

			Geometry:		Physics:	
Run title	--> example qh-mode plasma scenario		$R_0$	= 9.299 m	$I_p$	= 18.49 MA
PROCESS Version	--> 1.0.16		a	= 3 m	Vacuum $B_T$ at $R_0$	= 5.316 T
Date:	--> 16/08/2019		A	= 3.1	$q_{95}$	= 3.5
Time:	--> 15:15		$K_{95}$	= 1.65	$\beta_{N, \text{thermal}}$	= 2.337 % m T MA <sup>-1</sup>
User:	--> apearce		$\delta_{95}$	= 0.3333	$\beta_{N, \text{total}}$	= 2.816 % m T MA <sup>-1</sup>
Optimising:	--> Plasma major radius		Surface area	= 1535 m <sup>2</sup>	$\beta_P, \text{thermal}$	= 1.066
Plasma composition:			Plasma volume	= 2776 m <sup>3</sup>	$\beta_P, \text{total}$	= 1.285
Number densities relative to electron density:			No. of TF coils	= 16	$\langle t_e \rangle$	= 12.87 keV
D + T	= 0.8377		inboard blanket+shield	= 1.055 m	$\langle n_e \rangle$	= 7.172e+19 m <sup>-3</sup>
He	= 0.06839		ouboard blanket+shield	= 1.782 m	$\langle n_{e, \text{line}} \rangle / n_G$	= 1.2
Xe	= 0.0004622		Fusion power	= 2167 MW	$T_{e0} / \langle T_e \rangle$	= 2.275
W	= 5e-05				$n_{e0} / \langle n_{e, \text{vol}} \rangle$	= 1.275
Colour Legend:					$Z_{\text{eff}}$	= 2.387
ITR					$n_Z / \langle n_{e, \text{vol}} \rangle$	= 0.0005122
OP					$\tau_e$	= 3.546 s
					H-factor	= 1.1
					Scaling law	= IPB98(y,2)
Coil currents etc:			Power flows:		Neutral Beam Current Drive:	
PF 1	= 16.27 MA		Nominal neutron wall load	= 1.038 MW m <sup>-2</sup>	Steady state auxiliary power	= 76 MW
PF 3	= -8.484 MA		Normalised radius of 'core' region	= 0.75	Power for heating only	= 76 MW
PF 5	= -5.588 MA		Electron density at pedestal	= 5.887e+19 m <sup>-3</sup>	Bootstrap fraction	= 0.3846
Startup flux swing	= 375 Wb		r/a at density pedestal	= 0.94	Auxiliary fraction	= 5.521e-05
Available flux swing	= -649.2 Wb		Helium fraction	= 0.06839	Inductive fraction	= 0.6153
Burn time	= 2 hrs		Core radiation	= 122.5 MW	NB gamma	= 0.285 10 <sup>20</sup> A W <sup>-1</sup> m <sup>-2</sup>
TF coil type is WST Nb3Sn			Total radiation	= 315.1 MW	NB energy	= 1000 keV
Peak field at conductor (w. rip.)	= 12.01 T		Nuclear heating in blanket	= 1674 MW	Plasma heating used for H factor	= 367.1 MW
I/I <sub>crit</sub>	= 0.6444		Nuclear heating in shield	= 1.634 MW	$\frac{P_{\text{div}}}{R_0}$	= 18.78 MW m <sup>-1</sup>
TF Temperature margin	=ERROR! Var missing		Power to divertor	= 174.6 MW	$\frac{P_{\text{div}}}{\langle n \rangle R_0}$	= 26.18 × 10 <sup>-20</sup> MW m <sup>2</sup>
CS Temperature margin	= 3.744 K		H-mode threshold	= 116.4 MW	$\frac{P_{\text{div}}}{P_{\text{LH}}}$	= 1.501
Conduit Von Mises stress	= 4.467e+08 Pa		Divertor life	= 4.26 years	H* (non-rad. corr.)	= 0.9893
Case Von Mises stress	= 5.8e+08 Pa		Primary (high grade) heat	= 2879 MW		
Allowable stress	= 5.8e+08 Pa		Gross cycle efficiency	= 37.5 %	Costs	
Mass per TF coil	= 1.528e+06 kg		Net cycle efficiency	= 31.41 %	Cost of electricity	
			Gross electric power	= 1080 MW	=ERROR! Var missing	
			Net electric power	= 500 MW		
			Fusion-to-electric efficiency $\frac{P_{e, \text{net}}}{P_{\text{fus}}}$	= 23.07 %		

- CS coil
- CS comp
- TF coil
- Th shield
- VV & shield
- Blanket
- First wall
- Plasma
- PF coils
- NB duct shield
- cryostat

