

Run title --> hybrid scenario example		Geometry:		Physics:	
PROCESS Version --> 1.0.17		R_0 = 9.69 m		I_p = 16.66 MA	
Date: --> 18/03/2020		a = 3.126 m		Vacuum B_T at R_0 = 5.909 T	
Time: --> 10:34		A = 3.1		q_{95} = 4.5	
User: --> apearce		κ_{95} = 1.65		$\beta_{N, \text{ thermal}}$ = 2.184 % m T MA ⁻¹	
Optimising: --> Plasma major radius		δ_{95} = 0.3333		$\beta_{N, \text{ total}}$ = 2.768 % m T MA ⁻¹	
Plasma composition:		Surface area = 1667 m ²		$\beta_P, \text{ thermal}$ = 1.268	
Number densities relative to electron density:		Plasma volume = 3142 m ³		$\beta_P, \text{ total}$ = 1.608	
D + T = 0.8391		No. of TF coils = 16		$\langle t_e \rangle$ = 13.76 keV	
He = 0.06629		inboard blanket+shield = 1.055 m		$\langle n_e \rangle$ = 5.881e+19 m ⁻³	
Xe = 0.0005139		ouboard blanket+shield = 1.782 m		$\langle n_{e, \text{ line}} \rangle / n_G$ = 1.2	
W = 5e-05		Fusion power = 1910 MW		$T_{e0} / \langle T_e \rangle$ = 2.314	
Colour Legend:				$n_{e0} / \langle n_{e, \text{ vol}} \rangle$ = 1.315	
ITR				Z_{eff} = 2.529	
OP				$n_Z / \langle n_{e, \text{ vol}} \rangle$ = 0.0005639	
				τ_e = 3.619 s	
				H-factor = 1.2	
				Scaling law = IPB98(y,2)	
Coil currents etc:		Power flows:		Neutral Beam Current Drive:	
PF 1 = 11.14 MA		Nominal neutron wall load = 0.8426 MW m ⁻²		Steady state auxiliary power = 120.8 MW	
PF 3 = -8.235 MA		Normalised radius of 'core' region = 0.75		Power for heating only = 50 MW	
PF 5 = -5.393 MA		Electron density at pedestal = 4.613e+19 m ⁻³		Bootstrap fraction = 0.434	
Startup flux swing = 372.4 Wb		r/a at density pedestal = 0.94		Auxiliary fraction = 0.226	
Available flux swing = -524.9 Wb		Helium fraction = 0.06629		Inductive fraction = 0.34	
Burn time = 2 hrs		Core radiation = 123.1 MW		NB gamma = 0.3024 10 ²⁰ A W ⁻¹ m ⁻²	
TF coil type is WST Nb3Sn		Total radiation = 274.6 MW		NB energy = 1000 keV	
Peak field at conductor (w. rip.) = 13.12 T		Nuclear heating in blanket = 1459 MW		Plasma heating used for H factor = 362 MW	
I/I _{crit} = 0.5986		Nuclear heating in shield = 1.526 MW		$\frac{P_{\text{div}}}{R_0} = 21.72 \text{ MW m}^{-1}$	
TF Temperature margin =ERROR! Var missing		Power to divertor = 210.4 MW		$\frac{P_{\text{div}}}{\langle n \rangle R_0} = 36.93 \times 10^{-20} \text{ MW m}^2$	
CS Temperature margin = 6.196 K		H-mode threshold = 119.6 MW		$\frac{P_{\text{div}}}{P_{\text{LH}}} = 1.76$	
Conduit Von Mises stress =ERROR! Var missing		Divertor life = 3.039 years		H* (non-rad. corr.) = 1.066	
Case Von Mises stress =ERROR! Var missing		Primary (high grade) heat = 2591 MW		Costs	
Allowable stress = 5.8e+08 Pa		Gross cycle efficiency = 37.5 %		Cost of electricity =ERROR! Var missing	
Mass per TF coil = 1.964e+06 kg		Net cycle efficiency = 31.54 %			
		Gross electric power = 971.6 MW			
		Net electric power = 200 MW			
		Fusion-to-electric efficiency $\frac{P_{e, \text{ net}}}{P_{\text{fus}}} = 10.47 \%$			

- CS coil
- CS comp
- TF coil
- Th shield
- VV & shield
- Blanket
- First wall
- Plasma
- PF coils
- NB duct shield
- cryostat

