

LIC-12-0117 August 6, 2012

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject:

Licensee Event Report 2012-011, Revision 0, for the Fort Calhoun

Station

Please find attached Licensee Event Report 2012-011, Revision 0, dated August 6, 2012. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B) and 10 CFR 50.73(a)(2)(v)(D).

No commitments are being made in this letter.

If you should have any questions, please contact me.

Sincerely,

D. J. Bannister

Vice President and CNO

DJB /sds

Attachment

E. E. Collins, Jr., NRC Regional Administrator, Region IV

L. E. Wilkins, NRC Project Manager

J. C. Kirkland, NRC Senior Resident Inspector

INPO Records Center

NRC FOF (10-2010)	LIC	LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)								Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.							
1. FACIL	IIYNA	ME	Fort Calhoun Station						2. DC	CKET NUMBER 3. PAGE 05000285 1					OF 3		
4. TITLE	Emergency Diesel Inoperability Due to Bus Loads During a LOOP																
5. E	VENT [ATE	6. LER NUMBER 7. REPORT DATE					ATE		8. OTHER FACILITIES INVOLVED							
MONTH	DAY	YEAR	YEAR	SEQUENTI NUMBER		MONTH	DAY	YEAR	1	CILITY NAME				DOCKET NUMBER 05000 DOCKET NUMBER			
4	16	2012	2012	- 011	- 0	8	6	2012	2	CILITY NAME					050	00	
9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)												ply)					
10. POW	5 ER LE	VEL .	□ 20.2201(b) □ 20.2203(a)(3)(□ 20.2201(d) □ 20.2203(a)(3)(□ 20.2203(a)(1) □ 20.2203(a)(4) □ 20.2203(a)(2)(i) □ 50.36(c)(1)(i)(i)(i)(i)(i)(i)(i)(i)(i)(i)(i)(i)(i)					(3)(ii) (4) (i)(A) (ii)(A) (ii)(A)	□ 50.73(a)(2)(i)(C) □ 50.73(a)(2)(vii) □ 50.73(a)(2)(ii)(A) □ 50.73(a)(2)(viii)(A) □ 50.73(a)(2)(ii)(B) □ 50.73(a)(2)(viii)(B) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ 50.73(a)(2)(iv)(A) □ 50.73(a)(2)(x) □ 50.73(a)(2)(v)(B) □ 73.71(a)(4) □ 50.73(a)(2)(v)(C) □ OTHER								
			□ 20.2203(a)(2)(vi) □ 50.73(a)(2)(i)(B) □ 50.73(a)(2)(v)(D) Specify in Abstrar or in NRC Form 3						stract rm 366	below SA							
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FACILITY N	FACILITY NAME Erick Matzke									TELEPHONE NUMBER (Include Area Code) 402-533-6855							
			13. CON	IPLETE ON	IE LINE F	OR EAC	H COMPO	NENT F	AILU	IRE DESCRIBED	IN THIS	REPO	RT				
CAUSE		SYSTEM	1 COMPONENT MANU FACTUR				CAI	USE	E SYSTEM	COMPONENT		MANU- FACTURER		REPORTABLE TO EPIX			
14. SUPPLEMENTAL REPORT EXPECTED								NO	SUBMI	15. EXPECTED MONTH SUBMISSION DATE 10		MONTH 10	3°		YEAR 2012		
ABSTRA	CT (Lin	nit to 1400	spaces,	i.e., approx	imately 1	5 single-s _l	paced type	ewritten i	lines)						<u> </u>		

An Engineering review identified that a potential issue existed concerning Emergency Diesel Generators (EDG) capability to power required loads in certain loss of offsite power (LOOP) scenarios, specifically those scenarios during which a Loss of Coolant Accident (LOCA) or Main Steam Line Break (MSLB) does not occur. In a LOOP without a concurrent accident signal, the 480 V load shed that would be initiated as a direct result of the accident signal does not occur. Therefore, the electrical load that the EDGs must pick up when the EDG output breaker automatically closes could be significantly higher than the dead load that exists in an accident scenario. A review of design basis calculations and engineering analyses has identified several evaluations that consider the EDG dead load during accidents. However, no documents evaluating EDG dead loads in non-accident conditions were found. If one EDG were inoperable due to maintenance or other activities and the electrical distribution system loading conditions were such that the other EDG could have reached the output breaker trip settings during a LOOP event, both EDGs would be inoperable and FCS would have to take action per Technical Specification (TS) 2.0.1. It is conservative to assume that such conditions existed for those EDG outages that exceeded six hours. However, actions were not taken for two inoperable EDGs per the requirements of TS 2.0.1, resulting in operation or condition prohibited by TS.

A cause analysis is in progress. The results of the analysis will be published in a supplement to this LER.

NRC FORM 366A

(10-2010)

LICENSEE EVENT REPORT (LER) U.S. NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6	. LER NUMBER	3. PAGE			
Fort Callegue Station	05000285	YEAR	SEQUENTIAL NUMBER	REV NO.	,	OF	3
Fort Calhoun Station		2012	- 011 -	0	2		

NARRATIVE

BACKGROUND

Fort Calhoun Station (FCS) is a two-loop reactor coolant system of Combustion Engineering (CE) design. There are two installed safety related Emergency Diesel Generators (EDGs). These units provide reliable in-plant AC electrical power when no off-site electrical power is available. Each EDG can provide adequate in-plant AC power for safe shutdown of the plant and operation of the required engineered safeguards equipment in the event of a design basis accident (DBA). The EDGs and auxiliary systems are designed so that a single failure coincident with a loss of offsite power will not prevent safe plant shutdown. The EDGs are maintained in standby during normal and shutdown plant operations. In the event that low voltage on a 4160 volt (V) bus is detected, the associated EDG will automatically start. Automatic load sequencing occurs with accident signals.

Technical Specification (TS) 2.7 specifies the limiting conditions for operations, specifications, and minimum requirements for FCS electrical systems. Two EDGs are required with provisions for single EDG inoperability. Inoperability of both EDGs would place FCS in TS 2.0.1 as there are no actions for two inoperable EDGs.

TS 2.0.1 applies to the operable status of all systems, subsystems, trains, components, or devices covered by the Limiting Conditions for Operation. This specification delineates corrective measures to be taken for circumstances not directly provided for in the system specific specifications and whose occurrence would violate the intent of the specification.

EVENT DESCRIPTION

An Engineering review identified that a potential issue existed concerning Emergency Diesel Generators (EDG) capability to power required loads in certain loss of offsite power (LOOP) scenarios, specifically those scenarios during which a Loss of Coolant Accident (LOCA) or Main Steam Line Break (MSLB) does not occur. In a LOOP without a concurrent accident signal, the 480 V load shed that would be initiated as a direct result of the accident signal does not occur. Therefore, the electrical load that the EDGs must pick up when the EDG output breaker automatically closes could be significantly higher than the dead load that exists in an accident scenario. A review of design basis calculations and engineering analyses has identified several evaluations that consider the EDG dead load during accidents. However, no documents evaluating EDG dead loads in non-accident conditions were found. If one EDG were inoperable due to maintenance or other activities and the electrical distribution system loading conditions were such that the other EDG could have reached the output breaker trip settings during a loss of offsite power (LOOP) event, both EDGs would be inoperable and FCS would have to take action per TS 2.0.1. It is conservative to assume that such conditions existed for those EDG outages that exceeded six hours. However, actions were not taken for two inoperable EDGs per the requirements of TS 2.0.1, resulting in operation or condition prohibited by TS.

This report is being submitted in accordance with 10 CFR 50.73(a)(2)(i)(B), operation or condition prohibited by Technical Specifications and 10 CFR 50.73(a)(2)(v)(D), event or condition that could have prevented the fulfillment of the safety function of structures or systems that are needed to mitigate the consequences of an accident.

This LER reports a condition where analyses have not been found that would support operating practices, resulting in periods of EDG inoperability in excess of TS allowed limits. The initial Operations review focused on the current operating conditions, noting that the condition would need to

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1. FACILITY NAME	2. DOCKET	6	6. LER NUMBER	3. PAGE			
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NARRATIVE

be resolved prior to start up. The station paradigm inappropriately concluded that reportability could be evaluated at a later date since current operating conditions were not challenged, and that the 60-day reporting window commenced when the event was determined to be reportable. FCS has been systematically addressing issues that have been identified since June 2011, in response to the flooding conditions, switchgear fire, and increased oversight. This LER is being submitted beyond the 60-day regulatory reporting requirement due to non-conservative decisions with respect to procedural and regulatory reportability requirements and resource constraints caused by the operating challenges which began in June 2011.

CONCLUSION

A cause analysis is in progress. The results of the analysis will be published in a supplement to this LER.

CORRECTIVE ACTIONS

A cause analysis is in progress. One interim action has been identified. Pending resolution of this issue, Operations will not cross tie 480 V buses except in an emergency. Other corrective actions will be published in a supplement to this LER.

SAFETY SIGNIFICANCE

A cause analysis is in progress. The results of the analysis will be published in a supplement to this LER.

SAFETY SYSTEM FUNCTIONAL FAILURE

This event does result in a safety system functional failure in accordance with NEI-99-02.

PREVIOUS EVENTS

A cause analysis is in progress. Previous events will be determined from the results of the cause analysis.