Dynamic System Modeling & PID Controller Design for a Molten Salt Microreactor

Sam J. Root

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University of Idaho

Department of Nuclear Engineering and Industrial Management

About the Author



Experience

B.S Chemical Engineering (2015-2019) - Michigan Technological University M.S. Nuclear Engineering (2021-2023) - University of Idaho - NRC Fellow Modeling and Simulation Intern at Idaho National Lab

Select Publications

MSNB Modeling & Control

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The Molten Salt Nuclear Battery

Process Control Engineering

Reactor Characterization

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Outline



- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- 3 Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions

- The Molten Salt Nuclear Battery
 - Introduction
 - Xenon-135 Stripping In-Brief
- 2 Process Control Engineering
- Reactor Characterization
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Control Engineering

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Results and Analysis

Background



Gen-IV

blah

Molten Salt Reactors

Microreactors

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Xenon-135 Stripping In-Brief

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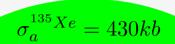
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Reactor Characterization

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Fission Product Poisoning





$$\sigma_a^{^{135}I} = 260mb$$

$$\sigma_f^{235}U = 180b$$

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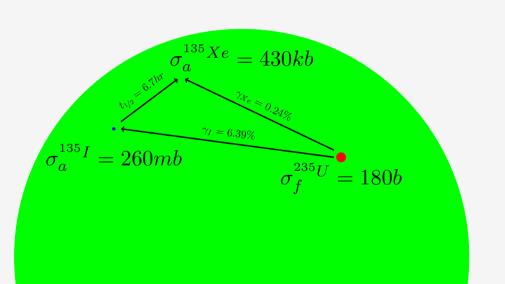
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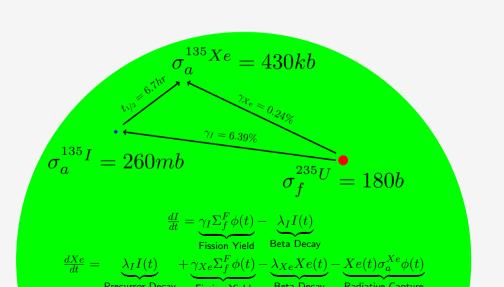
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- 2 Process Control Engineering
 - Control Theory
 - Transport Delay Problem
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Transport Delay Problem

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- Reactor Characterization
 - Neutronics Modeling
 - Process Simulation
- 4 Results and Analysis
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 - Control-Reactivity Curve
 - Controller Tuning
 - Demand Response
- 5 Conclusions



Control-Reactivity Curve

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This work and my coursework is being completed under a Graduate Fellowship funded by Nuclear Regulatory Commission (NRC).

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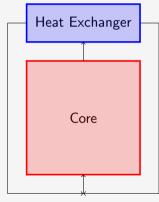
Reactor Characterization

Results and Analysis

Molten Salt Nuclear Battery (MSNB)



- Self-Contained liquid fueled molten salt micro-reactor 10 year design
- 1 MW design using HALEU UF_A dissolved in FLiNaK
- Criticality is manipulated using axial control drums
 - Neutron absorber plate covering cylinders of neutron reflector
 - Drums are rotated to point more absorber towards the core to insert negative control reactivity



Simplified schematic drawing of an MSNB

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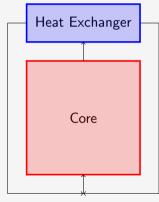
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Results and Analysis

- Self-Contained liquid fueled molten salt micro-reactor - 10 year design
- 1 MW design using HALEU UF_4 dissolved in FLiNaK
- Criticality is manipulated using axial control drums
 - Neutron absorber plate covering cylinders of neutron reflector

Background on MSNB

MSNB Modeling & Control

Neutronics [?]

- Control drums give a uniform axial and radial flux profile for all reactivity insertions
- Fission product poisoning is the biggest challenge to reach the desired 10-year lifespan
- Control drum vs. reactivity curve is sinusoidal

Thermal Hydraulics [?]

Process Control

[?]

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Background on MSNB



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Neutronics [?]

Thermal Hydraulics [?]

- The counteracting passive feedback effects of temperature reactivity and flow reactivity produce stable autonomous load following for relatively small ramp function power demand transients
- An in-core helix device can be used to manipulate temperature and power profiles in the core, as well as minimize advective loss of delayed-neutron precursors

Process Control

[?]

Background on MSNB



Neutronics [?]

Thermal Hydraulics [?]

Process Control

- Design controller compliment the autonomous capabilities provided by the passive feedback mechanisms
- Allow larger faster, more aggressive power changes
- Additional focus on time periods with a high degree of time variance (start-up, shut-down, re-start)

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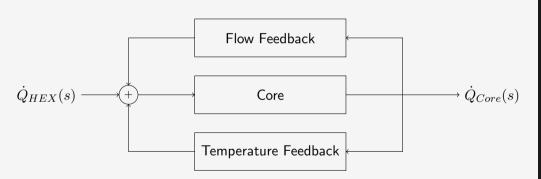
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Passive Feedback





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Conclusions

Simplified block diagram of two primary passive feedback mechanisms in an $\ensuremath{\mathsf{MSNB}}$

Passive Feedback

Temperature Reactivity



• Doppler broadening of resonance peaks results in more epithermal neutrons absorbed by ^{238}U etc. [? . Ch. 6]

- Molten salt fuels have high thermal expansion coefficient [?]
- Increased temperature leads to lower heavy metal density and smaller macroscopic fission cross-section at high temperature
- Similar to moderator thinning in LWRs
- These two effects combine to result in less power production at higher temperature

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Passive Feedback

Flow Reactivity



- Driven by advection of delayed neutron precursors [? , Ch. 3]
 - Most fission events release daughter neutrons *promptly*
 - Sometimes, unstable nuclides which decay by neutron emission are produced instead
 - ullet $t_{1/2}$ from less than a second to over a minute [? , Ch. 6]
- Precursors produced near the core exit and long lived precursors may emit their neutrons outside of the core, so they are effectively *lost* from the fission chain reaction
- In natural circulation, larger power transport requires a higher flow rate, and greater delayed neutron losses

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Main Operational Control Problem - Transport Delay



• Start design process by discussing dynamics associated with anticipated transients

- Natural circulation flow mode
- Passive feedback mechanisms
- Transport delays separating heat exchanger and core
- Thought Experiment
 - Step increase in power demand to a steady-state critical MSNB
 - Set-point is instantaneously equal to heat exchanger power consumption
 - Ideal controller which produces rapid load following with minimal overshoot

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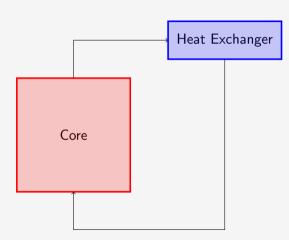
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Simplified schematic drawing of a natural circulation MSNB

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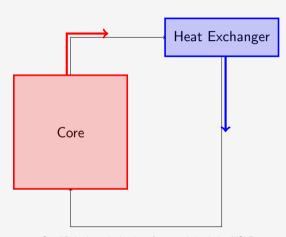
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Immediate Response





Simplified schematic drawing of a natural circulation MSNB

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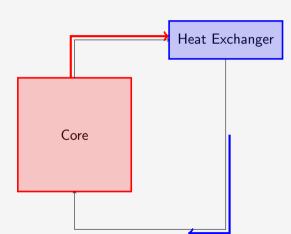
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Heat Exchanger Perturbation





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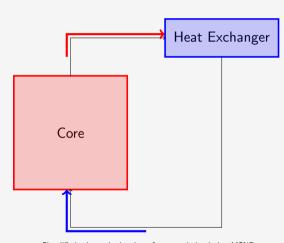
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Core Perturbation





Simplified schematic drawing of a natural circulation MSNB

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Design Decisions

Pre-Filter





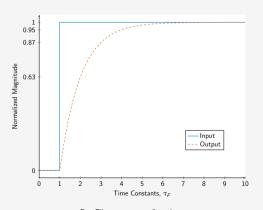
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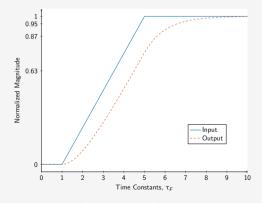
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Pre-Filter on a step-function



Pre-Filter on a ramp-function

Design Decisions

Dead-band



- Minor perturbations will cause power fluctuations
- Fine grain and constant control actuation would burn out servos prematurely
- When the error is small, allow the passive feedback mechanisms to fine tune

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Design Decisions

Disturbance Feedforward



• Disturbances, particularly temperature reactivity, are extremely high frequency - On order of mean neutron lifetime [? , Ch. 7]

- Control actuation is similarly quick
- Time delays on the order of dozens of seconds to minutes
- Inserting control reactivity to counteract temperature reactivity at the exact right moment would be difficult or impossible, so it is unlikely that a disturbance feedforward controller could reject the disturbance before it causes error [? , Ch. 10]
- Disturbance rejection best left to feedback PID controller

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Time-Variance and Non-Linearity

- Sinusoidal control drum angle vs. reactivity curve non-linear reactivity actuation Taylor Series approximation to linearize around the operating point [?, Ch. 2]
- Fissile depletion changes the amount of control reactivity required to make the core critical time-variant controller bias Bias and gain schedule [?]
- Control drums manipulate criticality, not power directly highly time-dependent control actuation

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• Equilibrium Poisoning

- ullet Poisons like ^{135}Xe and ^{149}Sa build-up after the reactor starts this causes a negative reactivity insertion
- Reach equilibrium over the first 100 hours
- Gain/Bias schedule may be used
- Alternatively, a burnable poison with appropriate *effective* half-life could be selected ^{157}Gd shows promise to counteract ^{135}Xe build-up
- Restart Poisoning
 - ^{135}Xe levels increase following shut-down because its beta-precursor (^{135}I) decays faster
 - Requires a lot of excess control reactivity and very good control to counteract the positive reactivity insertion of poison burn-out after restart
 - Low-flux burn-out instead of full shut-down
 - Stripping ^{135}Xe and other fission gasses before re-start

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- Use KCODE to develop control drum vs. reactivity curve at various points in the core lifespan
- Use Burn-up routine to study how the core criticality at different conditions effects the control drum vs. reactivity
 - Cold/clean start-up
 - Burnable poison start-up
 - Equilibrium poisoning
 - Long-term depletion of fissile isotopes
- Develop bias/unity point schedule
- Will to use HPC or Falcon

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Results and Analysis

• 1D+time finite element model that accounts for passive feedback mechanisms during unsteady state subcritical, critical, and supercritical modes to calculate the core power and flow loop temperature profile over time

- Simulate system response to:
 - Control actuation
 - Heat exchanger transients
- Empirical fitting of reactor transfer function
- Studies can be conducted locally or with cluster resources

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Results and Analysis

- Model control loop in Simulink
- Investigate system stability using frequency response tests
- Use built-in numerical methods to implement a PID controller tuning method
- Repeat for different core conditions to develop gain-schedule and/or look-up table for the controller parameters

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Reactor Characterization

Results and

- Implement control drum reactivity, pre-filter, and PID controller into the process simulation
- Test autonomous response to heat exchanger power demand transients
- Repeat with controller active
- Quantitatively compare response using settling time, dampening ratio, peak overshoot ratio etc.

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Timeline



Table: Timeframe for Execution of Project

Tasks	Oct.	Nov.	Dec.	Jan.	Feb.	Mar.	Apr.
Control Drums	X	X	X				
Process Simulation		X	X	X			
Controller Tuning				X	Х		
Implementation					X	X	
Cross-Cutting						X	X
Defend							X

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Other Considerations



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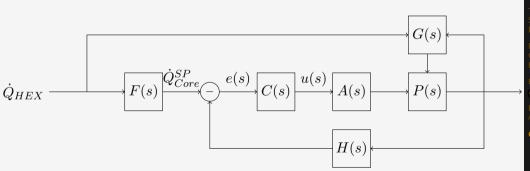
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- _
- [?

- Melting of *in-situ* frozen salt
- SCRAM system must be passive
- Decay heat system [?]
- Flow rate control

Control Loop



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