A Thesis

Presented in Partial Fulfillment of the Requirements for the

Degree of Master of Science

with a

Major in Nuclear Engineering

in the

College of Graduate Studies

University of Idaho

by

Sam J. Root

Major Professor: Michael McKellar, Ph.D.
Committee Members: Dakota Roberson, Ph.D.; Robert A. Borrelli, Ph.D.
Department Administrator: Indrajit Charit, Ph.D.

Abstract

Lorem ipsum dolor sit amet, consectetuer adipiscing elit. Ut purus elit, vestibulum ut, placerat ac, adipiscing vitae, felis. Curabitur dictum gravida mauris. Nam arcu libero, nonummy eget, consectetuer id, vulputate a, magna. Donec vehicula augue eu neque. Pellentesque habitant morbi tristique senectus et netus et malesuada fames ac turpis egestas. Mauris ut leo. Cras viverra metus rhoncus sem. Nulla et lectus vestibulum urna fringilla ultrices. Phasellus eu tellus sit amet tortor gravida placerat. Integer sapien est, iaculis in, pretium quis, viverra ac, nunc. Praesent eget sem vel leo ultrices bibendum. Aenean faucibus. Morbi dolor nulla, malesuada eu, pulvinar at, mollis ac, nulla. Curabitur auctor semper nulla. Donec varius orci eget risus. Duis nibh mi, congue eu, accumsan eleifend, sagittis quis, diam. Duis eget orci sit amet orci dignissim rutrum.

Nam dui ligula, fringilla a, euismod sodales, sollicitudin vel, wisi. Morbi auctor lorem non justo. Nam lacus libero, pretium at, lobortis vitae, ultricies et, tellus. Donec aliquet, tortor sed accumsan bibendum, erat ligula aliquet magna, vitae ornare odio metus a mi. Morbi ac orci et nisl hendrerit mollis. Suspendisse ut massa. Cras nec ante. Pellentesque a nulla. Cum sociis natoque penatibus et magnis dis parturient montes, nascetur ridiculus mus. Aliquam tincidunt urna. Nulla ullamcorper vestibulum turpis. Pellentesque cursus luctus mauris.

ACKNOWLEDGEMENTS

This work and my coursework was completed under a Graduate Fellowship funded by Nuclear Regulatory Commission (NRC).

DEDICATION

To my mother, Tammy, who planted and nurtured my love of science. To my father, Paul, who taught me how to design and build, and showed me that I am an engineer. To my cats, Babe and Bunyan, who stayed up with me all those late nights studying and writing. Thank you for your endless support.

Table of Contents

Abstract	ii
Acknowledgements	iii
Dedication	iv
Table of Contents	V
List of Tables	vii
List of Figures	riii
List of Codes	ix
List of Acronyms	Х
Chapter 1: Introduction	1
Microreactors	1
Molten Salt Reactors	2
Molten Salt Nuclear Battery	3
Scope	3
Outline	3
Chapter 2: Process Control Engineering	4
FEEDBACK	4
Feedforward	4
Time Variance	4
Chapter 3: Reactor Characterization	6
REACTOR DESIGN SELECTION	6
Neutronics Modeling	6
Process Simulation	6
Chapter 4: Controller Design	7
REACTOR TRANSFER FUNCTION	7
Tuning Methodology	7
Chapter 5: Results and Analysis	8
Chapter 6: Conclusions	9
LIMITATIONS	q

Future Work	9
Summary Remarks	9
References	10
Chapter A: Test	12
Chapter B: What	13

LIST OF TABLES

LIST OF FIGURES

LIST OF CODES

1	Hello!	12
2	F strings	12

ACRONYMS

INL Idaho National Laboratory.

 ${\bf LWR}\,$ Light Water Reactor.

 $\mathbf{MSNB}\,$ Molten Salt Nuclear Battery.

MSR Molten Salt Reactor.

MSRE Molten Salt Reactor Experiment.

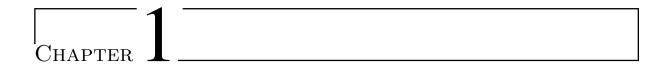
 \mathbf{NPP} Nuclear Power Plant.

NRC Nuclear Regulatory Commission.

NREL National Renewable Energy Laboratory.

ORNL Oak Ridge National Laboratory.

 ${\bf PID} \ \ {\bf Proportional\text{-}Integral\text{-}Derivative}.$



Introduction

The world is working to move away from fossil fuel as its main energy source [1]. National Renewable Energy Laboratory (NREL) has partnered with over 700 organizations, including large manufacturing companies, to de-carbonize supply chains [2]. Nuclear power has been well established as an alternative for base-load electrical generation with 93 facilities in the United States and 435 globally which generate on the order of 1 GWe, but there remains a need for smaller reactors to be deployed in more dynamic applications such as small remote grids, manufacturing, and power-peaking [3]. These small energy utilizers could turn to microreactors to fill their needs; to make this a reality, a robust control system for microreactors must be designed that is capable of ramping production up and down to meet demand.

1.1 Microreactors

Microreactors, as the name suggests, are small nuclear reactors which are designed to be fully assembled when shipped, rather than constructed on site. This is a hot area of research in the private sector as companies are working to capitalize on the growing need for clean and dependable small scale electrical generation [4]. They aim not to replace the utility scale Nuclear Power Plants (NPPs) which handle base-load electrical generation, but the diesel or natural gas engines that are found at countless manufacturing facilities, peaking stations, military bases, islands, and other locations where on-site generation is the primary or only source of electricity.

The goal is to be able to deliver a prefabricated microreactor to a site, integrate it to the necessary power cycles and process heat applications, and meet the needs of the site for a long period of time - up to a decade - without the need for refueling or significant maintenance. This would be quite convenient compared to building a pipeline to deliver fossil fuels to the site or scheduling regular delivery. One of the biggest challenges in implementing microreactors is the transients that these applications often require. Engines handle these quite well, simply adjusting the flow rates of fuel and combustion air. Nuclear reactor load following is a bit more complicated, as the reactor must be made supercritical to ramp up power or subcritical to decrease power. This necessitates valid characterization of the reactor's criticality control & actuation system and reactivity feedback mechanisms, so the controller can be designed and tuned to effectively and safely match power generation to consumption.

1.2 Molten Salt Reactors

Molten salts are highly desirable in high temperature applications due to their excellent thermophysical properties [5]. Salt mixtures have been developed to have very wide liquid temperature ranges (i.e. low melting point and high vaporization point). They also have very high volumetric heat capacities compared to other high temperature coolants (which tend to be gaseous), and are able to operate at or slightly above ambient pressure. These properties combine to make molten salts excellent choices in heat transfer and thermal storage applications. Furthermore, they are extremely strong electrolytes which on be useful as solvents, catalysts, or reagents in certain chemical reactions including a pyrometallurgical method for reprocessing spent nuclear fuel [6].

Molten Salt Reactors (MSRs) are a family of nuclear reactor in which a fuel salt (containing fissile and/or fertile nuclides) is dissolved in a coolant salt [7]. The concept was proven by the Molten Salt Reactor Experiment (MSRE) at Oak Ridge National Laboratory (ORNL) in the 1960s [8]. It has yet to take off beyond the research reactor sector, but it has re-emerged as a Gen-IV reactor concept, with a team at the Shangai Institute of Applied Physics gaining approval to operate a now fully constructed thorium breeding MSR [9]. Some of the benefits of MSRs over more conventional LWRs include:

- Higher operating temperatures allow for use in applications requiring high-grade process heat, and yield higher thermal efficiency [7];
- Lower pressure operating pressure lends itself to inherant safety, and less expensive (thinner) components [5];
- The ability to burn minor actinides supports the goal of reducing global stockpiles of high-level waste [5];
- Natural circulation of the fuel introduces an additional feedback mechanism that presents the possibility of autonomous load following of certain power demand transients [10];
- There is no concern of core melt-down as the reactor is designed for liquid fuel;
- The liquid state homogenizes nuclides throughout the core, which minimizes burn-up gradient to produce a flatter temperature and power profile within the core [11, 12]. The flowing nature also allows for online reprocessing, removing fission products and poisons during operation;

They also carry some demerits:

- Molten salts are very corrosive [13];
- The chemistry of the coolant (not only the fuel) is constantly changing due to fission, transmutation, and impurities from corrosion;
- Lithium is commonplace, so tritium production is unavoidable. Off-gas systems need to be robust to handle tritium as well as radionuclide noble gasses, halides, and interhalides [14];

1.3 Molten Salt Nuclear Battery

The Molten Salt Nuclear Battery (MSNB) is a self contained design for a liquid fueled molten salt microreactor [15, 4]. It is fueled by an inorganic form of uranium, UF_4 , dissolved in a coolant salt such as FLiNaK (a eutectic mixture of three alkali fluorides) or FLiBe (a mixture of LiF and BeF_2) [7]. Heat is generated in the core by fission, is transported by the natural circulation of the coolant/fuel salt, and rejected to a secondary working fluid in an integrated heat exchanger. Criticality is manipulated using axial control drums, which may be rotated to aim either a neutron reflecting material or a neutron absorbing material towards the core. The design studied in this work is intended to produce up to 1 MWth, although larger designs of up to 50 MWth also exist.

1.4 Scope

As a developing design, work has been done on neutronics [4], thermal-hydraulics and autonomous load following [15], and corrosion concerns [16]. However, until now, little to no work has been done on the control system. First and foremost, this work details a multiphysics characterization of the MSNB required to design a feedback controller capable of matching the core power generation to the secondary power demand. In addition to the main control mode of following power transients during normal operation, specific discussion is centered around more dynamic time periods, namely: 1) initial start-up; 2) shutdown, both planned and emergency; and 3) restart;

1.5 Outline

This report will begin by discussing the field of process control engineering, specifically the control methods which are most useful in the design of a controller for the MSNB, and the challenges inherent to a controlling a nuclear chain reaction, both in normal operational modes and special cases. The reactor will then be characterized, using a combination of stochastic neutron transport code to define the reactivity curve of the control drums and finite element process simulation to understand the reactivity feedback effects intrinsic to the design. The resulting model of the reactor will then be used to design and tune the controller, which will then be tested against the reactor's autonomous response to load demand changes. Finally, after the results of the simulation are analyzed, the limitations of this method, as well as future work that will be required to implement a Molten Salt Nuclear Battery will be discussed.



PROCESS CONTROL ENGINEERING

2.1 Feedback

2.2 Feedforward

The term 'Feedforward' can be used to refer to any element in the control block diagram that exists outside of the feedback loop.

DISTURBANCE FEEDFORWARD

Not that useful since disturbance transport delay is on the order of minutes and disturbance dynamics are on the order of milliseconds

PRE-FILTER

This could be electronic (less ideal) or physically realized by decoupling

2.3 TIME VARIANCE

Fissile depletion - time function parameters or look-up table to gain-schedule and turn the time variant system into a shift invariant system.

In addition to the relatively slow time variance of fissile fuel depletion during steady-state critical operation, there are specific times in a MSNB's expected operational life-cycle that exhibit a higher degree of time variance: 1. Start-up; 2. Shut-down; and 3. Re-start.

START-UP

Black-start may need to deal with thawing salt - main concern is fission product neutron poison build-up (discuss the burnable poison stuff)

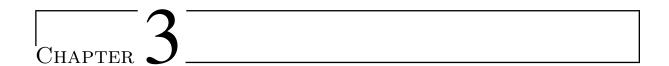
Shut-down

Planned shut-down

Emergency Shutdown/SCRAM(must be passive) Decay heat and keeping the salt liquid for restart

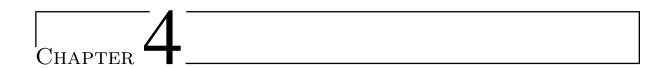
Re-start

 ^{135}Xe stripper



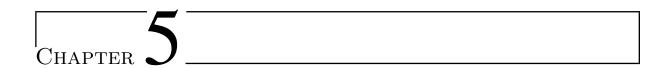
REACTOR CHARACTERIZATION

- 3.1 Reactor Design Selection
- 3.2 Neutronics Modeling
- 3.3 Process Simulation

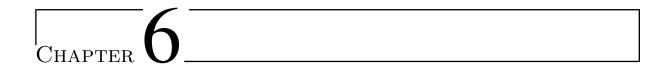


Controller Design

- 4.1 REACTOR TRANSFER FUNCTION
- 4.2 Tuning Methodology



Results and Analysis



Conclusions

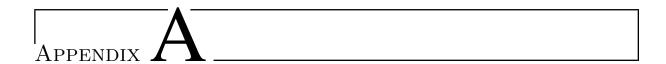
- 6.1 Limitations
- 6.2 Future Work
- 6.3 Summary Remarks

REFERENCES

- [1] Valluri, Sriram Kumar, 2021. Carbon Capture and Utilization. Ph.D. thesis, Michigan Technological University.
- [2] NREL, 2021. Technology partnerships. Technical report, National Renewable Energy Laboratory (NREL).
- [3] Nuclear Energy Institute, 2018. Roadmap for the deployment of micro-reactors for u.s. department of defense domestic installations. Technical report, Nuclear Energy Institute.
- [4] Peterson, John, 2019 8. An Analysis of the Nuclear Characteristics of a Molten Salt Microreactor. Master's thesis, University of Idaho.
- [5] Roper, Robin, Harkema, Megan, Sabharwall, Piyush, Riddle, Catherine, Chisholm, Brandon, Day, Brandon, Marotta, Paul, 2022. Molten salt for advanced energy applications: A review. Annals of Nuclear Energy 169, 108924. ISSN 0306-4549. doi:https://doi.org/10.1016/j.anucene.2021.108924. URL https://www.sciencedirect.com/science/article/pii/S030645492100801X
- [6] Simpson, Michael F., 2012. Development of spent nuclear fuel pyroprocessing technology at Idaho National Laboratory (INL). Technical report, Idaho National Laboratory (INL).
- [7] Roper, Robin V., Sabharwall, Piyush, Christensen, Richard, 2019. Chemical overview of molten salts. ANS Annual Meeting.
 URL https://www.ans.org/pubs/transactions/article-45524/
- [8] Haubenreich, P N, Engel, J R, Prince, B E, Claiborne, H C, 1964 2. Msre design and operations report. part iii. nuclear analysis. Technical report. doi:10.2172/4114686. URL https://www.osti.gov/biblio/4114686
- [9] World Nuclear News, 2022. Chinese molten-salt reactor cleared for start up. World Nuclear News.

 URL https://www.world-nuclear-news.org/articles/chinese-molten-salt-reactor-cleared-for-start-up.
- [10] Carter, John P., Christensen, Richard, Yoon, Sujong, 2022. Numerical analysis of dynamic load following response in a natural circulation molten salt power reactor system. Nuclear Engineering and Design.

- [11] Lamarsh, John R., Baratta, Anthony J., 2001. Introduction to Nuclear Engineering. Pretice Hall, Upper Sadle River, New Jersey, 3rd edition.
- [12] Todreas, Neil E., Kazimi, Mujid S., 1990. Nuclear Systems Volume I: Thermal Hydraulic Fundamentals. Taylor and Francis, USA.
- [13] Roper, Robin V., Christensen, Richard, 2019. Redox potential control for the molten salt reactor concept. ANS Winter Meeting. URL https://www.ans.org/pubs/transactions/article-47571/
- [14] Andrews, Hunter B., McFarlane, Joanna, Chapel, A. Shay, Ezell, N. Dianne Bull, Holcomb, David E., de Wet, Dane, Greenwood, Michael S., Myhre, Kristian G., Bryan, Samuel A., Lines, Amanda, Riley, Brian J., Felmy, Heather M., Humrickhouse, Paul W., 2021. Review of molten salt reactor off-gas management considerations. Nuclear Engineering and Design 385, 111529. ISSN 0029-5493. doi: https://doi.org/10.1016/j.nucengdes.2021.111529.
 - URL https://www.sciencedirect.com/science/article/pii/S0029549321004817
- [15] Carter, John P., 2022. Multi-Physics Investigation of a Natural Circulation Molten Salt Micro-Reactor that Utilizes an Experimental In-pile Device to Improve Core Physics and System Thermal-Hydraulic Performance. Ph.D. thesis, Univesity of Idaho.
- [16] Roper, Robin, 2022. The Effect of Impurities and Geometry on the Corrosion and Thermodynamic Behavior of Molten Salts. Ph.D. thesis, University of Idaho.



Test

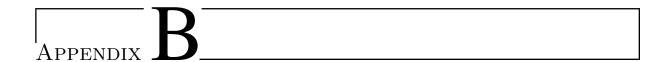
Code 1: Hello!

```
print("Hello World") #comment
try:
    a=2/x
except ZeroDivisionError:
print('undefined')
```

Inline codes like import numpy

Code 2: F strings

```
1  x = 4
2  print(f"The numeral four: {x}")
3  #comment
```



What

Straight Cash Homie