

# NUEN 301

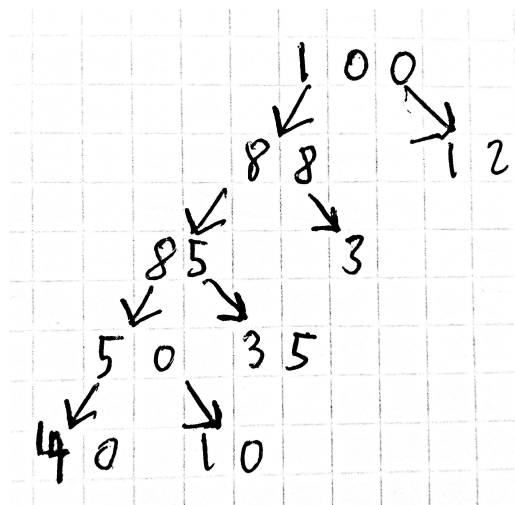
## Homework 2

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Exercise 1 [30 pts]: [chain reaction] In a certain thermal reactor, for every 100 neutrons emitted in fission, 12 escape while fast, and 3 escape after slowing down to thermal energies. No neutrons are absorbed while slowing down. The values of  $\eta_T$  and  $\nu_T$  in the fissile material are 2 and 2.5, respectively. The reactor is critical.

a. [5 pts] Fill the neutron tree of life, below.



b.[5 pts] Calculate the fast non-leakage probability ( $P_{FNL}$ ).

$$P_{FNL} = \frac{88}{100} = 0.88$$

c.[5 pts] What is the resonance-escape probability (p)?

$$p = \frac{88}{88} = 1$$

d.[5 pts] Calculate the thermal non-leakage probability ( $P_{TNL}$ ).

$$P_{TNL} = \frac{85}{88} = 0.966$$

e.[5 pts] Calculate the thermal utilization (f also known as  $u_T$ ).  $u_T = \frac{50}{85} = 0.588$

f.[5 pts] Calculate  $P_{TAF}$ .

$$P_{TAF} = \frac{\eta_F}{\nu_F} = \frac{2}{2.5} = 0.8$$

Exercise 2 [30 pts]: [RRD] Ni-63 is a beta- emitter, produced when a thermal neutron is captured in Ni-62 (molar mass 62 g/mol, density 8.9 g/cc). The radiative microscopic cross section of Ni-62 is 15 b when the neutron energy is  $E=1/40$  eV (the cross section has been averaged of the nucleus velocities). Assume no other types of interactions occur. A thin 0.05-gram target of pure Ni-62 is placed in a beam of thermal neutrons that has intensity  $6 \times 10^8$  n/(cm<sup>2</sup>-s).

a.[5 pts] What is the density of the neutrons in the incident beam?

$$E = \frac{1}{2}mv^2 \Rightarrow v = \sqrt{\frac{2E}{m}} = \sqrt{\frac{2(2.5 \times 10^{-8} [Mev])}{939.6 [Mev/c^2]}} = 7.29 \times 10^{-4} [cm/s]$$

$$I = nv \Rightarrow n = \frac{I}{v} = \frac{6 \times 10^8}{7.29 \times 10^{-6}} = 8.23 \times 10^{11} [n/cm^3]$$

b.[5 pts] What is the capture rate (captures/second) in the target?

$$Rate = (15 \times 10^{-24} [\frac{cm^2}{nucleus}]) (6 \times 10^8 [n/cm^2 - s]) (\frac{(8.9)(6.022 \times 10^{23})}{62} [\frac{atoms}{cm^3}]) (0.05 [cm^3]) = 1.38 \times 10^{11} [captures/s]$$

c.[5 pts] At what rate is Ni-63 produced (atoms/second)?

$$1 \text{ capture} = 1 \text{ production so } (8.9) = 4.01 \times 10^{-15} [atoms/s]$$

d.[5 pts] What is the maximum activity that this experiment can produce (the maximum decays of Ni-63 per second)? (Assume that the Ni-63 does not interact with the neutrons, but is lost only via decay.)

The maximum decay rate would be equal to the production rate assuming all the Ni-63s decay immediately so  $(8.9) = 4.01 \times 10^{-15} [decays/s]$

e.[10 pts] Suppose another Ni-62 target is now employed. It is thick enough that beam attenuation cannot be neglected. The exit intensity is 25% lower than the incident beam intensity.

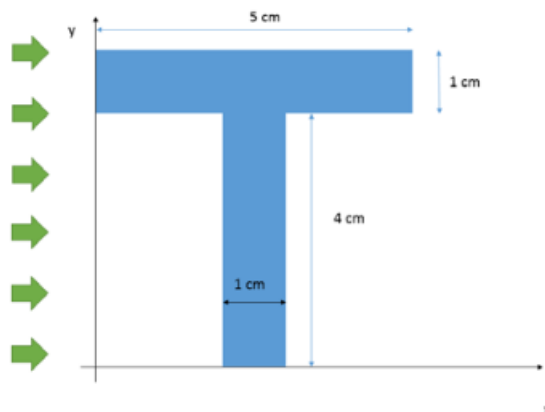
i. What is the target thickness?

$$I = I_0 e^{-\Sigma_t x} \Rightarrow x = -\Sigma_t \ln(\frac{I}{I_0}) = -(15 \times 10^{-24}) (\frac{(8.9)(6.022 \times 10^{23})}{62}) \ln(\frac{75}{100}) = 0.373 [cm]$$

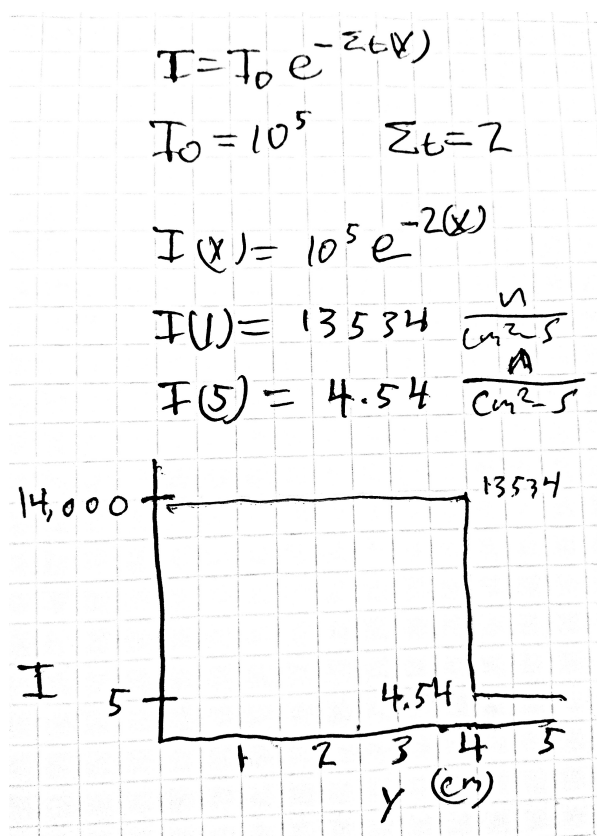
ii. What is the capture rate in this thick target (assume a beam-target interaction area of 3 cm<sup>2</sup>)?

$$Rate = (15 \times 10^{-24} [\frac{cm^2}{nucleus}]) (6 \times 10^8 [n/cm^2 - s]) (\frac{(8.9)(6.022 \times 10^{23})}{62} [\frac{atoms}{cm^3}]) (3 [cm^2]) (0.373 [cm]) = 8.71 \times 10^8 [captures/s]$$

Exercise 3 [20 pts]: [RRD] The letter T of the A & M logo is made of texasanmium. Its total cross section for thermal neutrons is 2 cm<sup>-1</sup>. A beam of thermal neutrons (intensity 105 n/(cm<sup>2</sup>-s)) is normally incident at x=0.



a.[10 pts]Plot the exiting uncollided intensity as a function of y for x=5cm



b.[10 pts]Assume that the letter T is 1-cm tall along the z-axis and that the beam fully covers the lateral faces, what is the reaction rate (reactions/s) in the letter T.

$$Rate = (10^5 [\frac{n}{cm^2-s}]) (2 [cm^{-1}]) ((5 [cm]) (1 [cm]) + (4 [cm]) (1 [cm])) (1 [cm]) = 1.80 \times 10^6 [reactions/s]$$

Exercise 4 [20 pts]: [Monte Carlo]Reproduce in python the Monte Carlo code shown in class. Requirements:

1. Slab thickness = 20 cm. Macroscopic total XS = 0.2 cm<sup>-1</sup>.
2. Number of bins = 12
- a.[5 pts]When running, the code you interactively request the user for the number of neutron histories.
- b.[5 pts]Provide 3 plots for the flux in the slab using (1) 100 histories, (2) 1,000 histories, (3) 10,000 histories.

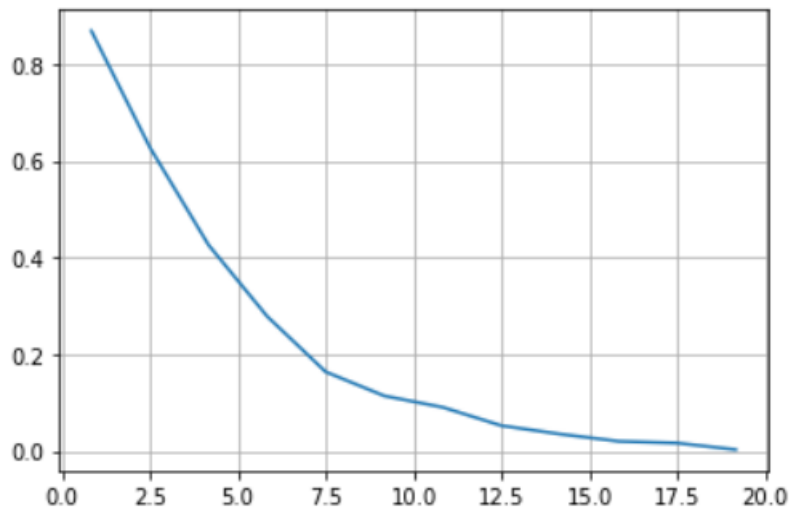


Figure 1: 100 histories

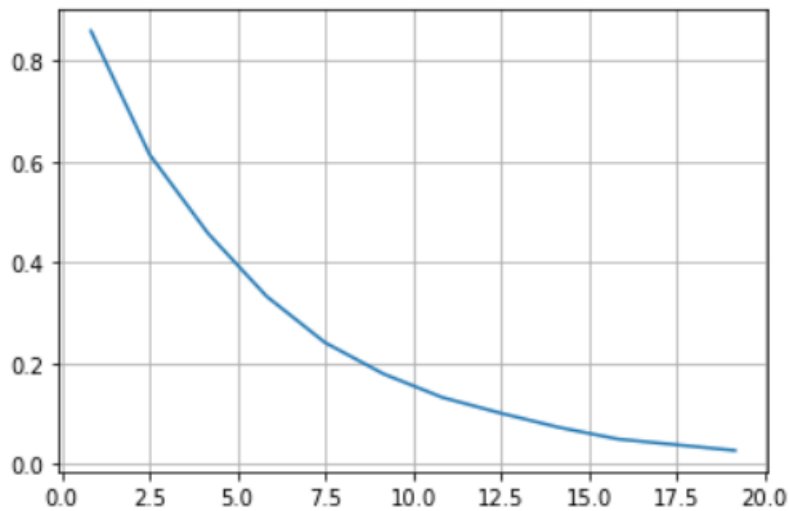


Figure 2: 1,000 histories

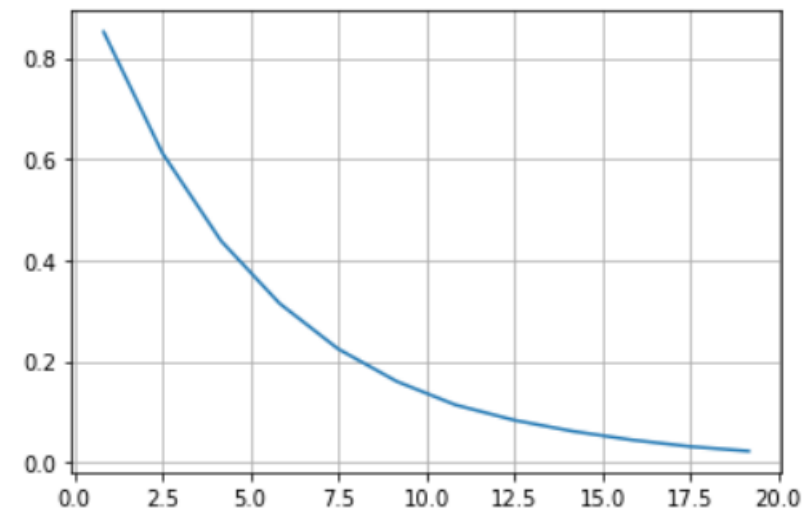


Figure 3: 10,000 histories

c.[5 pts]Provide your code in the HW submission (copy/paste). Code must be clean (meaningful variable names, no superfluous/unused lines of code, adequate comments/documentation)

```
1  import numpy as np
2
3  # slab description: width and total sigma (macroscopic cross section)
4  width = 20
5  sigt_t = 0.2
6  # numerical parameters
7  n_bins = 12
8  bin_width = width/n_bins
9
10 # number of histories to follow
11 neutron_histories = int(input("Input numebr of neutron histories to run: "))
12 # array to record track length left by neutrons
13 flux = np.zeros(n_bins)
14 # create value to count how many neutrons leak out
15 n_leak = 0
16
17 for i in range(neutron_histories):
18     absorbed = 0
19
20     # assume random number given
21     x = np.random.uniform(0,1,1)
22     # distance to be traveld
23     distance = -1/sigt_t * np.log(x)
24
25     if distance > width:
26         # if the neutron gets out of material
27         flux[:] += bin_width
28         n_leak += 1
29     else:
30         n_bins_traversed = int(np.floor(distance/bin_width))
31         flux[0:n_bins_traversed] += bin_width
32         # remainder of distance
33         distance_remainder = distance - bin_width * n_bins_traversed
34         flux[n_bins_traversed] += distance_remainder
35
36 # compute fraction of leaked neutons
37 frac = n_leak/neutron_histories
38 print('Fraction of neutrons that leaked out of slab: ')
39 print(frac)
40 # transform track length tally into a flux
41 flux /= bin_width
42 # normalize statistics this is now I(x)/I(0)
43 # Recall that I(x)/I(0) should be equal to: exp(-sigt*x)
44 flux /= neutron_histories
45
46 # plots
47 import matplotlib.pyplot as plt
48 x = np.linspace(0,width,n_bins+1)
49 xx = x[0:-1] + bin_width/2
50 plt.plot(xx, flux)
51 plt.grid()
52
```

d.[5 pts]Provide your python code (.py file to be submitted, we will be running the file)

Exercise 5 [10 EXTRA pts, MADATORY for Honors]: [Monte Carlo]Compute the fraction of neutrons that leak out of the slab. Provide a table that shows your Monte Carlo code results for 100, 1,000, and 10,000 histories. Compare it against the exact analytical answer.

For analytical solution  $I = I_0 e^{(-.2)(20)}$  for the fraction  $\frac{I}{I_0}$

From Code	Analytical
0.0	0.018
0.024	0.018
0.0193	0.018

As expected the Monte Carlo approaches the analytical solution as the number of histories increases.