Omaha Public Power District
444 South 16th Street Mall
Omaha, NE 68102-2247

LIC-11-0073
August 5, 2011

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Reference: Docket No. 50-285

Subject: Licensee Event Report 2011-008, Revision 0, for the Fort Calhoun Station

Please find attached Licensee Event Report 2011-008, Revision 0, dated, August 5, 2011. This report is being submitted pursuant to 10CFR50.73(a)(2)(x). If you should have any questions, please contact me.

Sincerely,

Jeffrey Reinhart
Site Vice President

JAR/epm

Attachment

c: E. E. Collins, Jr., NRC Regional Administrator, Region IV
L. E. Wilkins, NRC Project Manager
J. C. Kirkland, NRC Senior Resident Inspector
INPO Records Center

NRC FORM 366 (10-2010) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Fort Calhoun Station	05000285	1 OF 3

4. TITLE

Fire in Safety Related 480 Volt Electrical Bus

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
6	7	2011	2011	- 008 -	0	8	5	2011	FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE

5

10. POWER LEVEL

0

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check all that apply)*

- ☐ 20.2201(b)
- ☐ 20.2201(d)
- ☐ 20.2203(a)(1)
- ☐ 20.2203(a)(2)(i)
- ☐ 20.2203(a)(2)(ii)
- ☐ 20.2203(a)(2)(iii)
- ☐ 20.2203(a)(2)(iv)
- ☐ 20.2203(a)(2)(v)
- ☐ 20.2203(a)(2)(vi)
- ☐ 20.2203(a)(3)(i)
- ☐ 20.2203(a)(3)(ii)
- ☐ 20.2203(a)(4)
- ☐ 50.36(c)(1)(i)(A)
- ☐ 50.36(c)(1)(ii)(A)
- ☐ 50.36(c)(2)
- ☐ 50.46(a)(3)(ii)
- ☐ 50.73(a)(2)(i)(A)
- ☐ 50.73(a)(2)(i)(B)
- ☐ 50.73(a)(2)(i)(C)
- ☐ 50.73(a)(2)(ii)(A)
- ☐ 50.73(a)(2)(ii)(B)
- ☐ 50.73(a)(2)(iii)
- ☐ 50.73(a)(2)(iv)(A)
- ☐ 50.73(a)(2)(v)(A)
- ☐ 50.73(a)(2)(v)(B)
- ☐ 50.73(a)(2)(v)(C)
- ☐ 50.73(a)(2)(v)(D)
- ☐ 50.73(a)(2)(vii)
- ☐ 50.73(a)(2)(viii)(A)
- ☐ 50.73(a)(2)(viii)(B)
- ☐ 50.73(a)(2)(ix)(A)
- ☒ 50.73(a)(2)(x)
- ☐ 73.71(a)(4)
- ☐ 73.71(a)(5)
- ☐ OTHER

Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER *(Include Area Code)*
Erick Matzke	402-533-6855

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
	EC	52	NLI	Y					

14. SUPPLEMENTAL REPORT EXPECTED

☒ YES *(If yes, complete 15. EXPECTED SUBMISSION DATE)* ☐ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR
10	30	2011

ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

On June 7, 2011, at approximately 0930 CDT, a failure of a safety related 480 volt AC (VAC) load center supply breaker in the switchgear room occurred (Bus 1B4A). Fire alarms were received in the control room and the Halon system that protects the switchgear rooms discharged. The fire brigade responded and found the room filled with smoke, but no active fire. At 0940 CDT an Alert was declared for a fire affecting the operability of plant safety systems required to establish or maintain safe shutdown. At 1313 CDT on June 7, 2011, Fort Calhoun Station (FCS) exited the Alert after confirming that the fire was extinguished and the area was ventilated to restore access. Shutdown cooling remained in-service during the event.

FCS was also in an emergency classification of a Notification of Unusual Event (NOUE) due to high Missouri river level.

A root cause analysis is in progress. The results of the analysis and corrective actions will be reported in a revision to this LER.

LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REV NO.	
Fort Calhoun Station	05000285	2011	- 008	- 0	2 OF 3

NARRATIVE

BACKGROUND

Fort Calhoun Station (FCS) is a two-loop reactor coolant system of Combustion Engineering (CE) design. The plant has three 480V "island" buses. The island buses are fed from one side or the other of the main 480V buses.

EVENT DESCRIPTION

On June 7, 2011, at approximately 0930 CDT, a failure of a safety related 480 volt (V) AC load center supply breaker occurred (Bus 1B4A). FCS was operating in Mode 5 at about 82 degrees Fahrenheit with the plant fully depressurized. FCS was in an emergency classification of a Notification of Unusual Event (NOUE) due to high Missouri river level (NOUE declared June 6, 2011).

At 0930, the control room received multiple alarms that were indicative of a fire in the west electrical switchgear room. The Halon system that protects the switchgear rooms discharged. The fire brigade responded and found a room filled with smoke, but no active fire. The fire brigade confirmed that there was no active fire in the switchgear room. At about 0930 CDT, while de-energizing electrical buses to aid in damage mitigation and assessment, both trains of spent fuel pool cooling were de-energized. Spent fuel pool temperature increased 3 degrees Fahrenheit. At 0940 CDT, an Alert was declared for a fire affecting the operability of plant safety systems required to establish or maintain safe shutdown. During the course of the event a local offsite fire department responded to assist. At 1147 CDT, on June 7, 2011, power was restored to one train of spent fuel cooling.

At 1313 CDT on June 7, 2011, FCS exited the Alert. Plant shutdown cooling remained in-service during the event.

The circuit breaker that was damaged was the supply breaker to Bus 1B4A.

At 1021 CDT, the NRC Headquarters Operations Office (HOO) was notified per 10 CFR 50.72(a)(1)(i). This report is being made per 10 CFR 50.73(a)(2)(x).

CONCLUSION

A root cause analysis is in progress. The results of the analysis will be reported in a revision to this LER.

CORRECTIVE ACTIONS

The affected bus was de-energized and the Halon system extinguished the fire. The Halon system was recharged and restored to service.

The station will be replacing the affected bus (1B4A), which contains two 480V supply circuit breakers, 1B4A and BT-1B4A (supply breaker to the associated "island" bus).

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NARRATIVE

The station conducted inspections and testing of the 480V buses, the supply breakers to the 480V buses, and the 480V bus tie breakers. The circuit breakers in bus 1B4A were not tested as they are to be replaced. The tested 480V supply breakers and bus tie breakers passed their inspections and testing.

The station has completed inspections and testing of the 480V buses, as part of an extent of condition review. An indication of potential overheating in a bus connection was repaired. Additional corrective actions for the root cause and extent of condition will be identified by the stations corrective action system. These actions will be documented in a supplement to this LER.

SAFETY SIGNIFICANCE

The safety significance of the event will be provided in a revision to this LER.

PREVIOUS EVENTS

None