

Multiscale Model Reduction for Neutron Diffusion Equation

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Introduction

Problem statement

We consider a bounded 2D domain Ω ($\mathbf{x} = \{x_1, x_2\} \in \Omega$) with a convex boundary $\partial\Omega$. The one-group transport equation in the diffusion approximation with one-group delayed neutron source can be written as follows

$$\begin{aligned} \frac{1}{v} \frac{\partial \phi}{\partial t} - \nabla \cdot D \nabla \phi + \Sigma_r \phi &= \frac{1 - \beta}{K_{eff}} \nu \Sigma_f \phi + \lambda c, \\ \frac{\partial c}{\partial t} + \lambda c &= \frac{\beta}{K_{eff}} \nu \Sigma_f \phi. \end{aligned} \quad (1)$$

Here $\phi(\mathbf{x}, t)$ is the neutron flux at point \mathbf{x} and time t , v is the effective neutron velocity, $D(\mathbf{x})$ is the diffusion coefficient, $\Sigma_r(\mathbf{x}, t)$ is the removal cross-section, $\Sigma_f(\mathbf{x}, t)$ is the fission cross-section, β is the effective fraction of delayed neutrons, λ is the decay constant of the delayed neutron source.

Boundary and initial conditons

Let's determine the boundary and initial conditions for Equation (1). The albedo-type conditions are set at the boundary $\partial\Omega$:

$$D \frac{\partial \phi}{\partial n} + \gamma \phi = 0, \quad (2)$$

where n is an outer normal to the boundary $\partial\Omega$, γ is the albedo constant. Let's propose that at the initial time $t = 0$, the reactor is in steady-state critical condition

$$\phi(\mathbf{x}, 0) = \phi^0(\mathbf{x}). \quad (3)$$

Time approximation

To solve the problem within the domain Ω , we approximate the system of equations (1)-(3) using the finite element method. We discretize the time derivatives using finite-difference scheme, and then bring each stationary problem to a variational formulation. For approximation in time, we use a fully implicit scheme with time step τ .

Variational formulation

To specify the variational formulation, we multiply equations by the test function q and integrate over the domain Ω . Using the integration by parts, we obtain the following variational formulation: let's find $\phi^{n+1} \in V$ such that

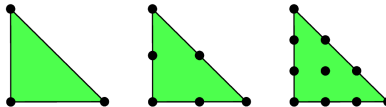
$$\begin{aligned} \int_{\Omega} \frac{1}{v} \frac{\phi^{n+1}}{\tau} q d\mathbf{x} + \int_{\Omega} D^{n+1} \nabla \phi^{n+1} \cdot \nabla q d\mathbf{x} + \int_{\Omega} \Sigma_r^{n+1} \phi^{n+1} q d\mathbf{x} - \\ \int_{\Omega} \frac{1 + \lambda\tau - \beta}{K_{\text{eff}}(1 + \lambda\tau)} \nu \Sigma_f^{n+1} \phi^{n+1} q d\mathbf{x} + \int_{\partial\Omega} \gamma \phi^{n+1} q ds = \end{aligned} \quad (4)$$

$$\int_{\Omega} \frac{1}{v} \frac{\phi^n}{\tau} q d\mathbf{x} + \int_{\Omega} \frac{\lambda c^n}{1 + \lambda\tau} q d\mathbf{x}, \quad \forall q \in V,$$

where $V = H^1(\Omega)$ is the Sobolev space consisting of scalar functions v such that v^2 and $|\nabla v^2|$ have a finite integral in Ω .

Discrete problem

Further, it's necessary to pass from the continuous variational problem (4) to the discrete problem. We introduce finite-dimensional space of finite elements $V_h \subset V$ and formulate a discrete variational problem. We use standard linear basis functions as basis functions to solve the problem on the fine grid.



Time step selection algorithm

- 1 Predictable time step: $\tilde{\tau}^{n+1} = \gamma \tau^n$ (eg $\gamma = 1.25$)
- 2 Predictive solution \tilde{y}^{n+1} : an explicit scheme, $\tilde{t}^{n+1} = t^n + \tilde{\tau}^{n+1}$
- 3 Estimation of approximation error: by found \tilde{y}^{n+1} from an implicit scheme
- 4 Step selection τ^{n+1} : $\|\psi^{n+1}\| \approx \delta$
- 5 Solution on a new time layer y^{n+1} : an implicit scheme, $t^{n+1} = t^n + \tau^{n+1}$

Neutron diffusion equation

One group diffusion approximation with one group delayed neutron sources

$$\frac{1}{v} \frac{\partial \phi}{\partial t} - \nabla \cdot D \nabla \phi + \Sigma_a \phi = (1 - \beta) \nu \Sigma_f \phi + \lambda c,$$

$$\frac{\partial c}{\partial t} + \lambda c = \beta \nu \Sigma_f \phi.$$

Boundary condition

$$D \frac{\partial \phi}{\partial n} + \gamma_a \phi = 0.$$

Initial conditions

$$\phi(0) = \phi^0, c(0) = c^0.$$

Calculated formulas

Denote vectors and matrix $\varphi = \{\varphi, s\}$, $\psi = \{\psi_1, \psi_2\}$,

$$A = \begin{pmatrix} -\nabla \cdot D \nabla + \Sigma_a - (1 - \beta) \nu \Sigma_f - \lambda & 0 \\ 0 & \lambda - \beta \nu \Sigma_f \end{pmatrix}.$$

The approximation error

$$\begin{aligned} \tilde{\psi}^{n+1} &= (A^{n+1} - A^n) \varphi^n + A^{n+1} (\tilde{\varphi}^{n+1} - \varphi^n) \\ &= \tilde{\tau}^{n+1} \left(\frac{A^{n+1} - A^n}{\tilde{\tau}^{n+1}} \varphi^n + A^{n+1} \frac{\tilde{\varphi}^{n+1} - \varphi^n}{\tilde{\tau}^{n+1}} \right). \end{aligned}$$

We match error $\tilde{\psi}^{n+1}$ with step $\tilde{\tau}^{n+1}$, and ψ^{n+1} with step τ^{n+1} :

$$\tilde{\tau}^{n+1} = \gamma_{n+1} \tau^n, \quad \gamma_{n+1} = \frac{\delta}{\|\tilde{\psi}^{n+1}\|} \gamma.$$

The needed time step

$$\tau^{n+1} \leq \bar{\tau}^{n+1}, \quad \tau^{n+1} \leq \tilde{\tau}^{n+1}, \quad \tau^{n+1} = \max \{ \tau^0, \min \{ \gamma_{n+1}, \gamma \} \tau^n \}.$$

The approximation error has the first order in time

$$\tilde{\psi}^{n+1} = \mathcal{O}(\tilde{\tau}_{n+1}).$$

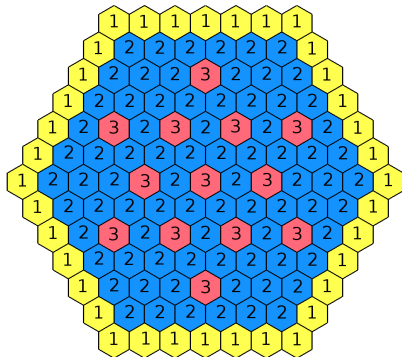
In view of this, we set

$$\|\tilde{\psi}^{n+1}\| \leq \|(A^{n+1} - A^n)\varphi^n + A^{n+1}(\tilde{\varphi}^{n+1} - \varphi^n)\|.$$

Calculated formula for time step

$$\gamma_{n+1} = \frac{\delta}{\|(A^{n+1} - A^n)\varphi^n + A^{n+1}(\tilde{\varphi}^{n+1} - \varphi^n)\|} \gamma.$$

IAEA-2D benchmark



- Without reflector
- One group of *instantaneous* and *delayed* neutrons
- Modeling effect of *immersion* or *extraction* of control rods

Scenario

Define the scenario of process:

- 1 The spectral problem is solved (initial condition);
- 2 Calculation for the non-stationary model in the range 0 to 0.5 sec;
- 3 At a moment of 0.1 sec the value Σ_a for the zone 3 changes to ± 0.000625 .

Software



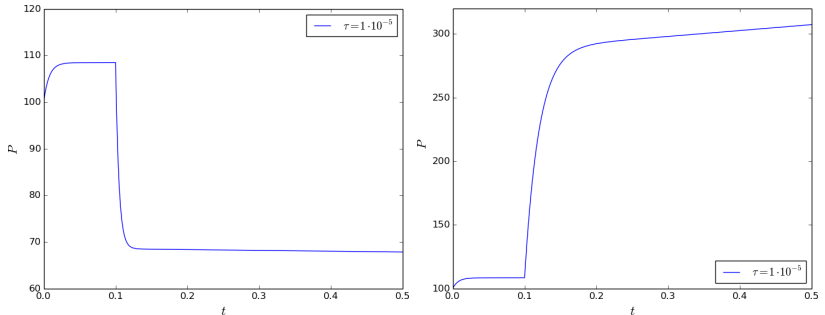
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FENICS
PROJECT



Nuclear power

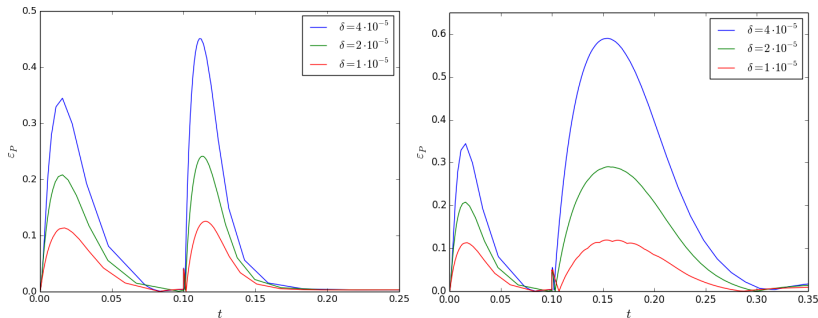


Nuclear power for immersion (left) and extraction (right).

$$P(t) = a \sum_{g=1}^G \int_{\Omega} \Sigma_{fg} \phi_g d\mathbf{x},$$

where a – normalization factor.

Error

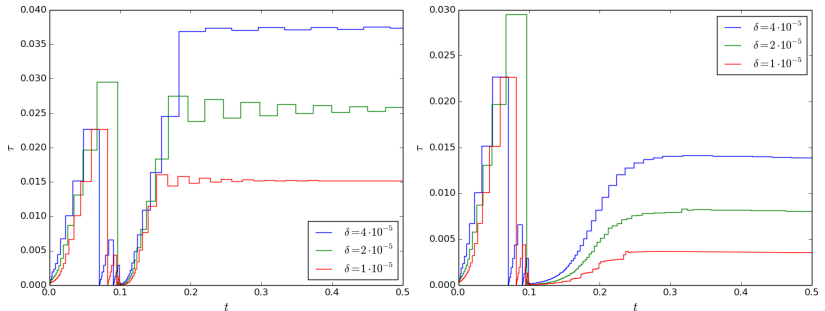


Error for immersion (left) and extraction (right).

$$\epsilon_P(t) = |P_{ref} - P|,$$

where P_{ref} – reference solution.

Time step



Time steps for immersion (left) and extraction (right).

Counting time and number of steps

	immersion			extraction		
δ	$\max(\epsilon_P)$	n	t , sec	$\max(\epsilon_P)$	n	t , sec
$4 \cdot 10^{-5}$	0.450	136	16	0.590	241	35
$2 \cdot 10^{-5}$	0.241	159	20	0.290	373	62
$1 \cdot 10^{-5}$	0.125	270	37	0.120	773	145

Reference solution: fixed time step 10^{-5} , number of steps – 50000, counting time – 2130 sec.

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Conclusion

- An algorithm for automatic time step selection for numerical solution of neutron diffusion problems has been developed.
- The solution is obtained using guaranteed stable implicit schemes, and the step choice is performed with the use of the solution obtained by an explicit scheme.
- Calculation results obtained for a neutron diffusion problems demonstrate reliability of the proposed algorithm for time step choice.

Thank you for your attention!