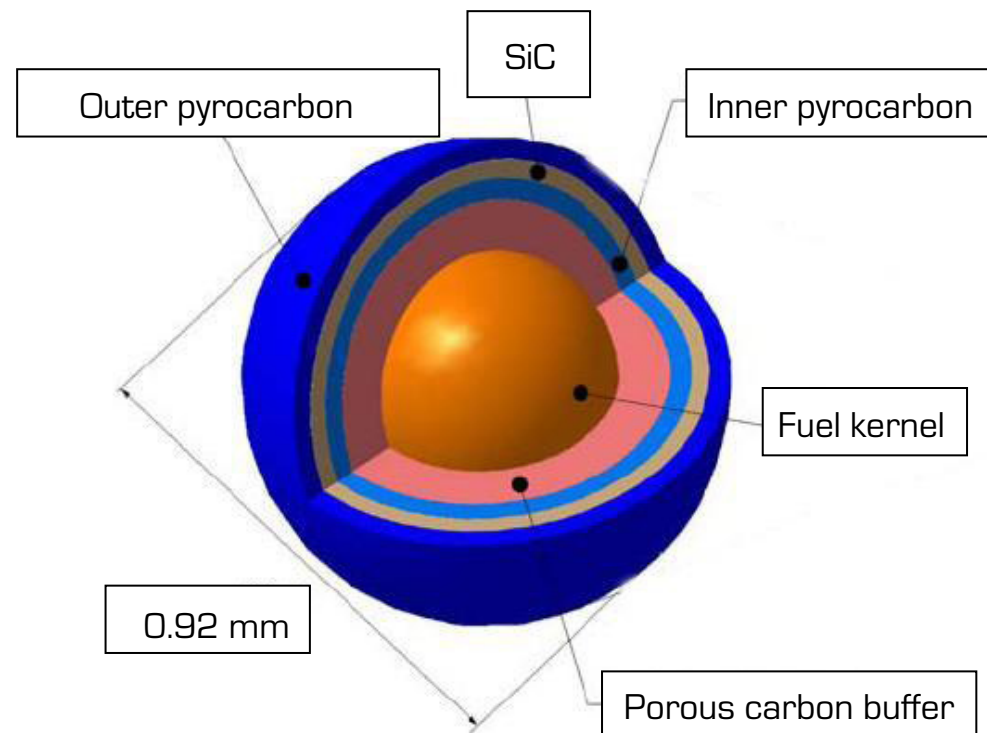


High temperature reactors (HTRs)



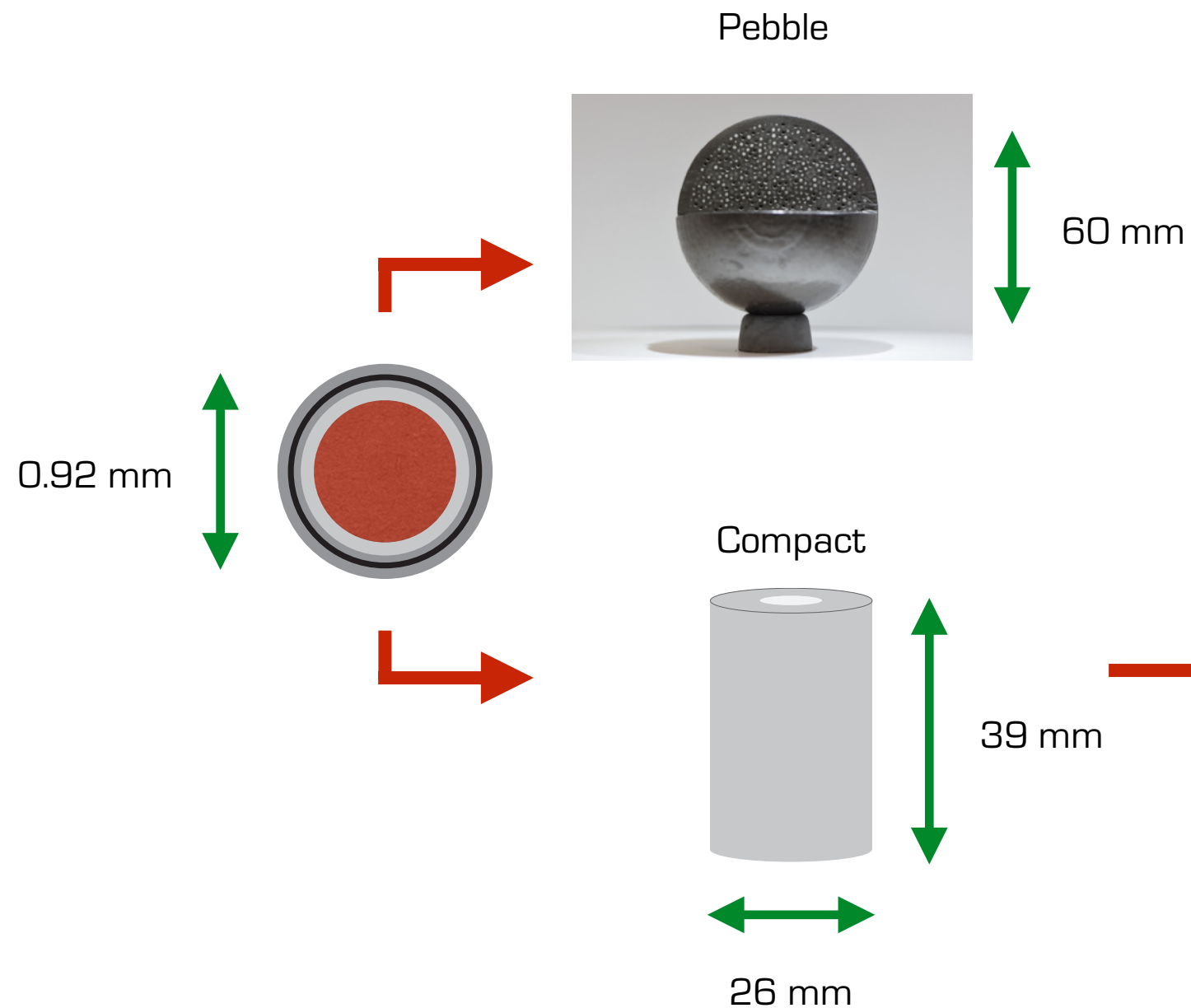
Helium cooled high temperature reactors are based on the use of coated particle fuels

- TRISO fuel coating withstands temperatures of up to 1600°C (cw 800°C for LWR cladding)
- Graphite moderator provides heat capacity
- Considered passively safe
- Two HTRs were in commercial operation:
- Fort St Vrain (330 MWe MWe, US)
- THTR (300 MWe, Germany)

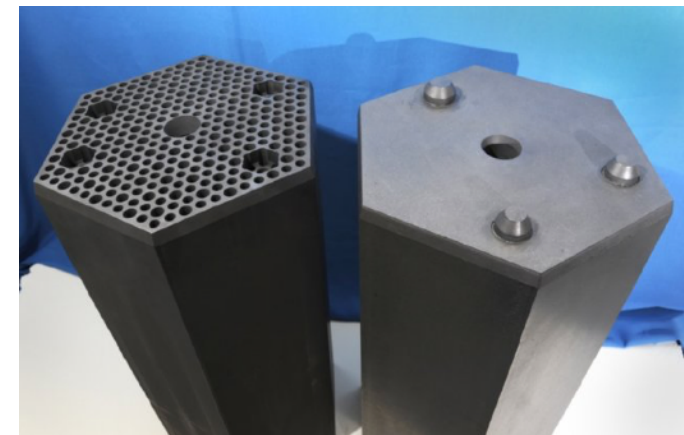


Fort St Vrain

Fuel elements in HTRs

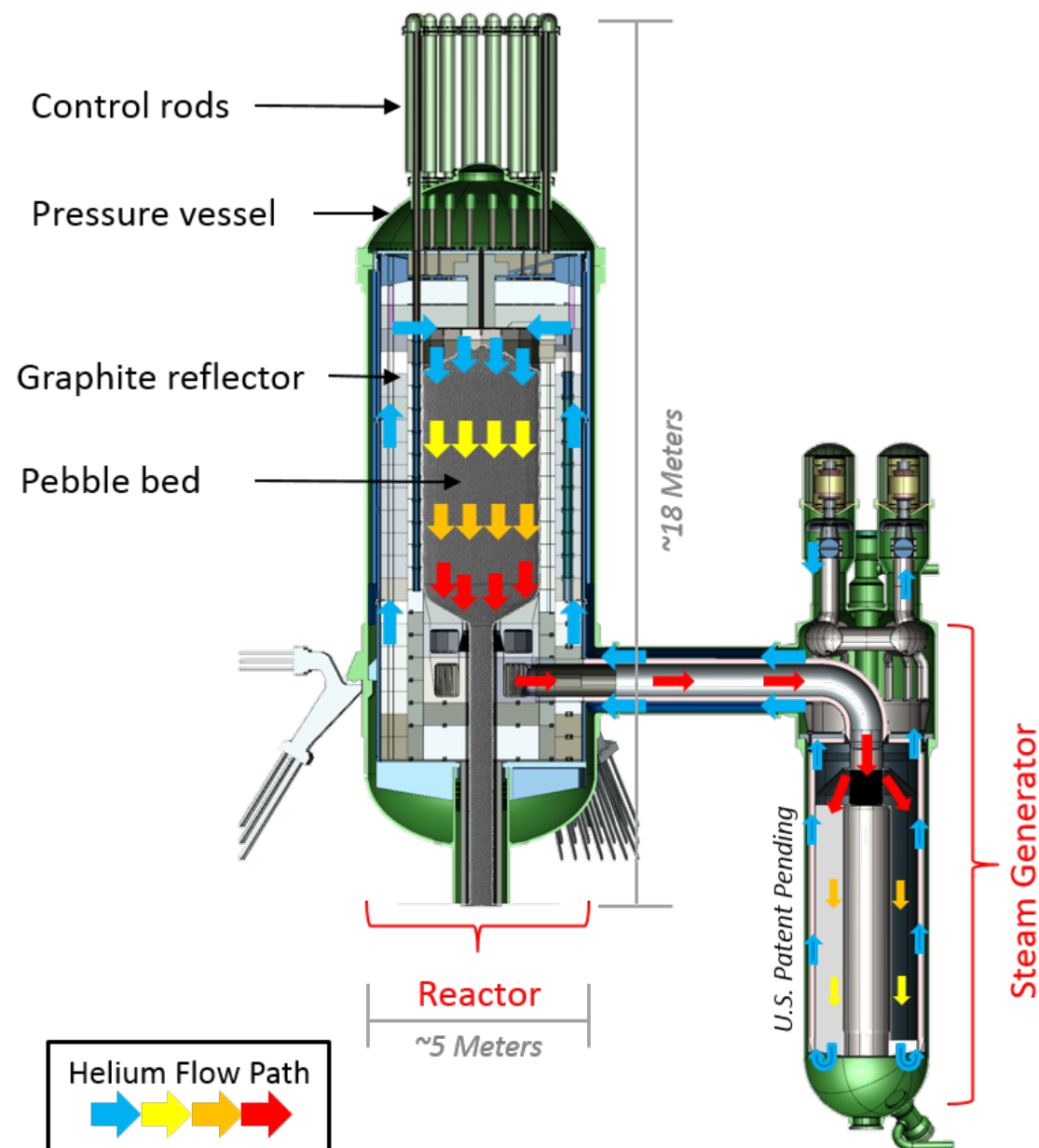


TRISO "microspheres" are embedded in a spherical [pebble] or cylindrical [compact] graphite matrix.



Compacts are stacked and inserted into a cylindrical holes in a "prismatic" graphite block.

Pebble bed HTR design

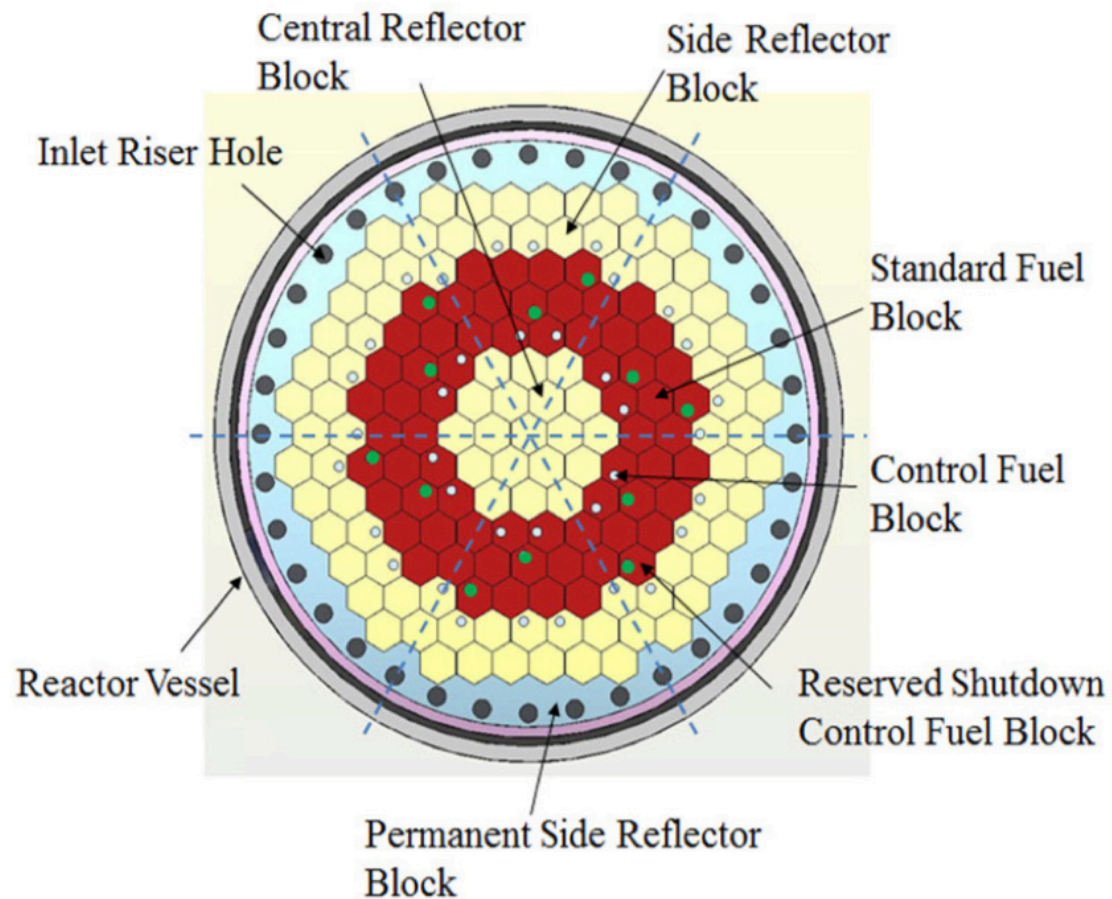


XE-100

Generic characteristics

- Pebbles extracted from bottom, reloaded at top
- He pressure: 7 MPa
- Fuel: UO_2 / UCO
- ^{235}U enrichment: $> 8\%$
- Power density: 3 MW/m^3 (2 kW/pebble)
- Coolant outlet temperature: 750°C
- Pebble residence time: 15 x 25 days
- Fuel burn-up: 90 GWd/ton

Prismatic HTR design



Control Rod
Drive Assembly

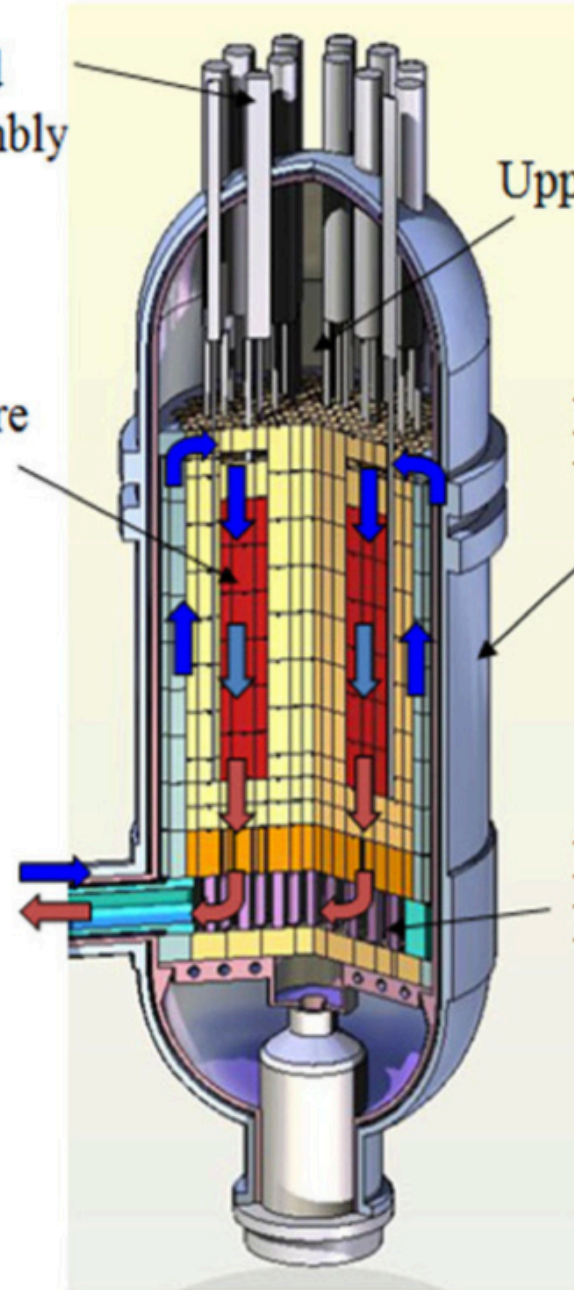
Active Core

Cold He
Hot He

Upper Plenum

Reactor
Vessel

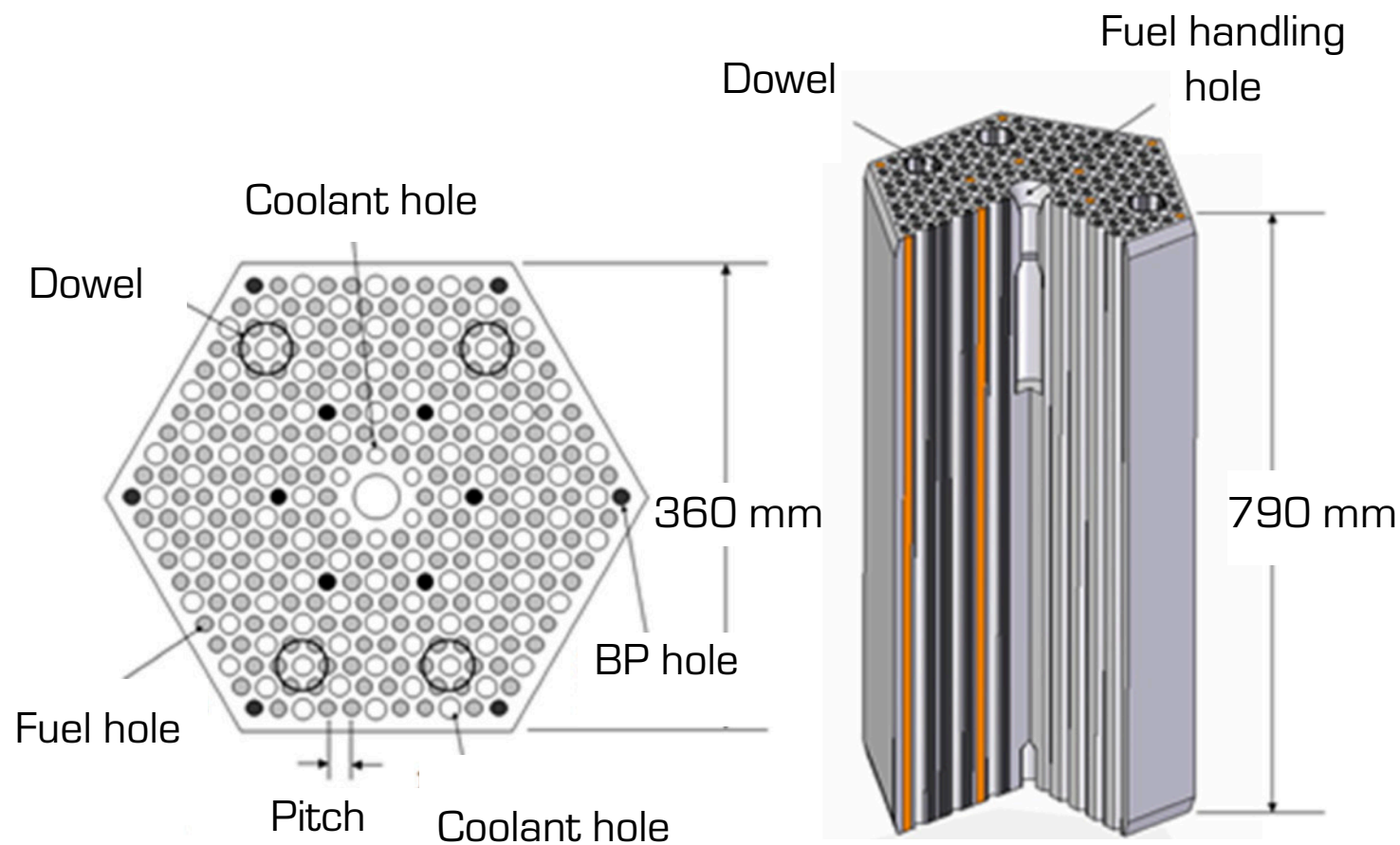
Lower
Plenum



- Annular core
- Coolant flow from top to bottom

Prismatic HTR fuel block design

Fuel block characteristics



- 204 fuel holes ($D = 12.7$ mm)
- 6 central coolant holes ($D = 12.7$ mm)
- 102 coolant holes ($D = 15.9$ mm)
- 12 burnable poison holes ($D = 12.7$ mm)
- Hole pitch: 18.8 mm
- Refueling cycle: 18 months
- Fuel burn-up: 100 GWd/ton

Pebble bed vs block design

- Advantages pertaining to pebble bed design

- Online-refuelling possible

- Dis-advantages of pebble-bed design

- Risk for mechanical damage to pebbles

- Spatial distribution of fuel-burnup difficult to determine

- Difficult to measure local gas temperature in core

- Need to measure pebble burn-up before re-load

- Dis-advantage of prismatic block design:

- Fuel reload is complicated

Incidents in HTRs



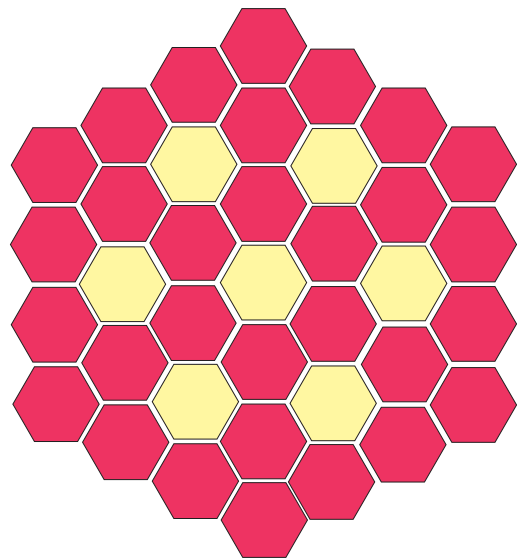
● Release of Sr-90 from AVR in Jülich

- 100 TBq of Sr-90 (2-3% of core inventory) released due to pebble temperatures exceeding design value.

● Damaged fuel pebbles in THTR, Hamm

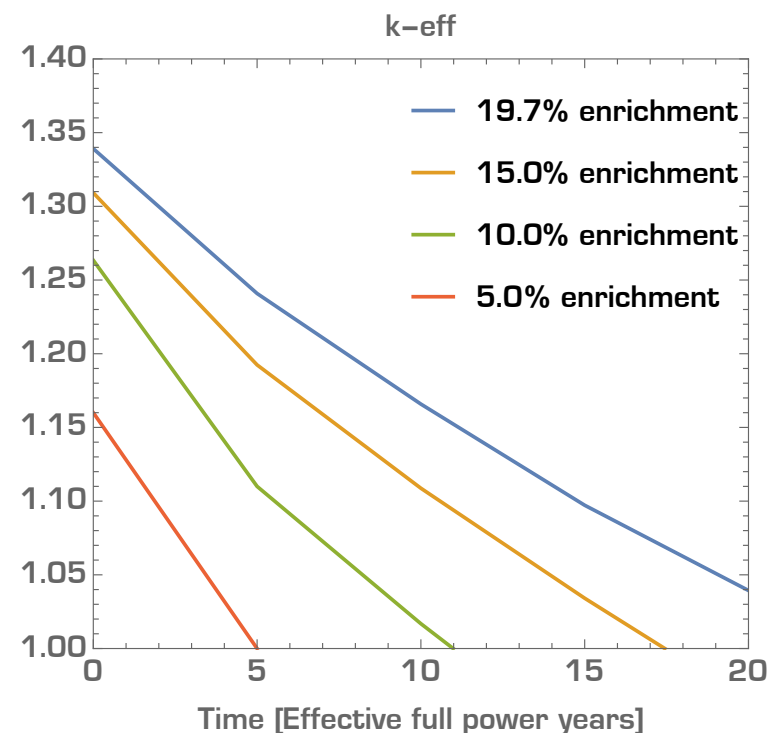
- 1-3 % of all pebbles were found to be mechanically damaged at extraction.
- On May 5th 1986, pebble damage occurred after forceful extraction of pebble stuck in fuel feed pipe channel, leading to a release of 100 MBq Co-60, Cs-137 and Pa-233.

Reactivity evolution in small prismatic core (U-Battery)



21 MWth \approx 9 MWe prismatic core

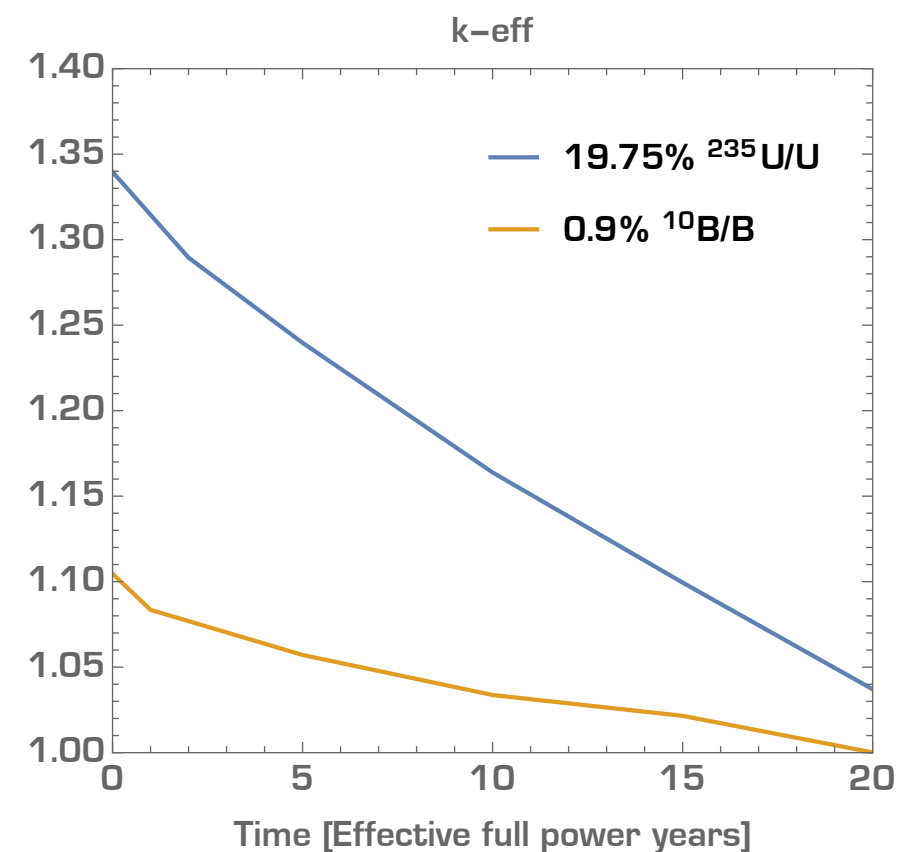
- 30 x 4 = 120 fuel blocks
- 1.8 tons of UO_2 fuel
- No burnable poison
- Internal moderator blocks
- Packing fraction of microspheres in compact: 30%
- > 20 years of residence time (9% burnup) w/o reload appears possible for 19.7% enrichment.



Burnable poisons in HTRs

Options studied for HTRs

- Gd_2O_3 particles - mixed with fuel spheres in pebble
- B_4C - in coated particles (pebble bed)
- B_4C - in poison channel (prismatic core)
- Er_2O_3 dispersed in graphite moderator



Small prismatic core having 6 channels filled with depleted B_4C