

LIC-13-0176 December 2, 2013

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject: Licensee Event Report 2013-013, Revision 0, for the Fort Calhoun

Station

Please find attached Licensee Event Report 2013-013, Revision 0. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B). There are no new commitments being made in this letter.

If you should have any questions, please contact Terrence W. Simpkin, Manager, Site Regulatory Assurance, at (402) 533-6263.

Sincerely,

Louis P. Cortopassi

Site Vice President and CNO

LPC/rjr

Attachment

M. L. Dapas, NRC Regional Administrator, Region IV

J. M. Sebrosky, NRC Senior Project Manager

J. C. Kirkland, NRC Senior Resident Inspector

L. E. Wilkins, NRC Project Manager

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION							ISSION /	APPRO	VED BY OMB: N	IO. 315	0-0104	E	XPIRE	S: 10	/31/2013		
(10-2010) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)										Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.							
1. FACILITY NAME Fort Calhoun Station										2. DOCKET NUMBER 3. PAGE 05000285 1 OF 3							
4. TITLE				Unquali	fied Co	mponen	ts used i	in Safet	y Sys	stem Control	Circu	uit					
5. EVENT DATE 6. LER NUMBER 7. REPORT DATE									8. OTHER FACILITIES INVOLVED								
MONTH DAY YEAR			YEAR SEQUENTIAL NUMBER NO.		MONTH			FACILITY NAME			71012111	DOCKET NUMBE 05000					
10	18	2012	2013	013	- 0	12	2	2013	FACIL	ITY NAME					05000		
9. OPERATING MODE 5 10. POWER LEVEL 0			11. THIS REPORT IS SUBMITTED PURSUANT □ 20.2201(b) □ 20.2203(a)(3)(i) □ 20.2203(a)(1) □ 20.2203(a)(4) □ 20.2203(a)(2)(i) □ 50.36(c)(1)(i)(A) □ 20.2203(a)(2)(ii) □ 50.36(c)(1)(ii)(A) □ 20.2203(a)(2)(iii) □ 50.36(c)(2) □ 20.2203(a)(2)(iv) □ 50.46(a)(3)(ii) □ 20.2203(a)(2)(v) □ 50.73(a)(2)(i)(A) □ 20.2203(a)(2)(vi) □ 50.73(a)(2)(i)(B)				(3)(i) (3)(ii) (4) (ii)(A) (iii)(A) (iii) (iii)	□ 50.73(a)(2)(i)(C) □ 50.73(a)(2)(vii) □ 50.73(a)(2)(ii)(A) □ 50.73(a)(2)(viii)(A) □ 50.73(a)(2)(iii)(B) □ 50.73(a)(2)(viii)(B) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ 50.73(a)(2)(iv)(A) □ 50.73(a)(2)(x) □ 50.73(a)(2)(v)(A) □ 73.71(a)(4) □ 50.73(a)(2)(v)(B) □ 73.71(a)(5) □ 50.73(a)(2)(v)(C) □ OTHER □ 50.73(a)(2)(v)(D) Specify in Abstract below)			
				203(a)(2)(v					в тыс		(V)(D)		or in NF				
TACILITY NAME 12. LICENSEE CONTACT FOR THIS LER FACILITY NAME Erick Matzke 12. LICENSEE CONTACT FOR THIS LER TELEPHONE NUMBER (Include Area Code) 402-533-6855																	
	13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																
CAUS	SE	SYSTEM	STEM COMPONENT MAN FACTU		_	ORTABLE O EPIX	CAU	USE SYSTEM		COMPONENT		MANU- FACTURI			RTABLE		
14		14.	. SUPPLEMENTAL REPORT EXPECTED							15. EXPECTED			MONTH	DA	Y	YEAR	
⊠YE	S (If yes,	complete	e 15. EXPECTED SUBMISSION DATE)						O	SUBMI DA	ISSION ATE	N	3	31		2014	
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) On October 3, 2013 station personnel identified that a condition with the control loop for HCV-1369, Turbine-Driven Auxiliary Feedwater Pump FW-10 Recirculation Valve, was incorrectly evaluated as not reportable. The original condition was identified on October 18, 2012, which identified unqualified components in the control loop whose failure could cause a spurious closure of HCV-1369 and result in pump damage. The station was shutdown in MODE 5 when discovered. The condition was entered in to the station's corrective action program as Condition Report 2013-18752. Engineering is reviewing this condition and the results of this review will be used to update this report.																	

LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET

U.S. NUCLEAR REGULATORY COMMISSION

1. FACILITY NAME	2. DOCKET	6	6. LER NUMBER		3. PAGE			
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NARRATIVE

BACKGROUND

Fort Calhoun Station (FCS) is a two-loop reactor coolant system of Combustion Engineering design.

FCS Technical Specification (TS) Section 2.5 states, in part that, two auxiliary feedwater (AFW) trains shall be OPERABLE when Tcold is above 300°F and with one AFW train inoperable for reasons other than Condition A, inoperable steam supply, restore the AFW train to OPERABLE status within 24 hours.

FCS Updated Safety Analysis Report (USAR) Section 9.4.1 states, in part that, FCS has two safety class auxiliary feed pumps, each capable of meeting system requirements and with diverse power sources; one electric motor driven and the other steam turbine driven.

EVENT DESCRIPTION

On October 3, 2013 station personnel identified that a condition with the control loop for HCV-1369, Turbine-Driven Auxiliary Feedwater Pump FW-10 Recirculation Valve, was incorrectly evaluated as not reportable. The original condition was identified on October 18, 2012. Although FCV-1369 is a fail-open valve, a closure due to a failure of a non-critical quality element (CQE) component coincident with a demand closure of HCV-1107B, Steam Generator RC-2A Auxiliary Feedwater Inlet Valve and HVC-1108B, Steam Generator RC-2B Auxiliary Feedwater Inlet Valve, could result in damage to FW-10 due to cavitation. FW-10 is the turbine-driven auxiliary feedwater pump. The station was shutdown in MODE 5 when discovered.

The current review determined that the components in question, although procured as CQE, had not been maintained as CQE. Additionally, the control loop is classified as non-CQE; therefore, the associated cables were not routed in safety related cable trays. The station was shutdown in MODE 5 when discovered. Although the condition only applies to the turbine-driven AFW pump (FW-10), during the last operating cycle the motor-driven AFW pump (FW-6) was taken out of service for testing.

This report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B) any operation or condition which was prohibited by the plant's Technical Specifications.

CONCLUSION

Engineering is reviewing this condition and the results of this review will be used to update this report.

CORRECTIVE ACTIONS

Engineering is reviewing this condition and the results of this review will be used to update this report.

SAFETY SIGNIFICANCE

Engineering is reviewing this condition and the results of this review will be used to update this report.

NRC FORM 366A

LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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NARRATIVE

SAFETY SYSTEM FUNCTIONAL FAILURE

This does not represent a safety system functional failure in accordance with NEI 99-02, Revision 6.

PREVIOUS EVENTS

Engineering is reviewing this condition and the results of this review will be used to update this report.