been generated from series of similar experiments from various facilities.

As will be elaborated below, the physical model under consideration for this research is the reflood model and the thermal-hydraulic code used is TRACE [13].

2.3   Bayesian Framework

System code is a complex tool to simulate complex problem. Usage of partial information, assumptions, subjectivity, and judgments are inevitable and are, in fact, crucial to make initial progress. The key then is to be as transparent as possible about these assumptions, to systematically check them, and in the light of new data, to allow the change in assumptions. These are certainly more constructive than merely being objective. All of these are accommodated naturally using Bayesian approach to quantify uncertainty.

Bayesian approach stated here does not correspond to a specific methodology. Instead, it refers to a framework which pose the inverse problem of parameter estimation in terms of probability. Beck [14] refers to this procedure as *stochastic embedding*. Posed in this way, Bayesian inference machinery can be used to infer the parameters of interest given the available data. Furthermore, as the key idea of Bayesian inference is to explore the posterior distribution (instead of finding single set of optimum parameters in the traditional deterministic calibration setting), the uncertainty of the parameters can be characterized. This characterization of parameter uncertainty of the model while simultaneously taking into account other sources of uncertainty is the ultimate goal of this thesis.

The first step to carry out Bayesian analysis is to cast the problem in terms of probability. The source of uncertainty that caused discrepancy (error) in the model prediction will be modeled as a random variable. The framework for error models used in this research is adapted and elaborated from the work of Huard and Mailhot [15] (see Fig. 4). It is assumed that there exists true process, true input, true output, true model parameters. However, in practice data, is finite and distorted, and models are an approximation due to lack of knowledge. That is, our analysis and prediction are distorted by uncertainties.

According to the figure there are three main types of errors due to uncertainty: input, output, and structural errors. They will have to be modeled probabilistically. The following paragraphs will describe each of the errors.

Input error refers to the difference between the true input $\tilde{\mathbf{x}}$ and the measured input $\mathbf{x}$. The measured input is limited by the imperfection of the measurement device. In the context of separate effect test, the input is the imposed initial and boundary conditions of the experiment. These conditions drive the system response during the test and they are measured with finite precision. Because the test conditions differ even within the same facility, these errors has to be taken into account for robust parameter estimation.
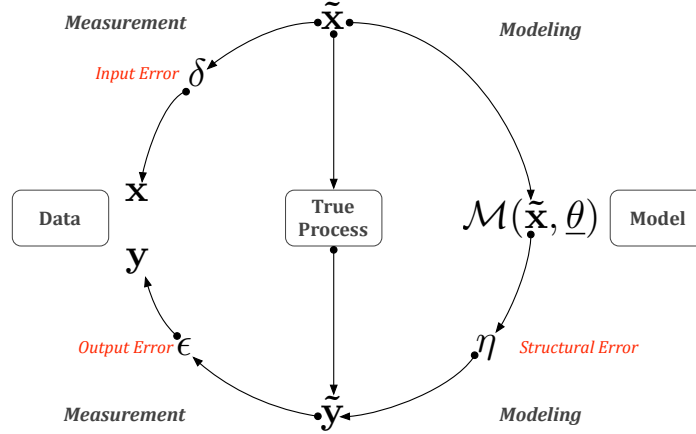
Figure 4: Errors and their relation for Bayesian uncertainty analysis (adapted from [15])

Output error is analogous to the input error described above. It is the difference between the true output $\tilde{\mathbf{y}}$ and the measured output $\mathbf{y}$. It is the system response for the imposed boundary conditions.

Structural error is defined as the difference between model results and the true output $\tilde{\mathbf{y}}$. Modeling process relates the input to the output via a model parametrized with $\underline{\theta}$. By definition of model, even with a perfect set of parameters (which in principle, are also unknown, uncertain), model will still be inadequate. There are processes which are not taken into account either because lack of knowledge, fallacy in the hypothesis or even error in the numerical solution. As shown in [16], not taking into account structural error during calibration biases the parameter estimates and reduces the model predictive capabilities.

The treatment of input and output errors belong to experimental data analysis and most of the time is straightforward. However, there is no standard approach to the model structural error. The structural error can be expected to show bias (due to incorrect or excluded process) as well as to be correlated in time and space, depending on the nature of problem. Several possibilities are available to model this error [17, 18]. In this context, an additional strength of Bayesian framework is that the plausibility of different error models can be checked and compared via their evidence as shown in [14].

In general, the above error model can be written as follows, independently of their forms:

$$
\begin{array}{llcc}
\text{Input} & p(\delta) & \Longleftrightarrow & p(\mathbf{x} \mid \tilde{\mathbf{x}}) \\
\text{Output} & p(\epsilon) & \Longleftrightarrow & p(\mathbf{y} \mid \tilde{\mathbf{y}}) \\
\text{Structure} & p(\eta) & \Longleftrightarrow & p(\tilde{\mathbf{y}} \mid \tilde{\mathbf{x}}, \underline{\theta}, \mathcal{M})
\end{array}
$$

Note that the structural error model is conditioned on a specific model class $\mathcal{M}$ that defines its parametrization. To obtain the posterior distribution of the parameters based on their priors and conditioned on the data (both output and input) and specific model class, Bayes' theorem is applied,

$$
\begin{aligned}
p(\underline{\theta} \mid \mathbf{y}, \mathbf{x}, \mathcal{M}) &= \int \int p(\underline{\theta}, \tilde{\mathbf{y}}, \tilde{\mathbf{x}} \mid \mathbf{x}, \mathbf{y}, \mathcal{M}) \, d\tilde{\mathbf{x}} d\tilde{\mathbf{y}} \\
&= \int \int p(\mathbf{x} \mid \tilde{\mathbf{x}}) \cdot p(\mathbf{y} \mid \tilde{\mathbf{y}}) \cdot p(\tilde{\mathbf{y}} \mid \tilde{\mathbf{x}}, \underline{\theta}, \mathcal{M}) \cdot p(\tilde{\mathbf{x}}) \cdot \frac{p(\underline{\theta} \mid \mathcal{M})}{p(\mathbf{x}, \mathbf{y})} \, d\tilde{\mathbf{x}} d\tilde{\mathbf{y}}
\end{aligned}
$$

Where the posterior is condiioned on the observed data and model class $\mathcal{M}$. The integration is carried out to marginalize the joint distribution. To characterize the posterior distribution, Monte Carlo simulation is usually performed. One of the popular approach is to use Markov Chain Monte Carlo (MCMC) simulation to generate sample representative of the posterior.

2.4   Context

Although the steps taken in this research can be applicable to any system code physical model calibration and validation, the attention will be focused on the models of particular importance during reflooding, the so-called Post-CHF flow regimes. The reasons for this emphasis are:

1. Reflooding is an important part in the simulation of nuclear power plant transient during LOCA. Modeling reflooding determines the appropriate representation of the dynamics of heat transfer phenomena during the effort to rewet an uncovered core. Of paramount interest is to estimate the time the rod can be expected to be rewet as well as the maximum temperature reached prior to rewet. Reflood is a transient with highly coupled hydrodynamic-heat transfer effects and it challenges the assumption made on the implemented closure laws. Indeed, several Separate Effect Test (SET) reflood programs existed and were designed to validate reflood models in system code. Unfortunately, no orchestrated effort so far to consolidate the generated data in general and into TRACE code in particular.

2. The models are adequately complex. It is complex that 5 flow regimes are involved in a single phenomena. But as the source of data is from reflooding separate effect test facilities (SETFs), system effect can be excluded and analysis can be concentrated on limited set of models. In fact, as already pointed out, reflood SETFs are designed to validate (and calibrate) reflood models in sytem codes.

3. Additional data from another reflooding facilities are also available, including data obtained from series of experiments at the Neptun facility conducted at Paul Scherrer Institut during the 1990s, as well as data from experiments at the PERICLES facility conducted by CEA.

4. It is the model considered in PREMIUM benchmark, thus there is possibility to compare the results of this research with the results of the other participants.

3 Premiers résultats / *First results*: (1-2 pages)

3.1 Bayesian Calibration of a Turbulence Model

As an illustrative example and to familiarize with the main idea of Bayesian framework and its method, simple case of physical model calibration were carried out during the first year. Here the RNG k-epsilon turbulence model (parametrized by 7 parameters) is calibrated using measurement data from backstep flow experiment [19]. The turbulence model is implemented in the OpenFOAM framework where the backstep flow simulation model is part of the OpenFOAM validation case.

The first step in statistical calibration is to develop a stochastic error model that probabilistically relates measured data and simulation results. There are two stochastic error models considered in the present analysis, it is either assumed to be independent or spatially correlated with an exponential form of correlation (with correlation length scale to be estimated). Then Markov Chain Monte Carlo simulation were carried out to characterize the posterior of the parameter conditioned on the data.

The likelihood (error function) is formulated in line with the procedure given in [20]. Here as opposed to approach given by [16], the multiplicative form discrepancy error was assumed such that no-slip boundary condition is retained. The discrepancy error was assumed to be Gaussian with mean= 1 and standard deviation to be estimated.

$$\text{prob}(\{D\}|\underline{\theta}, M_j) = \frac{1}{\sqrt{((2\pi)^2 \times |K|)}} \exp\left[-\frac{(d-y)^T K^{-1}(d-y)}{2}\right] \qquad (2)$$

Where the d is the measured velocity data, y is the simulation results with a given set of parameters, and K is the covariance matrix which depends on the model structure M_j. For $j = 1$, $K = Ke + Ki$ (measurement and model discrepancy independent variance), while for $j = 2$, $K = Ke + Kc$ (measurement and model discrepancy correlated variance).

This resulted in 8 parameters for M_1 and 9 parameters for M_2 to be calibrated. The priors are set to be uniform and independent with the mean for the turbulence model parameters corresponds to the default values in OpenFOAM and with $\pm 50\%$ upper and lower bound values for each parameter. 1000 samples (1000 simulation runs) from the posterior is then generated using Markov Chain Monte Carlo algorithm implemented in Matlab [21]. The priors and the results are tabulated in Table 1 and Table 2, respectively, while the marginal posterior distribution is given in Figure for the 9 parameters model.

Table 1: Bounds of Uniform PDFs

	Parameter	Initial	Lower	Upper
	C_μ	0.08450	0.04225	0.12675
	C_1	1.42000	0.71000	2.13000
	C_2	1.68000	0.84000	2.52000
RNG k-ϵ	σ_k	0.71942	0.35971	1.07913
	σ_ϵ	0.71942	0.35971	1.07913
	η_0	4.38000	2.19000	6.57000
	β	0.01200	0.00600	0.01800
M1 (Independent)	σ_{mod}	0.10000	0.00000	0.20000
M2 (Correlated)	σ_{mod}	0.10000	0.00000	0.20000
	$\log(\alpha)$	1.00000	0.00000	2.00000

As can be seen, all the parameters are informed by the data. The uncertainties of the parameter obtained from the posterior can then be propagated to obtain robust prediction in other quantities (*e.g.,*

Table 2: Statistics of MCMC samples of Correlated Model

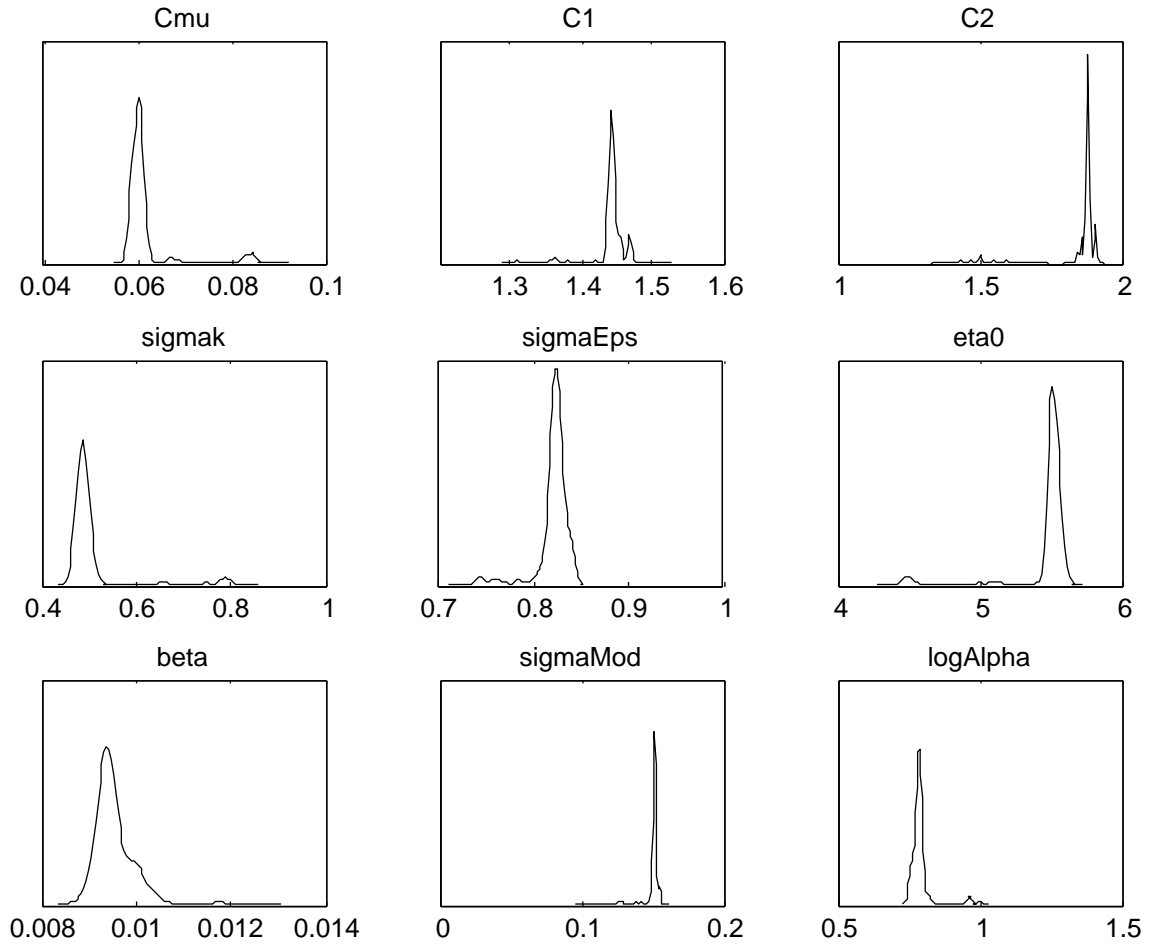| Parameter	mean	std.deviation	MC Error
C_μ	0.061714	0.0060212	0.0013311
C_1	1.43980	0.026246	0.0047692
C_2	1.8468	0.10874	0.023902
σ_k	0.51179	0.07683	0.017243
σ_ϵ	0.81953	0.01831	0.0039055
η_0	5.4496	0.24451	0.055019
β	0.0095692	0.00049043	9.8941×10^{-5}
σ_{mod}	0.14952	0.0072768	0.0015929
$\log(\alpha)$	0.79485	0.045782	0.0099883

Figure 5: Marginal Posterior PDFs for Model Parameters

the wall shear stress). However it was not checked the robustness of the parameter estimation using another data sets.

This exercise revealed some considerations in applying the Bayesian framework to calibrate TRACE reflood model with the data from reflood experiments:

1. The reflood process is inherently transient in nature, so in addition to measurement on several locations, the data will also be in time series. This need to be considered in deriving error models especially if there is correlation structure both in space and time assumed.

2. Incorporating several type of available data. In some facilities, several other quantities other than cladding temperature were also measured, including for example, pressure drop and void fraction. These call for several goodness-of-fit measures between data and model to be considered at once. Even if the importance of these additional data is less than clad temperature, they nevertheless contain additional information of reflood process.

3. The boundary conditions in reflood experiment are dynamic in nature, the imposed power as well as the injected water temperature is time-dependent. Their uncertainties should also be taken into account.

4. Parameters that need to be calibrated in TRACE reflood model are not part of the user input specification. Some were already hard-coded with values from a calibration with a data from a facility (without any statement of their uncertainties). Some modifications in the source code are required to accommodate sensitivity and uncertainty analysis of TRACE.

5. Aggregating the uncertainty estimation of parameters of reflood model from a facility with another might not be straightforward. Although each reflood experiment in different facility basically experimented a reflood process in a rod bundle, specificities exist and might bias the results from each.

6. Related to above concern, the statistical parameter estimation of reflood model in TRACE might not be identifiable with the available data from a facility, thus the results are very sensitive to a specific condition in a given facility and by no mean unique or can be generalized.

4   Planification future (plan de travail) / *Future planning (scheme of work)***:**

The proposed research plan, with emphasized placed on bottom reflood model and TRACE code, is expected to involve the following phases:

1. First, comprehensive review of TRACE closure laws related to reflood phenomena. Most are part of the post-CHF closure package, which includes interfacial drag, wall drag, interfacial heat transfer, and wall heat transfer models or correlations. The goal is to obtain understanding how reflood is conceptualized in TRACE, and along with it how the models are parametrized. This will result in list of parameters that is perceived to be important in reflood simulation.

2. Second, as these parameters are often not part of user specification and hard-coded in TRACE, modification of TRACE is required for it to accomodate sensitivity and uncertainty analyses. The goal here is to externalize important parameters hard-coded in TRACE, thus make them available for user to modify for each run.

3. Third, based on FEBA separate effect test facility, global sensitivity analysis of assumed important parameters (including the boundary and initial conditions) is carried out for the available type of measured data. In Bayesian framework, sensitivity analysis is an important for the purpose of system identification. In essence, if a measured value is insensitive to a (perceived) important parameter, then the uncertainty of this parameter cannot be inferred from this particular data. Excluding them prior to posterior sampling will save computational time or even reveal modeling error.

4. Fourth, based on several error models that can be assumed, posterior sampling is carried out to obtain the uncertainty in the parameters. The posterior sampling is performed via Markov Chain Monte Carlo (MCMC) illustrated in the example above. Further analysis for selecting (or averaging) the most plausible error model is done in this phase.

5. Next is the validation phase where the uncertainties obtained will be propagated in "forecasting" setting, using different data sets than the one used to derive the uncertainty of the model parameter.

6. Fifth, the uncertainties of parameters obtained from two different facilities need to be consistently combined while taking into account the facilities specificity and different range of experimental conditions. This phase involves a repetition of phase 3-5 but based on different facility (*e.g.*, PERICLES).

7. Finally, the results of combined uncertainties estimates need to be validated yet against another facility (*e.g.*, NEPTUN). The goal here is to investigate if indeed experimental data from different reflood SETFs can be consolidated into the current implementation of reflood model in TRACE to increase its predictive capability in simulating reflood process.

Basically for a given of reflood facility, the routine procedure would be: sensitivity analysis - uncertainty (backward) analysis for parametric calibration - uncertainty (forward) analysis for validation. The analyses at first are carried out separately for each reflood facilities. At the latter stage, however, the results will be consolidated where the uncertainty information of reflood physical model based on the data from various facilities are combined together in a consistent way, minimizing the specificity of the facilities and/or the experimental conditions.

Based on the phases identified above, a preliminary schedule for the 3-year dissertation work is proposed as follows:

1. **Task 1 (3 months)**

   - List the important parameters in TRACE that are perceived to be important for reflood simulation in SETF.
   - Externalize these parameters such that it can be specified in an input deck for the purpose of sensitivity and uncertainty analyses

2. **Task 2 (3 months)**

   - Test the modified TRACE for global sensitivity analysis.
   - Finalize the development of TRACE model for FEBA test facility.
   - Carried out global uncertainty analysis of FEBA test facility.

3. **Task 3 (10 months)**

   - Develop various plausible error models for input, output, and structural errors applicable for reflood model calibration. The nature of available data needs to be taken into account, that they are time series of measured values taken from different location. Possibility to incorporate information from different type of data (*e.g.,* temperature and void fraction) need also to be considered.
   - Develop or adapt available MCMC sampling algorithm that is efficient, can be parallelized, with informative diagnostics.
   - Carried out MCMC sampling based on each of the plausible structure of error models to characterize the uncertainty in the parameters conditioned on the data.
   - Select the more plausible model based on the evaluation of the evidence of these models.
   - If the reflood simulation proved to be too computationally demanding, develop a surrogate models to assist the calibration procedure.

4. **Task 4 (1 month)**. Validation of the obtain uncertainty estimates by propagating them for another data set within the same facility (FEBA).

5. **Task 5 (10 months)**. In principle, repeat most of the previous four tasks for other reflood facilities (*e.g.,* PERICLES and ACHILLES) to obtain the uncertainties in the important parameters.

6. **Task 3 (6 months)**: Consolidation and Validation

   - Develop procedure to consolidate parameter estimation of reflood models based on two/three different reflood facilities.
   - Validation of the consolidated uncertainty estimates of the parameters by propagating them in the simulation of reflood using data set from another facility (*e.g.,* NEPTUN).

7. **Thesis write-up (3 months)**:

5   Divers / *Miscellaneous*

5.1   Eventuelles publications / *Publications*

5.2   Présentation(s) orale(s) / *Oral presentation(s)*

5.3   Cours suivis / *Courses attended*

6   Références / *References*

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7   Signatures

Candidat(e)/*Candidate*   ....

Directeur de thèse/*Thesis director*   ....

Co-directeur de thèse/*Thesis co-director*   ....
(Officiellement habilité / *Officially authorized*)

Date du rapport/*Date of report*   ....

Directeur du programme/*Program director*

Signature: ....   Date: ....

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