# Dynamic System Modeling & PID Controller Design for a Molten Salt Microreactor

#### Sam J. Root

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Department of Nuclear Engineering and Industrial Management

## About the Author



# Experience

B.S Chemical Engineering (2015-2019) - Michigan Technological University M.S. Nuclear Engineering (2021-2023) - University of Idaho - NRC Fellow Modeling and Simulation Intern at Idaho National Lab

#### Select Publications

MSNB Modeling & Control

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The Molten Salt Nuclear Battery

Process Control Engineering

> Reactor Characterization

Results and Inalysis

# Outline



- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- 3 Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions

- The Molten Salt Nuclear Battery
  - Introduction
  - Xenon-135 Stripping In-Brief
- 2 Process Control Engineering
- Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions



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Control Engineering

Introduction

Reactor Characterization

Results and Analysis

# Background



Gen-IV

blah

Molten Salt Reactors

Microreactors

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# Xenon-135 Stripping In-Brief

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# Fission Product Poisoning





$$\sigma_a^{135}{}^I = 260mb$$

$$\sigma_f^{235}U = 180b$$

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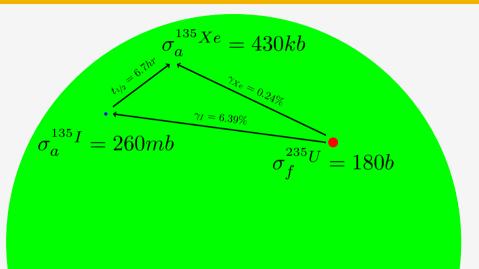
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Reactor Characterization

Results and analysis

# Fission Product Poisoning





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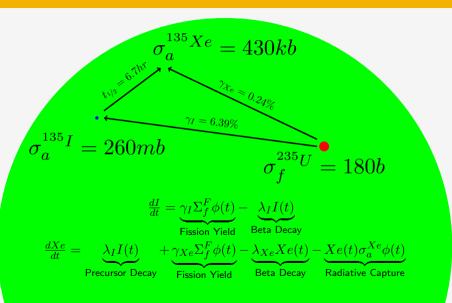
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Reactor Characterization

Results and Analysis

# Fission Product Poisoning





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Reactor Characterization

Results and Analysis

- The Molten Salt Nuclear Battery
- 2 Process Control Engineering
  - Control Theory
  - Transport Delay Problem
- Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions



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# Transport Delay Problem

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Engineering

Reactor Characterization

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- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- Reactor Characterization
  - Neutronics Modeling
  - Process Simulation
- 4 Results and Analysis
- 5 Conclusions



**Neutronics Modeling** 

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- 2 Process Control Engineering
- Reactor Characterization
- 4 Results and Analysis
  - Control-Reactivity Curve
  - Controller Tuning
  - Demand Response
- 5 Conclusions



# Control-Reactivity Curve

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Reactor Characterization

Results and Analysis

- The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions

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This work and my coursework is being completed under a Graduate Fellowship funded by Nuclear Regulatory Commission (NRC).

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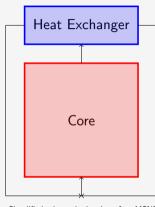
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# Molten Salt Nuclear Battery (MSNB)



- Self-Contained liquid fueled molten salt micro-reactor - 10 year design
- 1 MW design using HALEU  $UF_4$  dissolved in FLiNaK
- Criticality is manipulated using axial control drums
  - Neutron absorber plate covering cylinders of neutron reflector
  - Drums are rotated to point more absorber towards the core to insert negative control reactivity



Simplified schematic drawing of an MSNB

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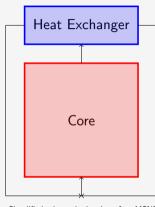
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Results and Analysis

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Reactor Characterization

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Reactor Characterization

Results and Analysis

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# Background on MSNB

# Neutronics [?]

- Control drums give a uniform axial and radial flux profile for all reactivity insertions
- Fission product poisoning is the biggest challenge to reach the desired 10-year lifespan
- Control drum vs. reactivity curve is sinusoidal

# Thermal Hydraulics [?]

#### Process Control

[?]

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# Background on MSNB



# Neutronics [? ]

# Thermal Hydraulics [?]

- The counteracting passive feedback effects of temperature reactivity and flow reactivity produce stable autonomous load following for relatively small ramp function power demand transients
- An in-core helix device can be used to manipulate temperature and power profiles in the core, as well as minimize advective loss of delayed-neutron precursors

#### Process Control

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# Background on MSNB



# Neutronics [?]

# Thermal Hydraulics [?]

#### Process Control

- Design controller compliment the autonomous capabilities provided by the passive feedback mechanisms
- Allow larger faster, more aggressive power changes
- Additional focus on time periods with a high degree of time variance (start-up, shut-down, re-start)

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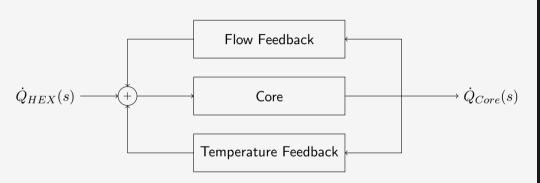
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# Passive Feedback





Simplified block diagram of two primary passive feedback mechanisms in an  $\ensuremath{\mathsf{MSNB}}$ 

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## Passive Feedback

## Temperature Reactivity



- Doppler broadening of resonance peaks results in more epithermal neutrons absorbed by  $^{238}U$  etc. [? , Ch. 6]
- Molten salt fuels have high thermal expansion coefficient [? ]
- Increased temperature leads to lower heavy metal density and smaller macroscopic fission cross-section at high temperature
- Similar to moderator thinning in LWRs
- These two effects combine to result in less power production at higher temperature

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## Passive Feedback

## Flow Reactivity



- Driven by advection of delayed neutron precursors [? , Ch. 3]
  - Most fission events release daughter neutrons *promptly*
  - Sometimes, unstable nuclides which decay by neutron emission are produced instead
  - ullet  $t_{1/2}$  from less than a second to over a minute [? , Ch. 6]
- Precursors produced near the core exit and long lived precursors may emit their neutrons outside of the core, so they are effectively *lost* from the fission chain reaction
- In natural circulation, larger power transport requires a higher flow rate, and greater delayed neutron losses

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# Main Operational Control Problem - Transport Delay



Start design process by discussing dynamics associated with anticipated transients

- Natural circulation flow mode
- Passive feedback mechanisms
- Transport delays separating heat exchanger and core
- Thought Experiment
  - Step increase in power demand to a steady-state critical MSNB
  - Set-point is instantaneously equal to heat exchanger power consumption
  - Ideal controller which produces rapid load following with minimal overshoot

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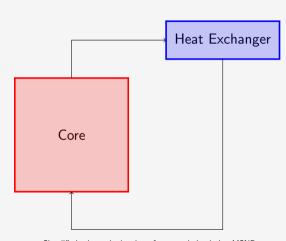
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Simplified schematic drawing of a natural circulation MSNB

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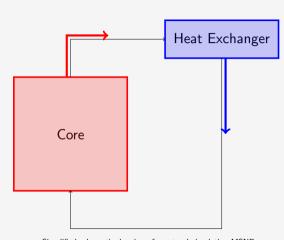
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Immediate Response





Simplified schematic drawing of a natural circulation  $\ensuremath{\mathsf{MSNB}}$ 

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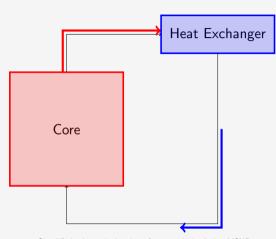
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Heat Exchanger Perturbation





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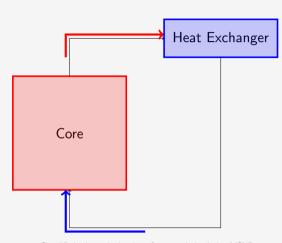
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Core Perturbation





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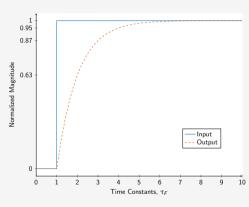
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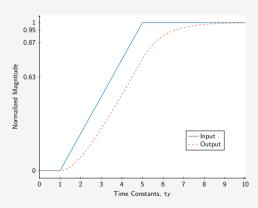
# Design Decisions

#### Pre-Filter





Pre-Filter on a step-function



Pre-Filter on a ramp-function

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### **Design Decisions**

Dead-band



- Minor perturbations will cause power fluctuations
- Fine grain and constant control actuation would burn out servos prematurely
- When the error is small, allow the passive feedback mechanisms to fine tune

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### **Design Decisions**

#### Disturbance Feedforward



- Disturbances, particularly temperature reactivity, are extremely high frequency On order of mean neutron lifetime [? . Ch. 7]
- Control actuation is similarly quick
- Time delays on the order of dozens of seconds to minutes
- Inserting control reactivity to counteract temperature reactivity at the exact right moment would be difficult or impossible, so it is unlikely that a disturbance feedforward controller could reject the disturbance before it causes error [?, Ch. 10]
- Disturbance rejection best left to feedback PID controller

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### Time-Variance and Non-Linearity

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**Operational Control** 

- Sinusoidal control drum angle vs. reactivity curve non-linear reactivity actuation - Taylor Series approximation to linearize around the operating point [?, Ch. 2]
- Fissile depletion changes the amount of control reactivity required to make the core critical time-variant controller bias Bias and gain schedule [?]
- Control drums manipulate *criticality*, not power directly highly time-dependent control actuation

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### Time-Variance and Non-Linearity

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#### Special Cases

- Equilibrium Poisoning
  - $\bullet$  Poisons like  $^{135}Xe$  and  $^{149}Sa$  build-up after the reactor starts this causes a negative reactivity insertion
  - Reach equilibrium over the first 100 hours
  - Gain/Bias schedule may be used
  - Alternatively, a burnable poison with appropriate *effective* half-life could be selected  $^{157}Gd$  shows promise to counteract  $^{135}Xe$  build-up
- Restart Poisoning
  - $^{135}Xe$  levels increase following shut-down because its beta-precursor ( $^{135}I$  ) decays faster
  - Requires a lot of excess control reactivity and very good control to counteract the positive reactivity insertion of poison burn-out after restart
  - Low-flux burn-out instead of full shut-down
  - ullet Stripping  $^{135}Xe$  and other fission gasses before re-start

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- Use KCODE to develop control drum vs. reactivity curve at various points in the core lifespan
- Use Burn-up routine to study how the core criticality at different conditions effects the control drum vs. reactivity
  - Cold/clean start-up
  - Burnable poison start-up
  - Equilibrium poisoning
  - Long-term depletion of fissile isotopes
- Develop bias/unity point schedule
- Will to use HPC or Falcon

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- 1D+time finite element model that accounts for passive feedback mechanisms during unsteady state subcritical, critical, and supercritical modes to calculate the core power and flow loop temperature profile over time
- Simulate system response to:
  - Control actuation
  - Heat exchanger transients
- Empirical fitting of reactor transfer function
- Studies can be conducted locally or with cluster resources

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MATLAB-Simulink

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table for the controller parameters

- Investigate system stability using frequency response tests
- Use built-in numerical methods to implement a PID controller tuning method
- Repeat for different core conditions to develop gain-schedule and/or look-up



- Implement control drum reactivity, pre-filter, and PID controller into the process simulation
- Test autonomous response to heat exchanger power demand transients
- Repeat with controller active
- Quantitatively compare response using settling time, dampening ratio, peak overshoot ratio etc.

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### Timeline



Table: Timeframe for Execution of Project

Tasks	Oct.	Nov.	Dec.	Jan.	Feb.	Mar.	Apr.
Control Drums	X	X	X				
<b>Process Simulation</b>		X	X	X			
Controller Tuning				X	X		
Implementation					X	X	
Cross-Cutting						X	X
Defend							X

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### Other Considerations

Melting of *in-situ* frozen salt
SCRAM system must be passive

• Decay heat system [?]

Flow rate control

• Poison perturbations following power transients [? ]



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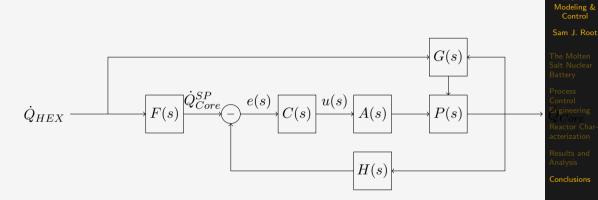
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Simplified control loop of a natural circulation MSNB