### HFNG Flux Calculation

Jonathan Morrell, Tyler Bailey, Mitch Negus

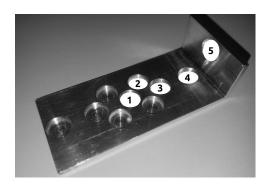
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### Overview

- Experiment summary
- DD fusion spectrum
- Solution method
- Implementation
- Results



### **Experiment Summary**



- Goal to measure <sup>35</sup>Cl(n,p)<sup>35</sup>S cross section
- 5x 11 mm OD NaCl pellets
- Monitor fluence using <sup>58</sup>Ni(n,p)<sup>58</sup>Co
- Need to determine energy spectrum for each sample

## **Experiment Summary**

Sample	Δx [mm]	Δy [mm]	Δz [mm]	Δ <i>θ</i> [°]
1	0.0	0.0	8.0	0
2	9.0	8.0	8.0	0
3	18.0	0.0	8.0	0
4	36.0	0.0	8.0	0
5	46.0	0.0	-7.0	90



- Δx, Δy, Δz relative to beam center
- Additional 1.5 mm ∆z due to thickness of sample holder
- 14 mm beam diameter



### DD Fusion Spectrum

$\overline{A_n}$	100 keV	200 keV
$\overline{A_0}$	2.4674	2.47685
$A_1$	0.30083	0.39111
$A_2$	0.01368	0.04098
$A_3$	0.0	0.02957

$$E_n(\theta) = A_0 + \sum_{n=1}^3 A_n \cos^n(\theta)$$

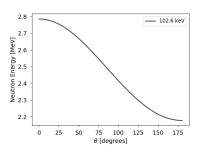
$A_n$	100 keV	200 keV
$\overline{A_1}$	0.01741	-0.03149
$A_2$	0.88746	1.11225
$A_3$	0.22497	0.38659
$A_4$	0.08183	0.26676
$A_5$	0.37225	0.11518
$R(\theta)$	$-1 \pm \nabla$	$\frac{1}{5}$ A $\cos^n(\theta)$

$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^5 A_n \cos^n(\theta)$$

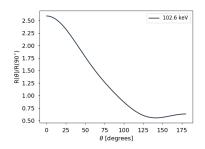
 Use neutron energy and intensity correlations as input to source definition



## DD Fusion Spectrum



$$E_n(\theta) = A_0 + \sum_{n=1}^{3} A_n \cos^n(\theta)$$
  $\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^{5} A_n \cos^n(\theta)$ 



$$\frac{R(\theta)}{R(90^\circ)} = 1 + \sum_{n=1}^{\infty} A_n \cos^n(\theta)$$

■ Use neutron energy and intensity correlations as input to source definition



#### Solution Method

Solve for average flux over sample (in vacuum) for given source definition and sample geometry using Monte Carlo

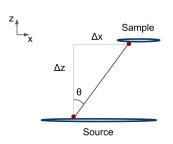
$$\begin{split} \bar{\phi}(E) &= \int \int S(\vec{r}, E, \hat{\Omega}) d^3 r d\hat{\Omega} \\ &= \int \int \phi_0 \frac{n(\vec{r}) \delta(\hat{\Omega} - \Omega_{sample}) R(\theta(E))}{|\vec{r} - r|^2} d^3 r d\hat{\Omega} \\ &= \frac{1}{N} \sum_{n=1}^{N} \phi_0 \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2} = \phi_0 \sum_{n=1}^{N} \frac{R(\theta(E_n)) \delta_{r\theta}}{(\Delta r_n)^2} \end{split}$$

where  $n(\vec{r})$  is PDF for source (e.g. Gaussian) and  $\delta_{r\theta}$  constrains neutron rays to source-sample paths.



#### Solution Method

Generate ray coordinates by randomly generating source point from (radial) Gaussian distribution, and sample point from (radial) uniform distribution.



In general:

$$\theta = arccos(\frac{\vec{r_1} \cdot \vec{r_2}}{|\vec{r_1}||\vec{r_2}|})$$

In 2D Cartesian:

$$\theta = \arccos(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta z^2}})$$

In 3D Cartesian:

$$\theta = \arccos(\frac{\Delta z}{\sqrt{\Delta x^2 + \Delta y^2 + \Delta z^2}})$$

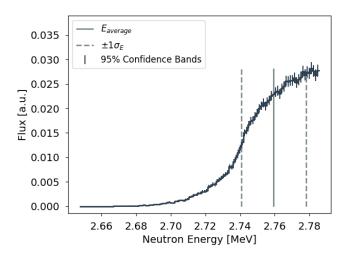


### Implementation

- Implemented in python 2.7
- Source on github:
- https://github.com/ jtmorrell/MC\_NE201
- Generates plots of flux spectrum for each sample and prints  $E_{average} \pm 1\sigma_E$



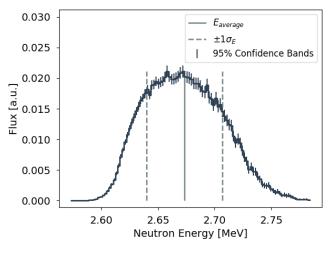
## Results (Sample 1)



$$E_n = 2.76 \pm 0.019$$
 [MeV]



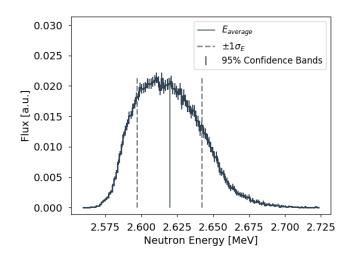
## Results (Sample 2)



$$E_n = 2.67 \pm 0.033$$
 [MeV]



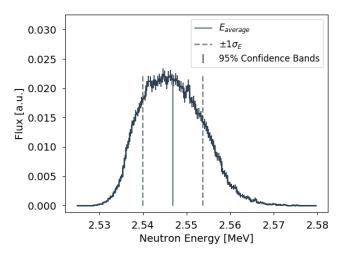
## Results (Sample 3)



$$E_n = 2.62 \pm 0.023$$
 [MeV]



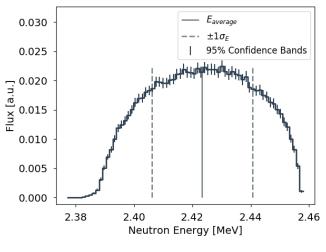
## Results (Sample 4)



$$E_n = 2.55 \pm 0.007$$
 [MeV]



## Results (Sample 5)



$$E_n = 2.47 \pm 0.018$$
 [MeV]



# Summary of Results

Sample	$E_{average}$ [MeV]	$1\sigma_E$ [MeV]	Relative Flux [a.u.]
1	2.76	0.019	46.78
2	2.67	0.033	16.61
3	2.62	0.023	8.21
4	2.55	0.007	1.98
5	2.47	0.018	1