**Problem:**

This assignment builds on your solution of home assignment 03. Your task is to compute, by the Monte Carlo method, the relative change in the distance the fission neutron flies to its first collision when the atomic concentration of the material is increased by 0.01% (compared to the concentration that you selected in home assignment 03).

- First, try to compute this result by running two independent simple sampling simulations (at the two different atomic concentrations). Repeat the experiment and evaluate the standard deviation of the relative change of the distance the fission neutron flies to its first collision.

- Second, compute the result by the correlated sampling method. Evaluate the standard deviation of the result and compare it to the result you got in the first part of this assignment.

**Nuclear Fission:**

Each uranium-235 (U-235) atom has 92 protons and 143 neutrons, for a total of 235. The particle arrangement within uranium-235 is somewhat unstable, and the nucleus can dissolve if energized by an outside source. When a U-235 nucleus absorbs an added neutron, it swiftly splits into two halves. This is known as fission. When a U-235 nucleus divides, two or three neutrons are released. As a result, the chance of starting a chain reaction exists.

The task at hand is to calculate the mean distance that the fission neutrons fly till their first collision. To carry out the said task, I took into consideration a particular system mentioned before in the problem i.e., an infinite system composed of a single fissile nuclide at a reasonable mass density.

**Transition Kernel:**

We know that transition kernel, , is the probability density function of the distance, , traversed by a neutron to next collision. To perform a simple sampling simulation to evaluate the mean distance that fission neutrons fly to the first collision, I made use of the cumulative distribution function derived from transformation of the given probability density function

where, is the total macroscopic cross-section of a given nuclide. In addition to that, we assumed that the total macroscopic cross section is constant along the flight in between the two collisions.

Knowledge concerning the physics of the fission process is summarized in various nuclear data libraries such as JEFF, ENDF, or ENSDF. JEFF and ENDF list the cross sections of a wide range of nuclear reactions as well as the fission yields. ENSDF, on the other hand, is primarily concerned with nuclear structural data. We can find the distance, , to first collision of neutrons using the microscopic cross-section values collected from ENDF B-VIII.0 including the incident neutron energies via the JANIS database. These data can be used to simulate the values of distances, , to first collision of neutrons by the help of cumulative distribution function and inverse transform of it.

We know that

where, is the atomic number density and is the microscopic cross-section.

Therefore, the cumulative distribution function of the transition kernel,

(Homogeneous material)

Let the generated random number be, which lives in the interval

From , we get

Therefore, we can make the subject of the equation shown above to obtain

Since the term is also a random number from interval (0, 1). We can rewrite as

By generating random values of , I got different values of using cross section values. The cross-section values are found for the nearest incident energy of the neutron by using simple mathematical tool of linear interpolation. I have used the built-in interpolation module of Python “NumPy” library to find the cross-section values for nearest incident energy values of neutrons.

**Graph:**

Chart, line chart

Description automatically generated

Fig. 1: NEA / Incident neutron data / ENDF/B-VIII.0 / Cross sections / U235 / MT=1: (n,total) / Cross section

**Mean and standard deviation for sampling using random variables:**

After sampling values of unknown random variable , the expectation value of can be estimated by the mean value of those generated sampling values.

According to the central limit theorem,

In addition to that, variance of mean values of generated samples of the unknown random variable , :

where

However, the value of is difficult to obtain but it can be estimated if a considerably substantial number of samples are taken e.g., .

Therefore, variance

Hence, we just need to update the values of and to estimate the and after collecting a new sample of .

And the standard deviations are found by,

**Correlated Sampling:**

When calculating the difference between two extremely similar models, and , correlated sampling might be employed.

Typically, is a significantly modified form of . Since this difference between the expected values could be exceedingly small, the sampling would need to calculate and to a high degree of accuracy, which would take a significant amount of computational time.

Nevertheless, by examining the random variable, correlated sampling can easily solve this problem.

Let,

Variance of is

Where covariance of and ,

is the value showing how and are related to each other.

Sampling is more efficient than sampling and separately using independent input random variables and when and are significantly positively correlated to each other. That implies that and are remarkably similar.

**Results:**

Sampling of relative distance traversed by neutrons till first collision for U-235 nuclide of atomic number density, [2] and when is increased by :

For simple sampling: