HFNG Flux Calculation

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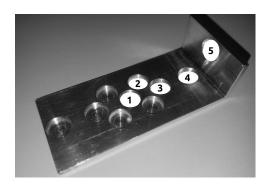
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Overview

- Experiment summary
- DD fusion spectrum
- Solution method
- Implementation
- Results



Experiment Summary



- Goal to measure ³⁵Cl(n,p)³⁵S cross section
- 5x 11 mm OD NaCl pellets
- Monitor fluence using ⁵⁸Ni(n,p)⁵⁸Co
- Need to determine energy spectrum for each sample

Experiment Summary

Sample	Δx [mm]	Δy [mm]	Δz [mm]	Δ <i>θ</i> [°]
1	0.0	0.0	8.0	0
2	9.0	8.0	8.0	0
3	18.0	0.0	8.0	0
4	36.0	0.0	8.0	0
5	46.0	0.0	-7.0	90



- Δx, Δy, Δz relative to beam center
- Additional 1.5 mm ∆z due to thickness of sample holder
- 14 mm beam diameter



DD Fusion Spectrum

$\overline{A_n}$	100 keV	200 keV
$\overline{A_0}$	2.4674	2.47685
A_1	0.30083	0.39111
A_2	0.01368	0.04098
A_3	0.0	0.02957

$$E_n(\theta) = A_0 + \sum_{n=1}^3 A_n \cos^n(\theta)$$

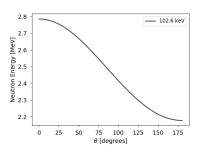
A_n	100 keV	200 keV
$\overline{A_1}$	0.01741	-0.03149
A_2	0.88746	1.11225
A_3	0.22497	0.38659
A_4	0.08183	0.26676
A_5	0.37225	0.11518
$R(\theta)$	$-1 \pm \nabla$	$\frac{1}{5}$ A $\cos^n(\theta)$

$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^5 A_n \cos^n(\theta)$$

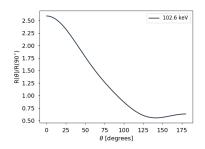
 Use neutron energy and intensity correlations as input to source definition



DD Fusion Spectrum



$$E_n(\theta) = A_0 + \sum_{n=1}^{3} A_n \cos^n(\theta)$$
 $\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^{5} A_n \cos^n(\theta)$



$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^{\infty} A_n \cos^n(\theta)$$

■ Use neutron energy and intensity correlations as input to source definition



Solution Method

Solve for average flux over sample (in vacuum) for given source definition and sample geometry using Monte Carlo

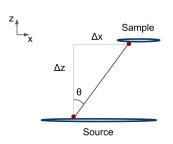
$$\begin{split} \bar{\phi}(E) &= \int \int S(\vec{r}, E, \hat{\Omega}) d^3 r d\hat{\Omega} \\ &= \int \int \phi_0 \frac{n(\vec{r}) \delta(\hat{\Omega} - \Omega_{sample}) R(\theta(E))}{|\vec{r} - r|^2} d^3 r d\hat{\Omega} \\ &= \frac{1}{N} \sum_{n=1}^{N} \phi_0 \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2} = \phi_0 \sum_{n=1}^{N} \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2} \end{split}$$

where $n(\vec{r})$ is PDF for source (e.g. Gaussian) and $\delta_{r\theta}$ constrains neutron rays to source-sample paths.



Solution Method

Generate ray coordinates by randomly generating source point from (radial) Gaussian distribution, and sample point from (radial) uniform distribution.



In general:

$$\theta = arccos(\frac{\vec{r_1} \cdot \vec{r_2}}{|\vec{r_1}||\vec{r_2}|})$$

In 2D Cartesian:

$$\theta = \arccos(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta z^2}})$$

In 3D Cartesian:

$$\theta = \arccos(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta y^2 + \Delta z^2}})$$

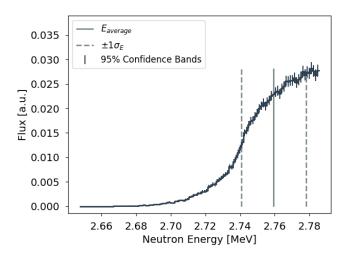


Implementation

- Implemented in python 2.7
- Source on github:
- https://github.com/ jtmorrell/MC_NE201
- Generates plots of flux spectrum for each sample and prints $E_{average} \pm 1\sigma_E$



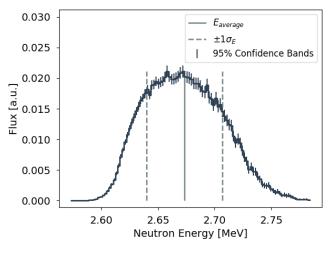
Results (Sample 1)



$$E_n = 2.76 \pm 0.019$$
 [MeV]



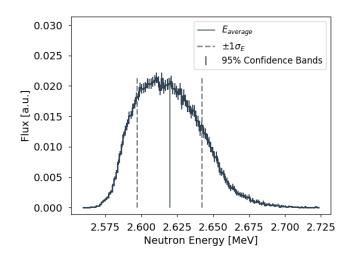
Results (Sample 2)



$$E_n = 2.67 \pm 0.033$$
 [MeV]



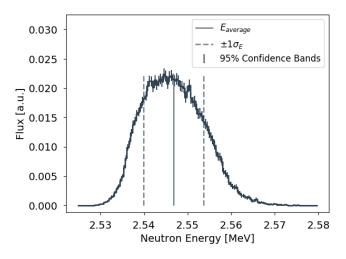
Results (Sample 3)



$$E_n = 2.62 \pm 0.023$$
 [MeV]



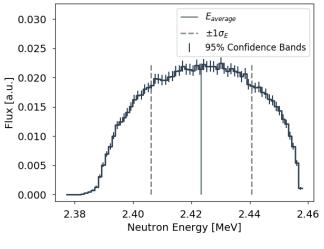
Results (Sample 4)



$$E_n = 2.55 \pm 0.007$$
 [MeV]



Results (Sample 5)



$$E_n = 2.47 \pm 0.018$$
 [MeV]