

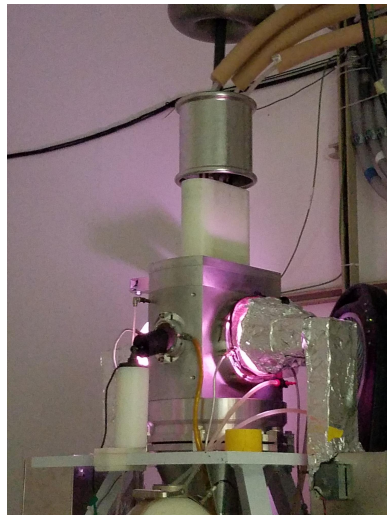
# HFNG Flux Calculation

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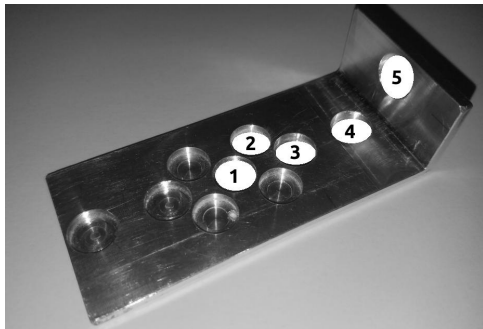
March 14, 2018

# Overview

- Experiment summary
- DD fusion spectrum
- Solution method
- Implementation
- Results



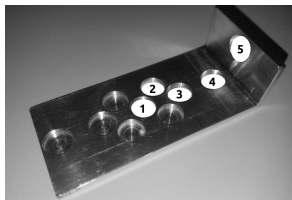
# Experiment Summary



- Goal to measure  $^{35}\text{Cl}(n,p)^{35}\text{S}$  cross section
- 5x 11 mm OD NaCl pellets
- Monitor fluence using  $^{58}\text{Ni}(n,p)^{58}\text{Co}$
- Need to determine energy spectrum for each sample

# Experiment Summary

Sample	$\Delta x$ [mm]	$\Delta y$ [mm]	$\Delta z$ [mm]	$\Delta\theta$ [°]
1	0.0	0.0	8.0	0
2	9.0	8.0	8.0	0
3	18.0	0.0	8.0	0
4	36.0	0.0	8.0	0
5	46.0	0.0	-7.0	90



- $\Delta x$ ,  $\Delta y$ ,  $\Delta z$  relative to beam center
- Additional 1.5 mm  $\Delta z$  due to thickness of sample holder
- 14 mm beam diameter

# DD Fusion Spectrum

$A_n$	100 keV	200 keV
$A_0$	2.4674	2.47685
$A_1$	0.30083	0.39111
$A_2$	0.01368	0.04098
$A_3$	0.0	0.02957

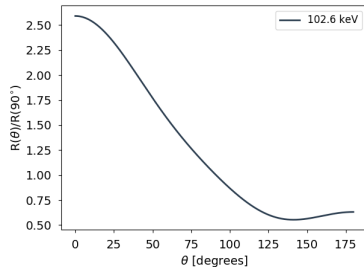
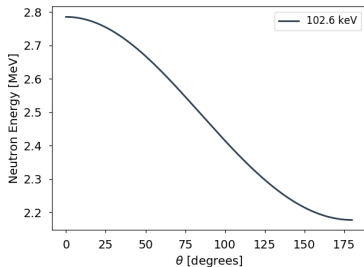
$$E_n(\theta) = A_0 + \sum_{n=1}^3 A_n \cos^n(\theta)$$

$A_n$	100 keV	200 keV
$A_1$	0.01741	-0.03149
$A_2$	0.88746	1.11225
$A_3$	0.22497	0.38659
$A_4$	0.08183	0.26676
$A_5$	0.37225	0.11518

$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^5 A_n \cos^n(\theta)$$

- Use neutron energy and intensity correlations as input to source definition

# DD Fusion Spectrum



$$E_n(\theta) = A_0 + \sum_{n=1}^3 A_n \cos^n(\theta)$$

$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^5 A_n \cos^n(\theta)$$

- Use neutron energy and intensity correlations as input to source definition

# Solution Method

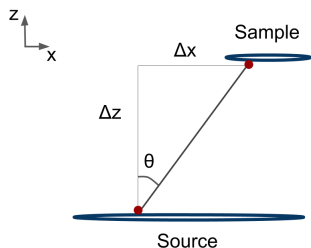
Solve for average flux over sample (in vacuum) for given source definition and sample geometry using Monte Carlo

$$\begin{aligned}\bar{\phi}(E) &= \int \int S(\vec{r}, E, \hat{\Omega}) d^3r d\hat{\Omega} \\ &= \int \int \phi_0 \frac{n(\vec{r}) \delta(\hat{\Omega} - \Omega_{sample}) R(\theta(E))}{|\vec{r} - r|^2} d^3r d\hat{\Omega} \\ &= \frac{1}{N} \sum_{n=1}^N \phi_0 \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2} = \phi_0 \sum_{n=1}^N \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2}\end{aligned}$$

where  $n(\vec{r})$  is PDF for source (e.g. Gaussian) and  $\delta_{r\theta}$  constrains neutron rays to source-sample paths.

# Solution Method

Generate ray coordinates by randomly generating source point from (radial) Gaussian distribution, and sample point from (radial) uniform distribution.



In general:

$$\theta = \arccos\left(\frac{\vec{r}_1 \cdot \vec{r}_2}{|\vec{r}_1||\vec{r}_2|}\right)$$

In 2D Cartesian:

$$\theta = \arccos\left(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta z^2}}\right)$$

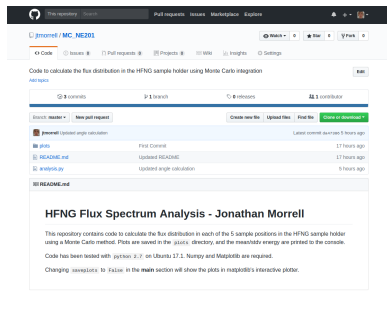
In 3D Cartesian:

$$\theta = \arccos\left(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta y^2 + \Delta z^2}}\right)$$

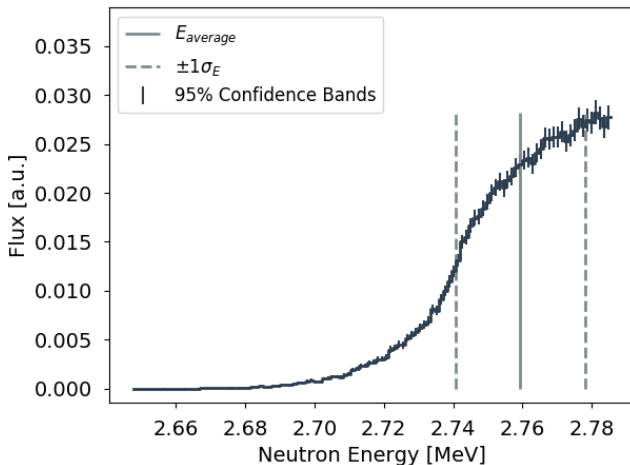


# Implementation

- Implemented in python 2.7
- Source on github:
- [https://github.com/jtmorrell/MC\\_NE201](https://github.com/jtmorrell/MC_NE201)
- Generates plots of flux spectrum for each sample and prints  $E_{average} \pm 1\sigma_E$

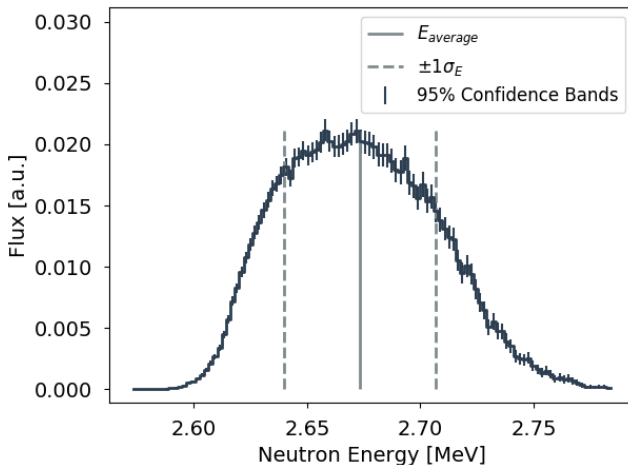


# Results (Sample 1)



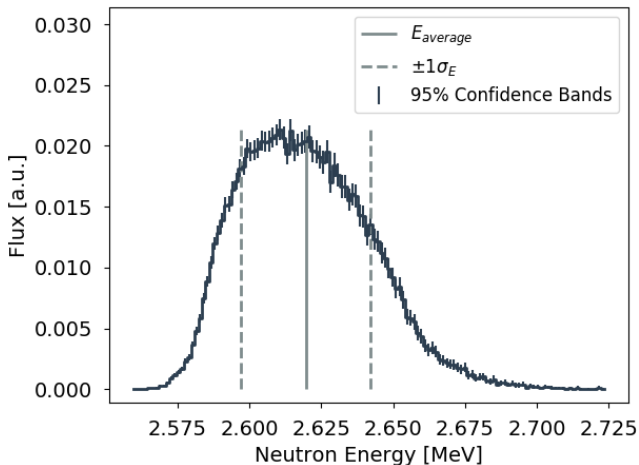
$$E_n = 2.76 \pm 0.019 \text{ [MeV]}$$

# Results (Sample 2)



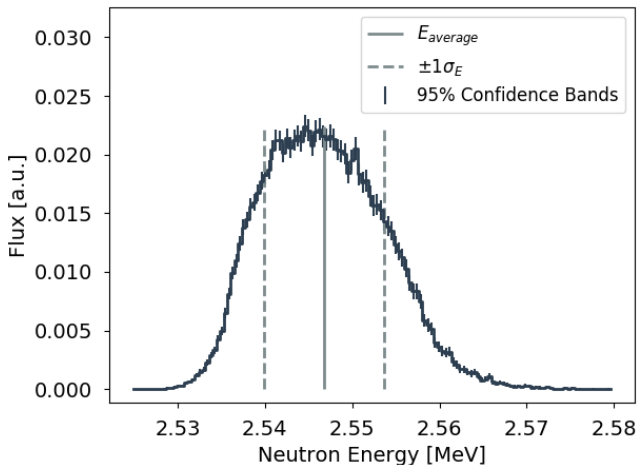
$$E_n = 2.67 \pm 0.033 \text{ [MeV]}$$

# Results (Sample 3)



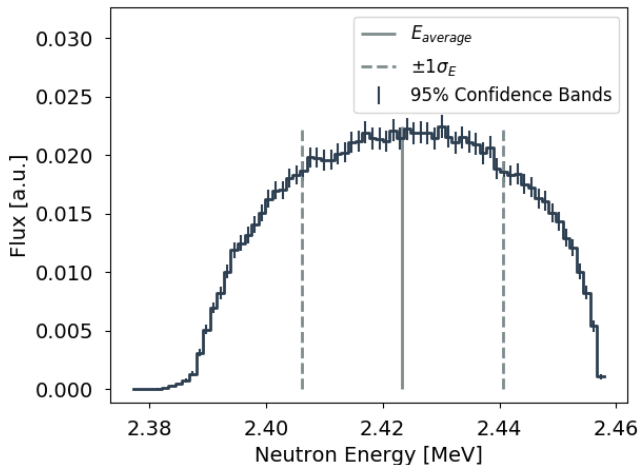
$$E_n = 2.62 \pm 0.023 \text{ [MeV]}$$

# Results (Sample 4)



$$E_n = 2.55 \pm 0.007 \text{ [MeV]}$$

# Results (Sample 5)



$$E_n = 2.47 \pm 0.018 \text{ [MeV]}$$

# Summary of Results

Sample	$E_{average}$ [MeV]	$1\sigma_E$ [MeV]	Relative Flux [a.u.]
1	2.76	0.019	46.78
2	2.67	0.033	16.61
3	2.62	0.023	8.21
4	2.55	0.007	1.98
5	2.47	0.018	1