

Rotation and Neoclassical Ripple Transport in ITER

November 27, 2016

Abstract

Neoclassical transport in the presence of non-axisymmetric magnetic fields cause a toroidal torque known as neoclassical toroidal viscosity (NTV). The toroidal symmetry of ITER will be broken by the finite number of toroidal field coils and the presence of perturbing ferromagnetic structures such as test blanket modules (TBMs) and ferritic inserts (FIs). 3D magnetic equilibria are calculated for an ITER steady-state scenario using the Variational Moments Equilibrium Code (VMEC), and neoclassical transport quantities in the presence of these error fields are calculated using the Stellarator Fokker-Planck Iterative Neoclassical Solver (SFINCS). As NTV is a complicated non-linear function of E_r , we study its behavior over a plausible range of E_r . We estimate the toroidal flow, and hence E_r , using a semi-analytic turbulent intrinsic rotation model. The magnitude of NTV torque density at large radii ($r/a \gtrsim 0.6$) is comparable to the NBI torque density at small radii ($r/a \lesssim 0.4$), but is opposite in direction and may significantly damp rotation in the edge. The FIs decrease neoclassical transport, while the TBMs do not produce significant NTV torque.

1 Introduction

Toroidal rotation is crucial to the experimental control of tokamaks: the magnitude of rotation is known to suppress resistive wall modes [2, 9], while rotation shear can decrease microinstabilities and promote formation of transport barriers [3, 32]. As some ITER scenarios will be above the no-wall limit [19], it is important to understand the sources and sinks of angular momentum for stabilization of external kink modes. One such sink is the toroidal torque caused by 3D non-resonant error fields. This change in rotation due to the breaking of toroidal symmetry of B has been observed in the Joint European Tokamak (JET) [6, 18], DIII-D [8], and NSTX [34].

In addition to the ripple due to the finite number (18) of toroidal field (TF) coils, the magnetic field in ITER will be perturbed by the addition of ferromagnetic components including ferritic inserts (FIs) and test blanket modules (TBMs). TBMs will be installed in three equatorial ports in order to test tritium breeding and extraction of heat from the blanket. The structural material for these modules is ferritic steel and will produce additional error fields in response to the background field. The TBMs will be installed during the H/He phase in order to test their performance in addition to their possible effects on confinement and transport [4]. It is important to understand their effect on transport during the early phases of ITER, including their influence on angular momentum transport. Experiments at DIII-D including mock-ups of TBMs indicated a reduction in toroidal rotation by as much as 60% due to magnetic braking [26]. However, neoclassical transport will likely differ in ITER due to its low collisionality. FIs are ferritic steel plates that will be installed in each of the toroidal field coil sections in order to mitigate energetic particle loss due to TF ripple [33]. As FIs will decrease toroidal field ripple, they may decrease the magnetic braking in ITER.

In the presence of non-axisymmetric error fields, the bounce-averaged radial particle drifts will not necessarily vanish. Several effects can give rise to non-zero radial current: particles trapped poloidally can experience a net radial drift (banana diffusion), and particles may become trapped in magnetic wells caused by the perturbing field (ripple trapping). For a general electric field, the electron and ion fluxes would not be identical. Ions tend to dominate neoclassical ripple transport, and the resulting particle flux is non-ambipolar. As ambipolarity must be restored for charge conservation, a radial electric field develops to hold the ions back. The radial current caused by outward ion flow induces a $\mathbf{J} \times \mathbf{B}$ torque which is typically counter-current, often referred to as neoclassical toroidal viscosity (NTV). Analytic expressions

for neoclassical fluxes in several rippled tokamak transport regimes have been derived, making assumptions about the magnitude of the perturbing field and the electric field, magnetic geometry, the collisionality, and the collision operator [27–29]. Rather than applying this simplified theory, in this paper a drift kinetic equation is solved using the Stellarator Fokker-Planck Iterative Neoclassical Solver (SFINCS) [17] to calculate neoclassical torque and heat flux for an ITER steady-state scenario. The SFINCS code does not exploit any expansions in collisionality, size of perturbing field, or magnitude of the radial electric field. It also allows for realistic experimental magnetic geometry rather than using simplified flux surface shapes.

In addition to NTV torque, the rotation profile in ITER will be determined by turbulence and neutral beam injection (NBI). As torque scales as $P/E^{1/2}$ for input power P and particle energy E , ITER’s neutral beams, with $E = 1$ MeV and $P = 33$ MW, will provide less momentum than in other tokamaks such as JET, with $E = 125$ keV for $P = 34$ MW [5]. NBI-driven rotation will also be smaller in ITER because of its relatively large moment of inertia, with major radius $R = 6$ m compared to 3 m for JET for comparable densities.

However, spontaneous rotation may be significant. Turbulence can drive significant flows in the absence of external momentum injection, known as intrinsic or spontaneous rotation, and this effect may be significant in ITER. This can be understood as a redistribution of turbulent momentum to produce large directed flows; in the case of tokamaks this must be in the (approximate) symmetry direction. According to gyrokinetic orderings and inter-machine comparisons by Parra [22], intrinsic rotation scales as $\sim \Delta T_i/I_p$, and core rotations may be on the order of 100 km/s in ITER. Rice’s scaling based on multi-experiment data [24] predicts a much larger rotation, $V_\zeta \propto \Delta W_p/I_p \approx 700$ km/s, where W_p is the stored plasma energy and I_p is the plasma current. Co-current toroidal rotation appears to be a common feature of H-mode plasmas and has been observed in electron cyclotron heated (ECH) [7], ohmic [7], and ion cyclotron range of frequencies (ICRF) [21] heated plasmas. Gyrokinetic GS2 simulations have also shown that low collisionality tokamaks have an inward radial momentum flux, corresponding to a rotation profile peaked in the core toward the co-current direction [1]. In an up-down symmetric tokamak, radial intrinsic angular momentum flux can be shown to vanish to lowest order in $\rho_* = \rho_i/a$ where a is the minor radius, but neoclassical departures from an equilibrium Maxwellian can break this symmetry and cause non-zero rotation in the absence of input momentum [1]. This type of symmetry-breaking will be considered in calculating anomalous rotation in section 4.

In section 2 the ITER steady state scenario considered is discussed. In section 3 free boundary magnetohydrodynamic (MHD) equilibrium in the presence of field ripple are presented. Section 4 presents estimation of NBI-driven rotation as well as turbulent rotation. This flow velocity is related to E_r in section 5. These calculations are used to determine neoclassical transport, as NTV torque is a non-linear function of E_r . The influence of TF ripple, TBMs, and FIs on neoclassical transport is evaluated, and a radial profile of torque is presented in section 6. In section 7 the scaling of transport calculated with SFINCS is compared with that predicted by NTV theory, and in section 8 neoclassical heat fluxes in the presence of ripple are presented. In section 9, we assess the influence of several radially-local magnetic drift schemes on the transport for this ITER scenario. In section 10 we summarize the results and conclude.

2 ITER Steady State Scenario

We consider an advanced ITER steady state scenario with significant bootstrap current and reversed magnetic shear [23]. The input power includes 33 MW NBI, 20 MW electron cyclotron (EC), and 20 MW lower hybrid (LH) heating for a fusion gain of $Q = 5$ and toroidal current of 9 MA. The discharge was simulated using the Tokamak Simulation Code (TSC) [15] and TRANSP [11] using a Coppi-Tang [14] transport model and EPED1 [31] pedestal modeling. The density, temperature, and safety factor q profiles are shown in figure 2.1. For neoclassical calculations, impurity species are ignored and it is assumed that $n_D = n_T = n_e/2$. Neoclassical transport will be analyzed at the radial locations indicated by dashed horizontal lines ($r/a = 0.5, 0.7, 0.9$). Throughout we will use the radial coordinate $r/a \propto \sqrt{\Psi_T}$ where Ψ_T is the toroidal flux.

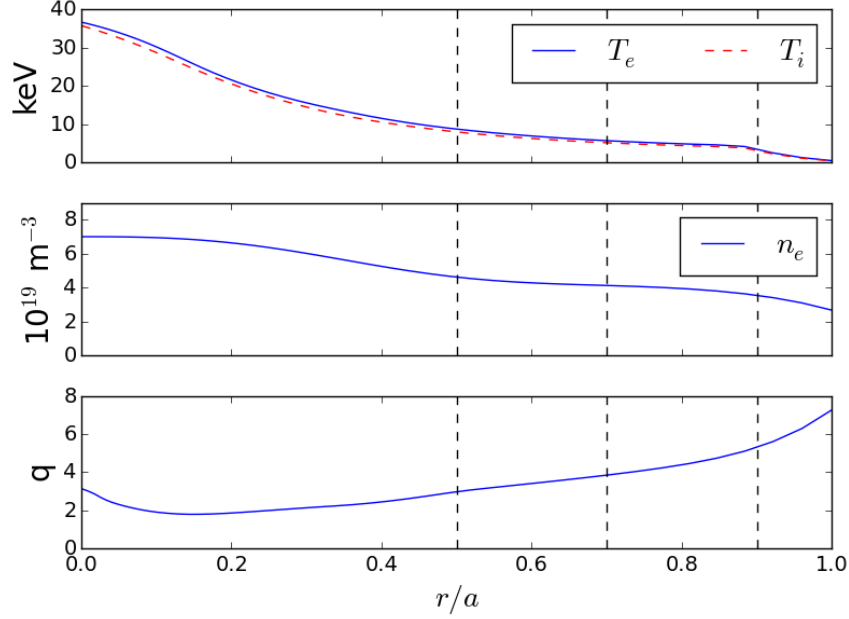


Figure 2.1: Radial profiles of temperature, density, and safety factor for the ITER steady state scenario [23]. Black dashed lines indicate the radial locations that will be considered for neoclassical calculations.

3 Free Boundary Equilibrium Calculations and Ripple Magnitude

The magnetic equilibrium was computed using the density, temperature, and q profiles from TRANSP along with filamentary models of the toroidal field (TF), poloidal field (PF), and center stack (CS) coils and their corresponding currents. The vacuum fields produced by the three TBMs and the FIs have been modeled using FEMAG [30]. The VMEC free-boundary equilibrium [13] is computed for four geometries: (i) including only the toroidal field ripple (ii) including TF ripple, TBMs, and FIs, (iii) TF ripple and FIs, and (iv) axisymmetric geometry.

We define the magnitude of the magnetic field ripple to be $\delta_B = (B_{max} - B_{min}) / (B_{max} + B_{min})$, where the maximum and minimum is evaluated at fixed radius and poloidal angle θ . In figure 3.1, δ_B is plotted on the poloidal plane for the three rippled VMEC equilibria. The components of B with toroidal mode number $n = \pm 18$ were removed from the case with TBMs and FIs in order to demonstrate the magnitude of the ripple produced by the TBMs alone (bottom right). When only toroidal field ripple is present, significant ripple persists over the entire outboard side, while in the configurations with ferritic inserts the ripple is much more localized in θ . When TBMs are also present the ripple is higher in magnitude (1.4%) around the outboard midplane, while in the other magnetic configurations $\delta_B \approx 1\%$ near the outboard midplane. For comparison, the TF ripple during standard operations is 0.08% in JET [6] and 0.6% in ASDEX Upgrade [20].

In figure 3.2, the magnitude of B is plotted as a function of toroidal angle ζ at $\theta = 0$ and $\theta = \pi/4$. Away from the midplane ($\theta = \pi/4$) the ferritic inserts greatly decrease the magnitude of the 18th harmonic of the perturbing field due to the finite number of TF coils. Near the midplane the FIs do not decrease the magnitude of this toroidal ripple as strongly, as the number of steel plates will be higher away from the midplane [30]. The TBMs add an additional perturbation around $\theta = 0$, as the TBM ports will be located about the midplane.

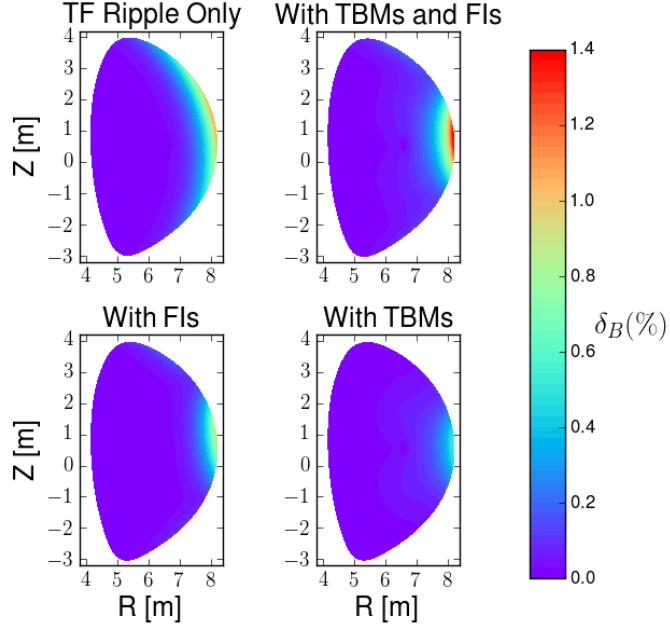


Figure 3.1: Magnetic field ripple, $\delta_B = (B_{max} - B_{min})/(B_{max} + B_{min})$, is plotted on the poloidal plane for VMEC free boundary equilibria (i) including only the toroidal field ripple (top left), (ii) including TF ripple, TBMs, and FIs (top right), (iii) including TF ripple and FIs (bottom left), and (iv) with TBMs only (bottom right). FIs decrease the radial extent of the ripple, while TBMs add an additional ripple near the outboard midplane.

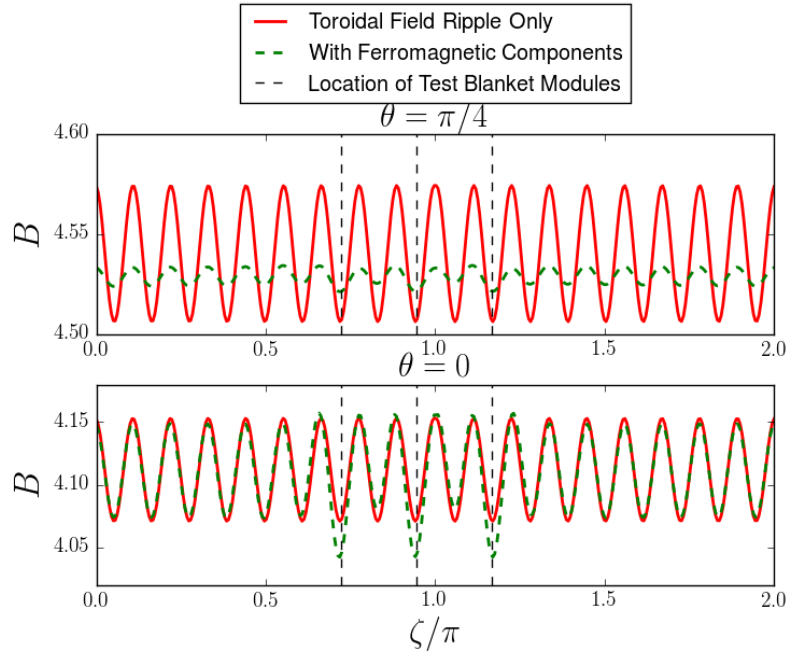


Figure 3.2: The magnitude of B as a function of toroidal angle (ζ) at $r/a = 1$, $\theta = 0$ and $\pi/4$. Vertical dashed lines indicate the toroidal locations of the TBM ports. The mitigating effect of the FIs is stronger away from the midplane, where an increased number of steel plates are inserted. The TBMs add an additional ripple near their locations at the $\theta = 0$.

4 Estimating Toroidal Rotation

In order to make predictions of the ripple transport in ITER, the radial electric field, $E_r = -\Phi'(r)$, upon which neoclassical transport depends non-linearly, must first be estimated. This is equivalent to predicting the parallel flow velocity, $V_{||}$, which scales monotonically with E_r . Throughout the paper, the $||$ subscript will denote the component of a vector in the $\mathbf{b} = \mathbf{B}/B$ direction. As we simply wish to determine a plausible value of E_r , the difference between $V_{||}$ and V_ζ , the toroidal flow, will be unimportant for our estimates. In our calculation of V_ζ the neoclassical torque is ignored: instead E_r is viewed as an input to neoclassical calculations from which the NTV torque can be obtained.

For this rotation calculation, angular momentum transport due to neutral beams and turbulence will be considered. There will be an additional torque caused by the radial current of orbit-lost alphas [25], but it will be negligible ($\approx 0.006 \text{ Nm/m}^3$). The following time-independent momentum balance equation is considered in determining V_ζ ,

$$\nabla \cdot \Pi_\zeta^{turb}(V_\zeta) + \nabla \cdot \Pi_\zeta^{NC}(V_\zeta) = \tau^{NBI}, \quad (4.1)$$

where Π_ζ^{turb} and Π_ζ^{NC} are the toroidal angular momentum flux densities due to turbulence and neoclassical effects and τ^{NBI} is the NBI torque density. For this paper the feedback of Π_ζ^{NC} on V_ζ will not be calculated. The determination of the change in rotation due to NTV would require iteratively solving this equation for V_ζ , as both Π_ζ^{turb} and Π_ζ^{NC} are functions of V_ζ . Π_ζ^{turb} consists of a diffusive term as well as a term independent of V_ζ which accounts for turbulent intrinsic rotation. An angular momentum pinch will not be considered for this analysis.

$$\Pi_\zeta^{turb} = -m_i n_i \chi_\zeta \langle R \rangle \frac{\partial V_\zeta}{\partial r} + \Pi_{int} \quad (4.2)$$

Here R major radius, m_i is the ion mass, and n_i is the ion density. The flux surface average is denoted by $\langle \dots \rangle$,

$$\langle \dots \rangle = \frac{1}{V'} \int_0^{2\pi} d\theta \int_0^{2\pi} d\zeta \frac{(\dots)}{\mathbf{B} \cdot \nabla \zeta} \quad (4.3)$$

$$V' = \int_0^{2\pi} d\theta \int_0^{2\pi} \frac{d\zeta}{\mathbf{B} \cdot \nabla \zeta}. \quad (4.4)$$

As it is difficult to model the interactions between external torques and intrinsic rotation, these two effects are considered separately in order to generate two estimates for V_ζ . To estimate NBI-drive rotation, TRANSP evolves the rotation profiles by balancing turbulent diffusion (assuming $\chi_\zeta = \chi_i$) and models of neutral beam torques from NUBEAM.

$$\tau^{NBI} = -m_i n_i \chi_i \langle R \rangle \frac{\partial V_\zeta}{\partial r}, \quad (4.5)$$

where τ^{NBI} is the total beam torque density calculated by NUBEAM including collisional, $\mathbf{J} \times \mathbf{B}$, thermalization, and recombination torques.

For the second estimate of toroidal rotation a semi-analytic intrinsic rotation model is used [12]:

$$V_\zeta(r) = -\langle R \rangle \int_r^a \frac{v_{ti} \rho_{*,\theta}}{2P_r L_T^2} \tilde{\Pi}(\nu_*) dr', \quad (4.6)$$

where $v_{ti} = \sqrt{2T_i/m_i}$ is the ion thermal velocity, $\rho_{*,\theta} = v_{ti} m_i / (eB_\theta \langle R \rangle)$, the Prandtl number $P_r = \chi_\zeta / \chi_i$ is taken to be 0.7. The normalized collision frequency $\nu_* = qRv_{ti} / (\nu_{ii} \epsilon^{3/2})$ where $\epsilon = r/\langle R \rangle$ is the inverse aspect ratio and $L_T = -(\partial \ln T_i / \partial r)^{-1}$ is the temperature gradient scale length. Equation 4.6 is obtained from assuming that Π_{int} balances turbulent momentum diffusion in steady state, $\Pi_{int} = m_i n_i \chi_\zeta \langle R \rangle \partial V_\zeta / \partial r$. This model considers the intrinsic torque driven by the neoclassical diamagnetic flows, such that $V_\zeta \sim \rho_{*,\theta} v_{ti}$ and $(V_\zeta / \langle R \rangle) \Pi_{int} / Q \sim \rho_{*,\theta}$. It is also assumed that $V_\zeta = 0$ at the wall. $\tilde{\Pi}(\nu_*)$ is an order unity function which characterizes the collisionality dependence of rotation reversals, determined from gyrokinetic turbulence

simulations [1]. Because of the ITER's low collisionality, we do not expect a rotation reversal, which is correlated with transitioning between the banana and plateau regimes. Equation 4.6 was integrated using profiles for the ITER steady state scenario.

The rotation predicted by these models is shown in figure 4.1. NBI torque contributes to significant rotation at $r/a \lesssim 0.4$, where the torque density also peaks (see figure 6.4), while turbulent torque produces much rotation in the pedestal. At the radii that will be considered for neoclassical calculations (indicated by dashed vertical lines), intrinsic turbulent rotation may dominate over that due to NBI. Therefore, for the purposes of estimating E_r the intrinsic rotation calculation of V_ζ will be used. However, it should be noted that V_ζ shown in figure 4.1 is an estimate based on scaling arguments, as much uncertainty is inherent in predicting turbulent rotation. We simply use this to estimate the value of E_r in ITER which determines the neoclassical transport regime. The intrinsic rotation calculated is comparable to that predicted from theoretical scaling arguments by Parra [22].

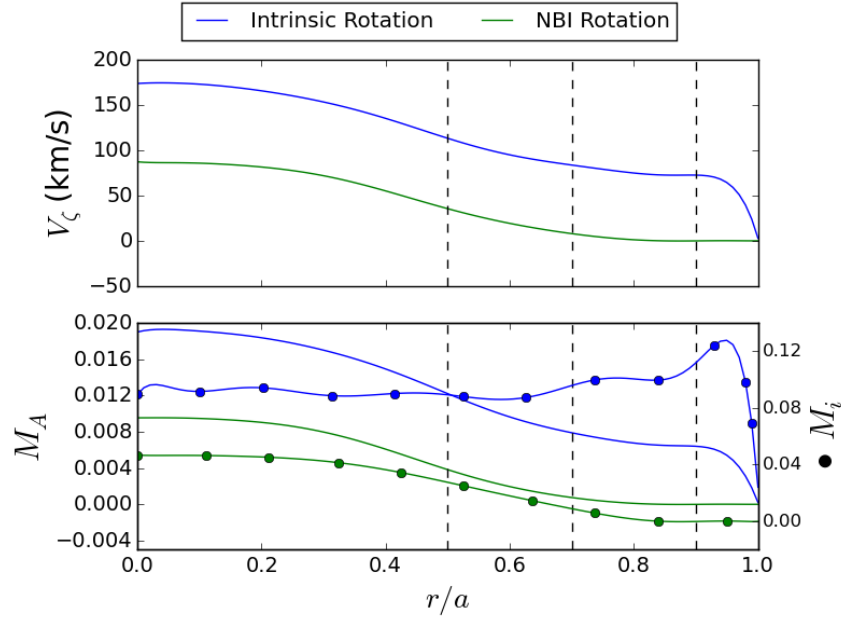


Figure 4.1: Toroidal rotation V_ζ due to turbulence and NBI (top) is shown along with corresponding Alfvén Mach number (bottom, solid), and sonic Mach number (bottom, bulleted). The intrinsic rotation calculation uses a semi-analytic model of turbulent momentum redistribution [12]. The NBI rotation is calculated from turbulent diffusion of NBI torque using NUBEAM and TRANSP [23]. Dashed vertical lines indicate the radial positions where SFINCS calculations are performed.

For stabilization of the resistive wall mode (RWM) in ITER, it has been estimated that a critical central rotation frequency $\omega_0 = V_\zeta(0)/\langle R \rangle \gtrsim 5\%$ of the Alfvén frequency, $\omega_A = B/(\langle R \rangle \sqrt{\mu_0 \rho_i})$, must be achieved given a peaked rotation profile. Given the central rotation $\approx 2\% M_A$ in figure 4.1, it may be difficult to suppress the RWM in ITER with rotation alone. As this calculation does not take into account magnetic braking, ω_0/ω_A is likely to be smaller than what is shown. Additionally, the TBM are known to increase the critical rotation frequency as they have a much shorter L/R time scale than the wall [19].

5 Relationship Between E_r and V_\parallel

Equipped with a profile of V_ζ , neoclassical calculations of V_\parallel are made in order to determine the radial electric field. The parallel flow velocity for species a is calculated from moments of the neoclassical distribution

function,

$$V_{||}^a = \left(\frac{1}{n_a} \right) \int d^3v v_{||} f_a, \quad (5.1)$$

which we calculate with the Stellarator Fokker-Planck Iterative Neoclassical Conservative Solver (SFINCS) [17] code. SFINCS is used to solve a radially-local drift kinetic equation for the gyro-averaged distribution function, f_{a1} , on a single flux surface including coupling between species.

$$(v_{||} \mathbf{b} + \mathbf{v}_E + \mathbf{v}_{ma}) \cdot (\nabla f_{a1}) - C(f_{a1}) = -\mathbf{v}_{ma} \cdot \nabla \psi \left(\frac{\partial f_{a0}}{\partial \psi} \right) + \frac{Z_a e v_{||} B \langle E_{||} B \rangle}{T_a \langle B^2 \rangle} f_{a0}. \quad (5.2)$$

Here a indicates species, f_{a0} is an equilibrium Maxwellian, $\psi = \Phi_T/2\pi$, and C is a linearized Fokker-Planck collision operator. The $\mathbf{E} \times \mathbf{B}$ drift is

$$\mathbf{v}_E = \frac{1}{B^2} \mathbf{B} \times \nabla \Phi \quad (5.3)$$

and the radial magnetic drift is

$$\mathbf{v}_{ma} \cdot \nabla \psi = \frac{m_a}{2Z_a e B^2} \left(v_{||}^2 + \frac{v_{\perp}^2}{2} \right) \mathbf{b} \times \nabla B \cdot \nabla \psi, \quad (5.4)$$

where $v_{||}$ and v_{\perp} are the components of the velocity coordinate parallel and perpendicular to \mathbf{b} , respectively. Transport quantities have been calculated using the steady state scenario ion and electron profiles and VMEC geometry. The $E_{||}$ term is negligible for this non-inductive scenario with loop voltage $\approx 10^{-4}$ V. For the calculations presented in this section and in section 8 the $\mathbf{v}_{ma} \cdot \nabla f_{a1}$ term is dropped. The effect of keeping this term is shown to be small in section 9.

The relationship between E_r and $V_{||}$ for electrons and ions at $r/a = 0.9$ is shown in figure 5.1. Note that only one curve is shown for each species as the addition of ripple fields did not change the dependence of $V_{||}$ on E_r significantly ($\leq 5\%$). While radial transport of heat and particles changes significantly in the presence of small ripple fields (see section 6), the parallel flow is much less sensitive to the perturbing field. It can be shown (see appendix A) that the component of f_1 that contributes to $V_{||}$ is of higher order in $\nu_* \ll 1$ than the component that contributes to Γ_{ψ} .

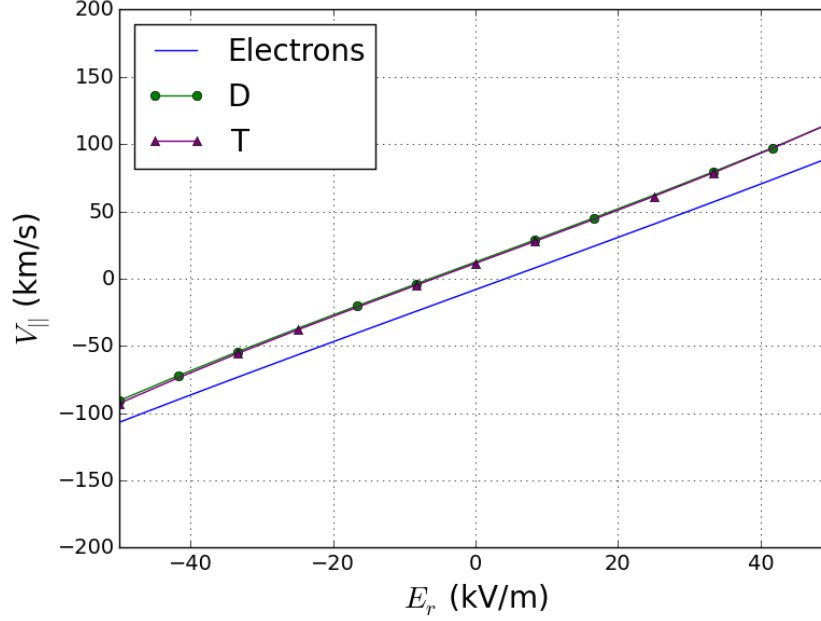


Figure 5.1: SFINCS calculation of the parallel flow $V_{||}$ at $r/a = 0.9$ for ions and electrons. Addition of ripple does not change tokamak neoclassical relationship between E_r and $V_{||}$ by a discernible amount on this scale although the radial particle fluxes, Γ_{ψ} , are sensitive to perturbing field.

6 Torque Calculation

The NTV torque density, τ^{NTV} , is calculated from radial particle fluxes, Γ_{ψ} , using the flux-friction relation,

$$\tau^{NTV} = -B^{\theta} \sum_a n_a q_a \Gamma_{\psi,a}, \quad (6.1)$$

where $B^{\theta} = \mathbf{B} \cdot \nabla \theta$ and the summation is performed over species. The calculation of τ^{NTV} for three geometries at $r/a = 0.9$ is shown in figure 6.1. The axisymmetric geometry does not contribute to NTV torque, as expected. Overall, the magnitude of the torque density with only TF ripple is larger than that with the addition of both the FIs and the TBMs. In figure 6.2 we show that the TBMs alone do not provide significant radial current, so the decrease in torque magnitude with both FIs and TBMs can be attributed to the decrease in ripple in the presence of FIs. The dashed vertical line indicates the value of $V_{||}$ and E_r predicted from the intrinsic rotation model. At this radius for the TF only case, $\tau^{NTV} = -0.058 \text{ Nm/m}^3$ while for the case with TBMs and FIs it is -0.012 Nm/m^3 . The circle indicates the offset rotation at the ambipolar E_r . If no other angular momentum source were present in the system, the NTV torque would drive the plasma to rotate at this velocity. Although the τ^{NTV} varies significantly between the two geometries they predict similar offset rotation velocities: $V_{\zeta} = -10 \text{ km/s}$ with TF ripple only and -6 km/s with TBMs and FIs. Note that for E_r greater than this ambipolar value, τ^{NTV} is counter-current while neutral beams and turbulence drive rotation in the co-current direction, so τ^{NTV} is a damping torque.

The NTV torque due to TF ripple only is larger in magnitude than τ^{NBI} while that with TBMs and FIs is of similar magnitude (see figure 6.4). NTV may also compete with the turbulent torque, $\tau^{turb} = -\nabla \cdot \Pi_{int} \approx 0.05 \text{ Nm/m}^3$ at this radius. Therefore, NTV torque may be key in determining the edge rotation in ITER. Note that the magnitude of τ^{NTV} peaks near E_r where $1/\nu$ transport dominates.

In order to decouple the influence of the FI ripple and the TBM ripple, the torque density is calculated at $r/a = 0.9$ for geometries including (i) TF ripple with FIs and TBMs, (ii) TF ripple with FIs, and (iii) only TBMs, shown in figure 6.2. The ripple due to the TBMs drives much less torque than the TF ripple with FIs does. While the FIs decrease the magnitude of the $n = 18$ component of B , the TBM contributes to low mode numbers (mostly $n = 1$). In the $\sqrt{\nu}$ regime ion transport scales as $\Gamma_{\psi} \sim \sqrt{n}$ and in the $1/\nu$

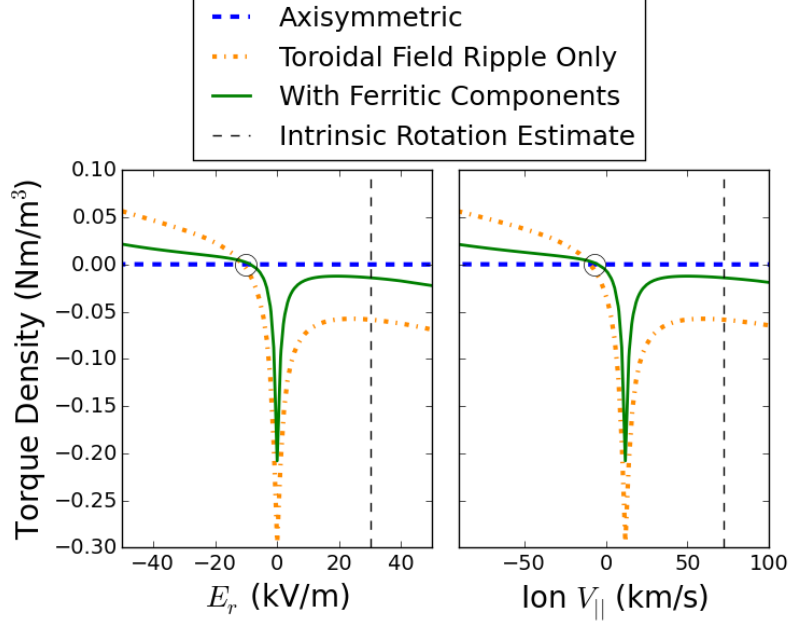


Figure 6.1: SFINCS calculation of NTV torque density as a function of E_r and $V_{||}$ at $r/a = 0.9$ is shown for 3 VMEC geometries: (i) axisymmetric (blue dashed), (ii) with TF ripple only (orange dash-dot), and (iii) TF ripple with FIs and TBMs (green solid). The vertical dashed line indicates the estimate of E_r and $V_{||}$ based on the intrinsic rotation model. The circle denotes the offset rotation at $V_{||} \sim -10$ km/s. The magnitude of τ^{NTV} at this radius is of similar magnitude to the NBI and turbulent torques but is opposite in direction (see figure 6.4).

regime $\Gamma_\psi \sim n^2$ [27], so it is reasonable to expect that the higher harmonic ripple of the FIs would drive more torque. Moreover, when transport is dominated by the $\mathbf{E} \times \mathbf{B}$ precession frequency ($\nu_{\text{eff}}/\omega_E \ll 1$), if the poloidal extent of the ripple $\Delta\theta$ is decreased, the characteristic time scale for detrapping $\Delta t \sim \Delta\theta/\omega_E$ will decrease. If diffusion scales as $D \sim f_t \Delta x^2 / \Delta t$, reducing the poloidal extent of ripple will decrease transport.

In figure 6.3, the SFINCS calculation of τ^{NTV} with TF ripple only is shown at $r/a = 0.5, 0.7$, and 0.9 . For these three radii the maximum $\delta_B = 0.26\%$, 0.51% , and 0.82% respectively. It is reasonable to expect that the magnitude of transport will decrease with decreasing ripple, as particles fluxes scale in the $1/\nu$ and ν regimes scale as $\Gamma_\psi \sim (\delta_B)^2$ and in the $\sqrt{\nu}$ regimes they scale as $\sim (\delta_B)$ [27]. Moreover, the poloidal extent of the ripple decreases with decreasing radius, so particles spend less time in the rippled region. On the other hand, transport in the $\sqrt{\nu}$ regime also scales with T_i , $\Gamma_\psi \sim v_{ti}^4 \sqrt{\nu_{ii}} \sim T_i^{5/4}$. This $\sqrt{\nu}$ regime applies to the region outside the thin ‘valley’ near $E_r = 0$, where $\omega_E > \nu_{ii}/\epsilon$ ($\omega_E = \frac{E_r}{B}$). The combined effect of decreased ripple and increased temperature with decreasing radius leads to comparable torques with decreasing radius in this region. However, the scaling with T_i is even stronger in the $1/\nu$ regime, where $\Gamma_\psi \sim v_{ti}^4/\nu_{ii} \sim T^{7/2}$. Indeed, we find that the magnitude of τ_{NTV} at $E_r = 0$ increases with decreasing radius.

In figure 6.4, we compare the magnitude of τ^{NTV} with τ^{NBI} and the turbulent momentum source causing intrinsic rotation, $\tau^{int} = -\nabla \cdot \Pi_{int}$. For the τ^{NTV} profile, the intrinsic rotation model is used for determining E_r at each radius. τ^{NBI} is determined from NUBEAM calculations [23]. τ^{int} is estimated using $\Pi_{int} \sim (\rho_\theta/L_T)\tilde{\Pi}(\nu_*)Q(\langle R \rangle/v_{ti})$ (see appendix B for details). Note that in the pedestal this model of turbulent torque is large as $\tau^{turb} \sim 1/L_T$.

Near the edge ($0.5 \lesssim r/a \lesssim 0.9$), NTV torque may dominate the rotation profile and will likely significantly damp rotation, decreasing MHD stability. However, the resulting rotation profile may be sheared because of the significant counter-current source at the edge and co-current NBI source in the core. This may provide a rotation shear as high as $\Delta V_\zeta/\Delta r \approx 0.8(v_{ti}/R)$ and may large enough to suppress micorturbulence and decrease turbulent transport [10].

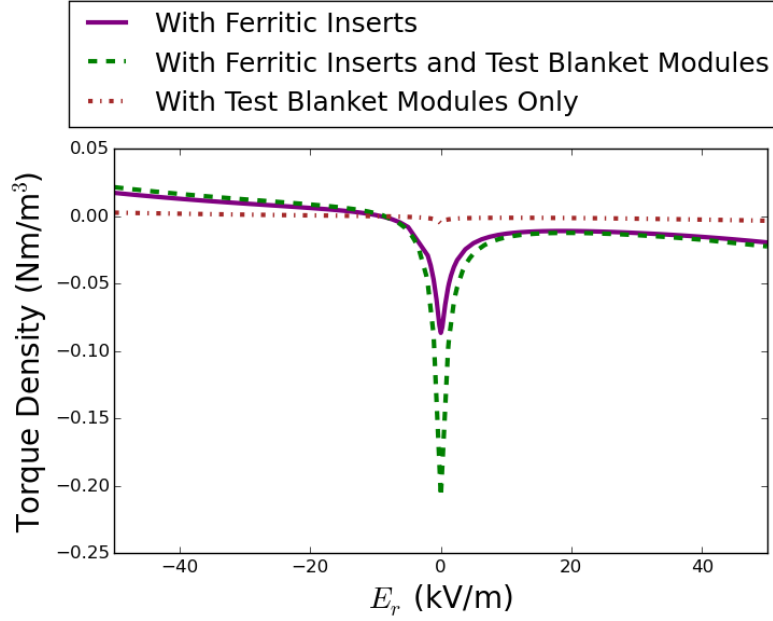


Figure 6.2: The NTV torque density at $r/a = 0.9$ for three magnetic geometries: (i) with FIs (purple solid), (ii) with TBMs (brown dash dot), (iii) and with FIs and TBMs (green dashed). The TBMs alone do not contribute significantly to neoclassical transport.

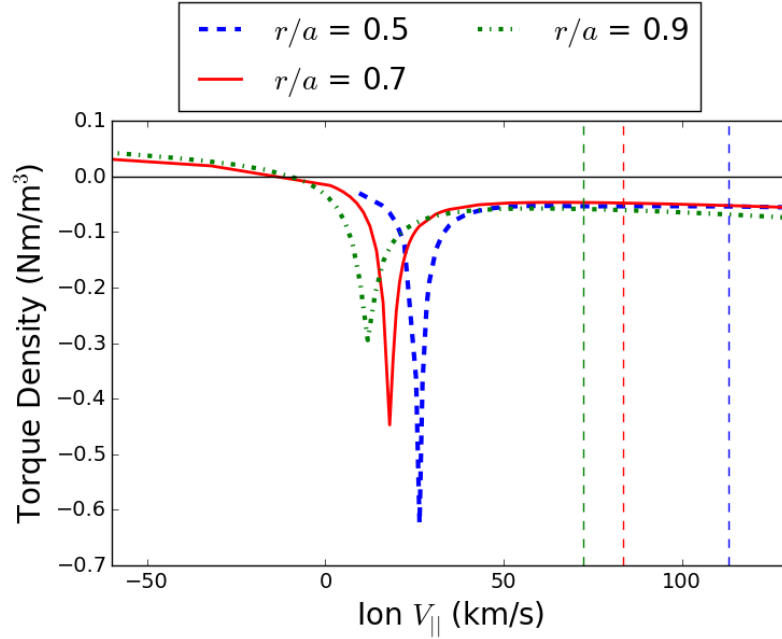


Figure 6.3: SFINCS calculation of NTV torque density (τ^{NTV}) as a function of ion $V_{||}$ for VMEC geometry with TF ripple only at $r/a = 0.5$ (blue solid), 0.7 (red dashed), and 0.9 (green dash-dot). Although the field ripple decreases with radius ($\delta_B = 0.82\%$ at $r/a = 0.9$, $\delta_B = 0.51\%$ at $r/a = 0.7$, $\delta_B = 0.26\%$ at $r/a = 0.5$), transport near $E_r = 0$ increases with decreasing radius because of strong scaling of neoclassical transport with temperature in the $1/\nu$ regime.

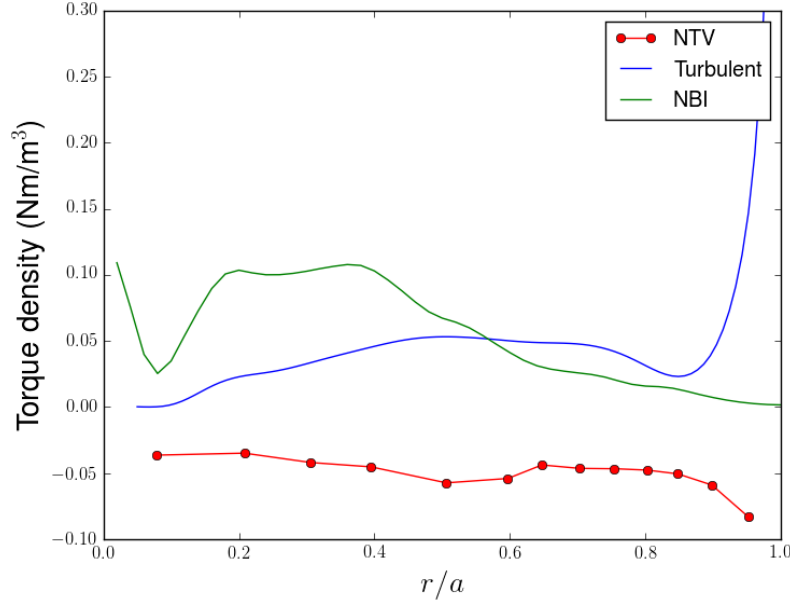


Figure 6.4: Torque density profiles of NTV torque density (τ^{NTV}) calculated with SFINCS, NBI torque density calculated from NUBEAM (τ^{NBI}), and estimate of turbulent intrinsic rotation momentum source (τ^{turb}). τ^{NTV} is calculated using SFINCS at E_r determined by intrinsic rotation model described in section 4. Turbulent torque is estimated using $\tau^{turb} \sim -\Pi_{int}/a$ where $\Pi_{int} \sim \rho_{*,\theta} \tilde{\Pi}(\nu_*) Q\langle R \rangle / v_{ti}$.

7 Scaling with Ripple Magnitude

The scaling of NTV transport with the magnitude of δ_B shows some agreement with that predicted by Shaing [29]. In figure 7.1, the NTV torque density calculated by SFINCS is shown as a function of the magnitude of the ripple, δ_B , for TF only geometry. The additional ferromagnetic ripple is not included, while the $n = \pm 18$ component of \mathbf{B} is rescaled. τ^{NTV} is calculated at $r/a = 0.9$ with $E_r = 30$ kV/m, corresponding to the intrinsic rotation estimate. The color-shaded background indicates the approximate regions of applicability of the $\sqrt{\nu}$ and ν regimes. The $1/\nu$ regime does not apply at this E_r , as $\omega_E \gg \nu/\epsilon$. E_r is also large enough that the resonance between v_E and v_m cannot occur, so the superbanana plateau and superbanana regimes are avoided.

In the collisional boundary layer, neoclassical fluxes scale as $\sqrt{\nu}$. This regime becomes relevant when the poloidal $\mathbf{E} \times \mathbf{B}$ precession frequency is larger than the effective collision frequency of detrapping banana particles, $\nu/\epsilon < \omega_E$, and the ripple magnitude is not large enough for collisionless detrapping to take effect, $\delta_B < (\epsilon\nu/\omega_E)^{1/2}$ [29]. In the collisionless trapping-retrapping regime, neoclassical fluxes scale with ν . This regime becomes relevant when $\delta_B > (\epsilon\nu/\omega_E)^{1/2}$, i.e. the perturbing field becomes large enough that poloidally trapped particles can become detrapped and retrapped by the ripple [27]. In the $\sqrt{\nu}$ regime $\tau^{NTV} \sim \delta_B^2$ and in the ν regime $\sim \delta_B$. For δ_B smaller than $\delta_B^* = 0.82\%$, the actual value of ripple at $r/a = 0.9$ for ITER geometry, the scaling of τ^{NTV} with δ_B is slightly shallower than what is predicted. As δ_B nears the boundary between the $\sqrt{\nu}$ and ν regimes in figure 7.1, the scaling of τ^{NTV} with δ_B becomes even less stiff and shows some agreement within the $\sqrt{\nu}$ regime. When $\delta_B \sim \epsilon$, the poloidal field variation no longer becomes the dominant perturbation, so perturbations this large have not been considered. The departure of numerical results from predicted scaling can be attributed to the analytic models which use a bounce-averaged kinetic equation and assume large aspect ratio.

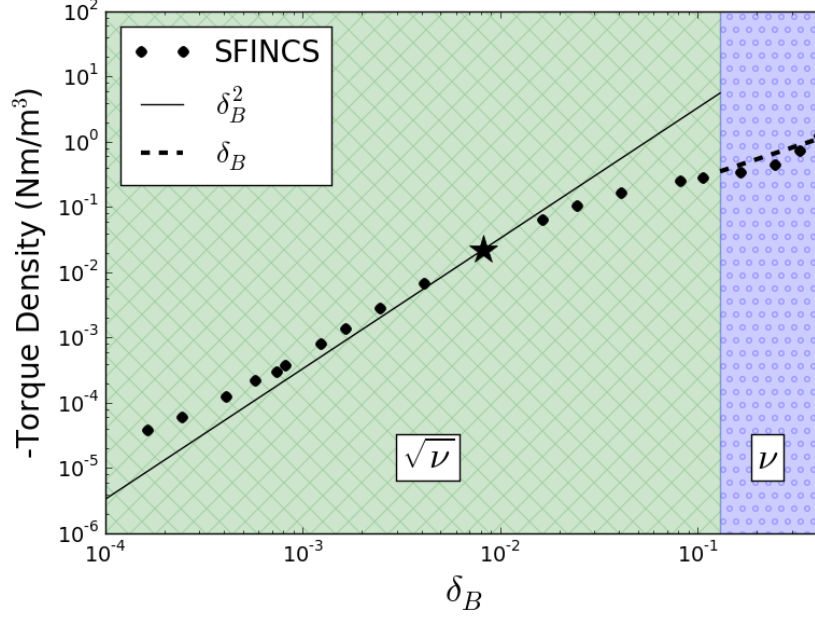


Figure 7.1: SFINCS calculations of NTV torque density as a function of δ_B at $r/a = 0.9$. A single value of $E_r = 30$ kV/m is used corresponding to the intrinsic rotation estimate. Color-shaded area indicates the approximate regions of applicability for rippled tokamak regimes described by Shaing [27, 29]: the $\sqrt{\nu}$ regime where $\tau^{NTV} \sim \delta_B^2$ and the ν regime where $\tau^{NTV} \sim \delta_B$.

8 Heat Flux Calculation

The breaking of toroidal symmetry drives an additional neoclassical heat flux as well as driving non-ambipolar particle fluxes. In figure 8.1, the SFINCS calculation of heat flux is shown for three magnetic geometries: (i) axisymmetric (blue solid), (ii) with TF ripple only (green dashed), and (iii) TF ripple with TBMs and FIs (red dash-dot). In the presence of TF ripple, the ripple drives an additional heat flux that is comparable to the axisymmetric heat flux. However, with the addition of the FIs the heat flux is reduced to the magnitude of the axisymmetric value, except near $E_r = 0$ where $1/\nu$ transport dominates.

While the radial ripple-drive particle fluxes will significantly alter the ITER angular momentum transport, the neoclassical heat fluxes are insignificant in comparison to the turbulent heat flux. Note that the neoclassical heat flux is $\lesssim 5\%$ of the heat flux calculated from heating and fusion rate profiles (see appendix B), ≈ 0.2 MW/m². Thus we can attribute $\gtrsim 95\%$ of the heat transport to turbulence. If ITER ripple were scaled up to $\delta_B \gtrsim 30\%$, the neoclassical ripple heat transport would be comparable to the anomalous transport.

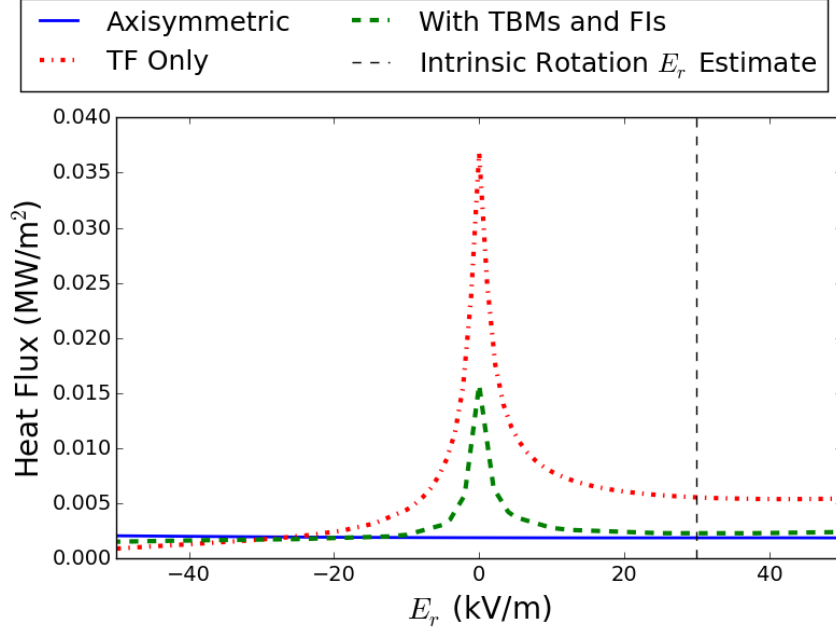


Figure 8.1: SFINCS calculation of neoclassical heat flux at $r/a = 0.9$ for three magnetic geometries: (i) axisymmetric (blue solid), (ii) with TF ripple only (green dashed), and (iii) TF ripple with TBMs and FIs (red dash-dot). The vertical dashed line corresponds to the intrinsic rotation E_r at this radius. These heat fluxes are much smaller than the anomalous heat transport is likely to be, $\approx 0.2 \text{ MW/m}^2$,

9 Radially Local Approximation of Magnetic Drifts

Although $(\mathbf{v}_E + \mathbf{v}_m) \cdot \nabla f_1$ is formally of lower order than the other terms in eq. 5.2, it has been found to be important when $\rho_* \sim \nu_*$ and has been included in other calculations of 3D neoclassical transport. In the SFINCS calculations shown above the $\mathbf{v}_m \cdot \nabla f_1$ term has not been included. As SFINCS does not maintain radial coupling of f_1 , there are several radially-local models of toroidal and poloidal \mathbf{v}_m that can be applied. The Kasilov form (eq. 67 in [16]) ignores the toroidally varying components of \mathbf{B} and the radial variation of q . The Martitsch form (eq. 16 in [20]) is the same as the Kasilov form but maintains magnetic shear dependence. This non-local effect has been shown to be important in some NTV transport regimes [20]. The Sugama form is obtained by subtracting off the radial component of \mathbf{v}_m and multiplying by a factor which ensures that $\mathbf{v}_m(\mu = 0) = 0$ in order to remove sources/sinks in velocity space. Each of these magnetic drift schemes are independent of the choice of Boozer angle.

An E_r scan at $r/a = 0.9$ is shown in figure 9.1. When $\mathbf{v}_m \cdot \nabla f_1$ is added to the kinetic equation, the $1/\nu$ valley is shifted toward a slightly negative E_r , corresponding to the region where $\mathbf{v}_E + \mathbf{v}_M = 0$. However, the addition of $\mathbf{v}_m \cdot \nabla f_1$ has a negligible effect on transport overall, and choice of magnetic drift scheme would not dramatically change the results in previous sections.

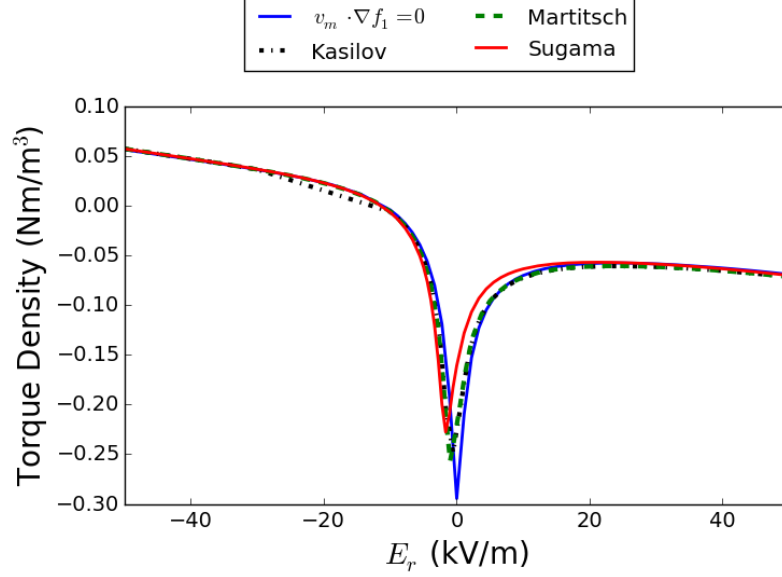


Figure 9.1: Comparison of magnetic drift models at $r/a = 0.9$. NTV torque density is calculated for the ITER steady state scenario as a function of E_r . The addition of $\mathbf{v}_m \cdot \nabla f_1$ in the kinetic equation solved by SFINCS has little effect on the neoclassical transport at this radius.

10 Summary

We calculate neoclassical transport in the presence of 3D magnetic fields, including toroidal field ripple and ferromagnetic components, for an ITER steady state scenario. We use an intrinsic turbulent rotation model described in section 4 to estimate E_r for neoclassical calculations. We find that even without τ^{NTV} , toroidal rotation will be $\lesssim 2\% M_A$, which is likely not large enough to suppress resistive wall modes [19]. We use VMEC free boundary equilibria in the presence of ripple fields to calculate neoclassical particle and heat fluxes using the drift-kinetic solver, SFINCS. At large radii $r/a \gtrsim 0.5$, τ^{NTV} is comparable to τ^{NBI} in magnitude but opposite in sign, which may result in flow damping at the edge and a decrease in MHD stability. However, the torque profile may also produce a significant rotation shear which could suppress turbulent transport. While the addition of FIs significantly reduces the transport ($\approx 80\%$ reduction at $r/a = 0.9$), the TBMs themselves produce very little NTV torque. Further analysis is needed to quantify how the addition of τ^{NTV} will change the rotation profile in ITER.

A Parallel Flow is Less Sensitive to Perturbing Field

In this appendix we will show that the contribution to $V_{||}$ from f_1 is much smaller than the contribution to Γ_ψ . As $V_{||}$ is an odd moment of $v_{||}$,

$$V_{||} = \left(\frac{1}{n}\right) \int d^3v v_{||} f, \quad (\text{A.1})$$

only the component of f_1 that is odd in $v_{||}$ will contribute. Similarly, only the component of f_1 that is even in $v_{||}$ will contribute to Γ_ψ as it is an even moment of $v_{||}$,

$$\Gamma_\psi = \left\langle \int d^3v (\mathbf{v}_m \cdot \nabla \psi) f \right\rangle. \quad (\text{A.2})$$

Starting with the following drift kinetic equation,

$$v_{||} \mathbf{b} \cdot \nabla f_1 + \mathbf{v}_d \cdot \nabla \psi \frac{\partial f_0}{\partial \psi} = C(f_1), \quad (\text{A.3})$$

we perform a secondary expansion of $f_1 = f_1^0 + f_1^1 + \dots$ in $\nu_* = \nu_{ii} R q / (\epsilon^{3/2} v_{ti}) \ll 1$. To lowest order we have

$$v_{||} \mathbf{b} \cdot \nabla f_1^0 = 0. \quad (\text{A.4})$$

To next order we have

$$v_{||} \mathbf{b} \cdot \nabla f_1^1 + \mathbf{v}_d \cdot \nabla \psi \frac{\partial f_0}{\partial \psi} = C(f_1^0). \quad (\text{A.5})$$

Bounce averaging eq. A.5 for banana trapped particles, we obtain

$$\langle \mathbf{v}_d \cdot \nabla \psi \rangle_b \frac{\partial f_0}{\partial \psi} = \langle C(f_1^0) \rangle_b, \quad (\text{A.6})$$

where

$$\langle \dots \rangle_b = \int_{\theta_-}^{\theta_+} \frac{d\theta B}{B^\theta} (\dots) \quad (\text{A.7})$$

where integration is taken between the bounce points. Note that in axisymmetry the left hand side of equation A.6 vanishes. We can expand f_1^0 in Legendre polynomials, $f_1^0 = \sum_l f_{1,l}^0 P_l(\xi)$ where $\xi = v_{||}/v$, which are eigenfunctions of the pitch angle scattering operator,

$$\langle C(f_1^0) \rangle_b = \frac{\nu}{2} \frac{\partial}{\partial \xi} \left((1 - \xi^2) \frac{\partial \langle f_1^0 \rangle_b}{\partial \xi} \right) = -\frac{\nu}{2} \sum_l l(l+1) P_l(\xi) \langle f_{1,l}^0 \rangle_b. \quad (\text{A.8})$$

As the left hand side of equation A.6 has terms proportional to ξ^2 and ξ^0 (see eq. 5.4), only Legendre modes P_0 and P_2 will survive after Legendre expansion. Thus $\langle f_1^0 \rangle_b$ will be even in $v_{||}$. On the other hand, f_1^1 must have a component that is odd in $v_{||}$,

$$f_1^1 = \int \frac{d\theta B}{B^\theta v_{||}} \left[C(f_1^0) - \mathbf{v}_d \cdot \nabla \psi \frac{\partial f_0}{\partial \psi} \right], \quad (\text{A.9})$$

as the bounce average of the first term in the integrand is even in $v_{||}$ and the second term is even in $v_{||}$. For $\nu_* \ll 1$, the departure of f_1^0 from its bounce average will be small, so departure of $V_{||}$ from the axisymmetric value will occur at order ν_* higher than the departure of Γ_ψ from its axisymmetric value.

B Approximate Turbulent Heat Flux and Torque

As Π_{int} is proportional to Q in our model, we must estimate Q using the input heating power and D-T fusion rates calculated with TRANSP and TSC. The LH, NBI, and ECH power densities (P_{LH} , P_{NBI} , and P_{ECH}) are integrated along with the fusion reaction rate density (R_{DT}) to calculate the total integrated heating source ($H(r)$),

$$\int_0^r dV H(r) = \int_0^r dV (P_{LH} + P_{NBI} + P_{ECH} + R_{DT}(3.5\text{MeV})). \quad (\text{B.1})$$

As $\int Q \cdot dS = \int H dV$,

$$Q(r) = \frac{\int_0^r dV (P_{LH} + P_{NBI} + P_{ECH} + R_{DT}(3.5\text{MeV}))}{A(r)}, \quad (\text{B.2})$$

where $A(r)$ is the surface area of a flux surface. We have shown in section 8 that the neoclassical heat flux is insignificant in comparison to $Q(r)$, so we can attribute $Q(r)$ to turbulent heat transport. The calculated Q is shown in figure B.1.

We estimate $\tau^{NBI} = -\nabla \cdot \Pi_{int} \sim -\Pi_{int}/a$ using

$$\Pi_{int} \sim \frac{\rho_\theta \tilde{\Pi}(\nu_*) Q \langle R \rangle}{v_{ti} L_T} \sim \frac{m_i}{e B_\theta a} \tilde{\Pi}(\nu_*) Q \langle R \rangle. \quad (\text{B.3})$$

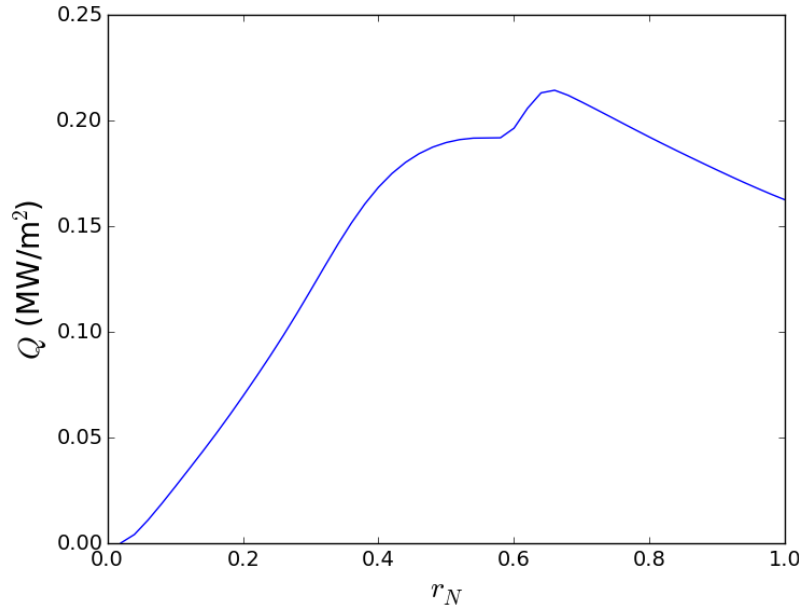


Figure B.1: Heat flux Q calculated with input heating and fusion rate profiles from TRANSP and TSC.

Acknowledgements

The authors would like to thank F. Parra, J. Hillesheim, J. Lee, and H. Smith for helpful input and discussions. This work was supported by the US Department of Energy through grants DE-FG02-93ER-54197 and DE-FC02-08ER-54964. The computations presented in this paper have used resources at the National Energy Research Scientific Computing Center (NERSC).

References

- [1] M. Barnes, F. I. Parra, J. P. Lee, E. A. Belli, M. F. F. Nave, and A. E. White. Intrinsic rotation driven by non-maxwellian equilibria in tokamak plasmas. *Physical Review Letters*, 111(5):1–5, 2013.
- [2] A. Bondeson and D. J. Ward. Stabilization of external modes in tokamaks by resistive walls and plasma rotation. *Physical Review Letters*, 72(17):2709–2712, 1994.
- [3] K. H. Burrell. Effects of $e \times b$ velocity shear and magnetic shear on turbulence and transport in magnetic confinement devices. *Physics of Plasmas*, 4(5):1499–1518, 1997.
- [4] V. A. Chuyanov, D. J. Campbell, and L. M. Giancarli. TBM Program implementation in ITER. *Fusion Engineering and Design*, 85(10-12):2005–2011, 2010.
- [5] D. Ćirić, A. D. Ash, B. Crowley, I. E. Day, S. J. Gee, L. J. Hackett, D. A. Homfray, I. Jenkins, T. T. C. Jones, D. Keeling, D. B. King, R. F. King, M. Kovari, R. McAdams, E. Surrey, D. Young, and J. Zacks. Performance of upgraded JET neutral beam injectors. *Fusion Engineering and Design*, 86(6-8):509–512, 2011.
- [6] P. C. de Vries, M.-D. Hua, D. C. McDonald, C. Giroud, M. Janvier, M. F. Johnson, T. J. J. Tala, K.-D. Zastrow, and the JET EFDA Contributors. Scaling of rotation and momentum confinement in JET plasmas. *Nuclear Fusion*, 48(6):65006, 2008.
- [7] J. S. DeGrassie, J. E. Rice, K. H. Burrell, R. J. Groebner, and W. M. Solomon. Intrinsic rotation in DIII-D. *Physics of Plasmas*, 14(5), 2007.
- [8] A. M. Garofalo, K. H. Burrell, J. C. Deboo, J. S. Degrassie, G. L. Jackson, M. Lanctot, H. Reimerdes, M. J. Schaffer, W. M. Solomon, and E. J. Strait. Observation of plasma rotation driven by static nonaxisymmetric magnetic fields in a tokamak. *Physical Review Letters*, 101(19), nov 2008.

- [9] A. M. Garofalo, E. J. Strait, L. C. Johnson, R. J. La Haye, E. A. Lazarus, G. A. Navratil, M. Okabayashi, J. T. Scoville, T. S. Taylor, and A. D. Turnbull. Sustained stabilization of the resistive-wall mode by plasma rotation in the DIII-D tokamak. *Physical Review Letters*, 89:235001, 2002.
- [10] T. S. Hahm. Rotation shear induced fluctuation decorrelation in a toroidal plasma. *Physics of Plasmas*, 1(9):2940, 1994.
- [11] R. Hawryluk et al. An empirical approach to tokamak transport. *Physics of plasmas close to thermonuclear conditions*, 1:19–46, 1980.
- [12] J. Hillesheim, F. Parra, M. Barnes, N. Crocker, H. Meyer, W. Peebles, R. Scannell, and a. Thornton. Dependence of intrinsic rotation reversals on collisionality in MAST. *Nuclear Fusion*, 55(3):032003, 2015.
- [13] S. Hirshman, W. van Rij, and P. Merkel. Three-dimensional free boundary calculations using a spectral Green’s function method. *Computer Physics Communications*, 43(1):143–155, 1986.
- [14] S. C. Jardin, M. G. Bell, and N. Pomphrey. TSC simulation of Ohmic discharges in TFTR. *Nuclear Fusion*, 33(3):371, 1993.
- [15] S. C. Jardin, N. Pomphrey, and J. Delucia. Dynamic modeling of transport and positional control of tokamaks. *Journal of Computational Physics*, 66(2):481–507, 1986.
- [16] S. V. Kasilov, W. Kernbichler, A. F. Martitsch, H. Maassberg, and M. F. Heyn. Evaluation of the toroidal torque driven by external non-resonant non-axisymmetric magnetic field perturbations in a tokamak. *Physics of Plasmas*, 21(9), 2014.
- [17] M. Landreman, H. M. Smith, A. Mollén, and P. Helander. Comparison of particle trajectories and collision operators for collisional transport in nonaxisymmetric plasmas. *Physics of Plasmas*, 21(4), 2014.
- [18] E. Lazzaro, R. J. Buttery, T. C. Hender, P. Zanca, R. Fitzpatrick, M. Bigi, T. Bolzonella, R. Coelho, M. DeBenedetti, S. Nowak, O. Sauter, and M. Stamp. Error field locked modes thresholds in rotating plasmas, anomalous braking and spin-up. *Physics of Plasmas*, 9(9):3906, 2002.
- [19] Y. Liu, A. Bondeson, Y. Gribov, and A. Polevoi. Stabilization of resistive wall modes in iter by active feedback and toroidal rotation. *Nuclear Fusion*, 44(2):232, 2004.
- [20] A. F. Martitsch, S. V. Kasilov, W. Kernbichler, G. Kapper, C. G. Albert, M. F. Heyn, H. M. Smith, E. Strumberger, S. Fietz, W. Suttrop, and M. Landreman. Effect of 3D magnetic perturbations on the plasma rotation in ASDEX Upgrade. *Plasma Physics and Controlled Fusion*, 58(7):074007, 2016.
- [21] J. M. Noterdaeme, E. Righi, V. Chan, J. DeGrassie, K. Kirov, M. Mantsinen, M. F. F. Nave, D. Testa, K. D. Zastrow, R. Budny, R. Cesario, A. Gondhalekar, N. Hawkes, T. Hellsten, P. Lamalle, F. Meo, F. Nguyen, and E.-J.-E. Contributors. Spatially resolved toroidal plasma rotation with ICRF on JET. *Nuclear Fusion*, 43:274–289, 2003.
- [22] F. I. Parra, M. F. F. Nave, A. A. Schekochihin, C. Giroud, J. S. De Grassie, J. H. F. Severo, P. De Vries, and K. D. Zastrow. Scaling of spontaneous rotation with temperature and plasma current in tokamaks. *Physical Review Letters*, 108(9):1–5, 2012.
- [23] F. Poli, C. Kessel, P. Bonoli, D. Batchelor, R. Harvey, and P. Snyder. External heating and current drive source requirements towards steady-state operation in ITER. *Nuclear Fusion*, 54(7):073007, jul 2014.
- [24] J. Rice, A. Ince-Cushman, J. DeGrassie, L.-G. Eriksson, Y. Sakamoto, A. Scarabosio, A. Bortolon, K. Burrell, B. Duval, C. Fenzi-Bonizec, M. Greenwald, R. Groebner, G. Hoang, Y. Koide, E. Marmar, A. Pochelon, and Y. Podpaly. Inter-machine comparison of intrinsic toroidal rotation in tokamaks. *Nuclear Fusion*, 47(11):1618–1624, nov 2007.
- [25] M. N. Rosenbluth and F. L. Hinton. Plasma rotation driven by alpha particles in a tokamak reactor. *Nuclear Fusion*, 36(1):55–67, 1996.
- [26] M. Schaffer, J. Snipes, P. Gohil, P. de Vries, T. Evans, M. Fenstermacher, X. Gao, A. Garofalo, D. Gates, C. Greenfield, W. Heidbrink, G. Kramer, R. La Haye, S. Liu, A. Loarte, M. Nave, T. Osborne, N. Oyama, J.-K. Park, N. Ramasubramanian, H. Reimerdes, G. Saibene, A. Salmi, K. Shinohara, D. Spong, W. Solomon, T. Tala, Y. Zhu, J. Boedo, V. Chuyanov, E. Doyle, M. Jakubowski, H. Jhang, R. Nazikian, V. Pustovitov, O. Schmitz, R. Srinivasan, T. Taylor, M. Wade, K.-I. You, and L. Zeng. ITER test blanket module error field simulation experiments at DIII-D. *Nuclear Fusion*, 51(10):103028, 2011.
- [27] K. Shaing, S. Sabbagh, and M. Chu. An approximate analytic expression for neoclassical toroidal plasma viscosity in tokamaks. *Nuclear Fusion*, 50(2):025022, feb 2010.
- [28] K. C. Shaing. Magnetohydrodynamic-activity-induced toroidal momentum dissipation in collisionless regimes in tokamaks. *Physics of Plasmas*, 10(5 1):1443–1448, 2003.

- [29] K. C. Shaing, P. Cahyna, M. Becoulet, J. K. Park, S. A. Sabbagh, and M. S. Chu. Collisional boundary layer analysis for neoclassical toroidal plasma viscosity in tokamaks. *Physics of Plasmas*, 15(8), 2008.
- [30] K. Shinohara, T. Oikawa, H. Urano, N. Oyama, J. Lonroth, G. Saibene, V. Parail, and Y. Kamada. Effects of ferromagnetic components on energetic ion confinement in ITER. *Fusion Engineering and Design*, 84(1):24–32, 2009.
- [31] P. Snyder, R. Groebner, J. Hughes, T. Osborne, M. Beurskens, a.W. Leonard, H. Wilson, and X. Xu. A first-principles predictive model of the pedestal height and width: development, testing and ITER optimization with the EPED model. *Nuclear Fusion*, 51(10):103016, 2011.
- [32] P. W. Terry. Suppression of turbulence and transport by sheared flow. *Reviews of Modern Physics*, 72(1):109–165, 2000.
- [33] K. Tobita, T. Nakayama, S. V. Konovalov, and M. Sato. Reduction of energetic particle loss by ferritic steel inserts in ITER. *Plasma Physics and Controlled Fusion*, 45(2):133–143, 2003.
- [34] W. Zhu, S. A. Sabbagh, R. E. Bell, J. M. Bialek, M. G. Bell, B. P. Leblanc, S. M. Kaye, F. M. Levinton, J. E. Menard, K. C. Shaing, A. C. Sontag, and H. Yuh. Observation of plasma toroidal-momentum dissipation by neoclassical toroidal viscosity. *Physical Review Letters*, 96(22):1–4, 2006.