

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

March 11, 2010 NOC-AE-10002529 10 CFR 50.73

U. S. Nuclear Regulatory Commission Attention: Document Control Desk One White Flint North 11555 Rockville Pike Rockville, MD 20852-2738

South Texas Project
Unit 2
Docket No. STN 50-499
Licensee Event Report 2-2010-002
Unanalyzed Condition Regarding the Fire Protection System

Pursuant to 10 CFR 50.73, the STP Nuclear Operating Company (STPNOC) submits the attached Unit 2 Licensee Event Report 2-2010-002 as a result of the inadvertent isolation of the fire water supply header for fire areas in Unit 2 where the Fire Hazards Analysis credits the availability of water suppression for safe shutdown in the event of a fire. This condition is reportable under 10 CFR 50.73(a)(2)(ii)(B) as an unanalyzed condition that significantly degraded plant safety.

This event did not have an adverse effect on the health and safety of the public.

There are no commitments contained in this Licensee Event Report. Corrective actions were processed in accordance with the STP Corrective Action Program.

If there are any questions on this submittal, please contact either Ken Taplett (361) 972-8416 or me at (361) 972-7158.

L. W. Peter

Plant General Manager

KJT

Attachment: LER 2-2010-002, Unanalyzed Condition Regarding the Fire Protection System

STI: 32623787

IE22 INRR cc:

(paper copy)

Regional Administrator, Region IV U. S. Nuclear Regulatory Commission 612 East Lamar Blvd, Suite 400 Arlington, Texas 76011-4125

Mohan C. Thadani Senior Project Manager U.S. Nuclear Regulatory Commission One White Flint North (MS 8B1A) 11555 Rockville Pike Rockville, MD 20852

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 289, Mail Code: MN116
Wadsworth, TX 77483

C. M. Canady City of Austin Electric Utility Department 721 Barton Springs Road Austin, TX 78704 (electronic copy)

A. H. Gutterman, Esquire Morgan, Lewis & Bockius LLP

Mohan C. Thadani U. S. Nuclear Regulatory Commission

Kevin Howell Catherine Callaway Jim von Suskil NRG South Texas LP

Ed Alarcon
J. J. Nesrsta
R. K. Temple
Kevin Pollo
City Public Service

Jon C. Wood Cox Smith Matthews

C. Mele City of Austin

Richard A. Ratliff
Texas Department of State Health Services

Alice Rogers
Texas Department of State Health Services

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Corrective actions included revising procedures for establishing fire protection system functionality requirements and for establishing plant operations department administrative guidelines to provide clear guidance and direction for removal of fire protection system components and/or sections of the fire water main from service. A fire protection system owner was designated to coordinate all activities that affect system configuration.															
The inadvertent isolation of a portion of the fire protection main ring header adversely impacted three fire areas in Unit 2 containing redundant safe shutdown equipment. The Fire Hazards Analysis credits the availability of water suppression in these affected fire areas for safe shutdown in the event of a fire. This event resulted in no personnel injuries, no offsite radiological releases, and no damage to safety-related equipment.															

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NRC FORM 366A (9-2007) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE					
		YEAR	SEQUENTIAL NUMBER	REV NO.						
South Texas, Unit 2	05000499	2010	002	00	2	OF 5				

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

I. DESCRIPTION OF EVENT

A. REPORTABLE EVENT CLASSIFICATION

This event is reportable pursuant to 10 CFR 50.73(a)(2)(ii)(B). The fire water supply header was inadvertently isolated to fire areas in Unit 2 where the Fire Hazards Analysis credits water suppression for the achievement of safe shutdown in the event of a fire. This condition represented an unanalyzed condition that significantly degraded plant safety.

B. PLANT OPERATING CONDITIONS PRIOR TO THE EVENT

STP Unit 2 was in Mode 1 at 100% power.

C. STATUS OF STRUCTURES, SYSTEMS, OR COMPONENTS THAT WERE INOPERABLE AT THE START OF THE EVENT AND THAT CONTRIBUTED TO THE EVENT

No other inoperable structures, systems, or components contributed to the event.

D. NARRATIVE SUMMARY OF THE EVENT (include dates and approximate times)

On July 6, 2008 at 1325 hours, an equipment clearance order (ECO) was approved in Unit 2 that removed part of the fire protection main ring header from service. On July 7, 2008 at 1929 hours, a different ECO was approved in Unit 1 that removed additional parts of the fire protection main ring header from service.

On July 7, 2008 at 2211 hours, both Unit 1 and Unit 2 control rooms started to receive fire detection system alarms on the fire detection computer due to reduced pressure in the isolated portion of the fire protection main ring header. In response to the alarms, Unit 1 approved release of the ECO previously approved for this Unit. The number one diesel-driven fire pump was started to maintain system pressure. The fire protection main ring header was restored to service on July 8, 2008 at 0057 hours.

Subsequent investigation determined that an ECO in Unit 2 supported a different work package than the work package that was supported by the ECO in Unit 1. Although the work week management was aware that both work packages were scheduled for the same work week, work schedule reviews did not identify the aggregate impact on the fire protection main ring header functionality if both jobs were worked simultaneously.

The isolation of a portion of the fire protection main ring header adversely impacted three fire areas in Unit 2 containing redundant safe shutdown equipment. No fire areas in Unit 1 containing redundant safe shutdown equipment were adversely impacted. The Fire

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Hazards Analysis credits water suppression in these affected fire areas for the achievement of safe shutdown in the event of a fire. Specifically:

- Fire Area 03 is a multi-elevation area in the Mechanical/Electrical Auxiliary Building (MEAB). The analysis states that the consequence of a fire in this fire area will be mitigated by manual compensating operator action and adequate spatial separation with automatic suppression.
- Fire Area 20 is a nonradioactive pipe chase located at elevation 27 foot in the MEAB. The analysis states that the consequence of a fire in this fire area will be mitigated by manual compensating operator action and adequate spatial separation with automatic suppression.
- Fire Area 63 is the reactor containment building. A manually operated special hazard spray system is installed to prevent the vertical spread of a fire in containment from one safety train to another. The analysis states that the combination of protection (i.e. spray system) and separation, coupled with the low combustible concentrations, limited and controlled access, and the large containment free volume, provides protection from exposure fires and ensures the integrity of at least one pathway required for safe shutdown.
- E. THE METHOD OF DISCOVERY OF EACH COMPONENT OR SYSTEM FAILURE, OR PROCEDURAL OR PERSONNEL ERROR

The loss of part of the fire protection main ring header was self-revealing when the fire detection computer alarmed due to reduced pressure in the isolated portion of the fire protection main ring header.

II. COMPONENT OR SYSTEM FAILURES

- A. FAILURE MODE, MECHANISM, AND EFFECTS OF EACH FAILED COMPONENT

 No component failed. The fire protection main ring header was isolated.
- B. CAUSE OF EACH COMPONENT OR SYSTEM FAILURENot applicable.
- C. SYSTEMS OR SECONDARY FUNCTIONS THAT WERE AFFECTED BY FAILURE OF COMPONENTS WITH MULTIPLE FUNCTIONS

Not applicable.

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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

D. FAILED COMPONENT INFORMATION

Not applicable.

III. ANALYSIS OF THE EVENT

A. SAFETY SYSTEM RESPONSES THAT OCCURRED

No safety systems were required to respond during this event.

B. DURATION OF SAFETY SYSTEM TRAIN INOPERABILITY

Objective evidence from the security computer door card readers (i.e., to aid in determining the times that operators proceeded to hang the ECO) and the fire detection computer (i.e., fire protection system parameters returned to normal) conservatively indicate that the duration of this event was three hours and ten minutes.

C. SAFETY CONSEQUENCES AND IMPLICATIONS

System Information:

Plant fire protection systems which utilize water are supplied by an underground ring main system. The underground ring main supplies both Unit 1 and Unit 2. Redundant safe shutdown systems and components are generally compartmentalized with heavy concrete walls so that they are unlikely to be damaged from a single fire. In the cases where physical separation as a means of fire protection is not practicable, adequate detection and protection (e.g. water suppression) are installed. The STP Fire Hazards Analysis provides the safe shutdown analysis for each fire area containing safe shutdown systems and components.

Risk Assessment:

This event resulted in no personnel injuries, no offsite radiological releases, and no damage to safety-related equipment. The non-functional time of the removed part of the fire protection main ring header resulted in an incremental core damage probability of 3.0E-8 and an incremental large early release probability of 1.5E-9.

IV. CAUSE OF THE EVENT

The root cause of the inadvertent isolation of a portion of the fire protection main ring header was the general absence of adequate process controls for assessing the aggregate impact of

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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17) multiple work activities on the fire protection system functionality.

V. CORRECTIVE ACTIONS

Procedures for establishing fire protection system functionality requirements and for establishing plant operations department administrative guidelines were revised to provide clear guidance and direction for removal of fire protection system components and/or sections of the fire water main from service.

A fire protection system owner was designated to coordinate all activities that affect system configuration.

VI. PREVIOUS SIMILAR EVENTS

There have been no previous Licensee Event Reports within the last three years involving an unanalyzed condition that significantly degraded plant safety.

In 2007, the fire suppression system to the Unit 1 Emergency Safety Features (ESF) Diesel Generators was inadvertently isolated. This event was determined to not be an unanalyzed condition that significantly degraded plant safety. The Fire Hazards Analysis credits compartmentalization for ensuring that one safe shutdown train remains available in the event of a fire in any of the fire areas that contain the ESF Diesel Generators. Suppression is not credited in the achievement of safe shutdown for these areas.

VII. ADDITIONAL INFORMATION

This event was initially determined to be not reportable. A re-evaluation of the event determined that the condition described in this event report represented an unanalyzed condition that significantly degraded plant safety and therefore is reportable pursuant to 10 CFR 50.73(a)(2)(ii)(B). A notification of the event pursuant to 10 CFR 50.72(b)(3)(ii)(B) was made to NRC on January 14, 2010 at 1659 hours (Event Notification number 45631).