Dynamic System Modeling & PID Controller Design for a Molten Salt Microreactor

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University of Idaho
Department of Nuclear Engineering
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About the Author

MSNB
Modeling &
Control

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The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Experience

B.S Chemical Engineering (2015-2019) - Michigan Technological University
M.S. Nuclear Engineering (2021-2023) - University of Idaho - NRC Fellow
Modeling and Simulation Intern at Idaho National Lab

Select Publications

Root, S. J., et al., 2023. Thermodynamic analysis on xenon stripping to shorten restart time in molten salt microreactors.
Nuclear Engineering and Design 414, 112606

Root, S. J., et al., 2023. Cyber hardening of nuclear power plants with real-time nuclear reactor operation, 1. preliminary operational testing.
Progress in Nuclear Energy 162, 104742

Outline

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

3

1 The Molten Salt Nuclear Battery

2 Process Control Engineering

3 Reactor Characterization

4 Results and Analysis

5 Conclusions

6 Endmatter/Old

Background

Gen-IV

Molten Salt Reactors

Microreactors

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

5

Background

Gen-IV

Molten Salt Reactors

Microreactors

Background

Gen-IV

Molten Salt Reactors

Microreactors

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

5

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Process Control Engineering

Process Control Engineering

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

8

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Reactor Characterization

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Reactor Characterization

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Results and Analysis

Results and Analysis

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Results and Analysis

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Acknowledgements

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

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1. Root, S. J., et al., 2023. Thermodynamic analysis on xenon stripping to shorten restart time in molten salt microreactors. Nuclear Engineering and Design 414, 112606.
2. Root, S. J., et al., 2023. Cyber hardening of nuclear power plants with real-time nuclear reactor operation, 1. preliminary operational testing. Progress in Nuclear Energy 162, 104742.
3. Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis, University of Idaho.
4. Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho.
5. Duderstadt, J. J. et al., 1976. Nuclear Reactor Analysis. New York, NY: Wiley & Sons, first edition.
6. Kerlin, T. W. et al., 2019. Dynamics and Control of Nuclear Reactors. Knoxville, Tennessee: Elsevier Inc.
7. Lamarsh, J. R. et al., 2001. Introduction to Nuclear Engineering. Upper Sadle River, New Jersey: Pretice Hall, third edition.
8. Bequette, B. W., 2003. Process Control: Modeling Design and Simulation. Upper Sadle River, New Jersey: Pretice Hall.

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The Molten Salt Nuclear Battery

Process Control Engineering

Reactor Characterization

Results and Analysis

Conclusions

References

Endmatter/Old

References II

9. Gahinet, P. M. et al., 2013. Automated tuning of gain-scheduled control systems. IEEE Conference on Decision and Control .
10. Al Rashdan, A. et al., 2019. A frequency domain control perspective on xenon resistance for load following of thermal nuclear reactors. IEEE Transactions on Nuclear Science 66(9), 2034.
11. Wang, S., et al., 2019. A passive decay heat removal system for emergency draining tanks of MSRs. Nuclear Engineering and Design 341, 423.

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The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Molten Salt Nuclear Battery (MSNB)

- Self-Contained liquid fueled molten salt micro-reactor - 10 year design
- 1 MW design using HALEU UF_4 dissolved in $FLiNaK$
- Criticality is manipulated using axial control drums
  - Neutron absorber plate covering cylinders of neutron reflector
  - Drums are rotated to point more absorber towards the core to insert negative control reactivity

Simplified schematic drawing of an MSNB
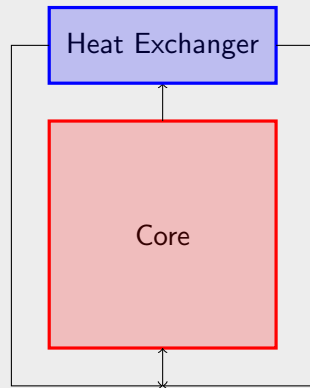
Molten Salt Nuclear Battery (MSNB)

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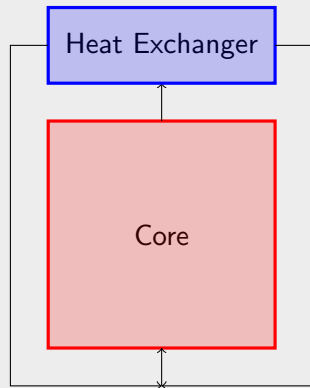

Simplified schematic drawing of an MSNB

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- Criticality is manipulated using axial control drums
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Background on MSNB

Neutronics [3]

- Control drums give a uniform axial and radial flux profile for all reactivity insertions
- Fission product poisoning is the biggest challenge to reach the desired 10-year lifespan
- Control drum vs. reactivity curve is sinusoidal

Thermal Hydraulics [4]

Process Control

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis, University of Idaho

[4] Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho

Background on MSNB

Neutronics [3]

Thermal Hydraulics [4]

- The counteracting passive feedback effects of temperature reactivity and flow reactivity produce stable autonomous load following for relatively small ramp function power demand transients
- An in-core helix device can be used to manipulate temperature and power profiles in the core, as well as minimize advective loss of delayed-neutron precursors

Process Control

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis, University of Idaho

[4] Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho

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The Molten Salt Nuclear Battery

Process Control Engineering

Reactor Characterization

Results and Analysis

Conclusions

References

Endmatter/Old

23

Background on MSNB

Neutronics [3]

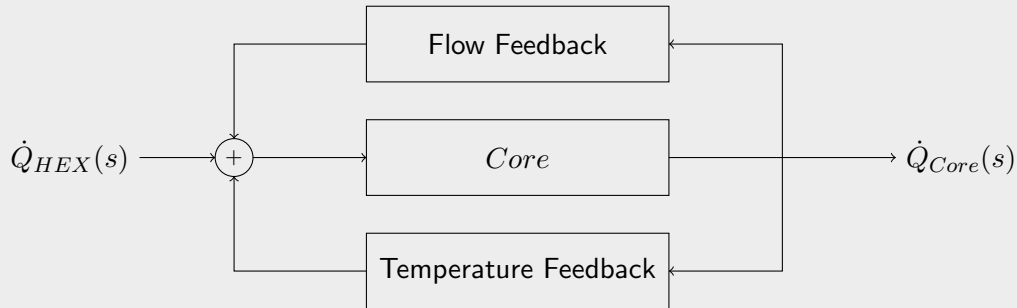## Thermal Hydraulics [4]

Process Control

- Design controller *compliment* the autonomous capabilities provided by the passive feedback mechanisms
- Allow larger faster, more aggressive power changes
- Additional focus on time periods with a high degree of time variance (start-up, shut-down, re-start)

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis, University of Idaho

[4] Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho

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The Molten Salt Nuclear Battery

Process Control Engineering

Reactor Characterization

Results and Analysis

Conclusions

References

Endmatter/Old

23

Passive Feedback

Simplified block diagram of two primary passive feedback mechanisms in an MSNB

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The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

24

- Doppler broadening of resonance peaks results in more epithermal neutrons absorbed by ^{238}U etc. [5, Ch. 6]
- Molten salt fuels have high thermal expansion coefficient [3]
- Increased temperature leads to lower heavy metal density and smaller macroscopic fission cross-section at high temperature
- Similar to moderator thinning in LWRs
- These two effects combine to result in less power production at higher temperature

[5] Duderstadt, J. J. et al., 1976. Nuclear Reactor Analysis.
New York, NY: Wiley & Sons, first edition

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor.
Master's thesis, University of Idaho

Passive Feedback
Flow Reactivity

- Driven by advection of delayed neutron precursors [6, Ch. 3]
  - Most fission events release daughter neutrons *promptly*
  - Sometimes, unstable nuclides which decay by neutron emission are produced instead
  - $t_{1/2}$ from less than a second to over a minute [7, Ch. 6]
- Precursors produced near the core exit and long lived precursors may emit their neutrons outside of the core, so they are effectively *lost* from the fission chain reaction
- In natural circulation, larger power transport requires a higher flow rate, and greater delayed neutron losses

[6] Kerlin, T. W. et al., 2019. Dynamics and Control of Nuclear Reactors.
Knoxville, Tennessee: Elsevier Inc

[7] Lamarsh, J. R. et al., 2001. Introduction to Nuclear Engineering.
Upper Sadle River, New Jersey: Pretice Hall, third edition

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Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
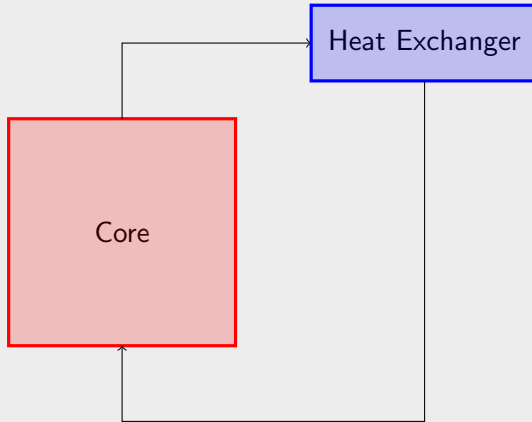Analysis

Conclusions

References

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Main Operational Control Problem - Transport Delay

- Start design process by discussing dynamics associated with anticipated transients
  - Natural circulation flow mode
  - Passive feedback mechanisms
  - Transport delays separating heat exchanger and core
- Thought Experiment
  - Step increase in power demand to a steady-state critical MSNB
  - Set-point is instantaneously equal to heat exchanger power consumption
  - Ideal controller which produces rapid load following with minimal overshoot

Main Operational Control Problem - Transport Delay

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Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
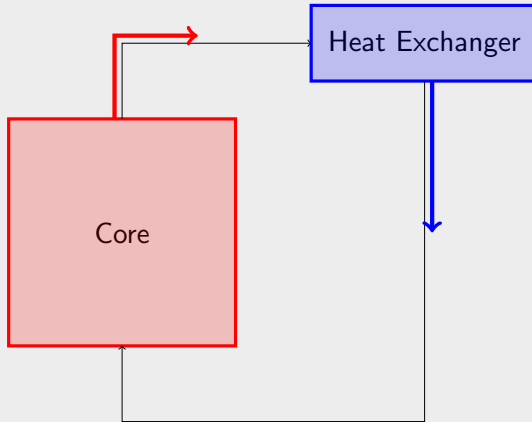Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

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Simplified schematic drawing of a natural circulation MSNB

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Design Decisions
Pre-Filter

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The Molten
Salt Nuclear
Battery

Process
Control
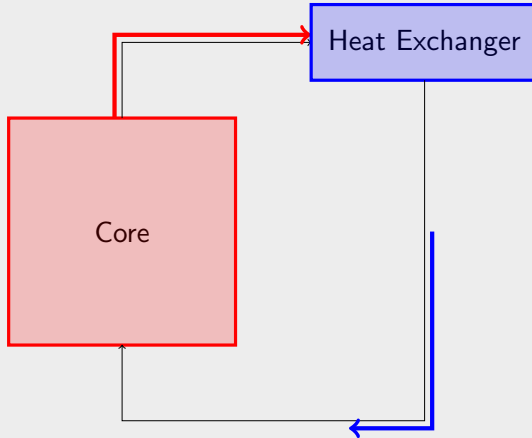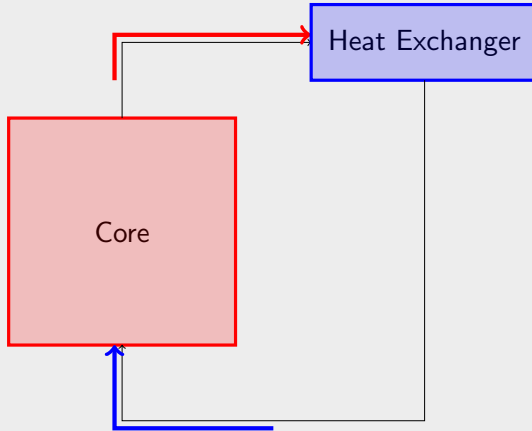Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

Pre-Filter on a step-function

Pre-Filter on a ramp-function

Design Decisions
Dead-band

- Minor perturbations will cause power fluctuations
- Fine grain and constant control actuation would burn out servos prematurely
- When the error is small, allow the passive feedback mechanisms to fine tune

Design Decisions
Disturbance Feedforward

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The Molten
Salt Nuclear
Battery

Process
Control
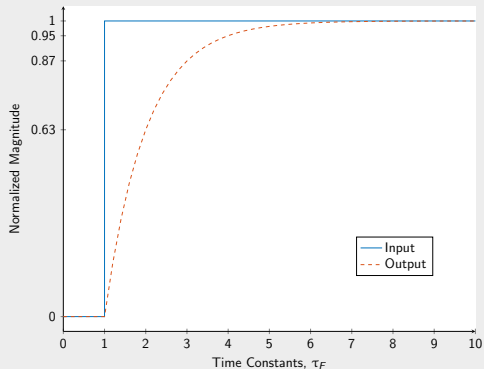Engineering

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acterization
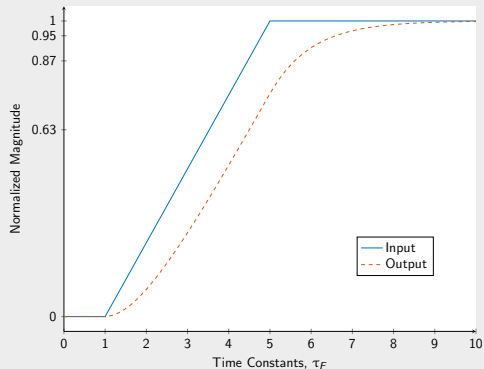
Results and
Analysis

Conclusions

References

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- Disturbances, particularly temperature reactivity, are extremely high frequency - On order of mean neutron lifetime [7, Ch. 7]

- Control actuation is similarly quick

- Time delays on the order of dozens of seconds to minutes

- Inserting control reactivity to counteract temperature reactivity at the exact right moment would be difficult or impossible, so it is unlikely that a disturbance feedforward controller could reject the disturbance before it causes error [8, Ch. 10]

- Disturbance rejection best left to feedback PID controller

[7] Lamarsh, J. R. et al., 2001. Introduction to Nuclear Engineering.
Upper Sadle River, New Jersey: Pretice Hall, third edition

[8] Bequette, B. W., 2003. Process Control: Modeling Design and Simulation.
Upper Sadle River, New Jersey: Pretice Hall

- Sinusoidal control drum angle vs. reactivity curve - non-linear reactivity actuation - Taylor Series approximation to linearize around the operating point [8, Ch. 2]

- Fissile depletion changes the amount of control reactivity required to make the core critical - time-variant controller bias - Bias and gain schedule [9]

- Control drums manipulate *criticality*, not power directly - highly time-dependent control actuation

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[9] Gahinet, P. M. et al., 2013. Automated tuning of gain-scheduled control systems.
IEEE Conference on Decision and Control

[8] Bequette, B. W., 2003. Process Control: Modeling Design and Simulation.
Upper Sadle River, New Jersey: Pretice Hall

Time-Variance and Non-Linearity
Special Cases

- Equilibrium Poisoning
  - Poisons like ^{135}Xe and ^{149}Sa build-up after the reactor starts - this causes a negative reactivity insertion
  - Reach equilibrium over the first $\tilde{1}00$ hours
  - Gain/Bias schedule may be used
  - Alternatively, a burnable poison with appropriate *effective* half-life could be selected - ^{157}Gd shows promise to counteract ^{135}Xe build-up
- Restart Poisoning
  - ^{135}Xe levels increase following shut-down because its beta-precursor (^{135}I) decays faster
  - Requires a lot of excess control reactivity and very good control to counteract the positive reactivity insertion of poison burn-out after restart
  - Low-flux burn-out instead of full shut-down
  - Stripping ^{135}Xe and other fission gasses before re-start

Control Drum Characterization
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Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

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- Use KCODE to develop control drum vs. reactivity curve at various points in the core lifespan
- Use Burn-up routine to study how the core criticality at different conditions effects the control drum vs. reactivity
  - Cold/clean start-up
  - Burnable poison start-up
  - Equilibrium poisoning
  - Long-term depletion of fissile isotopes
- Develop bias/unity point schedule
- Will to use HPC or Falcon

Process Simulation
Python

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Control

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The Molten
Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

- 1D+time finite element model that accounts for passive feedback mechanisms during unsteady state subcritical, critical, and supercritical modes to calculate the core power and flow loop temperature profile over time
- Simulate system response to:
  - Control actuation
  - Heat exchanger transients
- Empirical fitting of reactor transfer function
- Studies can be conducted locally or with cluster resources

Controller Tuning
MATLAB-Simulink

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Salt Nuclear
Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

Endmatter/Old

- Model control loop in Simulink
- Investigate system stability using frequency response tests
- Use built-in numerical methods to implement a PID controller tuning method
- Repeat for different core conditions to develop gain-schedule and/or look-up table for the controller parameters

- Implement control drum reactivity, pre-filter, and PID controller into the process simulation
- Test autonomous response to heat exchanger power demand transients
- Repeat with controller active
- Quantitatively compare response using settling time, dampening ratio, peak overshoot ratio etc.

Timeline

Table: Timeframe for Execution of Project

Tasks	Oct.	Nov.	Dec.	Jan.	Feb.	Mar.	Apr.
Control Drums	X	X	X				
Process Simulation		X	X	X			
Controller Tuning				X	X		
Implementation					X	X	
Cross-Cutting						X	X
Defend							X

Other Considerations

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The Molten
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Battery

Process
Control
Engineering

Reactor Char-
acterization

Results and
Analysis

Conclusions

References

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- Poison perturbations following power transients [10]
- Melting of *in-situ* frozen salt
- SCRAM system must be passive
- Decay heat system [11]
- Flow rate control

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[10] Al Rashdan, A. et al., 2019. A frequency domain control perspective on xenon resistance for load following of thermal nuclear reactors.
IEEE Transactions on Nuclear Science 66(9), 2034

[11] Wang, S., et al., 2019. A passive decay heat removal system for emergency draining tanks of MSRs.
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MSNB
Modeling &
Control

Sam J. Root

The Molten
Salt Nuclear
Battery

Process
Control
Engineering
Reactor Char-
acterization

Results and
Analysis

Conclusions

References
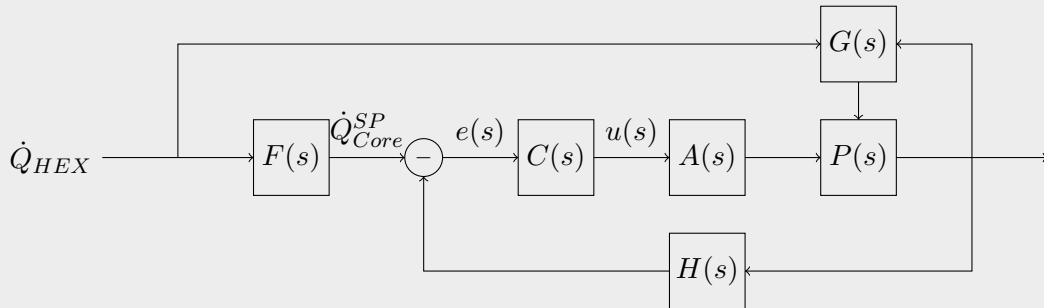
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Simplified control loop of a natural circulation MSNB