

# Evolution of Reactor Antineutrino Flux and Energy Spectrum at Daya Bay

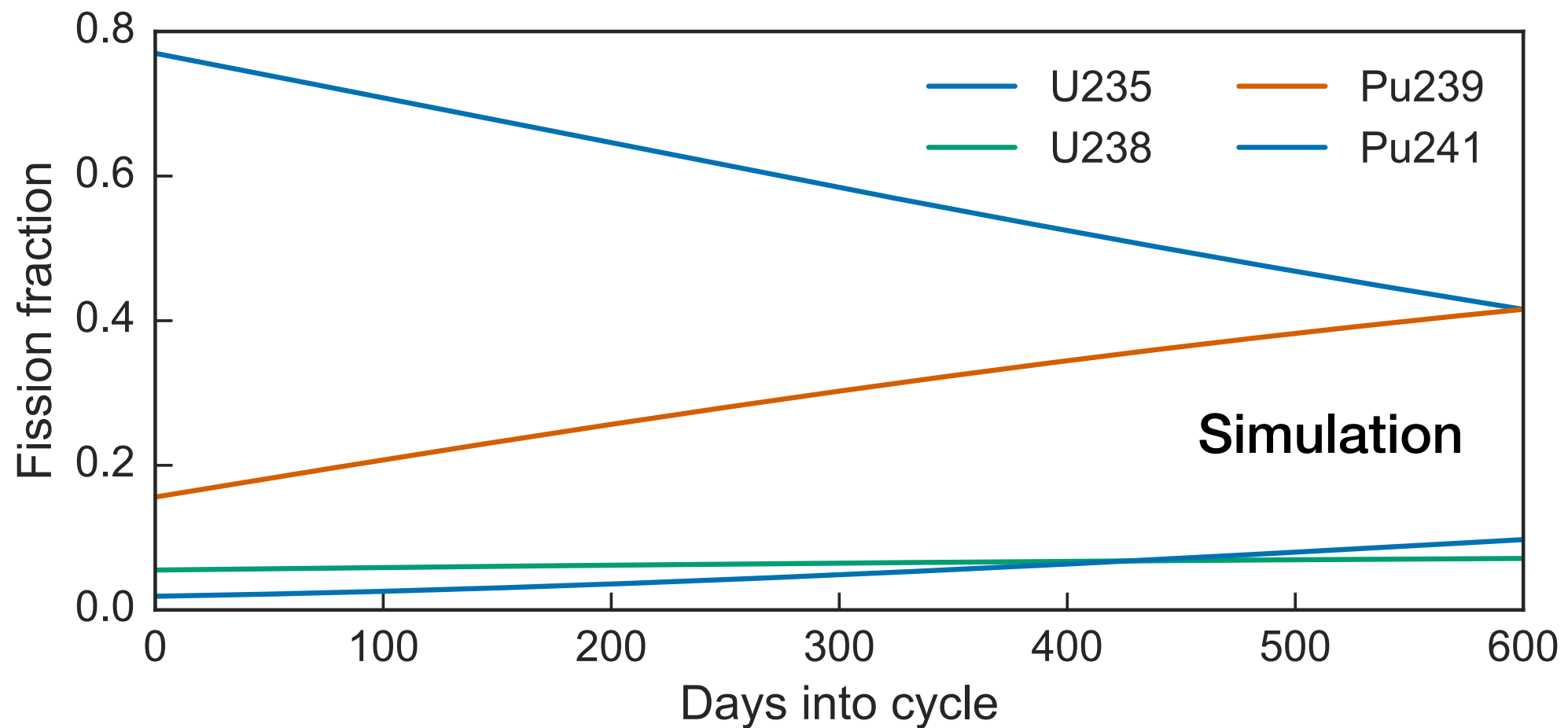
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Patrick Tsang (LBNL)

March 18, 2016 @ Daya Bay Collaboration Meeting

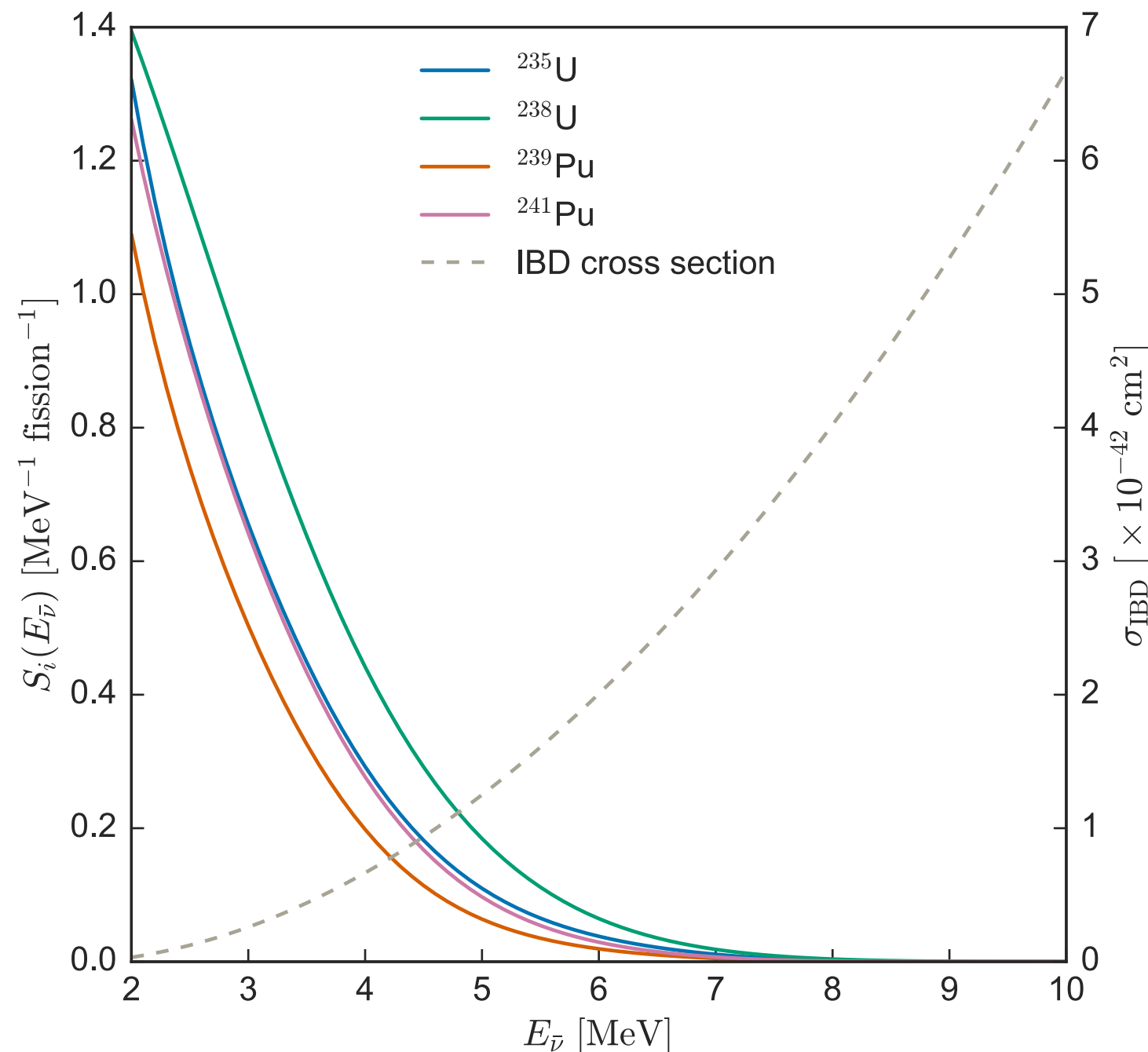
# Introduction

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- relative compositions of fission isotopes evolve with time
- U235 & Pu239 generate >85% reactor antineutrinos

# Introduction (cont.)



- energy spectra of U235 & Pu239 are significantly different
- expect less anti-nu and softer energy spectrum as the fuel is used up

**Can we observe and quantify the change at DYB?**

# Notations

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## Subscript

$c$  - reactor core {D1, D2, L1, ..., L4}

$i$  - fission isotope {U235, U238, Pu239, Pu241}

## Symbol

$f_i$  - fission fraction of isotope  $i$

$S_i$  - antineutrino energy spectrum due to isotope  $i$  [/MeV fission]

$\sigma_i$  - integral of  $S_i$  over energy weighted by IBD cross section [cm<sup>2</sup>/fission]

$\sigma_f$  - “effective cross section”  $\sum f_i \sigma_i$

$E_\nu$  - antineutrino energy [MeV]

$E_{vis}$  - visible (detected) energy [MeV]

# Evolution of Reactor Antineutrino Flux

# Effective fission fraction

Observed rate =  
sum of contribution  
from each core

$\sigma_f$  (to be measured)  
look like to a single  
core with effective  
fission fraction

Unless specified,  
effective fission fraction  
is interchangeable with  
“fission fraction” for the  
rest of the slides.

$$\begin{aligned}
 n_{\bar{\nu}} &= \epsilon N_p \sum_c \frac{\bar{p}_c}{4\pi L_c^2} \left( \frac{W_{th,c}}{E_{f,c}} \sum_i f_{i,c} \sigma_i \right) \\
 &= \sum_i \beta_i \sigma_i \\
 &= \bar{\beta} \sum_i \tilde{f}_i \sigma_i,
 \end{aligned}$$

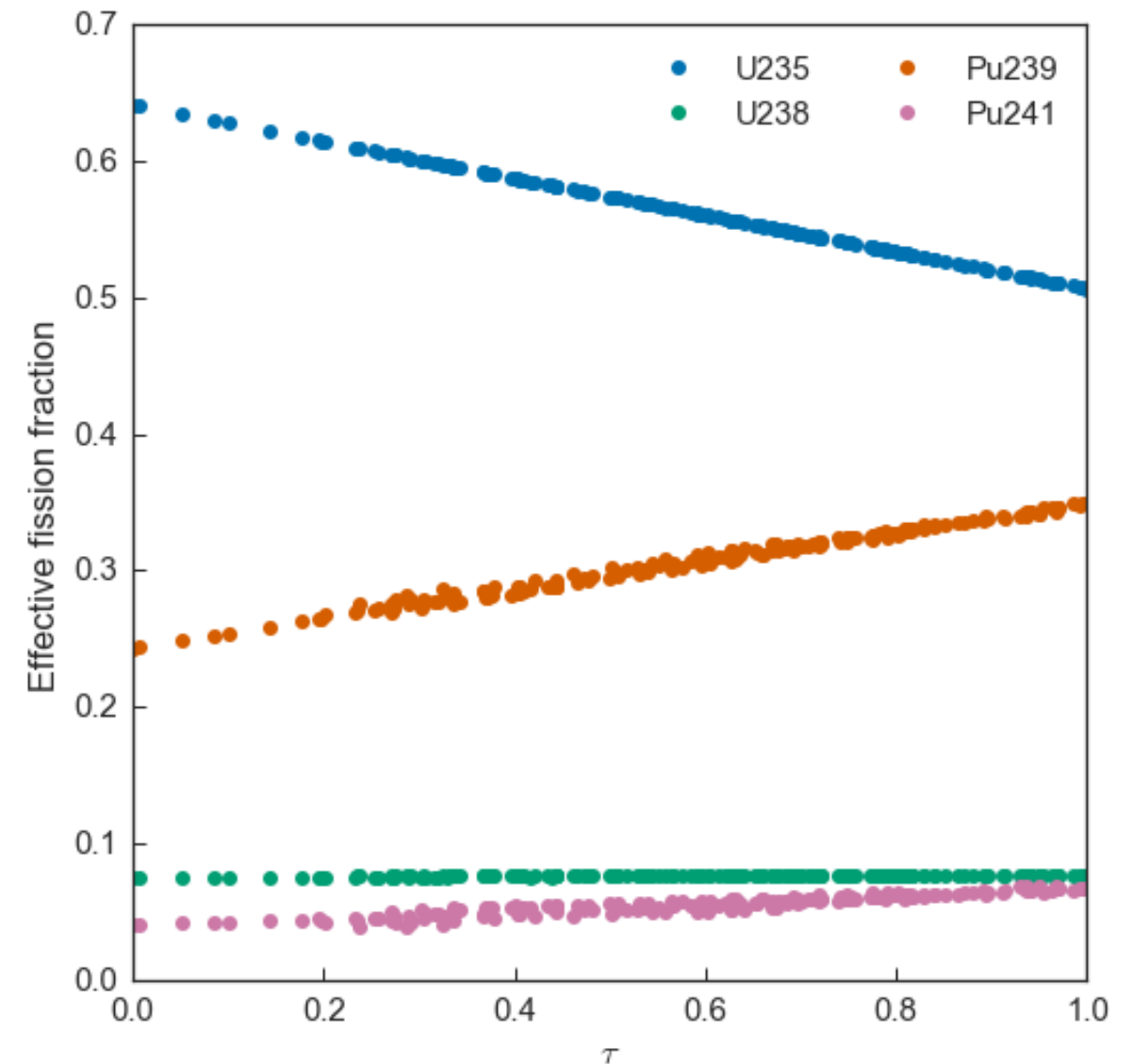
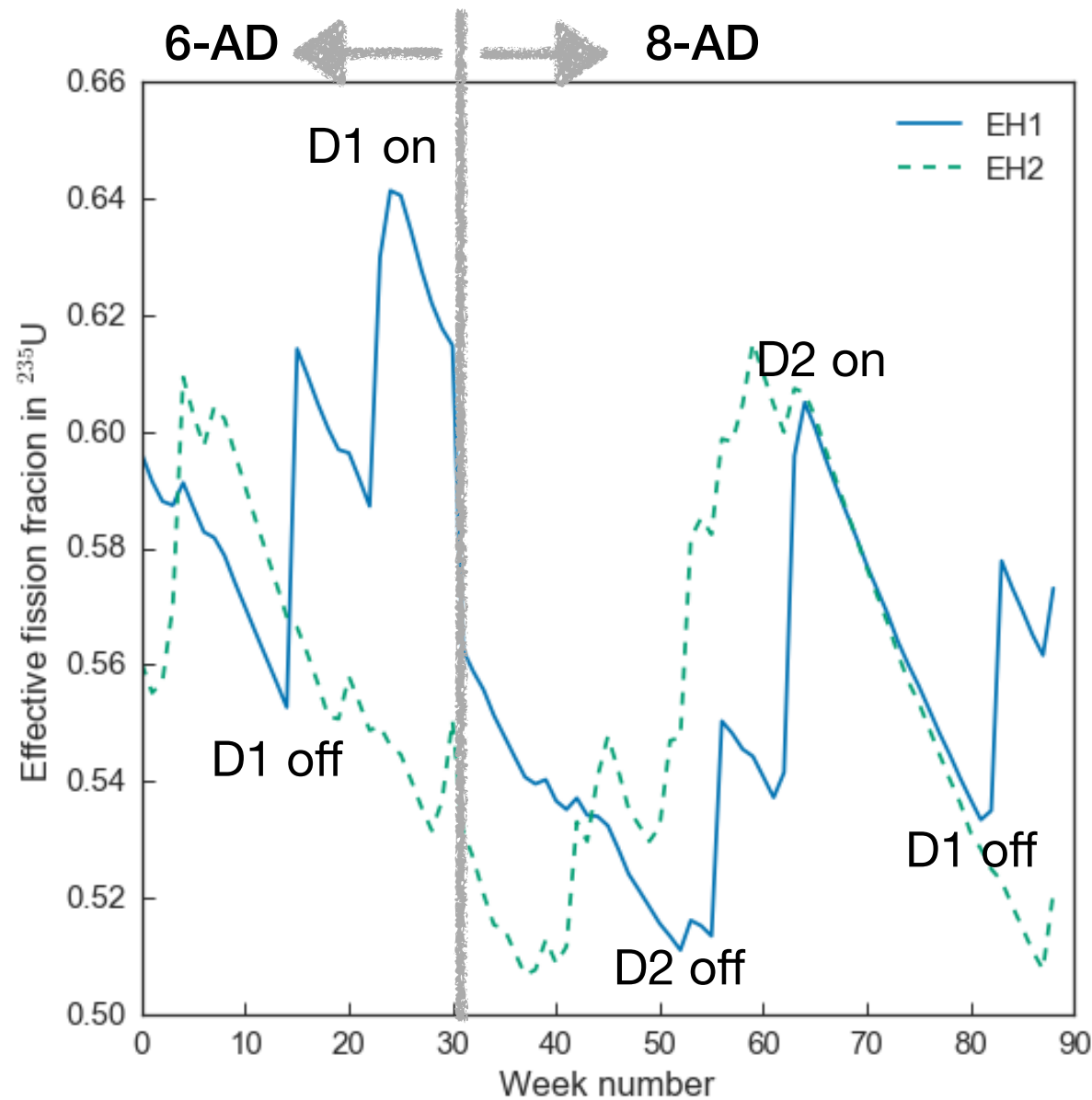
efficiency & # of targets  $\rightarrow \epsilon N_p$   
 mean survival probability  $\rightarrow \bar{p}_c$   
 thermal power  $\rightarrow W_{th,c}$   
 distance  $\rightarrow 4\pi L_c^2$   
 energy released per fission  $\rightarrow E_{f,c}$   
 sum over isotopes  $\rightarrow \sum_i f_{i,c} \sigma_i$

$$\beta_i = \epsilon N_p \sum_c \frac{\bar{p}_c}{4\pi L_c^2} \frac{W_{th,c}}{E_{f,c}} f_{i,c},$$

$$\bar{\beta} = \sum_i \beta_i,$$

$$\tilde{f}_i = \beta_i / \bar{\beta}.$$

# Effective fission fraction (Dec 2011 - Nov 2013)

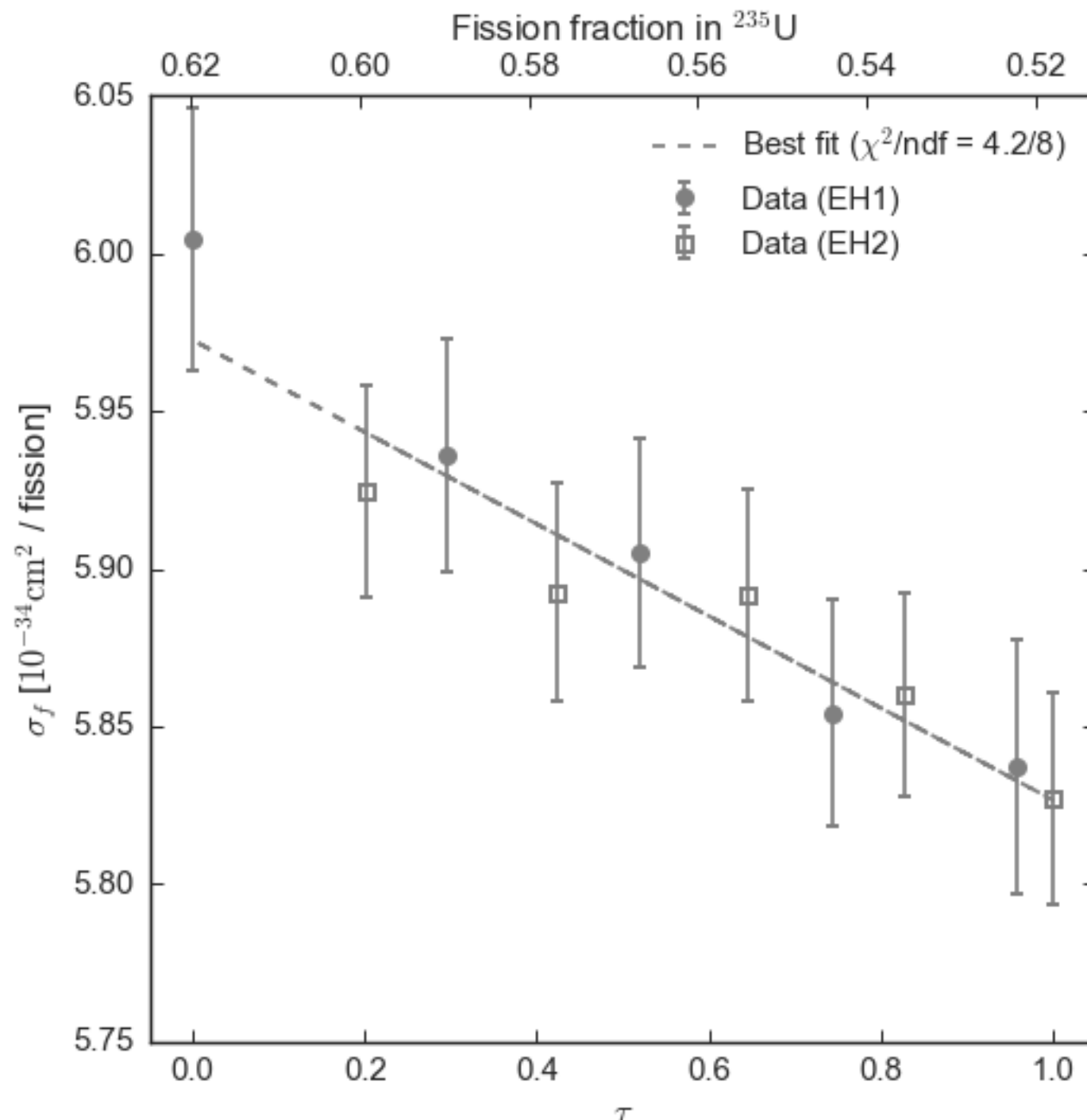


pseudo time (look like a single fuel cycle)

$$\tau = [f_{\text{U235}} - \max(f_{\text{U235}})] / [\min(f_{\text{U235}}) - \max(f_{\text{U235}})]$$

$$\Delta f_{\text{U235}} = -0.112 \Delta \tau$$

# Evolution of Antineutrino Flux (P14A dataset)



$$\sigma_f(\tau) \approx \bar{\sigma}_f + \frac{d\sigma}{d\tau} \times \left(\tau - \frac{1}{2}\right)$$

Average flux

$$\bar{\sigma}_f = \int_0^1 \sigma_f(\tau) d\tau$$

$$(5.90 \pm 0.02) \times 10^{-34} \text{ cm}^2 / \text{fission}$$

Total change

$$\Delta\sigma_f = \sigma_f(1) - \sigma_f(0) = \frac{d\sigma_f}{d\tau}$$

$$(-0.15 \pm 0.02) \times 10^{-34} \text{ cm}^2 / \text{fission}$$

**Uncertainty on absolute normalization  
(~2%, dominated by detection  
efficiency) is not included.**



# Why $\sigma_f$ is linear in $\tau$ or $f_{U235}$ ?

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$$\sigma_f = \sum f_i \sigma_i$$

## **We have**

4 unknowns  $\sigma_i$  and  $n > 4$  measurements.

Let  $F$  be a  $n \times 4$  matrix for the system of linear equations.

The singular values of  $F$  are  $\{s_1, s_2, s_3, s_4\}$  in descending order.

## **However,**

$$(s_1 + s_2 + s_3) / \sum s_i \approx 100\%$$

$$(s_1 + s_2) / \sum s_i = 99.6\%, \text{ using Daya Bay fission fraction history.}$$

## **Therefore,**

we measure only measure three components (at most) with DYB data.

Indeed, *two linear combinations* of  $\sigma_i$  (let's call them  $v_1$  and  $v_2$ ) are “good” approximation for the observations.

Same conclusion for spectral measurement.

# Why $\sigma_f$ is linear in $\tau$ ? (cont.)

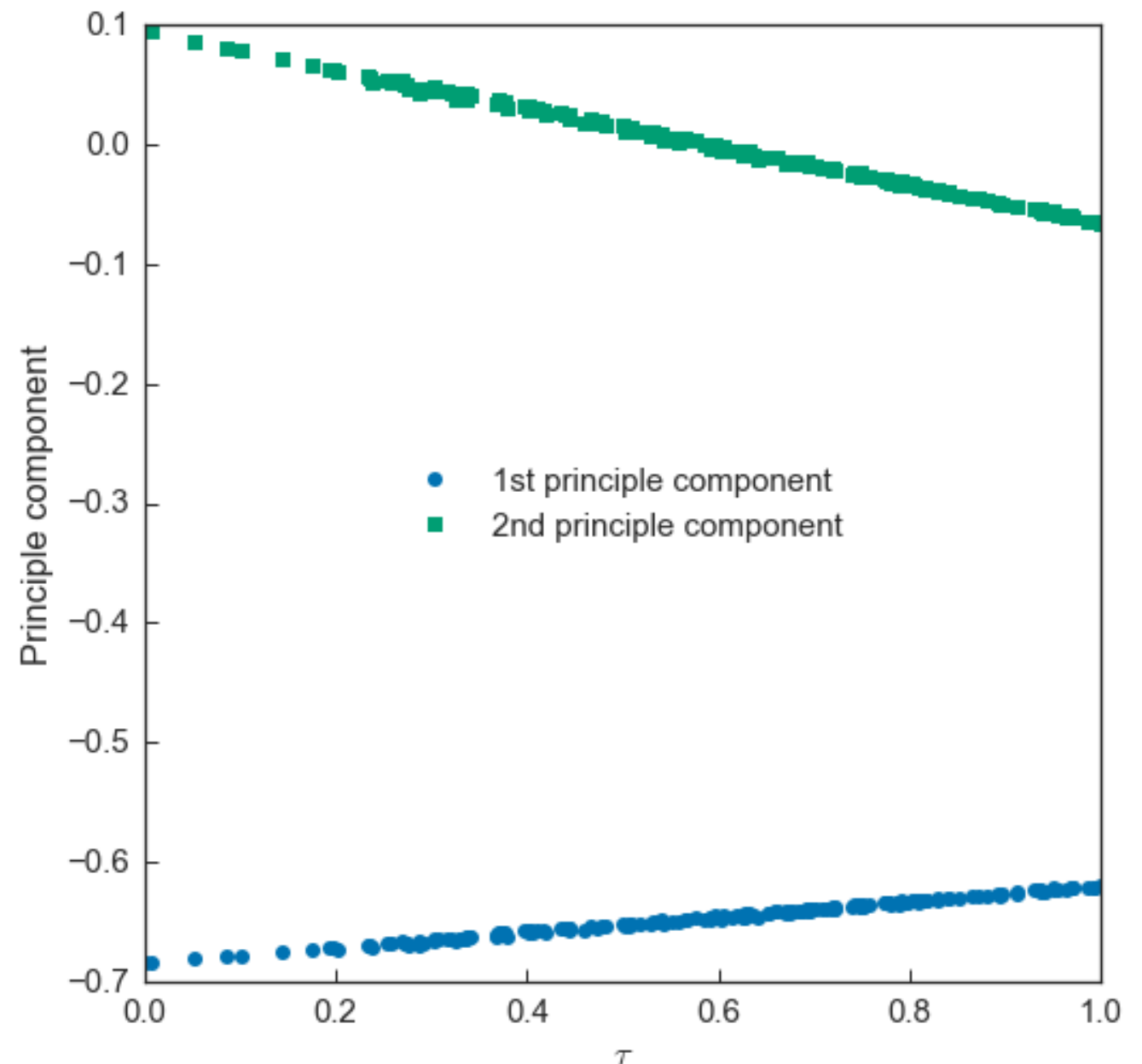
$$\sigma_f(\tau) \approx z_1(\tau) v_1 + z_2(\tau) v_2$$

Using *principle component analysis* (PCA), the linear system is decomposed into linear combinations  $v_1$  and  $v_2$  with coefficients  $z_1$  and  $z_2$ .

Applying PCA on the Daya Bay's fission fraction history shows that

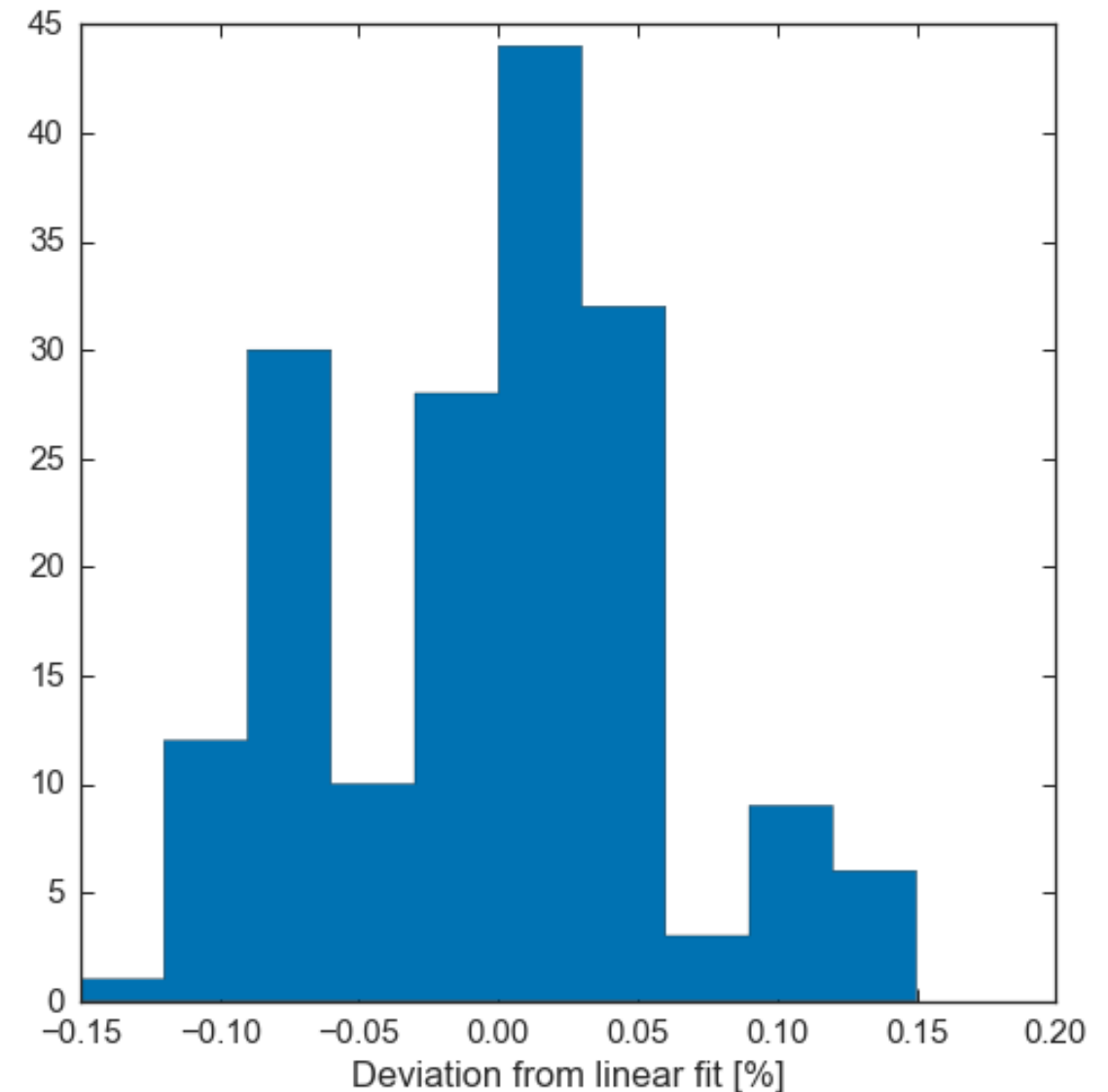
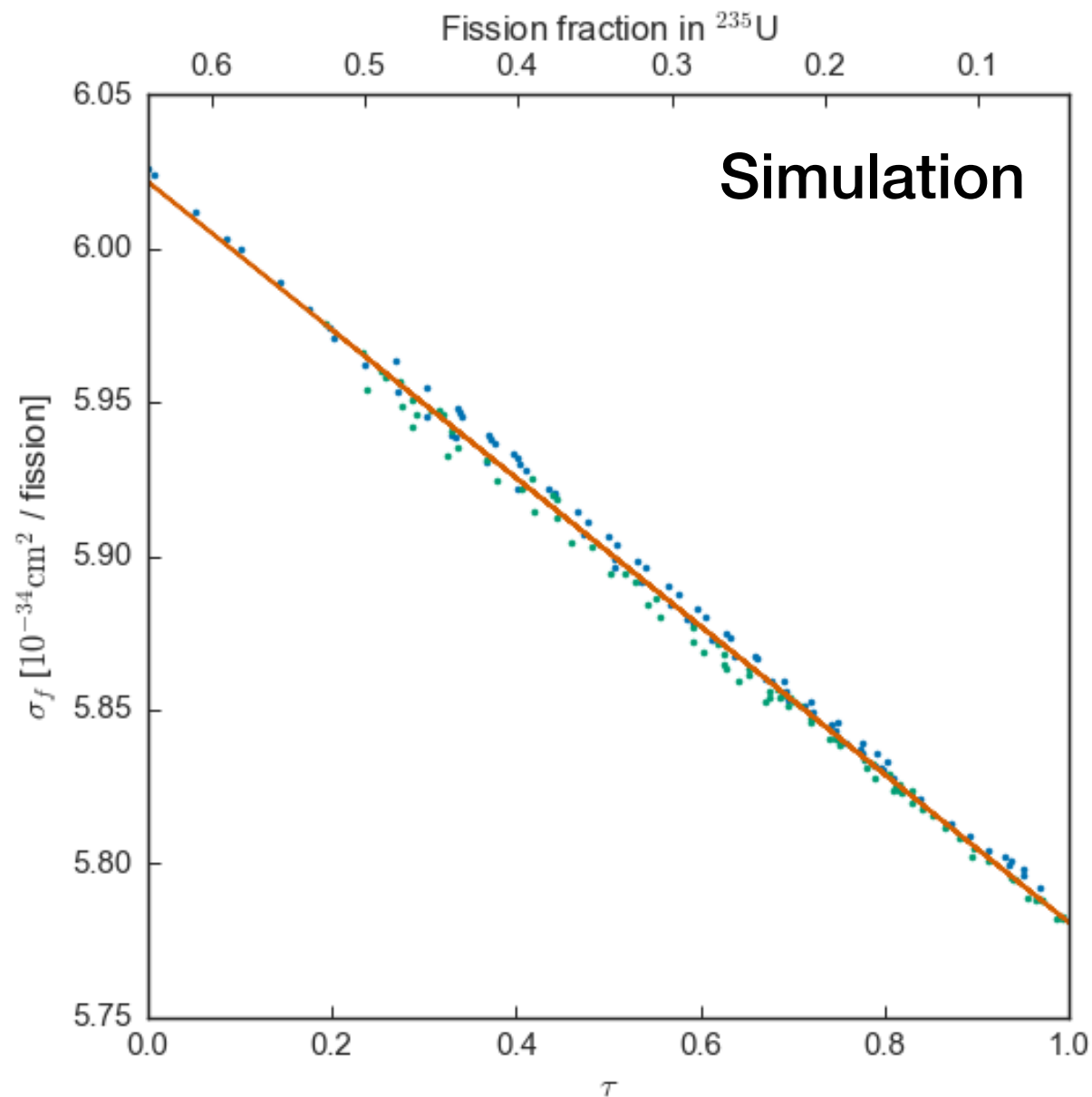
$z_1 \sim$  linear in  $\tau$ , and  
 $z_2 \sim$  linear in  $\tau$ .

Hence,  $\sigma_f \sim$  **linear in  $\tau$** . (Q.E.D.)



**Model independent - no assumption on  $\sigma_i$ .**

# How accurate is the linear approximation?

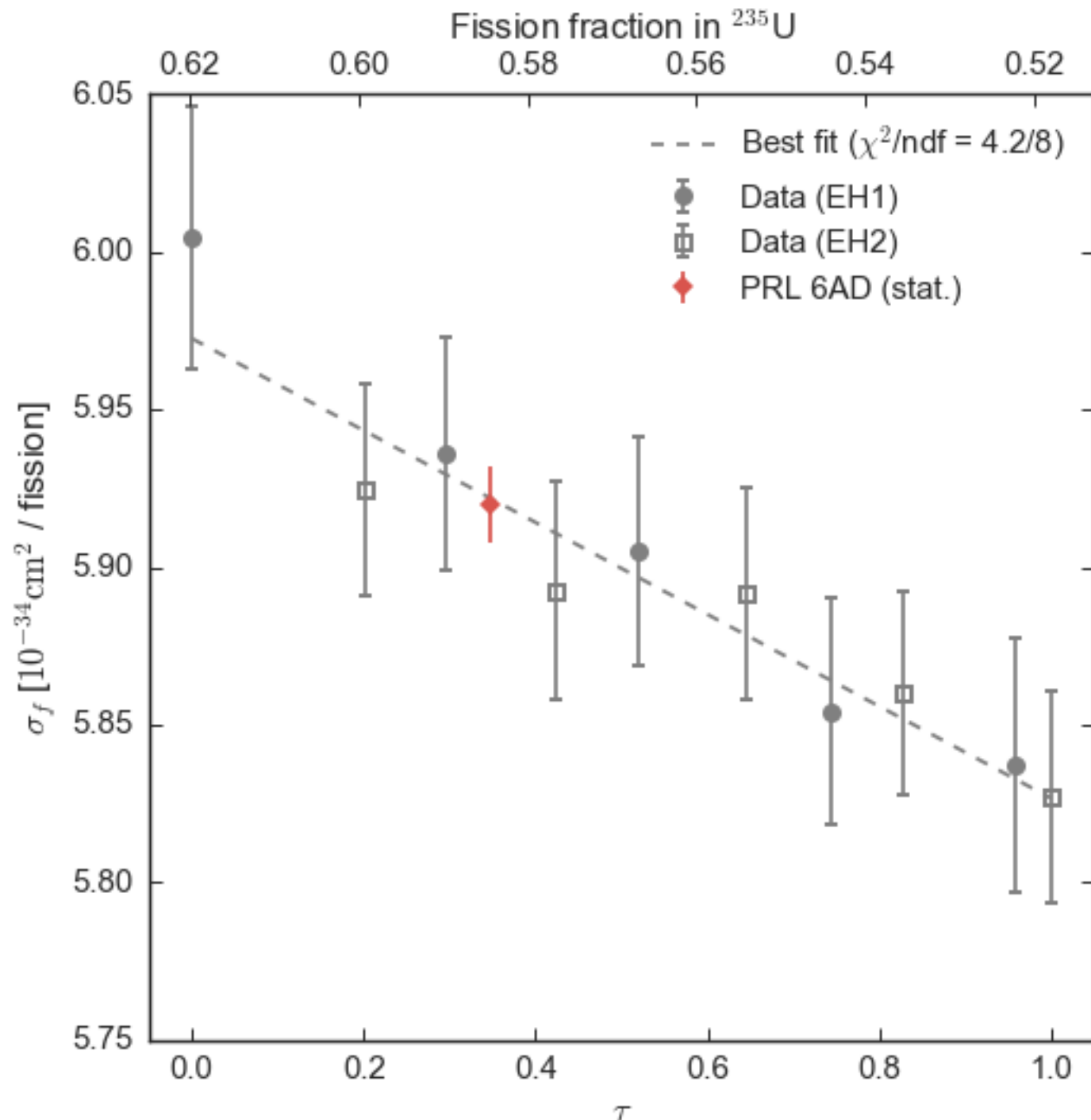


Overall bias is  $\sim 0.1\%$ .

Negligible for slope determination.

The deviation is further reduced if data is binned in  $\tau$ .

# Comparison with previous result (PRL 6AD)



At  $f_{\text{U}235} = 0.586$

## PRL 6AD

$$\sigma_f = 5.92 \times 10^{-34} \text{ cm}^2 / \text{fission}$$

## This analysis

$$\sigma_f = 5.92 \times 10^{-34} \text{ cm}^2 / \text{fission}$$

Uncertainty is dominated by normalization anyway. Statistical uncertainty is negligible.

# Uncertainties

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## Time-correlated (absolute scale)

- 2.1% absolute detection efficiency
- ~0.5% reactor-related
- ~0.2% neutrino oscillation

**Irrelevant for relative  
measurement  $\Delta\sigma_f / \sigma_f$**

Total systematic uncertainty on normalization ~2.2%

## Backgrounds

Accidentals are subtracted week-by-week.

Nominal background rates (other than accidentals) are constant in time (i.e. time variations are ignored).

Background systematic uncertainty is sub-dominant ~0.2%

## Energy model

Negligible - given that the detection efficiency is stable in time.

## Time-uncorrected

Dominated by statistical uncertainty

# Evolution of Antineutrino Energy Spectrum

# IBD Energy Spectrum

$$\frac{dN(E_\nu)}{dE_\nu} = \tilde{p}(E_\nu) \bar{\beta} \sum_i \tilde{f}_i \sigma(E_\nu) S_i(E_\nu)$$

Expressed as a sum over isotope  $i$ .  
 $\beta$  and  $f_i$  are defined in the flux analysis.

Correction due to oscillation weighted by core contributions.

i.e. dependence in thermal power, ... etc

$$\frac{dN(E_{\bar{\nu}})}{dE_{\bar{\nu}}} = \beta \sum_i \tilde{f}_i c_i(E_{\bar{\nu}}) \sigma(E_{\bar{\nu}}) S_i(E_{\bar{\nu}}), \quad (20)$$

184 where

$$c_i(E_{\bar{\nu}}) = \left( \sum_c \frac{p_c(E_{\bar{\nu}})}{4\pi L_c^2} \frac{W_{th,c}}{E_{f,c}} f_{i,c} \right) / \beta \tilde{f}_i \quad (21)$$

185 is an energy-dependent correction factor for isotope  $i$  absorbing all reactor-  
 186 dependent terms and survival probability due to oscillation. Equation 19 can  
 187 be simplified to

$$\frac{dN(E_{\bar{\nu}})}{dE_{\bar{\nu}}} \approx \beta \tilde{p}(E_{\bar{\nu}}) \sum_i \tilde{f}_i \sigma(E_{\bar{\nu}}) S_i(E_{\bar{\nu}}). \quad (22)$$

188 by averaging correction factors over all isotopes

$$\tilde{p}(E_{\bar{\nu}}) = \sum_i \tilde{f}_i c_i(E_{\bar{\nu}}). \quad (23)$$

189 Using numerical simulation, the approximation in Eqn. 21 is valid up to  $O(0.01\%)$ .  
 190 The antineutrino spectrum per fission weighted by IBD cross section is obtained  
 191 from observed spectrum with correction of oscillation effect

$$\begin{aligned} S(E_{\bar{\nu}}) &\equiv \frac{1}{\beta \tilde{p}(E_{\bar{\nu}})} \frac{dN_{\bar{\nu}}(E_{\bar{\nu}})}{dE_{\bar{\nu}}} \\ &\approx \sum_i \tilde{f}_i \sigma(E_{\bar{\nu}}) S_i(E_{\bar{\nu}}). \end{aligned} \quad (24)$$

# Correction of the Oscillation Effect

The correction is small:

~2% at low energy

<1% at high energy.

*Time variation* is even smaller  
~0.1%.

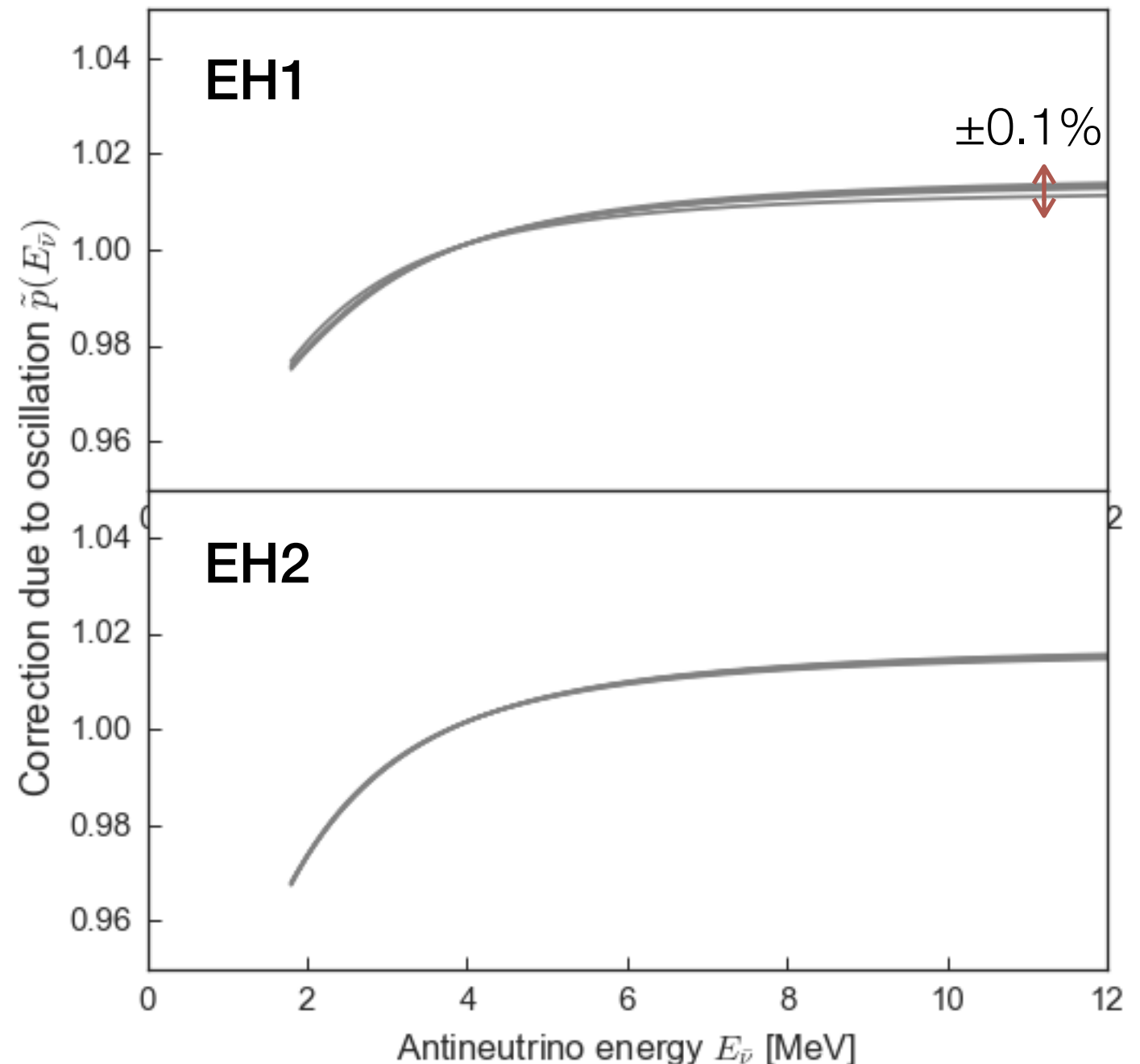
## Option 1

Use uncorrected energy spectrum (or apply a common correction).

Apply 0.1% uncertainty.

## Option 2

Apply individual correction for each time period.



**Either one has negligible impact for this study.**



# Analogy to Flux Analysis

IBD spectrum *with oscillation* in *visible* (observed) energy  
 R = Detector response (convert  $E_\nu$  to  $E_{vis}$ )

**Spectral  
analysis**

$$S^{osc}(E_{vis}) = \sum_i \tilde{f}_i R[\tilde{p}(E_\nu) \sigma(E_\nu) S_i(E_\nu)]$$

**Observations**



**4 unknowns**

**Flux  
analysis**

$$\sigma_f = \sum_i \tilde{f}_i \sigma_i$$

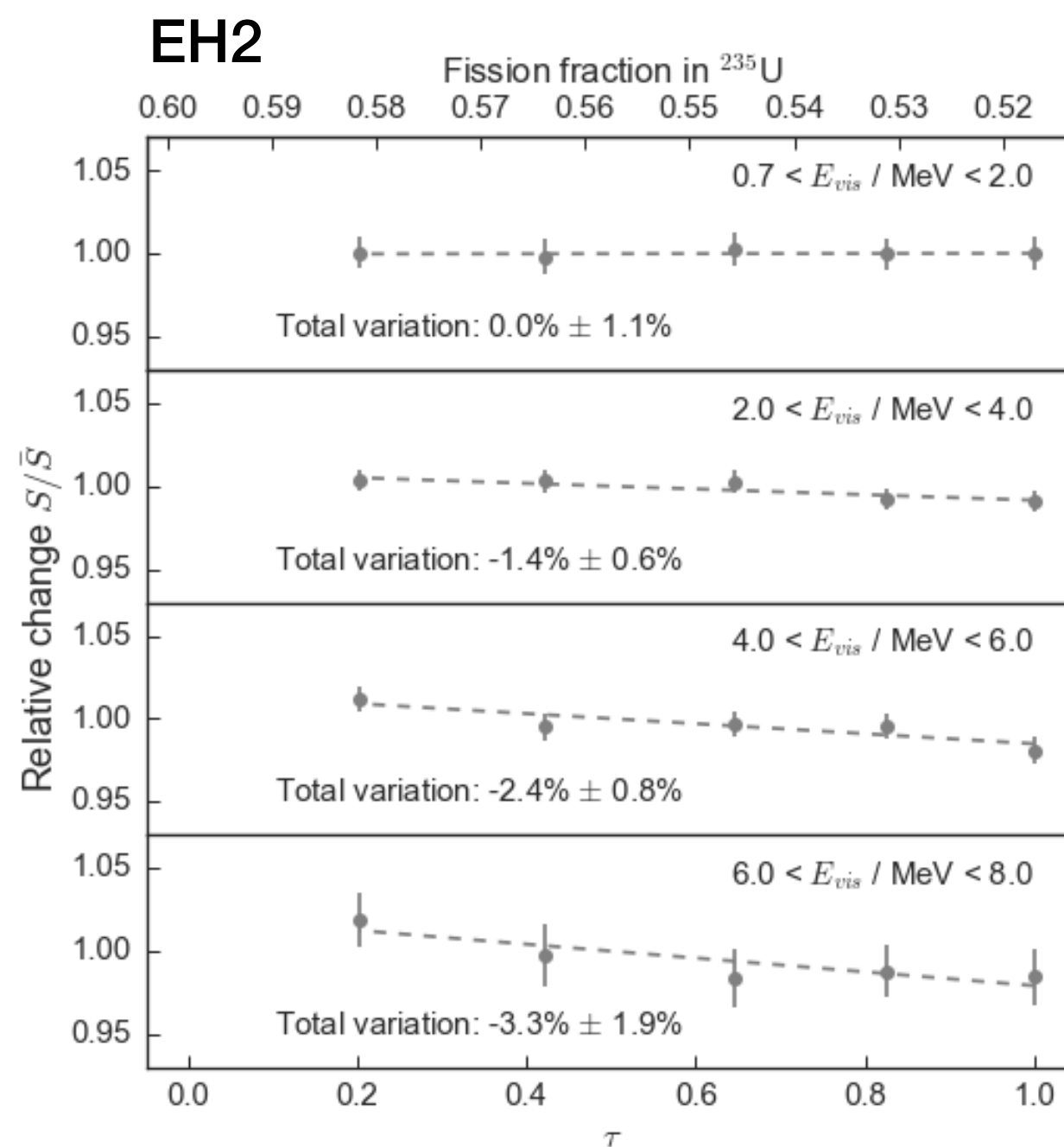
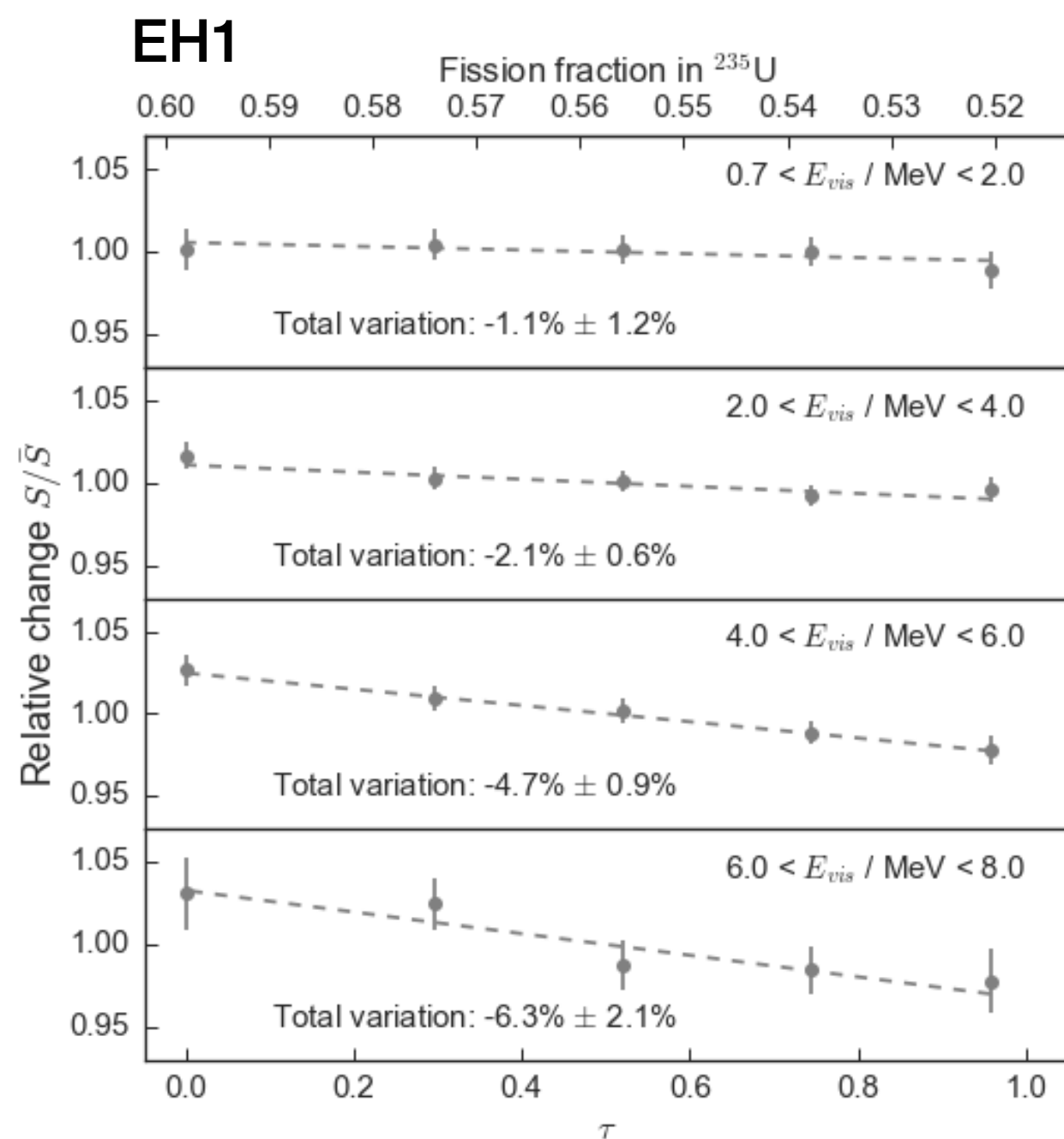
Measured quantity is a linear sum of effective fission fraction

$\sigma_f \sim \text{linear in } \tau$



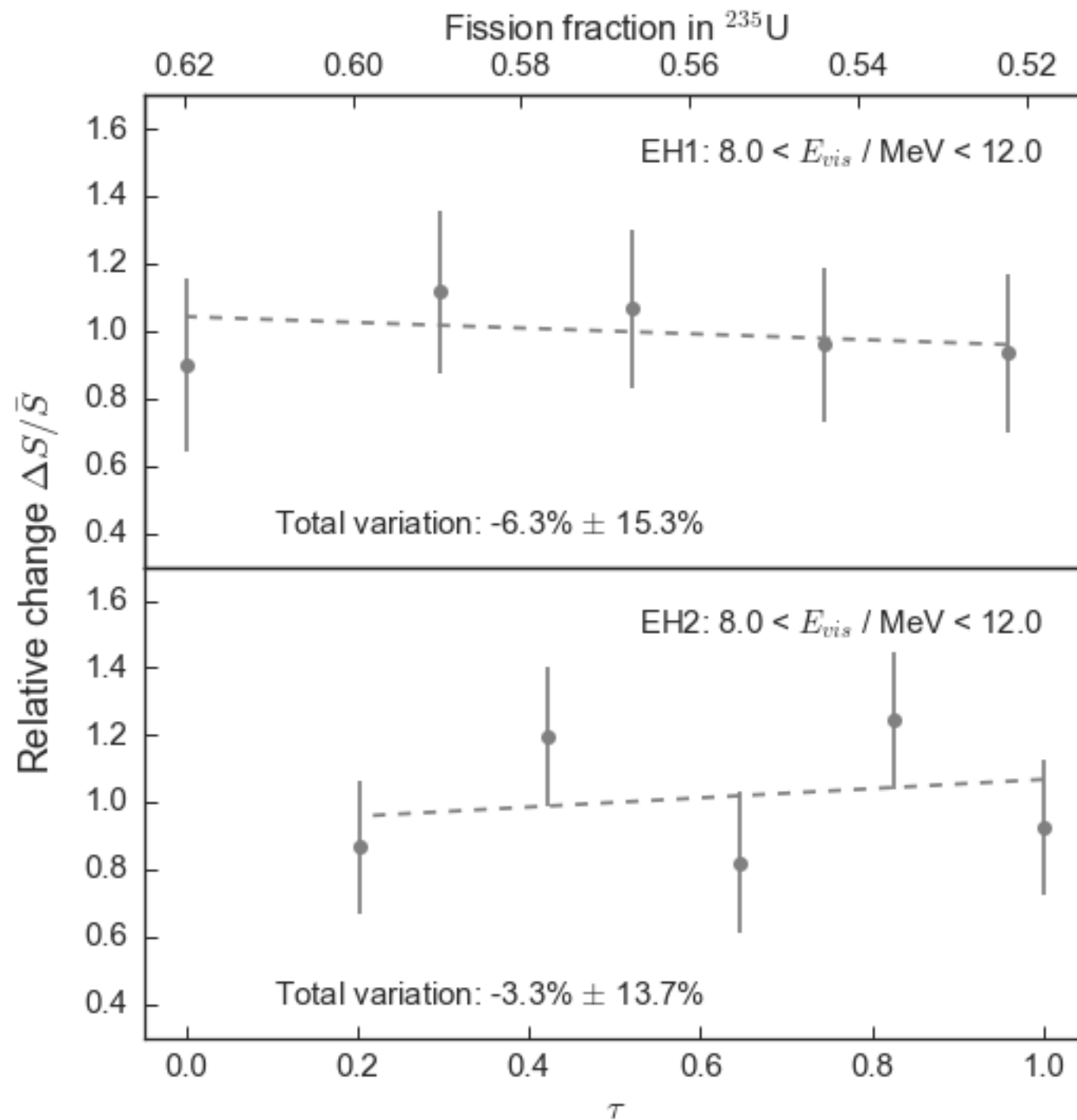
$S^{osc} \sim \text{linear in } \tau$

# Results for Oscillated Spectrum in $E_{vis}$



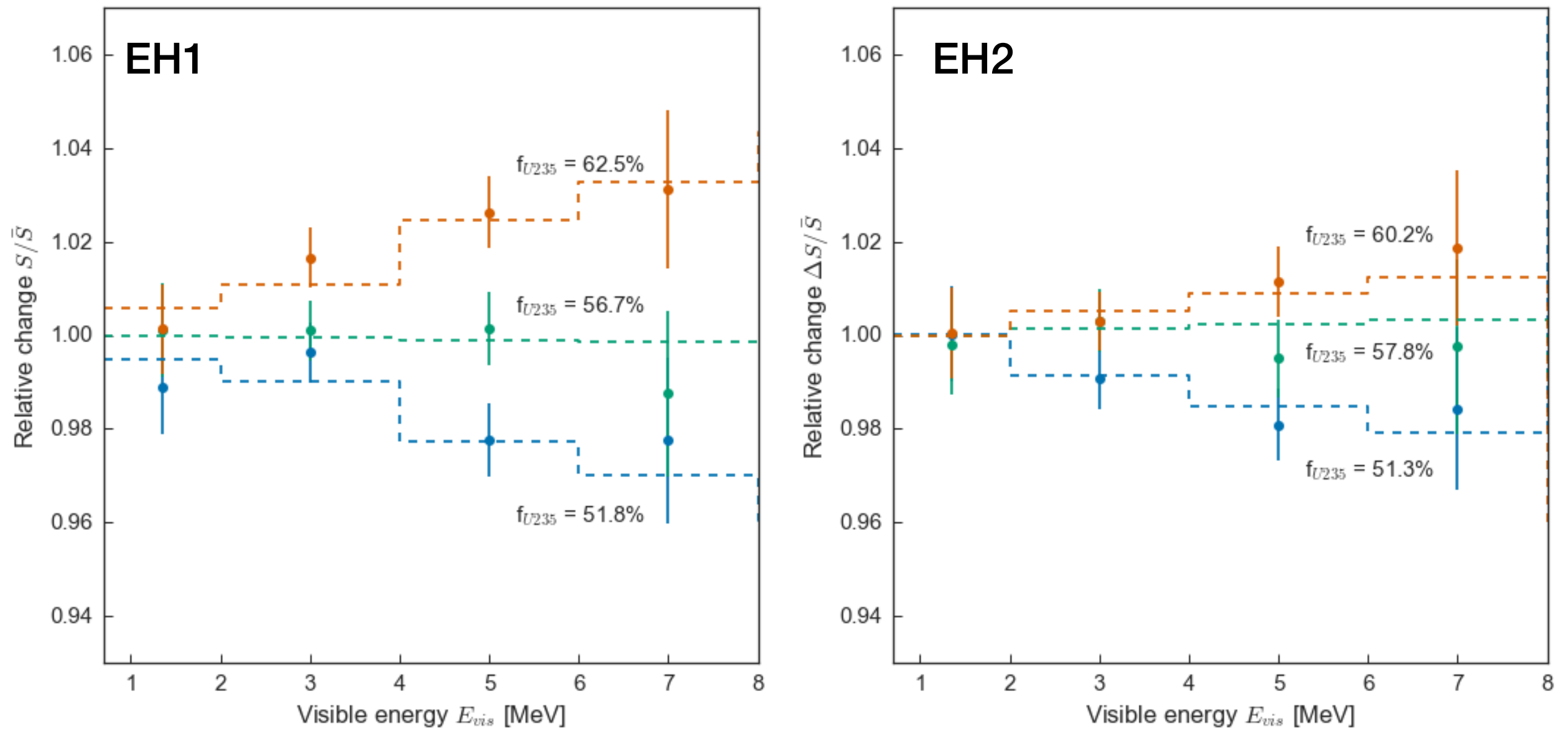
Simultaneous fits of all energy bins  
(individual fit for EH1 and EH2)

# High Energy Reactor Antineutrino



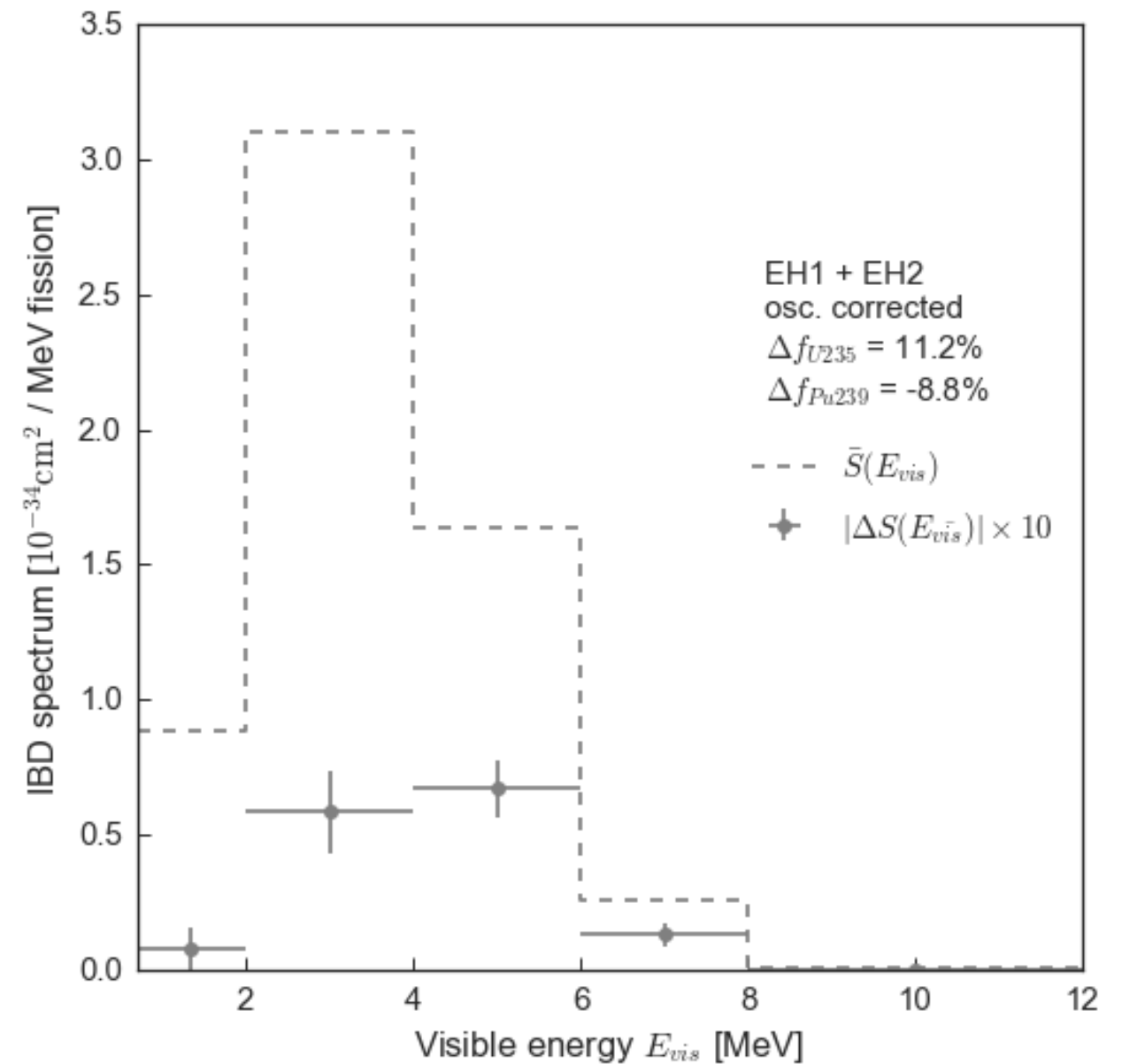
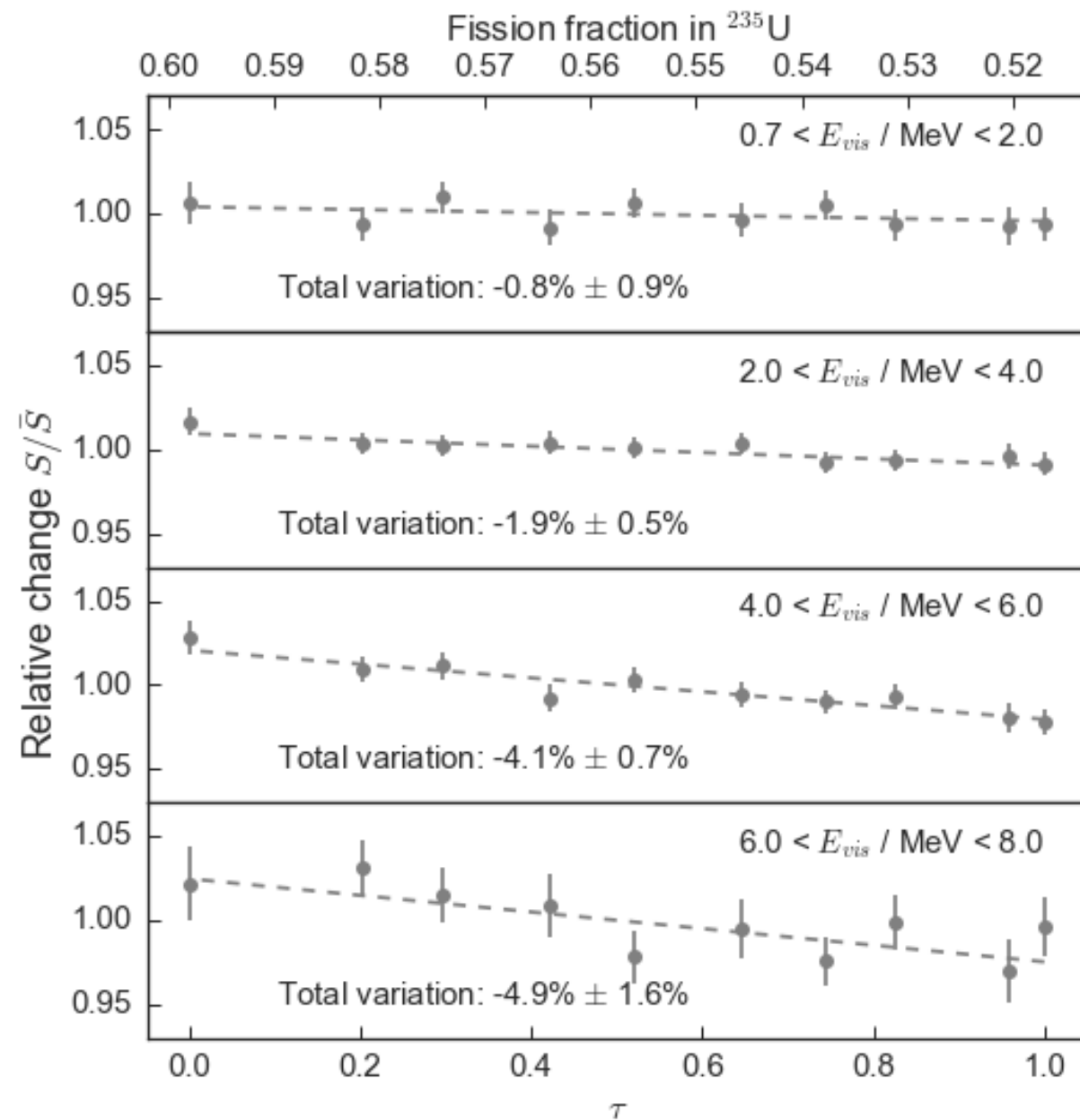
**Inconclusive.  
Limited by statistics.**

# Evolution of Visible Energy Spectrum (w/osc)



Data (points) with best fit (dotted) for evolution of oscillated energy spectrum in visible energy.

# Combined Fit



Evolution of EH1 & EH2 combined energy spectrum  
(corrected for oscillation) in visible energy.

# Uncertainties

## In addition to Flux Analysis

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### Energy model

- almost irrelevant for measurement in visible
  - absolute energy scale, non-linearity, resolution, ...
- shift in energy scale has negligible impact
  - $\pm 0.2\%$  variation in energy scale gives  $< 0.1\%$  change in spectrum (random in time)

### Oscillation

- correction due to oscillation is energy dependent
  - $\sim 0.3\%$  at  $E_\nu = 2 \text{ MeV}$
  - $\sim 0.05\%$  at  $E_\nu = 12 \text{ MeV}$
- subdominant effect

**Uncertainty on relative measurement  $\Delta S/S$  is  $O(10\%)$ ,  
dominated by statistics.**

# Documentations

## A model-independent method of measuring the evolution of antineutrino flux and energy spectrum from nuclear reactors

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### Abstract

A model-independent method is developed to characterize the evolution of antineutrino flux and energy spectrum at different stages of fuel composition for a standard commercial pressurized reactor. The method is also applicable to reactor antineutrino experiments with multiple reactors. Examples and prospects of such measurements for Daya Bay, Double Chooz, and RENO experiments are discussed.

**Keywords:** Daya Bay Double Chooz RENO reactor neutrino

### 1. Introduction

Antineutrinos from nuclear reactor have been essential in fundamental physics, including the initial detection of antineutrinos and the most distinct evidence for neutrino flavor oscillation. Detection of antineutrinos could also be applied to reactor monitoring and non-proliferation safeguards. Past experiments have observed a clear correlation between the antineutrino rate and the nuclear reactor operating condition. The expected evolution relies on the prediction of antineutrino emission from uranium and plutonium isotopes. However, not only are absolute measurements of antineutrino flux from reactor 6% lower than the

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### 1 Introduction

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A general description of the method.  
Examples with simulated data.

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Application of the method to DYB data.

# Conclusions

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- a novel model independent method to characterize the evolution of reactor antineutrino flux and energy spectrum
- flux measurement  $\sigma_f = (1 + k) \sigma_f^{\text{avg}}$ 
  - the evolution constant  $k$  is proportional to  $\Delta f_{\text{U235}}$
  - $k = \underline{(2.47 \pm 0.36)\%}$  for  $\Delta f_{\text{U235}} = 11.2\%$
- spectral measurement  $S = (1 + k) S^{\text{avg}}$ 
  - the evolution constant  $k$  is proportional to  $\Delta f_{\text{U235}}$
  - unambiguous variation for  $2 < E_{\text{vis}} < 8 \text{ MeV}$
- relative measurement of the evolution is dominant by statistical uncertainty

Visible Energy [MeV]	Variation [%]
$2 < E_{\text{vis}} < 4$	$-1.9 \pm 0.5$
$4 < E_{\text{vis}} < 6$	$-4.1 \pm 0.7$
$6 < E_{\text{vis}} < 8$	$-4.9 \pm 1.6$