

LIC-12-0073 May 29, 2012

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject: Licensee Event Report 2012-004, Revision 0, for the Fort Calhoun

Station

Please find attached Licensee Event Report 2012-004, Revision 0, dated May 29, 2012. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(ii)(B) and (a)(2)(v)(A,B,C,D)

No commitments are being made in this letter.

If you should have any questions, please contact me.

Sincerely,

D. J. Bannister

Vice President and CNO

DJB /epm

Attachment

E. E. Collins, Jr., NRC Regional Administrator, Region IV

L. E. Wilkins, NRC Project Manager

J. C. Kirkland, NRC Senior Resident Inspector

INPO Records Center

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB: NO. 3150-0104 EXPIRE							S: 10	0/31/2013								
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)								 	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are inco rporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Priv acy Section (T-5 F53), U.S. Nuclear Regulator y Commission, Washington, DC 205 55-0001, or b y internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sp onsor, and a person is not required to respond to, the information collection.							
4 FACILITY NAME									DOC	VET NUMBER	1	la na	\CE			
1. FACILITY NAME Fort Calhoun Station								2. DOCKET NUMBER 3. PAGE 1 OF 2								
4. TITLE Inadequate Analysis of Drift Affects Safety Related Equipment																
5. E	VENT D	ATE	6.	LER NUMI	BER	7.	7. REPORT DATE			8. OTHER FACILI						
MONTH DAY YEAR			YEAR SEQUENTIAL REV NUMBER NO.			ONTH DAY YEAR		FACILITY NAME				DOCKET NUMBER 05000				
03	29	2012	2012	- 004	- 0	05	29	2012	FACILITY NAME					DOCKET NUMBER 05000		
9. OPER	ATING I	MODE	11	. THIS REI	PORTIS	SUBMIT	TED PURS	UANT TO	THE	REQUIREMEN [®]	TS OF	10 CFR	§: (Check	all tha	at ap	ply)
5 10. POWER LEVEL 0			□ 20.2201(b) □ 20.2203(a)(3)(i) □ 20.2201(d) □ 20.2203(a)(3)(i) □ 20.2203(a)(1) □ 20.2203(a)(4) □ 20.2203(a)(2)(i) □ 50.36(c)(1)(i)(A □ 20.2203(a)(2)(ii) □ 50.36(c)(1) □ 20.2203(a)(2)(iii) □ 50.36(c)(2) □ 20.2203(a)(2)(iv) □ 50.46(a)(3)(ii) □ 20.2203(a)(2)(v) □ 50.73(a)(2)(i)(A □ 20.2203(a)(2)(vi) □ 50.73(a)(2)(i)(B)(3)(ii))(4))(i)(A))(ii)(A))(ii)(A) 2) 3)(ii)	□ 50.73(a)(2)(i)(C) □ 50.73(a)(2)(vii) □ 50.73(a)(2)(ii)(A) □ 50.73(a)(2)(viii)(A) □ 50.73(a)(2)(ii)(B) □ 50.73(a)(2)(viii)(B) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ 50.73(a)(2)(iv)(A) □ 50.73(a)(2)(x) □ 50.73(a)(2)(v)(A) □ 73.71(a)(4) □ 50.73(a)(2)(v)(B) □ 73.71(a)(5) □ 50.73(a)(2)(v)(C) □ OTHER □ 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A				oelow					
						12. LICE	NSEE CON	TACT FO	R THI	S LER						
FACILITY NAME Erick Matzke Telephone number (Include Area Code) 402-533-6855																
			13. CON	IPLETE O	NE LINI	FOR EA	СН СОМРО	ONENT F	AILUR	E DESCRIBED	IN TH	IS REPO	DRT			
CAUSE		SYSTEM	SYSTEM COMPON		MAI FACT			CAU	SE	SYSTEM	COMPONENT		MANU- FACTURER		REPORTABLE TO EPIX	
		14.	SUPPLE	MENTAL	REPOR	T EXPEC	TED			15. EXF	FCTE	D	MONTH	DA'	Y	YEAR
Myr	C (15 v.o.o.								NO	SUBM	ISSIO		-	10		
		•					⊏) -spaced typ		NO	DA	TE		8	10	,	2012
While investigating operating experience from another station concerning potential instrument drift it was determined that Fort Calhoun Station (FCS) is subject to similar conditions. It was determined that pressure switches that provide safety related signals for high containment pressure to the reactor protection system (RPS) and engineered safeguards actuation circuitry may be similarly affected at FCS. The impact of the potential drift was evaluated and it was determined that neither RPS nor the engineered safeguard circuitry may actuate at the required containment pressure of 5 psig. An evaluation determined that the actuation may not occur until slightly higher than the required pressure. Other systems are currently being evaluated for this condition. A cause analysis is being performed and will be provided in a supplement to this report. Corrective actions will be determined following the completion of the cause analysis.																

NRC FORM 366A

(10-2010)

LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6		3. PAGE			
Fort Callegue Station	05000385	YEAR	SEQUENTIAL REV NUMBER NO.		,	OF	2
Fort Calhoun Station	05000285	2012	- 004 -	00		OF	۷

NARRATIVE

EVENT DESCRIPTION

While investigating operating experience from another station it was determined that Fort Calhoun Station (FCS) is subject to similar conditions. The operating experience involved setpoint drift of safety-related pressure switches beyond what had been accounted for in the station's safety analyses.

Following investigation and evaluation, it was determined that pressure switches that provide safety-related signals for high containment pressure to the reactor protection system (RPS) and engineered safeguards actuation circuitry may be similarly affected at FCS. The impact of the potential drift was evaluated and it was determined that neither RPS nor the engineered safeguard circuitry may actuate at the required containment pressure of 5 psig. An evaluation determined that the actuation may occur at a slightly higher value than the required pressure. Other systems are currently being evaluated for the condition.

On May 2, 2012, an eight (8) hour report was made per 10 CFR 50.72(b)(3)(ii)(B) to the NRC Headquarters Operation Office (HOO) at 1802 CDT (Event Number (EN) 47892). After further evaluation this report is being made per 10 CFR 50.73(a)(2)(ii)(B) and (a)(2)(v)(A,B,C,D).

CONCLUSION

A cause analysis is being performed and the results will be provided in a supplement to this report.

CORRECTIVE ACTIONS

Corrective actions will be determined following the completion of the cause analysis.

SAFETY SIGNIFICANCE

A cause analysis is in progress. The safety significance of this event will be provided in a supplement to this LER following completion of the analysis.

SAFETY SYSTEM FUNCTIONAL FAILURE

This event does result in a safety system functional failure in accordance with NEI-99-02.

PREVIOUS EVENTS

No events of a similar nature have been identified.

Date

LICENSING CORRESPONDENCE REVIEW FORM

LIC-12-0073

Date Issued:	5/23/12		Requested Return Date:_	5/25/12					
F	Review/Approv	/al	Information						
Susan Baughn	'		Dave Bannister						
John Herman			Woody Goodell						
Steve Miller			Brad Blome						
Chris Sterba			L. Smith						
Corey Cameron									
John Steinke									
Ken Dunham									
review for our reco	ords, please s quested return	sign this form and r date, your concurre		re). In order to document your pordinator. If n o notification is assumed. 6855					
[] Approved with	no comment.	[] Approve noted.	d pending resolution of comr	nents as					
Comments:									
		_							

Reviewer's Signature

LICENSING CORRESPONDENCE REVIEW FORM SUMMARY

LIC-12-0073

Date Issued: 5/23/12 Requested Return Date: 5/25/12

Name	Date Comments Received	No Comments ¹	Comments - How Resolved ²		
Susan Baughn	5/28/12		corrected		
John Herman	none				
Steve Miller	5/27/12	Х			
Mike Smith	none				
Corey Cameron	none				
Scott Pallas	none				
Chris Sterba	none				
John Steinke	5/28/12	Х	verbal		
Ken Dunham	5/24/12	Х			
L. Smith	none				

Subject: LER 2012-004 Rev 0 "Inadequate Analysis of Drift Affects RPS and ECCS Equipment"							
NOTE – This submittal does does notX_ include documents/files on CD-ROM. ³							
NL Comment Coordinator Signature	Date						
Responsible Dept. Manager (if required)	Date						
Review by Nuclear Licensing Supervisor	Date						

Attach only signed Licensing Correspondence Review Form.
 Attach necessary documentation.

Ensure that the CD-ROM files are formatted properly for electronic information exchange (EIE) to the NRC. (Reference NL-17)