

444 South 16th Street Mall Omaha, NE 68102-2247

LIC-13-0093 July 2, 2013

U.S. Nuclear Regulatory Commission

Attn: Document Control Desk Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject:

Licensee Event Report 2013-010, Revision 0, for the Fort Calhoun

Station

Please find attached Licensee Event Report 2013-010, Revision 0. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(v)(B) and (D), and 10 CFR 50.73(a)(2)(vii). There are no new commitments being made in this letter.

If you should have any questions, please contact Terrence W. Simpkin, Manager, Site Regulatory Assurance, at (402) 533-6263.

Sincerely,

Louis P. Cortopassi

Site Vice President and CNO

LPC/rjr/epm

Attachment

A. T. Howell, NRC Regional Administrator, Region IV

J. M. Sebrosky, NRC Sr. Project Manager

L. E. Wilkins, NRC Project Manager

J. C. Kirkland, NRC Sr. Resident Inspector

| NRC FORM 366 | | | | U.S. NUC | LEAR R | EGULATO | RY COMM | ISSION | APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013 | | | | | | |
|--|------------|------------|---|---------------------------|----------------|---|--|-------------------------------|--|--------------------|--------------|---------------------|---------------------|----------------|--|
| (10-2010) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block) | | | | | | | | | Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are inco rporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Priv acy Section (T-5 F53), U.S. Nuclear Regulator y Commission, Washington, DC 205 55-0001, or b y internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sp onsor, and a person is not required to respond to, the information collection. | | | | | | |
| | | | | | | | | 2. | 2. DOCKET NUMBER 3. PAGE | | | | | | |
| Fort Calhoun Station | | | | | | | | | 05000285 1 OF 3 | | | | | | |
| 4. TITLE | | | | | | HPSI | Pump F | low Imba | aland | ce | | | | | |
| 5. E | VENT D | ATE | 6. LER NUMBER | | | 7. REPORT DATE | | | 8. OTHER FACILIT | | | | | | |
| MONTH | | | YEAR | SEQUENTI NUMBER | | MONTH | DAY | YEAR | FACILITY NAME | | | DOCKET NUMBER 05000 | | | |
| 5 | 03 | 2013 | 2013 | 010 | - 0 | 7 | 2 | 2013 | FACIL | LITY NAME | | | DOCKET 0 | NUMBER 5000 | |
| 9. OPER | ATING I | MODE | 11 | THIS REP | ORT IS | SUBMITT | ED PURSI | JANT TO | THE | REQUIREMENT | TS OF 10 CFR | §: (Check | all that | apply) | |
| 5 10. POWER LEVEL 0 | | | $\begin{array}{ c c c c c } \hline 20.2201(b) & & & 20.2203(a)(3)(i) \\ \hline 20.2201(d) & & 20.2203(a)(3)(ii) \\ \hline 20.2203(a)(1) & & 20.2203(a)(4) \\ \hline 20.2203(a)(2)(i) & & 50.36(c)(1)(i)(A) \\ \hline 20.2203(a)(2)(ii) & & 50.36(c)(1)(ii)(A) \\ \hline 20.2203(a)(2)(iii) & & 50.36(c)(2) \\ \hline 20.2203(a)(2)(iii) & & 50.36(c)(2) \\ \hline 20.2203(a)(2)(iv) & & 50.46(a)(3)(ii) \\ \hline 20.2203(a)(2)(v) & & 50.73(a)(2)(i)(A) \\ \hline 20.2203(a)(2)(vi) & & 50.73(a)(2)(i)(B) \\ \hline \end{array}$ | | | (3)(ii) (4) (ii)(A) (iii)(A) (iii) (iii) (iii) (iii)(A) (ii)(B) | □ 50.73(a)(2)(i)(C) □ 50.73(a)(2)(vii) □ 50.73(a)(2)(ii)(A) □ 50.73(a)(2)(viii)(A) □ 50.73(a)(2)(ii)(B) □ 50.73(a)(2)(viii)(B) □ 50.73(a)(2)(iii) □ 50.73(a)(2)(ix)(A) □ 50.73(a)(2)(iv)(A) □ 50.73(a)(2)(x) □ 50.73(a)(2)(v)(A) □ 73.71(a)(4) □ 50.73(a)(2)(v)(B) □ 73.71(a)(5) □ 50.73(a)(2)(v)(C) □ OTHER □ 50.73(a)(2)(v)(D) Specify in Abstract below or in NRC Form 366A | | | | |)(A))(B) (A) | | | |
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| FACILITY N | AME | | | | Erick | Matzke | | | | | TELEPHO | 402-53 | | | |
| | | | 13. COMPLETE ONE LINE FOR EACH COMPON | | | | NENT FA | LURE DESCRIBED IN THIS REPORT | | | | | | | |
| CAUSE | | SYSTEM | COM | MPONENT | MANI FACTUI | | PORTABLE TO EPIX | CAUS | USE SYSTEM COMPONENT | | | | PORTABLE TO EPIX | | |
| | | | | | | | | | | | | | | | |
| | , | | 14. SUPPLEMENTAL REPORT EXPECTED | | | | | | | 15. EXP | | MONTH | DAY | YEAR | |
| ⊠YE | S (If yes, | complete | 15. EXF | EXPECTED SUBMISSION DATE) | | | | | 10 | IO SUBMISSION DATE | | 8 | 9 | 13 | |
| ABSTRA | CT (Lim | it to 1400 | spaces, | i.e., approx | imately | 15 single-s | spaced type | ewritten lin | es) | • | | | | | |
| On May 03, 2013, at approximately 1759 CST, it was identified that the high pressure injection pump injection flows to the reactor coolant system are not balanced in accordance with the Fort Calhoun Station (FCS) Updated Safety Analysis Report Section 14.15.5.2. FCS is currently shutdown with fuel removed from the vessel. A causal analysis is in progress. The results of the analysis will be published in a supplement to this | | | | | | | | | | | | | | | |
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U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) **CONTINUATION SHEET**

| 1. FACILITY NAME | 1. FACILITY NAME 2. DOCKET 6. LER NUMBER | | | | 3. PAGE | | |
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| Fort Collegue Station | 05000285 | YEAR | SEQUENTIAL NUMBER | REV NO. | 2 | OF | 9 |
| Fort Calhoun Station | | 2013 | - 005 - | 1 | | OF | <u> </u> |

NARRATIVE

BACKGROUND

Fort Calhoun Station (FCS) is a two-loop reactor coolant system of Combustion Engineering (CE) design. The safety injection system (including three high pressure and two low pressure safety injection pumps and the four safety injection tanks, one safety injection and refueling water storage tank, and interconnecting piping).

EVENT DESCRIPTION

On May 03, 2013, at approximately 1759 CST, it was identified that the high pressure injection pump injection flows to the reactor coolant system are not balanced in accordance with the Fort Calhoun Station (FCS) Updated Safety Analysis Report (USAR) Section 14.15.5.2 method for small break loss of coolant analysis (LOCA). FCS is currently shutdown with fuel removed from the vessel.

A causal analysis is in progress. The results of the analysis will be published in a supplement to this LER.

This report is being submitted pursuant to 10 CFR 50.73(a)(2)(v): any event or condition that could have prevented the fulfillment of the safety function of structures or systems that are needed to (B) remove residual heat, and (D) mitigate the consequences of an accident, and 10 CFR 50.73(a)(2)(vii): "Any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system designed to: (B) Remove residual heat; (D) Mitigate the consequences of an accident."

CONCLUSION

A causal analysis is in progress. The results of the analysis will be published in a supplement to this LER.

CORRECTIVE ACTIONS

A causal analysis is in progress. The results of the analysis will be published in a supplement to this LER.

SAFETY SIGNIFICANCE

A causal analysis is in progress. The results of the analysis will be published in a supplement to this LER.

SAFETY SYSTEM FUNCTIONAL FAILURE

A causal analysis is in progress. The results of the analysis will be published in a supplement to this LER.

NRC FORM 366A (10-2010)

U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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