

Run title	--> hybrid scenario example	Geometry:		Physics:	
PROCESS Version	--> 1.0.16	$R_0$	= 9.656 m	$I_p$	= 16.64 MA
Date:	--> 24/09/2019	a	= 3.115 m	Vacuum $B_T$ at $R_0$	= 5.924 T
Time:	--> 14:43	A	= 3.1	$q_{95}$	= 4.5
User:	--> apearce	$K_{95}$	= 1.65	$\beta_N$ , thermal	= 2.185 % m T MA <sup>-1</sup>
Optimising:	--> Plasma major radius	$\delta_{95}$	= 0.3333	$\beta_N$ , total	= 2.768 % m T MA <sup>-1</sup>
Plasma composition:		Surface area	= 1655 m <sup>2</sup>	$\beta_P$ , thermal	= 1.269
Number densities relative to electron density:		Plasma volume	= 3109 m <sup>3</sup>	$\beta_P$ , total	= 1.608
D + T	= 0.8389	No. of TF coils	= 16	$\langle t_e \rangle$	= 13.75 keV
He	= 0.0663	inboard blanket+shield	= 1.055 m	$\langle n_e \rangle$	= 5.916e+19 m <sup>-3</sup>
Xe	= 0.0005182	ouboard blanket+shield	= 1.782 m	$\langle n_{e, \text{line}} \rangle / n_G$	= 1.2
W	= 5e-05	Fusion power	= 1911 MW	$T_{e0} / \langle T_e \rangle$	= 2.314
Colour Legend:				$n_{e0} / \langle n_{e, \text{vol}} \rangle$	= 1.315
ITR				$Z_{\text{eff}}$	= 2.539
OP				$n_Z / \langle n_{e, \text{vol}} \rangle$	= 0.0005682
				$\tau_e$	= 3.602 s
				H-factor	= 1.2
				Scaling law	= IPB98(y,2)
Coil currents etc:		Power flows:		Neutral Beam Current Drive:	
PF 1	= 11.26 MA	Nominal neutron wall load	= 0.8489 MW m <sup>-2</sup>	Steady state auxiliary power	= 121 MW
PF 3	= -8.243 MA	Normalised radius of 'core' region	= 0.75	Power for heating only	= 50 MW
PF 5	= -5.373 MA	Electron density at pedestal	= 4.641e+19 m <sup>-3</sup>	Bootstrap fraction	= 0.434
Startup flux swing	= 370.7 Wb	r/a at density pedestal	= 0.94	Auxiliary fraction	= 0.226
Available flux swing	= -522.3 Wb	Helium fraction	= 0.0663	Inductive fraction	= 0.34
Burn time	= 2 hrs	Core radiation	= 123.6 MW	NB gamma	= 0.3021 10 <sup>20</sup> A W <sup>-1</sup> m <sup>-2</sup>
TF coil type is WST Nb3Sn		Total radiation	= 276.2 MW	NB energy	= 1000 keV
Peak field at conductor (w. rip.)	= 13.15 T	Nuclear heating in blanket	= 1461 MW	Plasma heating used for H factor	= 361.8 MW
$I/I_{\text{crit}}$	= 0.5972	Nuclear heating in shield	= 1.519 MW	$\frac{P_{\text{div}}}{R_0}$	= 21.66 MW m <sup>-1</sup>
TF Temperature margin	=ERROR! Var missing	Power to divertor	= 209.2 MW	$\frac{P_{\text{div}}}{\langle n \rangle R_0}$	= 36.62 ×10 <sup>-20</sup> MW m <sup>2</sup>
CS Temperature margin	= 6.066 K	H-mode threshold	= 119.5 MW	$\frac{P_{\text{div}}}{P_{\text{LH}}}$	= 1.75
Conduit Von Mises stress	= 5.038e+08 Pa	Divertor life	= 3.062 years	H* (non-rad. corr.)	= 1.065
Case Von Mises stress	= 5.8e+08 Pa	Primary (high grade) heat	= 2592 MW	Costs	
Allowable stress	= 5.8e+08 Pa	Gross cycle efficiency	= 37.5 %	Cost of electricity	=ERROR! Var missing
Mass per TF coil	= 2.012e+06 kg	Net cycle efficiency	= 31.54 %		
		Gross electric power	= 972 MW		
		Net electric power	= 200 MW		
		Fusion-to-electric efficiency $\frac{P_{\text{e, net}}}{P_{\text{fus}}}$	= 10.47 %		

- CS coil
- CS comp
- TF coil
- Th shield
- VV & shield
- Blanket
- First wall
- Plasma
- PF coils
- NB duct shield
- cryostat

