your energy partner
OPPD
Omaha Public Power District
444 South 16th Street Mall
Omaha, NE 68102-2247

LIC-12-0143
September 25, 2012

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject: Licensee Event Report 2012-018, Revision 0, for the Fort Calhoun Station

Please find attached Licensee Event Report 2012-018, Revision 0, dated September 25, 2012. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B).

There are now new commitments being made in this letter.

If you should have any questions, please contact me.

Sincerely,

Louis P. Cortopassi,
Vice President and CNO

LC/epm/rjr

Attachment
c: E. E. Collins, Jr., NRC Regional Administrator, Region IV
L. E. Wilkins, NRC Project Manager
J. C. Kirkland, NRC Senior Resident Inspector

NRC FORM 366 (10-2010) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Fort Calhoun Station	05000285	1 OF 4

4. TITLE

Containment Air Cooling Units Operated Outside of Technical Specifications during Cycle 26

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
07	27	2012	2012	018 -	0	09	25	2012	FACILITY NAME	DOCKET NUMBER 05000

9. OPERATING MODE

5

10. POWER LEVEL

0

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check all that apply)*

- ☐ 20.2201(b)
- ☐ 20.2201(d)
- ☐ 20.2203(a)(1)
- ☐ 20.2203(a)(2)(i)
- ☐ 20.2203(a)(2)(ii)
- ☐ 20.2203(a)(2)(iii)
- ☐ 20.2203(a)(2)(iv)
- ☐ 20.2203(a)(2)(v)
- ☐ 20.2203(a)(2)(vi)
- ☐ 20.2203(a)(3)(i)
- ☐ 20.2203(a)(3)(ii)
- ☐ 20.2203(a)(4)
- ☐ 50.36(c)(1)(i)(A)
- ☐ 50.36(c)(1)(ii)(A)
- ☐ 50.36(c)(2)
- ☐ 50.46(a)(3)(ii)
- ☐ 50.73(a)(2)(i)(A)
- ☒ 50.73(a)(2)(i)(B)
- ☐ 50.73(a)(2)(i)(C)
- ☐ 50.73(a)(2)(ii)(A)
- ☐ 50.73(a)(2)(ii)(B)
- ☐ 50.73(a)(2)(iii)
- ☐ 50.73(a)(2)(iv)(A)
- ☐ 50.73(a)(2)(v)(A)
- ☐ 50.73(a)(2)(v)(B)
- ☐ 50.73(a)(2)(v)(C)
- ☐ 50.73(a)(2)(v)(D)
- ☐ 50.73(a)(2)(vii)
- ☐ 50.73(a)(2)(viii)(A)
- ☐ 50.73(a)(2)(viii)(B)
- ☐ 50.73(a)(2)(ix)(A)
- ☐ 50.73(a)(2)(x)
- ☐ 73.71(a)(4)
- ☐ 73.71(a)(5)
- ☐ OTHER

Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER *(Include Area Code)*
Erick Matzke	402-533-6855

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED

☒ YES *(If yes, complete 15. EXPECTED SUBMISSION DATE)* ☐ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR
1	21	13

ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

While performing NRC Inspection Manual Chapter 0350 checklist reviews, the recovery engineering team identified that the containment air cooling and filtering system was operated outside its design basis during cycle 26 resulting in Fort Calhoun Station being in a condition prohibited by Technical Specifications during that operating cycle.

A cause analysis is in-process. When completed, this LER will be supplemented.

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NARRATIVE

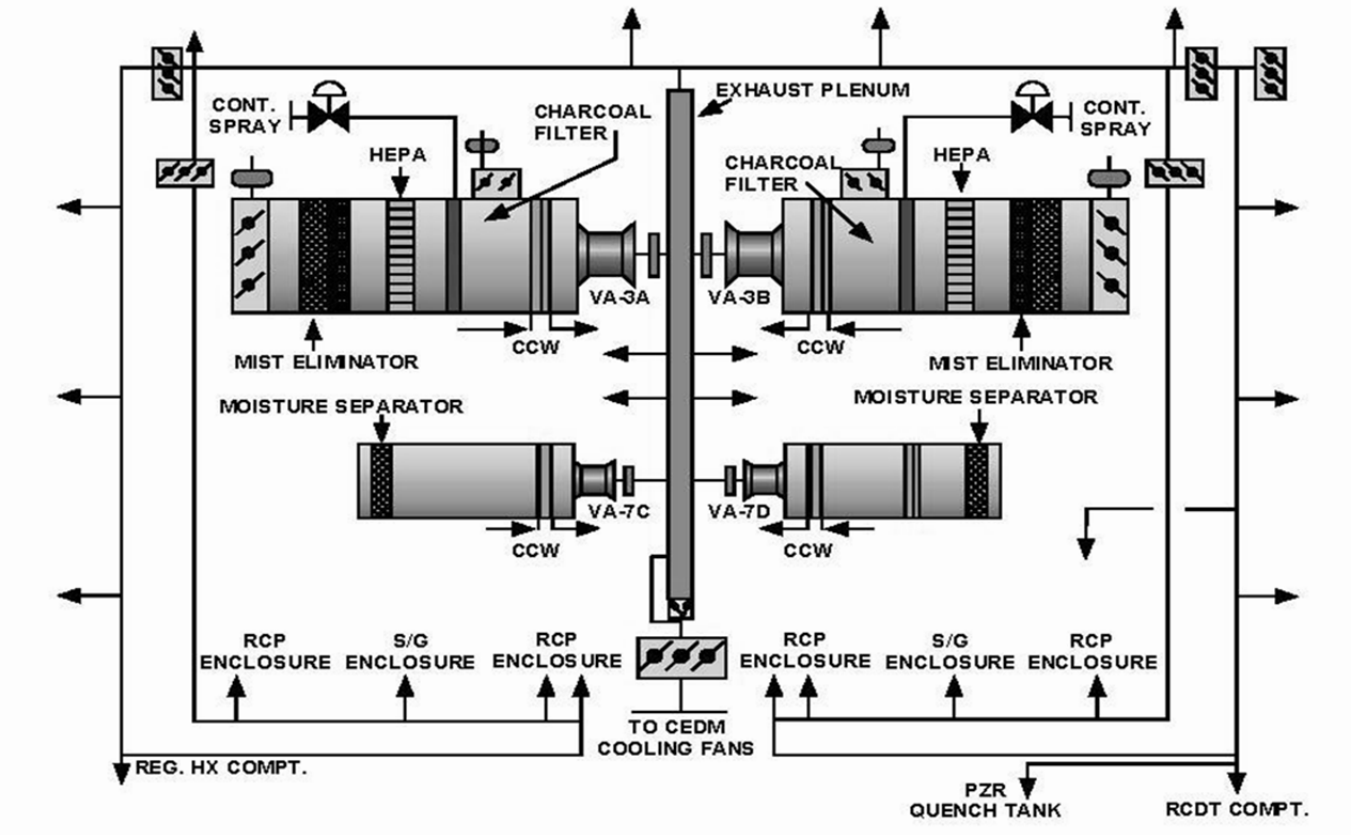
BACKGROUND

System Description

The containment air cooling and filtering system (CACFS) is designed to perform a nuclear safety function of filtering and cooling the containment building atmosphere during normal plant operation and accident conditions. During accident conditions, the CACFS is designed to limit the leakage of airborne activity from the containment and reduce containment pressures in the event of a LOCA. During normal plant operations, CACFS cools the containment atmosphere and provides filtration required prior to personnel access.

The CACFS consists of four air handling units each with a fan and heat exchanger. They discharge to a common plenum. There are two types of units. Two units (VA-3A, VA-3B) have filtering capacity while the other two air cooling units (VA-7C, VA-7D) do not have filtering capacity. The air cooling units are similar in design to the cooling and filtering units but do not include mist eliminators, HEPA filters, or charcoal filters. License Amendment No. 255 approved the change where the CACFS components are combined into a train consisting of one containment air filtering and cooling unit, and one containment cooling unit that are associated to the same emergency diesel generator. Therefore, the two trains of containment air cooling and filtering components are: (VA-3A and VA-7C) which are associated with diesel generator DG-1 and (VA-3B and VA-7D) which are associated with diesel generator DG-2

Below is a partial plant drawing of the CACFS components discussed in the Event Description.



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NARRATIVE

Technical specifications 2.4 and 3.6(3)g divide the four air handling units into two independent trains, each with 100% capacity that are to be shown to operate within +/- 10% of design flow on a refueling surveillance interval.

EVENT DESCRIPTION

On July 27, 2012, while performing NRC Inspection Manual Chapter 0350 checklist reviews, the recovery engineering team identified that the CACFS was operated outside its design basis during cycle 26 resulting in Fort Calhoun Station (FCS) being in a condition prohibited by Technical Specifications during that operating cycle. The review of Condition Report (CR) 2009-6610 identified that on the previous day VA-7C and VA-7D failed the acceptable pressure drop range of 3.3 to 5.0 inches water column (WC) as specified in surveillance test IC-ST-VA-0013.

Under Steps 7.3 and & 7.4 of the referenced surveillance is a note which states "*It is desired to have one fan in the ON position to minimize back pressure during VA-7C/D test". The following step (7.3.1 or 7.4.1) has Operations place fan combinations in service with desired and actual fan combinations. The desired position category allows Operations to interpret the note found above the step (3.4.1 or 7.4.1) (see below).

The test was re-performed a day later and during the re-test during the 2009 refueling outage the note was interpreted incorrectly and both VA-3A and VA-3C were left running during the individual testing of the VA-7C and VA-7D units. Running this fan configuration increases backpressure, reducing measured flow, resulting in what appeared to be a successful test. What was not recognized at the time was that the test violated train separation of the containment cooling fans, therefore, the FCS failed to meet the surveillance testing requirements and was outside the conditions described in technical specifications. The step is shown below:

7.3 [7.4] Pressure Drop of VA-7C (Data Sheet 3)
NOTE: * It is desired to have one fan in ON position to minimize back pressure during VA-7C/D test.
7.3.1 [7.4.1] Operations ensure following alignment:

	Desired	Actual
VA-3A	*	_____
VA-3B	*	_____
VA-7C	ON	_____
VA-7D	OFF	_____

Three of these four blanks were completed as "ON" by the performing crew, indicating that 3 of the 4 fans were running at the same time.

This condition is being submitted pursuant to: 10 CFR 50.73(a)(2)(i)(B), Any operation or condition which was prohibited by the plant's Technical Specifications.

CONCLUSION

A cause analysis is in-process. When completed, this LER will be supplemented.

CORRECTIVE ACTIONS

A cause analysis is in-process. When completed, this LER will be supplemented.

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NARRATIVE

SAFETY SIGNIFICANCE

A cause analysis is in-process. When completed, this LER will be supplemented.

SAFETY SYSTEM FUNCTIONAL FAILURE

This event does not result in a safety system functional failure in accordance with NEI-99-02.

PREVIOUS EVENTS

A cause analysis is in progress. Previous Events will be determined from the results of the cause analysis.