Dynamic System Modeling & PID Controller Design for a Molten Salt Microreactor

Sam J. Root

University of Idaho - Idaho Falls Center for Higher Education Department of Nuclear Engineering and Industrial Management

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Department of Nuclear Engineering and Industrial Management

About the Author



Experience

B.S Chemical Engineering (2015-2019) - Michigan Technological University M.S. Nuclear Engineering (2021-2023) - University of Idaho - NRC Fellow Modeling and Simulation Intern at Idaho National Lab

Select Publications

Root, S. J., et al., 2023. Thermodynamic analysis on xenon stripping to shorten restart time in molten salt microreactors.

Nuclear Engineering and Design 414, 112606

Root, S. J., et al., 2023. Cyber hardening of nuclear power plants with real-time nuclear reactor operation, 1. preliminary operational testing. Progress in Nuclear Energy 162, 104742

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The Molten Salt Nuclear Battery

Control Engineering

Reactor Characterization

Analysis

Conclusions

Outline



- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- 3 Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions

- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- Reactor Characterization
- 4 Results and Analysis
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Background



Gen-IV

- Complete re-designs
- Smaller footprint
- Deployability
- Versatility

Molten Salt Reactors

Microreactors

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Background



Gen-IV

Molten Salt Reactors

- High temperature
- Low pressure
- High specific heat

Microreactors

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Background



Gen-IV

Molten Salt Reactors

Microreactors

- Less than 50 MW
- Assembly line manufacturing
- Deliver/installation vs. construction

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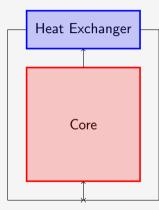
Molten Salt Nuclear Battery



Fuel/Primary Coolant

- Self-Contained liquid fueled molten salt micro-reactor
- 10 year design
- ullet 10 MWth using HALEU UF_4 dissolved in FLiNaK
- Natural circulation driven

Control Drums



Simplified schematic drawing of an MSNB

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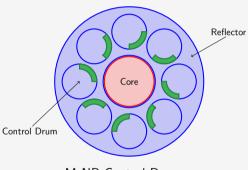
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Fuel/Primary Coolant

Control Drums

- Criticality is manipulated using axial control drums
- Neutron absorber plate covering cylinders of neutron reflector
- Drums are rotated to point more absorber towards the core to insert negative control reactivity



MsNB Control Drums

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Previous and Present Work



Neutronics [3]

- Control drums give a uniform axial and radial flux profile for all reactivity insertions
- Control drum vs. reactivity curve is sinusoidal

Thermal Hydraulics [4]

Process Control

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis. University of Idaho

[4] Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho MSNB Modeling & Control

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Conclusions

Previous and Present Work



Neutronics [3]

Thermal Hydraulics [4]

- Transient simulations can be reduced to 1D and still correspond to STAR-CCM+
- Stable autonomous load following for relatively small ramp function power demand transients

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Previous and Present Work



Neutronics [3]

Thermal Hydraulics [4]

Process Control

- Design controller *compliment* the autonomous capabilities provided by the passive feedback mechanisms
- Allow larger faster, more aggressive power changes
- Return MSNB to steady-state operation faster

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- The Molten Salt Nuclear Battery
- 2 Process Control Engineering
 - Control Theory
 - Transport Delay Problem
- Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions



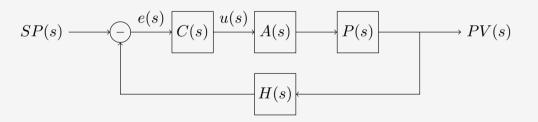
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Control

Control Theory

Feedback Control



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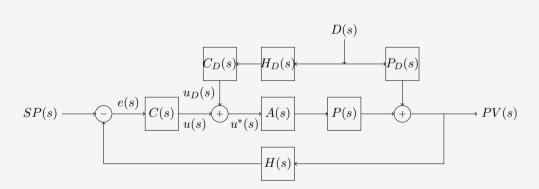
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Feedforward Control





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 $u(t) = \underbrace{K_P e(t)}_{\text{Proportional}} + \underbrace{K_I \int_0^t e(t) dt}_{\text{Integral}} + \underbrace{K_D \frac{de(t)}{dt}}_{\text{Derivative}}$

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$$u(t) = \underbrace{K_P e(t)}_{\text{Proportional}} + \underbrace{K_I \int_0^t e(t) dt}_{\text{Integral}} + \underbrace{K_D \frac{de(t)}{dt}}_{\text{Derivative}}$$

Proportional

- Control output is manipulated in proportion to the error defined by the proportional gain constant
- High gain yields an aggressive controller that is prone to overshooting the setpoint
- Low gain may result in steady-state offset

Integral

Derivative

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Proportional

Integral

- Considers cumulative error to help eliminate steady-state offset
- As the process variable settles around the set-point, the cumulative error approaches a constant value and the effect of the integral controller diminishes.

Derivative

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Proportional

Integral

Derivative

- Estimates the time rate of change of the error to dampen overshoot
- Backs-off the proportional response when the process variable rapidly approaches the set-point
- Can be difficult to tune

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Transport Delay Problem

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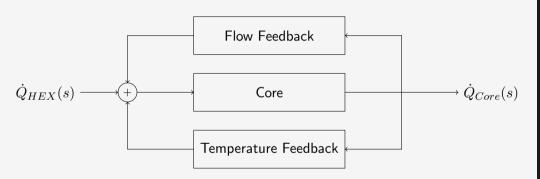
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Passive Feedback





Simplified block diagram of two primary passive feedback mechanisms in an MSNB

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Passive Feedback Temperature Reactivity



Doppler broadening

- Resonance peaks lower and broaden with increased temperature
- High kinetic energy of target nucleus introduces more relative uncertainty of the center-of-mass energy [5, Ch. 7]
- ullet More epithermal neutrons absorbed by $^{238}U\,$ etc. [6, Ch. 6]

Thermal Expansion

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^[5] Kerlin, T. W. et al., 2019. Dynamics and Control of Nuclear Reactors. Knoxville. Tennessee: Elsevier Inc

Passive Feedback Temperature Reactivity



Doppler broadening

Thermal Expansion

- Increased temperature leads to lower heavy metal density and smaller macroscopic fission cross-section at high temperature [3]
- Similar to moderator thinning in LWRs

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Delayed Neutron Precursors

- Most fission events release daughter neutrons promptly
- Sometimes, unstable nuclides which decay by neutron emission are produced instead
- $t_{1/2}$ from less than a second to over a minute [7, Ch. 6]

$$^{87}Br \xrightarrow{\beta^{-}} ^{87}Kr^{*} \rightarrow ^{86}Kr + n$$

Flowing Fuel

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Passive Feedback

Flow Reactivity



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Delayed Neutron Precursors

Flowing Fuel

- Precursors produced near the core exit and long lived precursors may emit their neutrons outside of the core
- These neutrons are effectively lost from the fission chain reaction [5, Ch. 3]
- Larger power transport requires a higher flow rate
- Greater delayed neutron losses
- Negative feedback

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Main Operational Control Problem - Transport Delay



Dynamics associated with anticipated transients

- Natural circulation flow mode
- Passive feedback mechanisms
- Transport delays separating heat exchanger and core

Thought Experiment

- Step increase in power demand to a steady-state critical MSNB
- Set-point is instantaneously equal to heat exchanger power consumption
- Ideal controller which produces rapid load following with minimal overshoot

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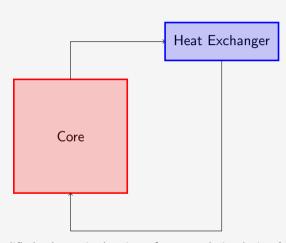
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Main Operational Control Problem - Transport Delay





Simplified schematic drawing of a natural circulation MSNB

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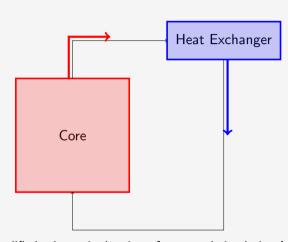
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Main Operational Control Problem - Transport Delay Immediate Response





Simplified schematic drawing of a natural circulation $\ensuremath{\mathsf{MSNB}}$

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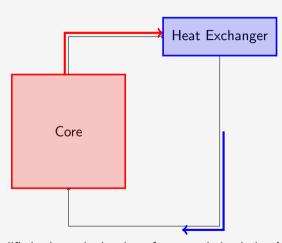
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Main Operational Control Problem - Transport Delay Heat Exchanger Perturbation





Simplified schematic drawing of a natural circulation $\ensuremath{\mathsf{MSNB}}$

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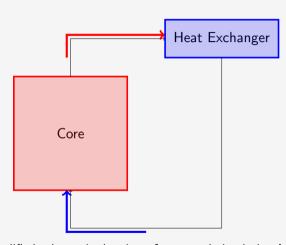
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Main Operational Control Problem - Transport Delay





Simplified schematic drawing of a natural circulation MSNB

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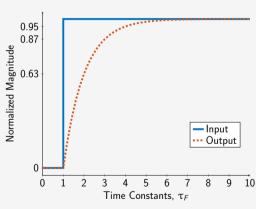
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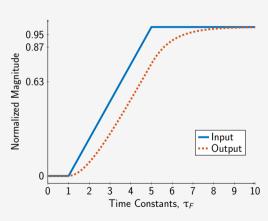
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Pre-Filter



Pre-Filter on a step-function



Pre-Filter on a ramp-function

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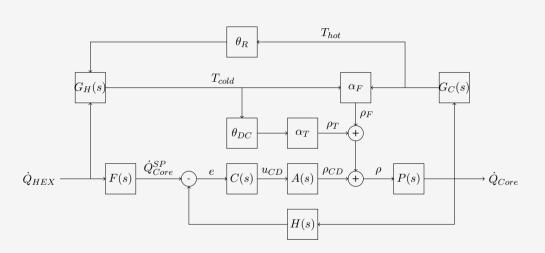
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MSNB Control Loop





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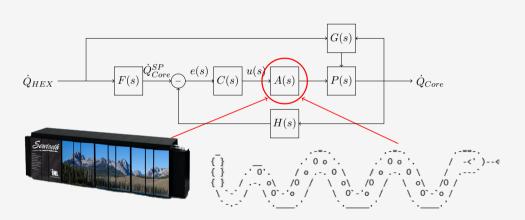
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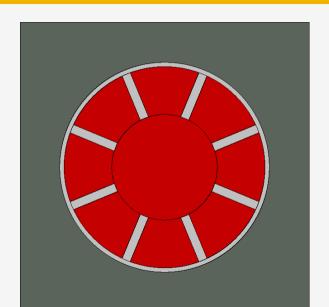
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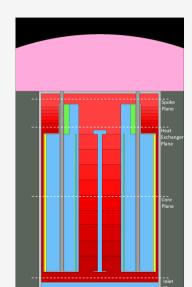
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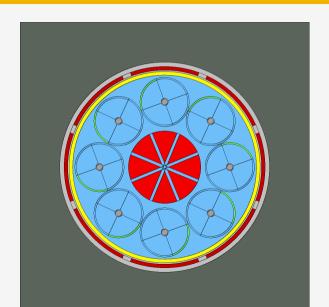
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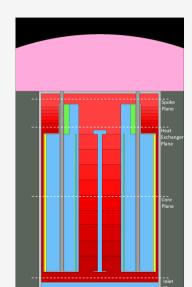
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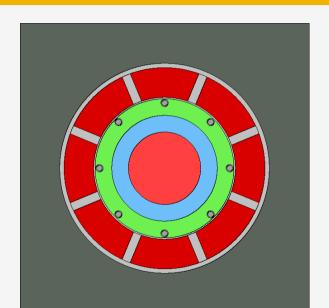
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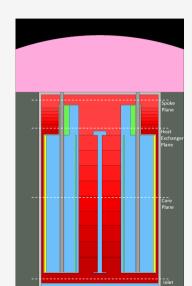
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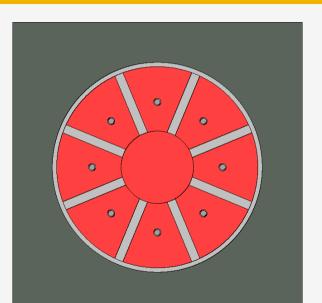
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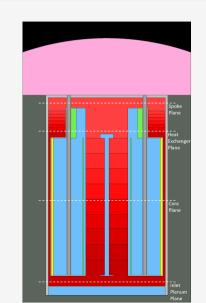
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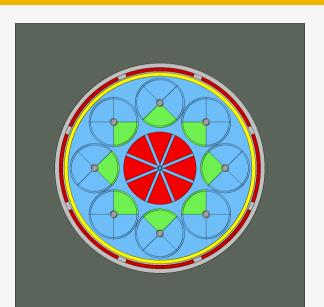
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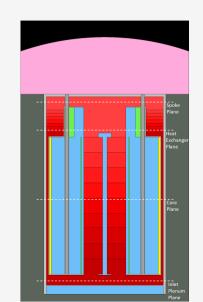
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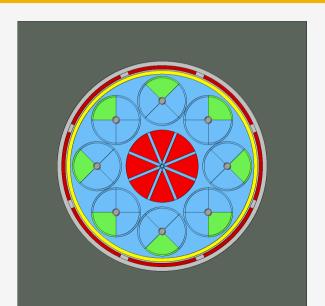
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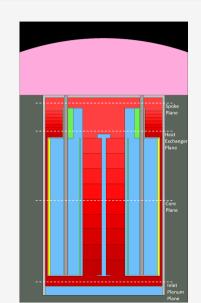
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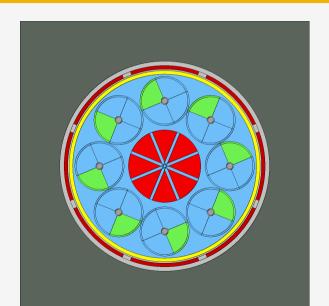
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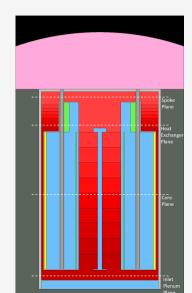
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Physics Cards



Criticality

- Set Pop 1,000,000 500 100 1
- Set Power 1e7 %10MWth

Burn-Up

- Set mcvol 10,000,000
- Set nbuf
- Set printm 1 1e-10
- Set inventory "all"

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Control-Reactivity Curve

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Acknowledgements



This work and my coursework is being completed under a Graduate Fellowship funded by Nuclear Regulatory Commission (NRC).

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