



**Omaha Public Power District**

444 South 16<sup>th</sup> Street Mall  
Omaha, NE 68102-2247

LIC-13-0166

November 12, 2013

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555-0001

Reference: Docket No. 50-285

**Subject: Licensee Event Report 2013-015, Revision 0, for the Fort Calhoun Station**

Please find attached Licensee Event Report 2013-015, Revision 0. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(ii)(B). No commitments are being made in this letter.

If you should have any questions, please contact Terrence W. Simpkin, Manager, Site Regulatory Assurance, at (402) 533-6263.

Sincerely,

Louis P. Cortopassi  
Site Vice President and CNO

LPC/rjr

**Attachment**

c: M. L. Dapas, NRC Regional Administrator, Region IV  
J. M. Sebrosky, NRC Sr. Project Manager  
L. E. Wilkins, NRC Project Manager  
J. C. Kirkland, NRC Sr. Resident Inspector

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the

1. FACILITY NAME Fort Calhoun Station								2. DOCKET NUMBER 05000285	3. PAGE 1 OF 3	
4. TITLE Unqualified Coating used as a Water Tight Barrier in Rooms 81 and 82										
5. EVENT DATE			6. LER NUMBER		7. REPORT DATE			8. OTHER FACILITIES INVOLVED		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME DOCKET NUMBER 05000	
09	13	2013	2013	- 015 - 0		11	12	2013	FACILITY NAME DOCKET NUMBER 05000	
9. OPERATING MODE 5			11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)							
			<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)				
			<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)				
			<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)				
			<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)				
10. POWER LEVEL 0			<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)				
			<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)				
			<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)				
			<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER				
			<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A				
12. LICENSEE CONTACT FOR THIS LER FACILITY NAME Erick Matzke										
TELEPHONE NUMBER (Include Area Code) 402-533-6855										
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	
14. SUPPLEMENTAL REPORT EXPECTED <input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)					15. EXPECTED SUBMISSION DATE		MONTH DAY YEAR			
					<input type="checkbox"/> NO		02 03 2014			
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)										
<p>On September 13, 2013, it was identified that the floor coatings in Rooms 81 and 82 may not maintain its integrity during a high energy line break environment allowing water to migrate into the rooms below which contain the diesel generators and safety related switchgear. This was reported on September 23, 2013, under 10 CFR 50.72(b)(3)(ii)(B), Unanalyzed Condition (Event Notification 49378). Fort Calhoun Station was shutdown in MODE 5 when the condition was identified and entered into the station's corrective action program as Condition Report 2013-17605.</p> <p>Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results of this review will be used to update this report.</p>										

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**NARRATIVE****BACKGROUND**

Fort Calhoun Station (FCS) is a two-loop reactor coolant system of Combustion Engineering (CE) design.

**EVENT DESCRIPTION**

On September 13, 2013, it was identified that the floor coatings in Rooms 81 and 82 may not maintain its integrity during a high energy line break environment allowing water to migrate into the rooms below which contain the diesel generators and safety related switchgear. This was reported on September 23, 2013, under 10 CFR 50.72(b)(3)(ii)(B), Unanalyzed Condition (Event Notification 49378). Fort Calhoun Station was shutdown in MODE 5 when the condition was identified and entered into the station's corrective action program as Condition Report 2013-17605.

The FCS Updated Safety Analysis Report, Appendix M, Postulated High Energy Line Rupture, states, in part, that the evaluations of Room 81 flooding assume that leakage through cracks in the concrete is minimal. A number of cracks in the floor of Room 81 and Room 82 were identified and subsequently repaired prior to any significant leakage occurring.

Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results will be used to update this report.

This report is submitted pursuant to 10 CFR 50.73(a)(2)(ii)(B), any event or condition that resulted in: the nuclear power plant being in an unanalyzed condition that significantly degraded plant safety.

**CONCLUSION**

Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results of this review will be used to update this report.

**CORRECTIVE ACTIONS****Actions Completed to Address Floor Leaks:**

The floor in Room 82 was recoated.

The seismic gap between containment and the auxiliary building was sealed.

All penetrations that had openings below 2 feet above the floor were coated and the area around the impingement plate was sealed.

Cracks in the ceilings of the switchgear and upper electrical penetration rooms were repaired.

**Actions Remaining to Address Floor Leaks:**

The floor in Room 81, the hallway between Room 82 and the filter area, and the filter area floor will be recoated.

Update USAR Appendix M to reflect calculation FC08313, Fort Calhoun Room 81 Flooring Analysis.

Develop and implement a quarterly preventive maintenance task to inspect the floor coatings in Rooms 81 and 82.

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**NARRATIVE****SAFETY SIGNIFICANCE**

Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results of this review will be used to update this report.

**SAFETY SYSTEM FUNCTIONAL FAILURE**

Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results of this review will be used to update this report.

**PREVIOUS EVENTS**

Engineering is reviewing this condition and the evaluation performed in 2009 for a previous condition. The completed results of this review will be used to update this report.