Dynamic System Modeling & PID Controller Design for a Molten Salt Microreactor

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December 6th, 2023



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Department of Nuclear Engineering and Industrial Management

About the Author



Experience

B.S Chemical Engineering (2015-2019) - Michigan Technological University M.S. Nuclear Engineering (2021-2023) - University of Idaho - NRC Fellow Modeling and Simulation Intern at Idaho National Lab

Select Publications

Root, S. J., et al., 2023. Thermodynamic analysis on xenon stripping to shorten restart time in molten salt microreactors.

Nuclear Engineering and Design 414, 112606

Root, S. J., et al., 2023. Cyber hardening of nuclear power plants with real-time nuclear reactor operation, 1. preliminary operational testing. Progress in Nuclear Energy 162, 104742

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The Molten Salt Nuclear Battery

Control Engineering

Reactor Characterization

Analysis

Conclusions

Outline



- 1 The Molten Salt Nuclear Battery
- 2 Process Control Engineering
- 3 Reactor Characterization
- 4 Results and Analysis
- 5 Conclusions

- The Molten Salt Nuclear Battery
 - Introduction
 - Xenon-135 Stripping In-Brief
- 2 Process Control Engineering
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Background



Gen-IV

blah

Molten Salt Reactors

Microreactors

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Previous and Present Work



Neutronics [3]

- Control drums give a uniform axial and radial flux profile for all reactivity insertions
- Control drum vs. reactivity curve is sinusoidal

Thermal Hydraulics [4]

Process Control

[3] Peterson, J., 8 2019. An analysis of the nuclear characteristics of a molten salt microreactor. Master's thesis. University of Idaho

[4] Carter, J. P., 2022. Multi-physics investigation of a natural circulation molten salt micro-reactor that utilizes an experimental in-pile device to improve core physics and system thermal-hydraulic performance. Ph.D. thesis, University of Idaho MSNB Modeling & Control

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Conclusions

Previous and Present Work



Neutronics [3]

Thermal Hydraulics [4]

- Transient simulations can be reduced to 1D and still correspond to STAR-CCM+
- Stable autonomous load following for relatively small ramp function power demand transients

Process Control

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Previous and Present Work



Neutronics [3]

Thermal Hydraulics [4]

Process Control

- Design controller *compliment* the autonomous capabilities provided by the passive feedback mechanisms
- Allow larger faster, more aggressive power changes
- Additional focus on time periods with a high degree of time variance (start-up, shut-down, re-start)

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Xenon-135 Stripping In-Brief

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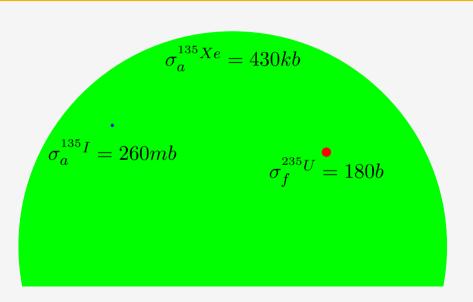
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Deference

Fission Product Poisoning





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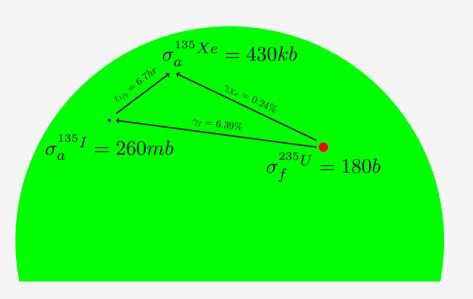
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Fission Product Poisoning





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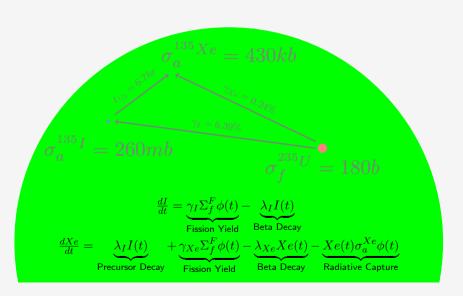
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Fission Product Poisoning





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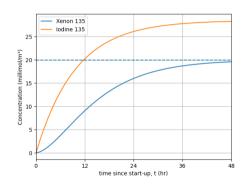
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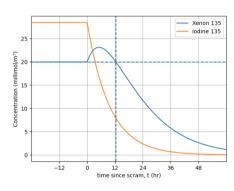
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Xenon Dynamics



Concentration of ^{135}I and ^{135}Xe vs. time following start-up



Concentration of ^{135}I and ^{135}Xe vs. time following reactor scram

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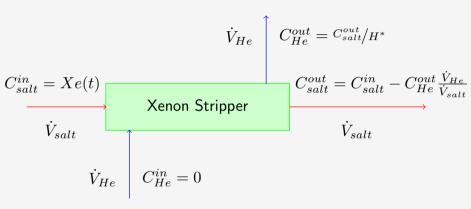
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Xenon Stripper





Schematic Drawing of Xenon Stripping Module

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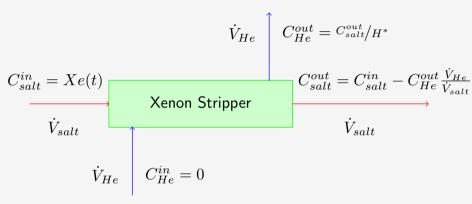
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Xenon Stripper





Schematic Drawing of Xenon Stripping Module

$$\frac{dXe}{dt} = \underbrace{\gamma_{Xe}\Sigma_f^F\phi(t)}_{\text{Fission Yield}} + \underbrace{\lambda_II(t)}_{\text{Precursor Decay}} - \underbrace{\lambda_{Xe}Xe(t)}_{\text{Beta Decay}} - \underbrace{Xe(t)\sigma_a^{Xe}\phi(t)}_{\text{Radiative Capture}} - \underbrace{\frac{Xe(t)}{\tau_{salt}}\left(H^*\frac{\dot{V}_{salt}}{\dot{V}_{He}} + 1\right)^{-1}}_{\text{Stripping}}$$

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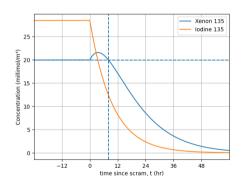
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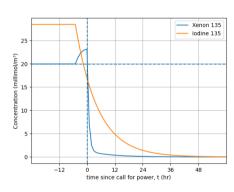
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Xenon Stripping Dynamics



Concentration of ^{135}I and ^{135}Xe vs. time following reactor scram - Restart Mode



Concentration of ^{135}I and ^{135}Xe vs. time following reactor scram - Standby Mode

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Transport Delay Problem

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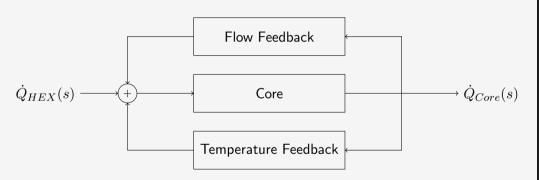
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Passive Feedback





Simplified block diagram of two primary passive feedback mechanisms in an MSNB

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Passive Feedback Temperature Reactivity



Doppler broadening

- Resonance peaks lower and broaden with increased temperature
- High kinetic energy of target nucleus introduces more relative uncertainty of the center-of-mass energy [5, Ch. 7]
- ullet More epithermal neutrons absorbed by $^{238}U\,$ etc. [6, Ch. 6]

Thermal Expansion

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^[5] Kerlin, T. W. et al., 2019. Dynamics and Control of Nuclear Reactors. Knoxville, Tennessee: Elsevier Inc

Passive Feedback Temperature Reactivity



Doppler broadening

Thermal Expansion

- Increased temperature leads to lower heavy metal density and smaller macroscopic fission cross-section at high temperature [3]
- Similar to moderator thinning in LWRs

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Delayed Neutron Precursors

- Most fission events release daughter neutrons promptly
- Sometimes, unstable nuclides which decay by neutron emission are produced instead
- $t_{1/2}$ from less than a second to over a minute [7, Ch. 6]

$$^{87}Br \xrightarrow{\beta^{-}} ^{87}Kr^{*} \rightarrow ^{86}Kr + n$$

Flowing Fuel

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Passive Feedback

Flow Reactivity



Delayed Neutron Precursors

Flowing Fuel

- Precursors produced near the core exit and long lived precursors may emit their neutrons outside of the core
- These neutrons are effectively lost from the fission chain reaction [5, Ch. 3]
- Larger power transport requires a higher flow rate
- Greater delayed neutron losses
- Negative feedback

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Main Operational Control Problem - Transport Delay



Dynamics associated with anticipated transients

- Natural circulation flow mode
- Passive feedback mechanisms
- Transport delays separating heat exchanger and core

Thought Experiment

- Step increase in power demand to a steady-state critical MSNB
- Set-point is instantaneously equal to heat exchanger power consumption
- Ideal controller which produces rapid load following with minimal overshoot

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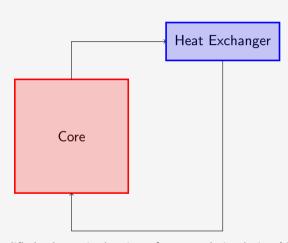
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Main Operational Control Problem - Transport Delay





Simplified schematic drawing of a natural circulation MSNB

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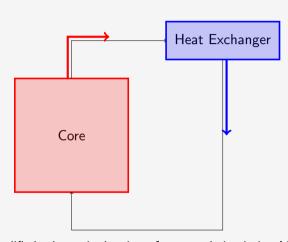
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Main Operational Control Problem - Transport Delay Immediate Response





Simplified schematic drawing of a natural circulation $\ensuremath{\mathsf{MSNB}}$

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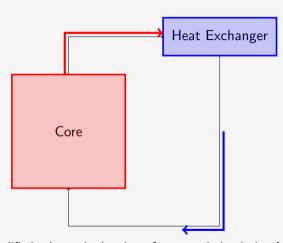
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Main Operational Control Problem - Transport Delay Heat Exchanger Perturbation





Simplified schematic drawing of a natural circulation $\ensuremath{\mathsf{MSNB}}$

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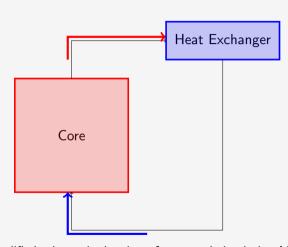
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Simplified schematic drawing of a natural circulation MSNB

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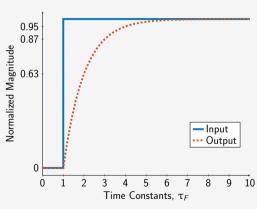
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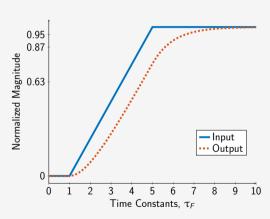
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Pre-Filter



Pre-Filter on a step-function



Pre-Filter on a ramp-function

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Process Simulation

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Control-Reactivity Curve

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Controller Tuning



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Acknowledgements



This work and my coursework is being completed under a Graduate Fellowship funded by Nuclear Regulatory Commission (NRC).

This research made use of the resources of the High Performance Computing Center at Idaho National Laboratory, which is supported by the Office of Nuclear Energy of the U.S. Department of Energy and the Nuclear Science User Facilities under Contract No. DE-AC07-05ID14517.

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- 1. Root, S. J., et al., 2023. Thermodynamic analysis on xenon stripping to shorten restart time in molten salt microreactors. Nuclear Engineering and Design 414, 112606.
- 2. Root, S. J., et al., 2023. Cyber hardening of nuclear power plants with real-time nuclear reactor operation, 1. preliminary operational testing. Progress in Nuclear Energy 162, 104742.
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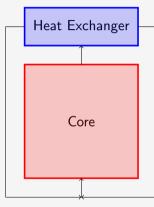
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Molten Salt Nuclear Battery



- Self-Contained liquid fueled molten salt micro-reactor - 10 year design
- 1 MW design using HALEU UF_4 dissolved in FLiNaK
- Criticality is manipulated using axial control drums
 - Neutron absorber plate covering cylinders of neutron reflector
 - Drums are rotated to point more absorber towards the core to insert negative control reactivity



Simplified schematic drawing of an MSNB

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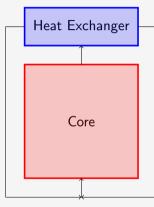
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MsNB Control Drums

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