

# FISPACT-II

## Case Study : Waste Assessments

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# Overview

1. FISPACT-II and waste assessments
2. Fusion Waste Prospects
  - EU DEMO waste assessments with FISPACT-II
  - Waste expectations from fusion
3. Assessment of possible steels for fusion
4. Conclusions

# FISPACT-II and waste assessments

- Before disposal radioactive materials are classified based on their nuclide content and activity.
- These classifications determine how the waste will be disposed of.
- For example low level waste (LLW) can typically undergo near surface disposal.
- Knowledge of activities and sources of activity is required for waste assessments.

**How can what a sample contains be determined before disposal?**

- FISPACT-II can model the nuclide evolution of a material under irradiation. Providing information on activities and sources of activity during and after decommissioning.
- From knowledge of a material's starting composition and the irradiation conditions a component's possible disposal routes can be determined.

# How to do a waste assessment with FISPACT-II?

1. Run a FISPACT-II calculation:
  - Define the material to be studied.
  - Define the flux and irradiation schedule.
  - Define cooling times, these are when the material will be assessed.
2. Apply waste criteria to FISPACT-II output to determine the waste classification

Waste criteria are typically limits on activity from a given sources.

e.g. UK LLW  $\beta+\gamma$  activity limit =  $1.2 \times 10^7$  Bq kg $^{-1}$

Need to sum  $\beta$  and  $\gamma$  Bq and divided my mass in kg, then compare to limit.

Output file			
0 TOTAL NUMBER OF NUCLIDES PRINTED IN INVENTORY = 117			
0 TOTAL CURIOS TOTAL ALPHA TOTAL BETA TOTAL GAMMA			
CURIE-MeV CURIE-MeV CURIE-MeV			
5.02240E-05 0.00000E+00 5.20711E-05 6.88742E-05			
0 ALPHA BECQUERELS = 0.00000E+00	BETA BECQUERELS = 1.763358E+06	GAMMA BECQUERELS = 9.492956E+04	
0 TOTAL ACTIVITY FOR ALL MATERIALS	1.85829E+06 Bq		
	3.99783E-04 Ci/cc	DENSITY	7.96E+00 gm/cc
TOTAL ACTIVITY EXCLUDING TRITIUM	1.85829E+06 Bq		
	3.99783E-04 Ci/cc		
0 TOTAL ALPHA HEAT PRODUCTION	0.00000E+00 kW	TOTAL HEAT PRODUCTION	7.16970E-10 kW
TOTAL BETA HEAT PRODUCTION	3.08680E-10 kW		
TOTAL GAMMA HEAT PRODUCTION	4.08290E-10 kW	TOTAL HEAT EX TRITIUM	7.16970E-10 kW
0 INITIAL TOTAL MASS OF MATERIAL	1.00000E-03 kg		
0 TOTAL MASS OF MATERIAL	1.00000E-03 kg		
NEUTRON FLUX DURING INTERVAL	1.11600E+10 n/cm^2/s		

$\beta+\gamma$  activity (in Bq kg $^{-1}$ ) =  $1.86 \times 10^9$  Bq kg $^{-1}$   
 In this example the limit is breached

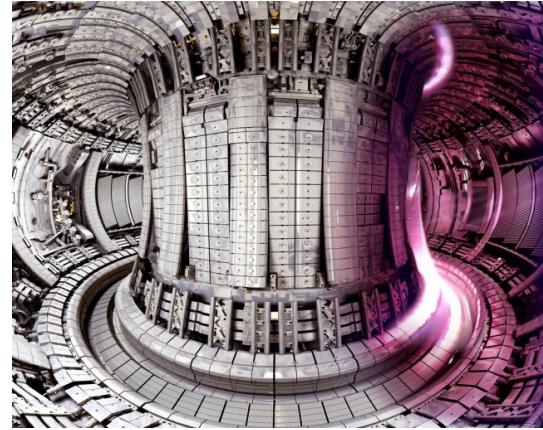
For a complete waste assessment all relevant criteria need to be considered at all relevant timesteps

# Fusion Waste Prospects

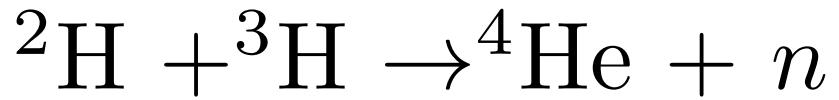
Waste assessment of complete reactor systems, example with EU DEMO

# Radioactive Waste and Fusion

- Current Public perception is that fusion is a source of abundant clean nuclear energy.
- Fission power plants can produce extremely long lived and highly active waste. Primarily from the nuclear fuel.
- Fusion power plants won't produce radioactive waste, right?



- Currently planned Fusion reactors will use Deuterium-Tritium (DT) reactions to generate energy.

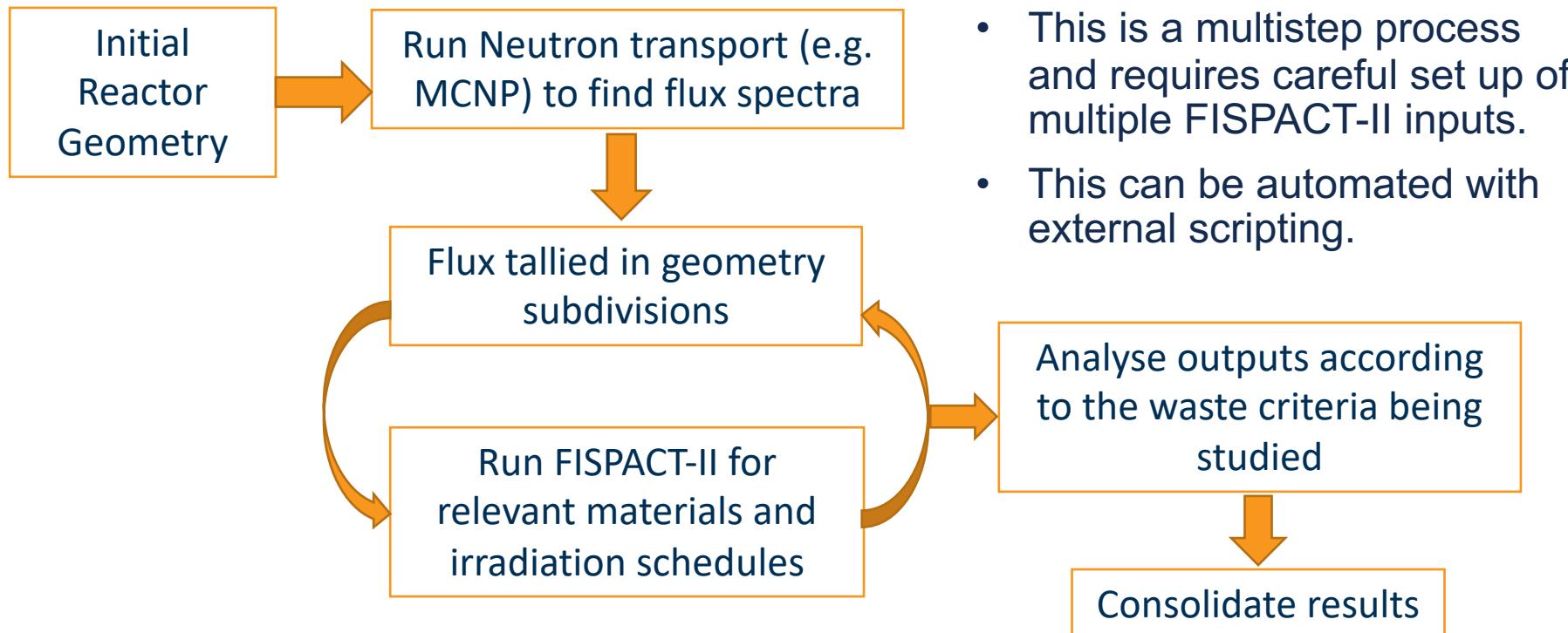


- Most of the energy carried by the neutron. These impinge on to the reactor components causing activation and material damage.
- Leaving radioactive waste at reactor end of life. Currently hoped to be low-level waste after 100 years.

# Complete Reactor waste assessments

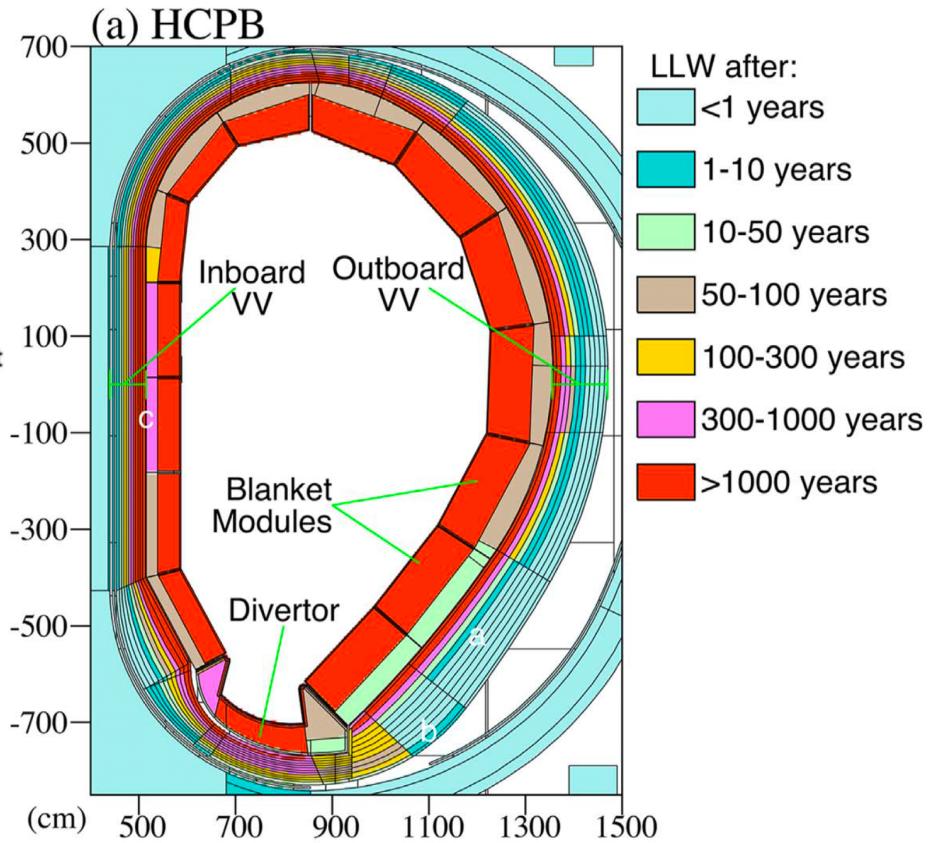
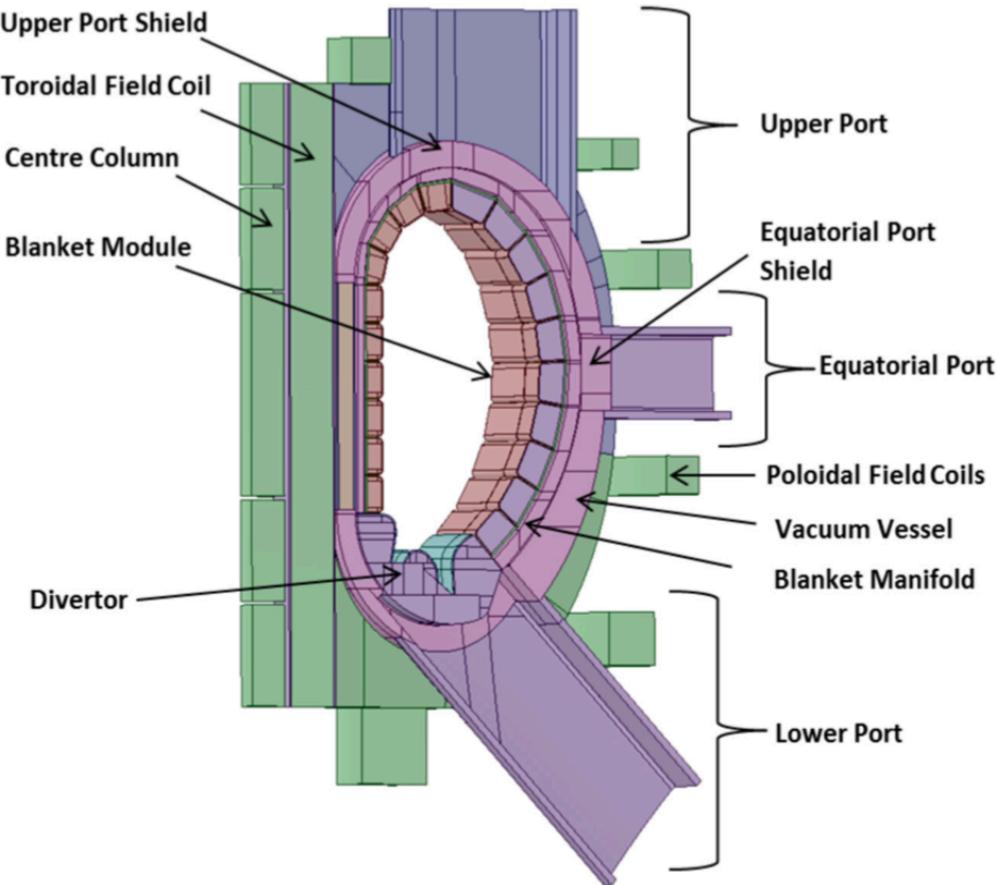
- Complex nuclear devices will need complete waste assessment of their materials prior to decommissioning.
- EU DEMO, a fusion reactor concept, has had several waste assessment performed on its geometry using FISPACT-II.

How is a complete reactor assessment performed?



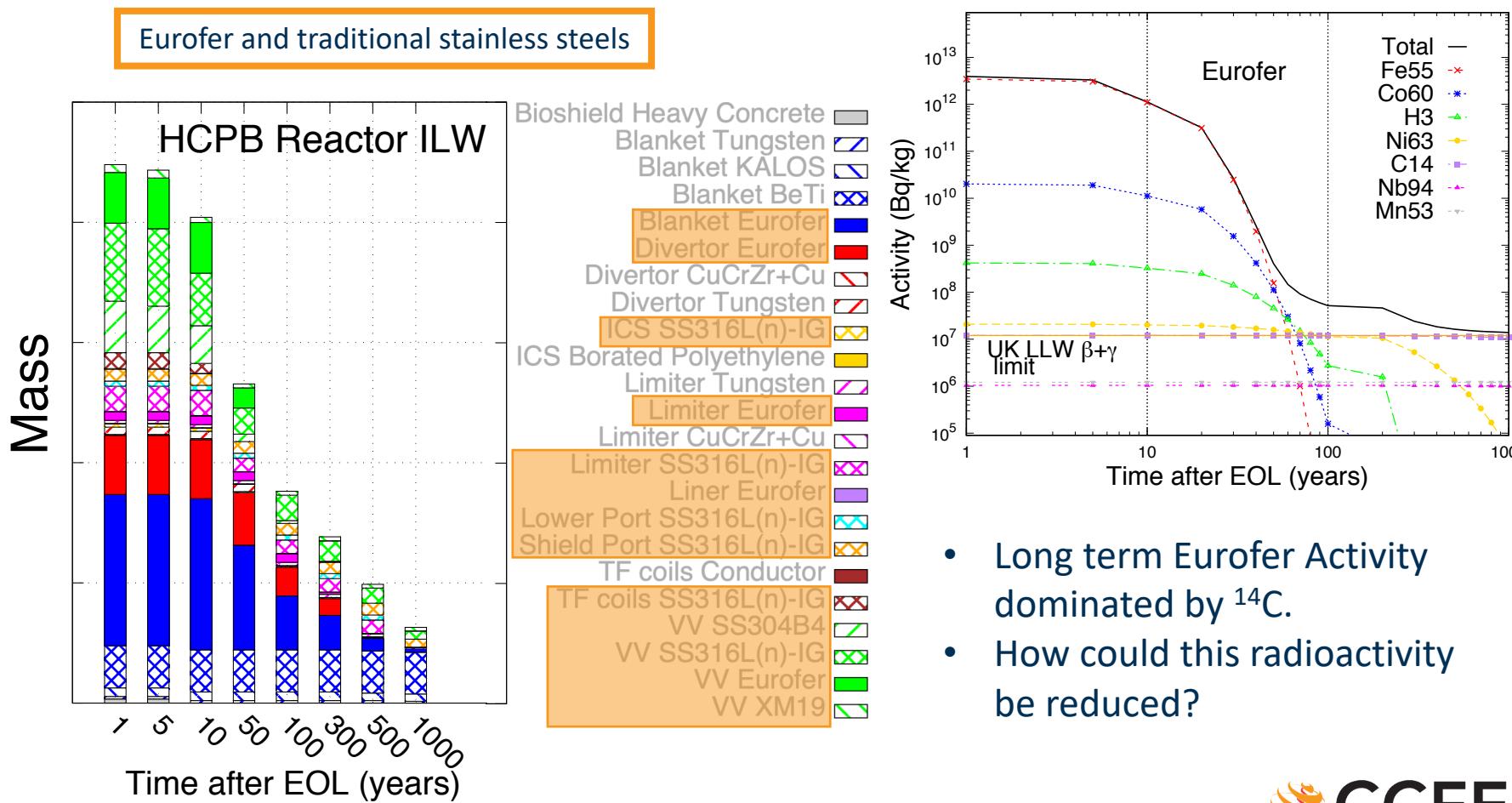
# Current Waste Expectations for DEMO

- FISPACT-II simulations suggest that near-plasma components will fail to be Low Level Waste (LLW) for over 1000 years!



# Current Waste Expectations for DEMO

- Further analysis reveals that a major contributor to expected DEMO waste are activated steels.
- If steel radioactivity could be reduced DEMO's waste prospects should improve.



# **Waste Assessment of possible steels for fusion**

Waste prospects of possible steels for fusion applications

# Waste Expectations for Fusion Steels

- Proportions of DEMO's Blanket and Vacuum Vessel are not expected to be called Low Level waste, by UK waste standards, for over 1000 years.
- What if different materials are used? DEMO is mostly planning to use the steels Eurofer and SS316.
- Are other possibilities available which could have improved waste prospects?

European Developed Reduced Activation(RA) Steel

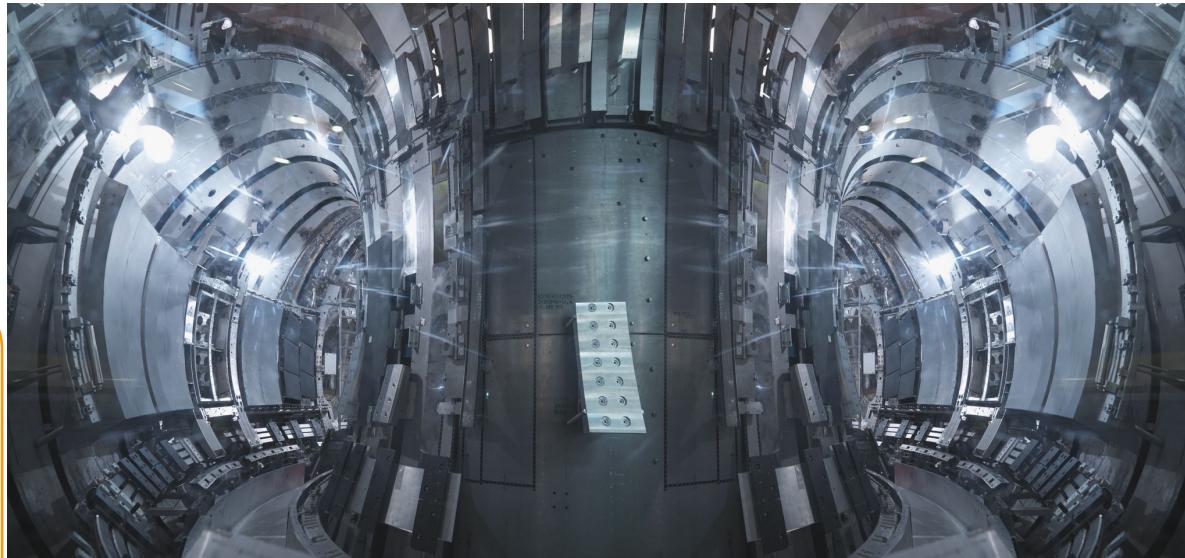
Traditional Stainless Steel

RA steels

- Hiperfer
- CLAM
- F82H
- ...

Stainless steels

- XM19
- Inconel 718
- T91
- ...



# Waste Expectations for Fusion

- What is Low Level Waste really? Near Surface but what else?
- Different countries have differing definitions of what classes as Low-Level waste, so which one should be used?

Country	Activity Limits ( $\text{Bqkg}^{-1}$ ) [ $\dagger \text{Bqm}^{-3}$ ]							Example limits from complex waste management systems
	$\alpha$	$\beta + \gamma$	$^3\text{H}$	$^{14}\text{C}$	$^{63}\text{Ni}$	$^{94}\text{Nb}$	$^{99}\text{Tc}$	
UK	$4 \times 10^6$	$1.2 \times 10^7$	-	-	-	-	-	
US	-	-	$\dagger 1.48 \times 10^{12}$	$\dagger 2.96 \times 10^{12}$	$\dagger 2.59 \times 10^{14}$	$\dagger 7.4 \times 10^9$	$\dagger 1.11 \times 10^{11}$	
France	-	-	$2 \times 10^8$	$9.2 \times 10^7$	$3.2 \times 10^9$	$1.2 \times 10^5$	$4.4 \times 10^7$	
Russia	$1 \times 10^3$	$1 \times 10^7$ ( $-^3\text{H}$ )	$\dagger 1 \times 10^{11}$	$\dagger 3 \times 10^{12}$	$\dagger 2.59 \times 10^{14}$	$\dagger 7.4 \times 10^9$	$\dagger 1.1 \times 10^{11}$	
Spain	$3.7 \times 10^6$	$3.7 \times 10^7$	$1 \times 10^9$	$2 \times 10^8$	$1.2 \times 10^{10}$	$1.2 \times 10^5$	$1 \times 10^6$	
Japan	$1 \times 10^7$	-	-	$1 \times 10^{13}$	$1 \times 10^{10}$	-	$1 \times 10^{11}$	

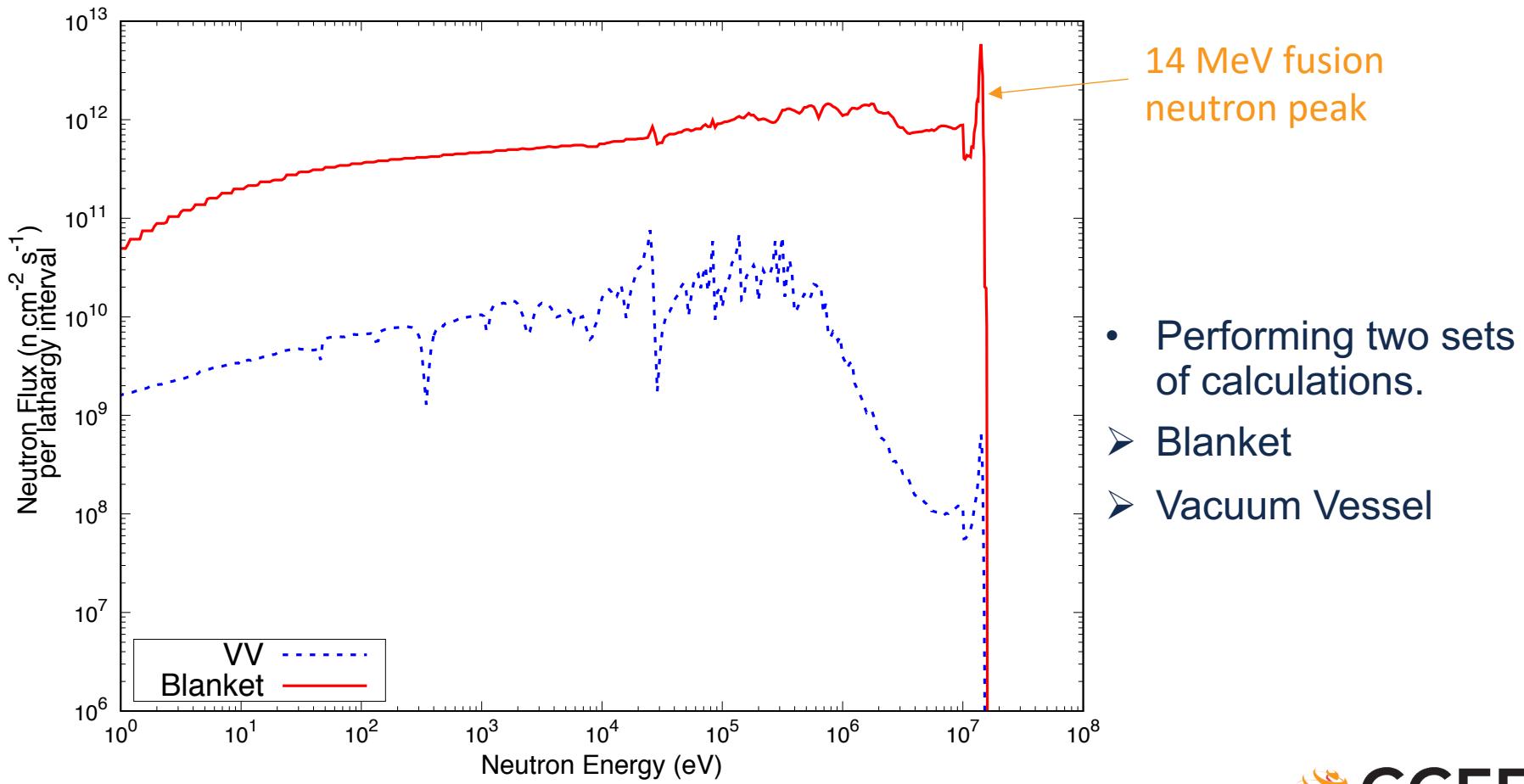
- Countries can include limits on over 100 nuclides in waste criteria. Need to study them all.
- Some systems divide low level waste into multiple categorisations.
- E.g. Spain uses 2 levels of Low Intermediate Level waste and a Very low-level waste class.
- Analyze FISPACT-II outputs with different criteria to understand classification possibilities.



Source: U.S. Nuclear Regulatory Commission

# FISPACT-II calculations for fusion waste assessments – Flux Spectra

- Use the DEMO model. It is the closest we have to a commercial fusion plant.
- Fluxes derived from MCNP calculations, tallied in 709 energy groups.



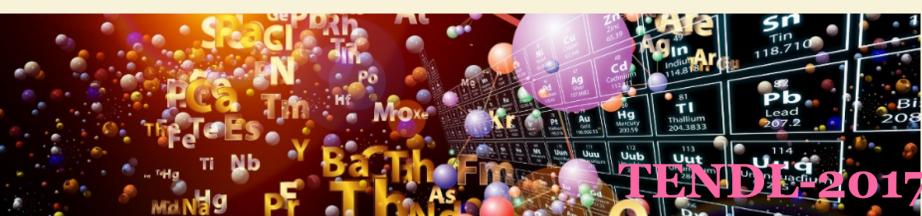
# FISPACT-II calculations for fusion waste assessments – Nuclear Data

Select Nuclear Data set.

- Will use TENDL 2017, contains about 2800 isotopes and uses 709 group structure.

TALYS-based evaluated nuclear data library

Home Reference & us Citations



We believe that our great goal can be achieved with systematism and reproducibility. We are so outside the box, that the box is a point! 

**How to reference**

**Sub-library files**

1. neutron
2. Proton
3. Deuteron
4. Triton
5. He3

**TENDL-2017: (release date: December 30, 2017)**

Last update: 29 December 2017

TENDL is a nuclear data library which provides the output of the TALYS nuclear model code system for direct use in both basic physics and applications. The 9<sup>th</sup> version is TENDL-2017, which is based on both default and adjusted TALYS calculations and data from other sources (previous releases can be found here: [2008](#), [2009](#), [2010](#), [2011](#), [2012](#), [2013](#), [2014](#), and [2015](#)).

## Example from “files” file

```
# index of nuclides to be included
ind_nuc /opt/fispact/nuclear_data/TENDL2017data/tendl17_decay12_index

# Library cross section data
xs_endf /opt/fispact/nuclear_data/TENDL2017data/tal2017-n/gxs-709

# Library probability tables for self-shielding
prob_tab /opt/fispact/nuclear_data/TENDL2017data/tal2017-n/tp-709-294

#fluxes
fluxes fluxes
```

- Also use UK decay 2012 decay data library.

Remember that the choice of Nuclear Data can affect the outputs.  
 Both of these library are available for use with FISPACT-II.

# FISPACT-II calculations for fusion waste assessments – Steel Definitions

- Create input file for each steel studied.
- Studying 1kg of steel under DEMO conditions.
  - Steel Compositions are a mixture of engineering standards and experimental compositions.

## Steel

Eurofer

Hiperfer

Rusfer

CLAM

F82H

XM19

Inconel 718

SS316

Steel 660

ASTM G91 T1

ASTM G91 T2

RA steels



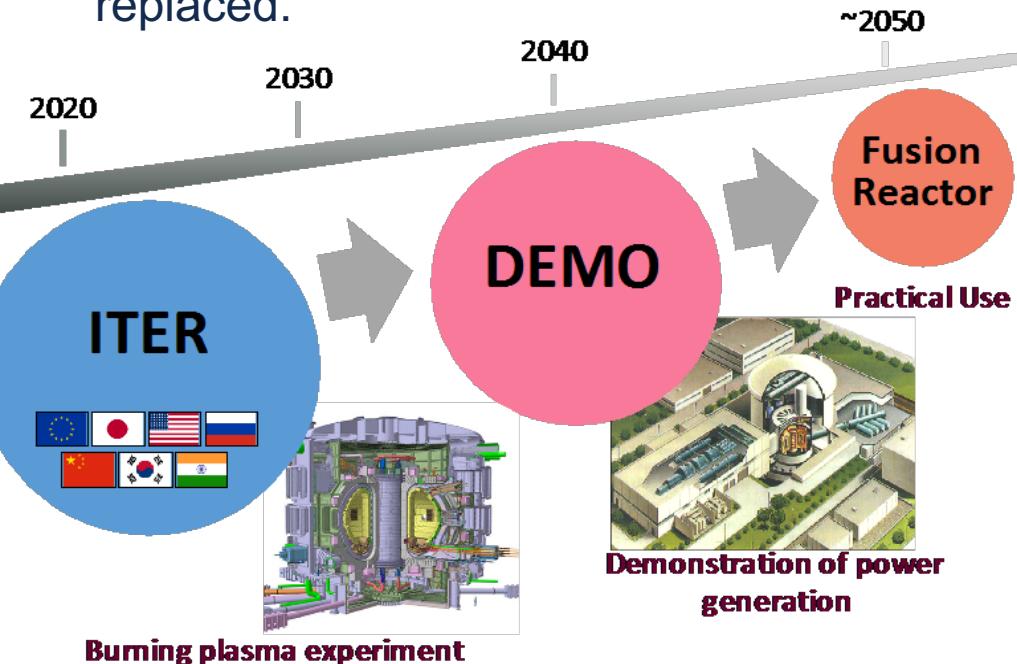
DENSITY	7.9
FUEL	57
Fe54	5.5986142E+23
Fe56	8.7886269E+24
Fe57	2.0296772E+23
Fe58	2.7011278E+22
B10	1.1028313E+20
B11	4.4670238E+20
C12	5.2060516E+22
C13	5.8435810E+20
N14	1.7134914E+22
N15	6.2944161E+19
O16	3.7550434E+20
O17	1.4303206E+17
O18	7.5280034E+17
Al27	8.9277948E+20
Si28	5.1418120E+21
Si29	2.6107671E+20
Si30	1.7209990E+20
P31	3.8885435E+20
S32	5.3538258E+20
S33	4.2258149E+18
S34	2.3720908E+19
S36	1.1268840E+17
Ti46	1.0379370E+19
Ti47	9.3603041E+18
Ti48	9.2747530E+19
Ti49	6.8063502E+18
Ti50	6.5169859E+18
V50	5.9108439E+19
V51	2.3584267E+22
Cr50	4.5290826E+22
Cr52	8.7338850E+23
Cr53	9.9035245E+22

Cr54	2.4651969E+22
Mn55	6.0289324E+22
Co59	5.1092946E+20
Ni58	6.9849353E+20
Ni60	2.6905703E+20
Ni61	1.1696794E+19
Ni62	3.7286095E+19
Ni64	9.5010797E+18
Cu63	1.9665464E+20
Cu65	8.7651618E+19
Nb93	1.2963892E+20
Mo92	2.7947682E+19
Mo94	1.7420220E+19
Mo95	2.9981611E+19
Mo96	3.1412894E+19
Mo97	1.7985200E+19
Mo98	4.5443233E+19
Mo100	1.8135861E+19
Ta180	4.7924757E+17
Ta181	3.9932505E+21
W180	4.3239498E+19
W182	9.5487224E+21
W183	5.1563101E+21
W184	1.1040485E+22
W186	1.0244158E+22

Example: Eurofer's composition

# FISPACT-II calculations for fusion waste assessments – Irradiation Schedule

- Set Irradiation and Cooling Schedules.
- DEMO is planned to operate for over 20 years, the Vacuum Vessel will not be replaced.



Burning plasma experiment

**DEMO aim** : Waste to be ‘Low Level’ 100 years after reactor End of Life.

- Sample should be allowed to cool for up to 1000 years.

```

PULSE 10
FLUX 7.47205E+13
TIME 179.19 DAYS
ATOMS
ENDPULSE
PULSE 48
FLUX 0.00000E+00
TIME 1 HOURS
ATOMS
FLUX 2.49068E+14
TIME 4 HOURS
ATOMS
ENDPULSE
FLUX 0.00000E+00
PULSE 10
TIME 24.35 DAYS
ATOMS
ENDPULSE
PULSE 10
FLUX 7.47205E+13
TIME 179.19 DAYS
ATOMS
ENDPULSE
PULSE 48
FLUX 0.00000E+00
TIME 1 HOURS
ATOMS
  
```

Need short pulses to ensure short lived nuclide populations are reproduced correctly.

Need to account for maintenance shutdown periods.

# FISPACT-II's alternative outputs

```
"inventory_data": [
  {
    "irradiation_time": 0.0E+0,
    "cooling_time": 0.0E+0,
    "flux": 0.747205E+14,
    "total_activity": 0.25114433516057053E-1,
    "alpha_activity": 0.19925778721290189E-2,
    "beta_activity": 0.23121855643928033E-1,
    "gamma_activity": 0.0E+0,
    "total_mass": 0.99999999999999978E+0,
    "total_heat": 0.19150250262055421E-17,
    "alpha_heat": 0.42655940255359924E-18,
    "beta_heat": 0.1485312690273825E-17,
    "gamma_heat": 0.31529333781175787E-20,
    "ingestion_dose": 0.38057061641231115E-8,
    "inhalation_dose": 0.59980236337468839E-7,
    "dose_rate": {
      "type": "",
      "distance": 0.0E+0,
      "mass": 0.0E+0,
      "dose": 0.0E+0
    },
    "nuclides": [
      {
        "element": "B",
        "isotope": 10,
        "state": "",
        "half_life": 0.0E+0,
        "zai": 50100,
        "atoms": 0.22056680437981689E+21,
        "grams": 0.3667335116426904E-2,
        "activity": 0.0E+0,
        "alpha_activity": 0.0E+0,
        "beta_activity": 0.0E+0,
        "gamma_activity": 0.0E+0,
        "heat": 0.0E+0,
        "alpha_heat": 0.0E+0,
        "beta_heat": 0.0E+0,
        "gamma_heat": 0.0E+0,
        "dose": 0.0E+0,
        "ingestion": 0.0E+0,
        "inhalation": 0.0E+0
      },
      {
        "element": "C",
        "isotope": 12,
        "state": "",
        "half_life": 0.0E+0,
        "zai": 60100,
        "atoms": 0.1102833021995895E+21,
        "grams": 0.1834722036659825E-2,
        "activity": 0.0E+0,
        "alpha_activity": 0.0E+0,
        "beta_activity": 0.0E+0,
        "gamma_activity": 0.0E+0,
        "heat": 0.0E+0,
        "alpha_heat": 0.0E+0,
        "beta_heat": 0.0E+0,
        "gamma_heat": 0.0E+0,
        "dose": 0.0E+0,
        "ingestion": 0.0E+0,
        "inhalation": 0.0E+0
      }
    ]
  }
]
```

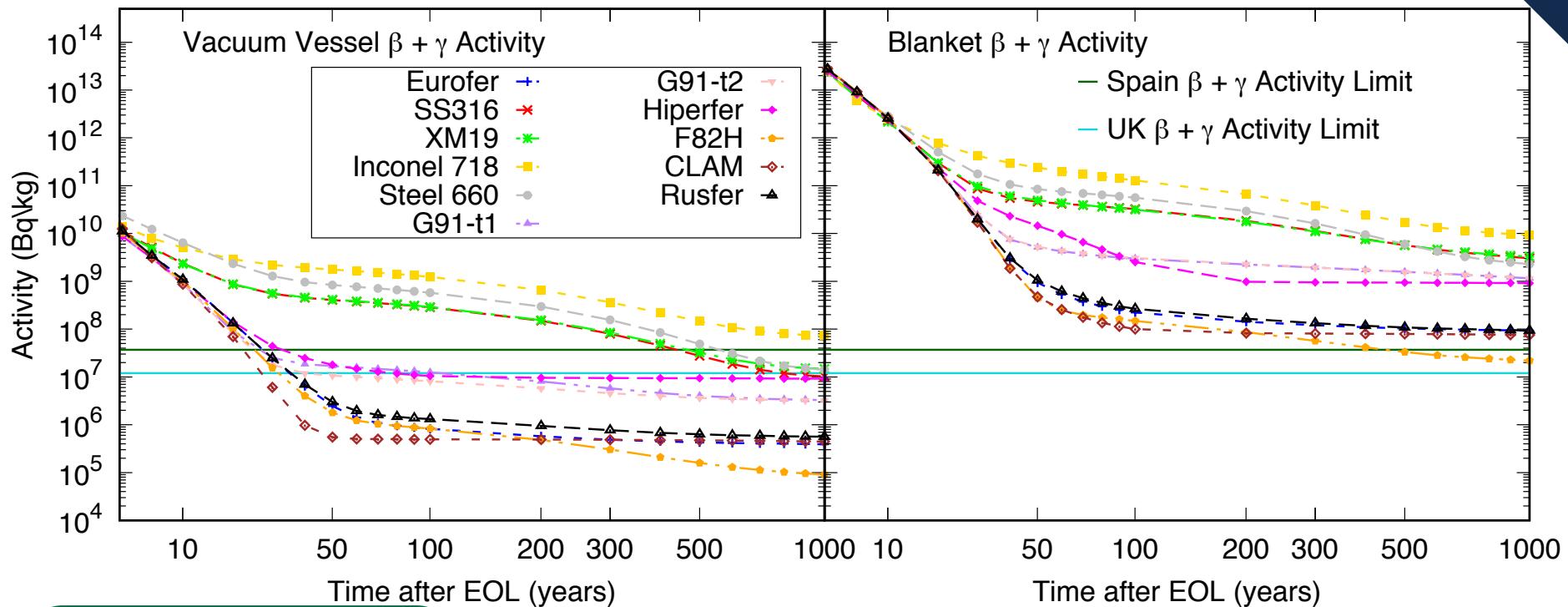
## JSON

- JSON (**JavaScript Object Notation**) is a serialised data format. This is computer readable, but not necessarily easily human readable.
- FISPACT-II can output the nuclide inventory in JSON format with the **JSON** keyword.
- Python has the ability to read JSON files, parsers exist for C++ and Fortran.

## TAB files

- The TAB files output inventory quantities in a simpler format for easier processing.
- For example “**TAB2**” will produce a file which has the activities (Bq) and the Dose rate (Sv/hr) of each nuclide.
- These can be output using the “**TAB1**”, “**TAB2**”, “**TAB3**” , “**TAB4**” keywords

# Results from FISPACT-II: total $\alpha$ and $\beta + \gamma$ activities



For all steels the alpha activity  $< 10^{-1}$  Bq/kg after 1 year. Not a concern to waste classification

All RA and at least one G91 steels meet UK and Spain activity after 100 years.

No Blanket steel can be UK limits. Only F82H meets Spain's limits after 500 years

# Results from FISPACT-II: total $\alpha$ and $\beta + \gamma$ activities

Country	$\alpha$	$\beta + \gamma$	${}^3\text{H}$
UK	$4 \times 10^6$	$1.2 \times 10^7$	-
US	-	-	$\dagger 1.48 \times 10^{12}$
France	-	-	$2 \times 10^8$
Russia	$1 \times 10^3$	$1 \times 10^7$ ( $- {}^3\text{H}$ )	$\dagger 1 \times 10^{11}$
Spain	$3.7 \times 10^6$	$3.7 \times 10^7$	$1 \times 10^9$
Japan	$1 \times 10^7$	-	-

- All steels meet these limits
- $\alpha$  emitters are not a large contributor to activated fusion waste

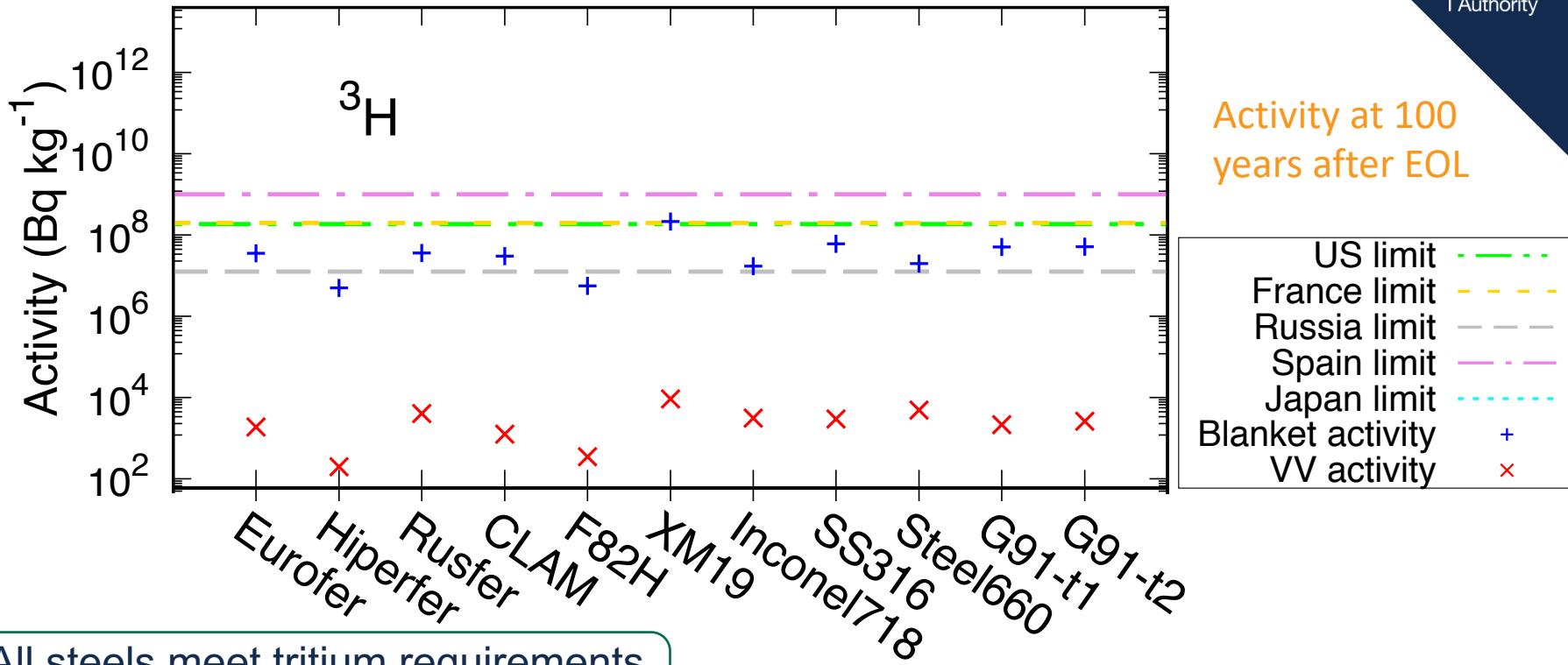
Spain's  $\beta + \gamma$  limits only prevent some LLW classifications, not all

- Blanket steels breach these limits and
- $\beta$  emitters are a large problem.

**Conclusion :** None of the Blanket Steels studied can be called Low Level under UK criteria may struggle under Spanish criteria.

Need tritium activity for Russian limit

# Results from FISPACT-II: Tritium Activity



All steels meet tritium requirements in Vacuum Vessel.

Russian limit:  $\beta + \gamma \leq 1 \times 10^7 (-^3\text{H})$

After 100 years:

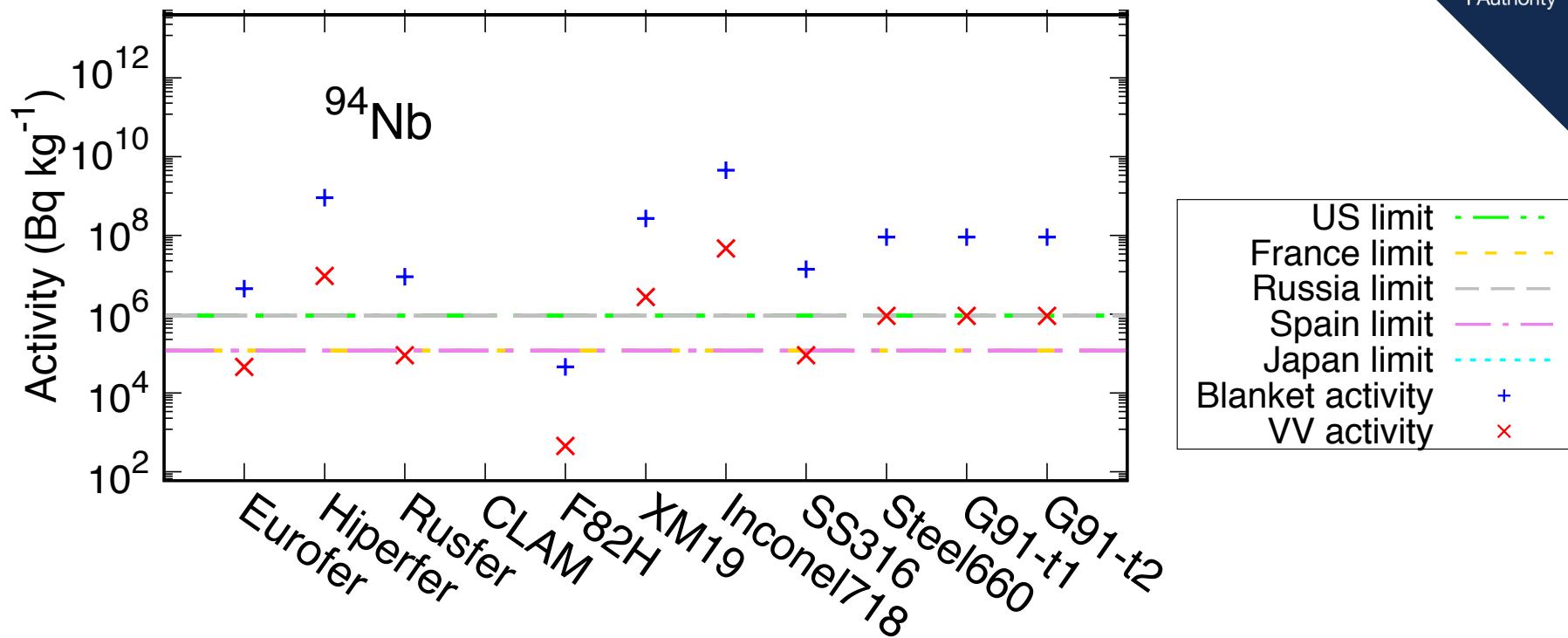
VV:  $\beta + \gamma \gtrsim 10^5 \text{ Bq kg}^{-1}$  →

Blanket:  $\beta + \gamma \gtrsim 10^8 \text{ Bq kg}^{-1}$

After 100 years Hiperfer and F82H meet all limits under blanket conditions.

**Conclusion :** No Blanket Steels studied can be called Low Level under Russian criteria.  
Some may in the Vacuum Vessel.

# Results from FISPACT-II: $^{94}\text{Nb}$ Activity



F82H and CLAM meet all  $^{94}\text{Nb}$  limits. Several VV steels meet the limits

Most Blanket steels fail to meet required criteria

CLAM produces no  $^{94}\text{Nb}$

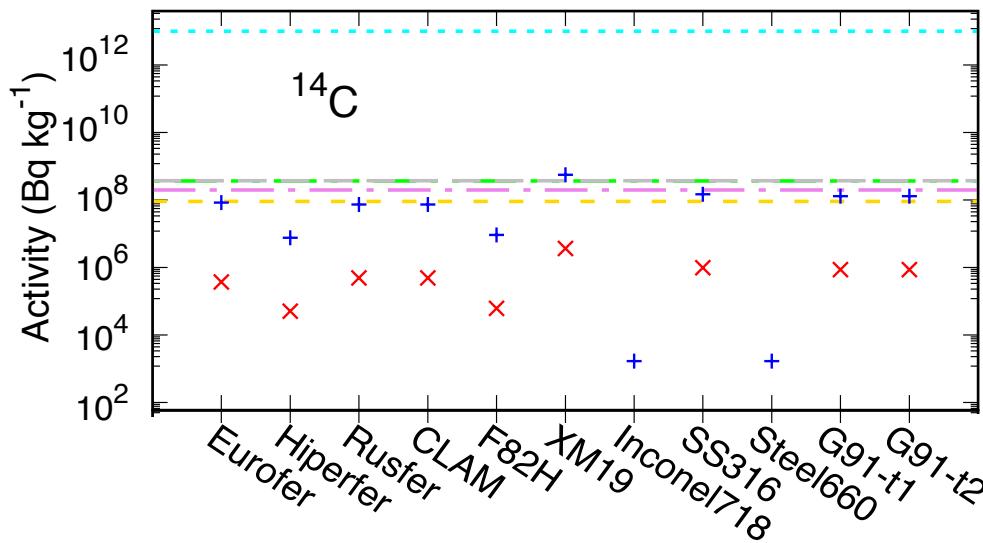
**Conclusion :** Only two steels can meet all limits, meaning most will struggle to be LLW, especially in the blanket.

The VV results may allow some steels to be LLW under some LLW criteria.

# Results from FISPACT-II: $^{99}\text{Tc}$ and $^{14}\text{C}$

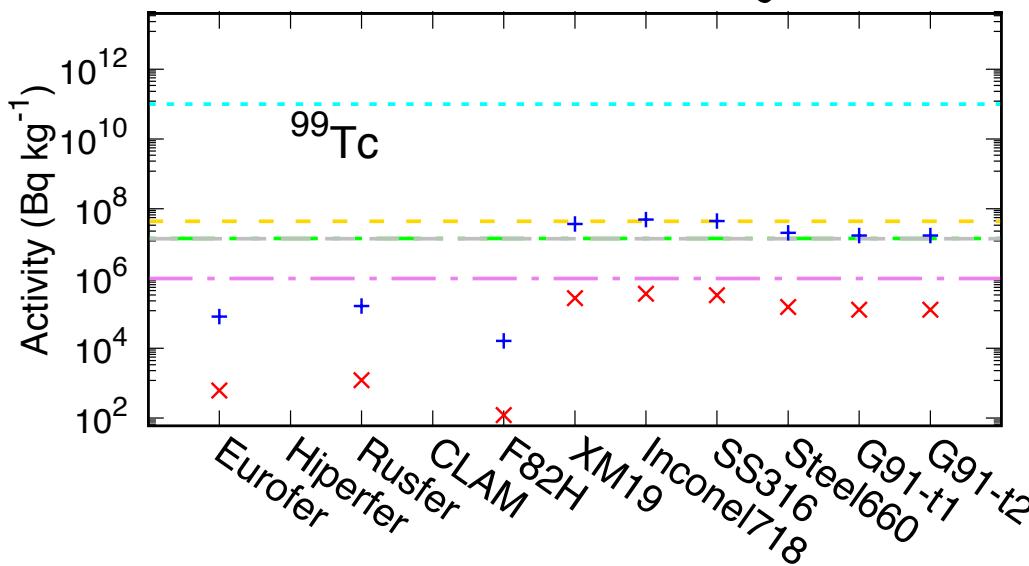
$^{14}\text{C}$

All steels but XM19 can meet limits in the blanket. All steel can meet limits in the VV



$^{99}\text{Tc}$

Traditional steels struggle to meet limits in the blanket



US limit  
 France limit  
 Russia limit  
 Spain limit  
 Japan limit  
 Blanket activity  
 VV activity

**Conclusion :** All Steels could possibly be Low Level under Japanese criteria

# Results from FISPACT-II: Final Waste Classification after 100 years

- Full waste criteria used to assess possible classifications

No Blanket steel can be low level under UK or Russian criteria.

All steels in both regions are LLW by Japans requirements

Steels perform better in the Vacuum Vessel

Steel	Blanket							Vacuum Vessel						
	UK	US	France	Russia	Spain	Japan	UK	US	France	Russia	Spain	Japan		
Eurofer	ILW	ILW	ILW	ILW	ILW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW
Hiperfer	ILW	ILW	ILW	ILW	ILW	LLW	LLW	ILW	ILW	ILW	ILW	ILW	ILW	LLW
Rusfer	ILW	ILW	ILW	ILW	ILW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW
CLAM	ILW	LLW	LLW	ILW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW
F82H	ILW	LLW	ILW	ILW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW	LLW
XM19	ILW	ILW	ILW	ILW	ILW	LLW	ILW	ILW	ILW	ILW	ILW	ILW	ILW	LLW
Inconel 718	ILW	ILW	ILW	ILW	ILW	LLW	ILW	ILW	ILW	ILW	ILW	ILW	ILW	LLW
SS316	ILW	ILW	ILW	ILW	ILW	LLW	ILW	LLW	LLW	ILW	ILW	LLW	LLW	LLW
Steel 660	ILW	ILW	ILW	ILW	ILW	LLW	ILW	ILW	ILW	ILW	ILW	ILW	ILW	LLW
G91 T1	ILW	ILW	ILW	ILW	ILW	LLW	ILW	LLW	ILW	ILW	LLW	ILW	ILW	LLW
G91 T2	ILW	ILW	ILW	ILW	ILW	LLW	ILW	LLW	ILW	ILW	ILW	ILW	ILW	LLW

# Fusion Waste Classification: What does it all mean?

Does these results mean current fusion reactors need to make near plasma components from F82H or CLAM and dispose of them only in the US, Japan or Spain No.

Not practical to transport waste over large distances.

Definition of Low Level varying across the globe

No single steels will meet a fusion reactors structural requirements

- Near plasma fusion waste should not be expected to be low level 100 years after shut down by most international standards. Better prospects in the Vacuum Vessel
- It can be expected to be **Intermediate level** waste
- What can be done? Several possibilities:

Change components more regularly to avoid build up of activation

Change reactor running schedule to lower activation

Develop specific waste repositories for activated steels

Lowering of material impurities

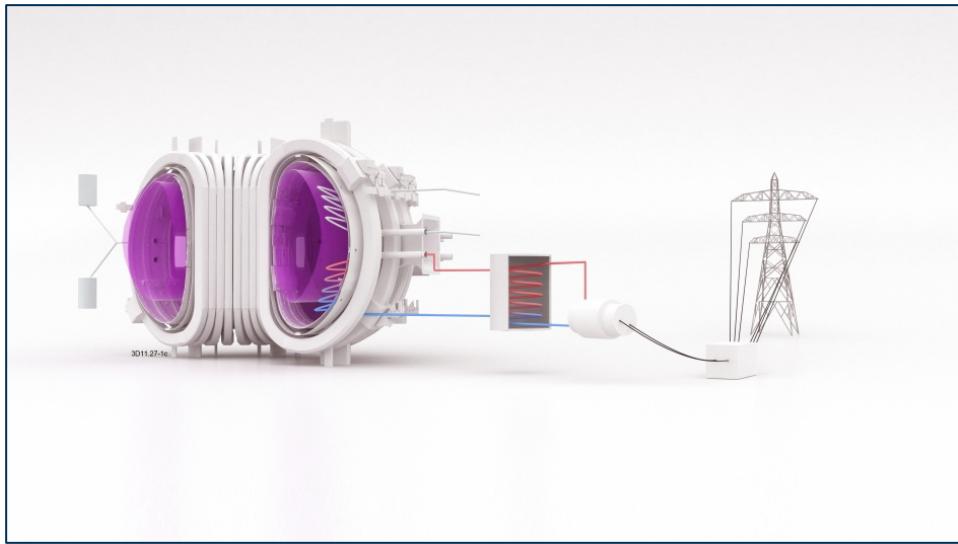
Additional shielding to isolate activity to aid disposal



Possible processing of steels e.g Tritium or Carbon removal

# Conclusion

- FISPACT-II is an ideal tool for activation analysis and waste classification.
- Straightforward to use for many calculations and is used on projects such as the DEMO fusion reactor.



- Fusion power plants are expected to produce activated steel waste from near plasma components
- Use of currently available low activation materials will not solve this problem.

- Steel specific repositories probably needed for test/1<sup>st</sup> generation fusion reactors
- Long term solution to fusion waste has yet to be found but is being studied.

# FISPACT-II

**Thank you for listening.  
Now try the exercises**

**FISPACT-II workshop**

**Online, November/December 2020**



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