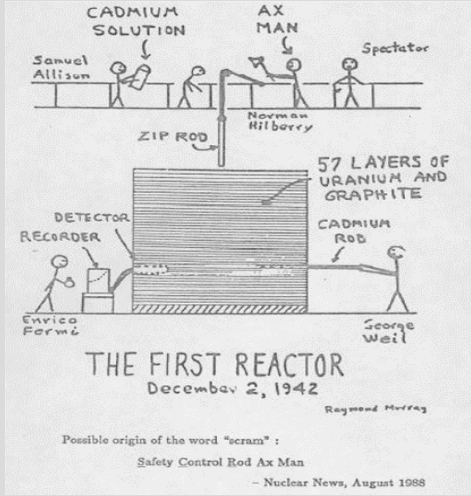
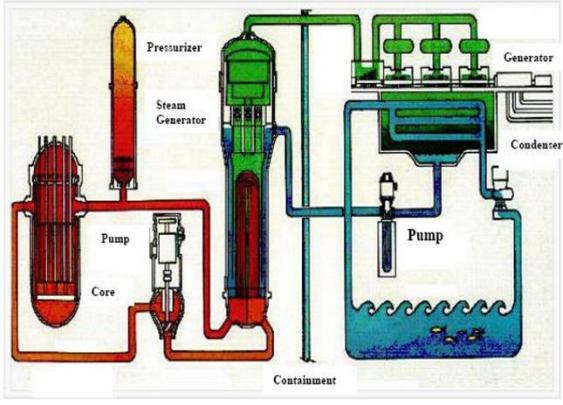
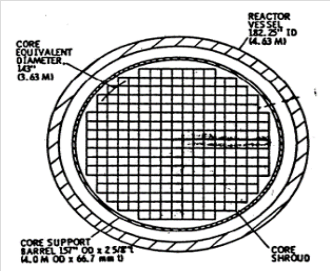
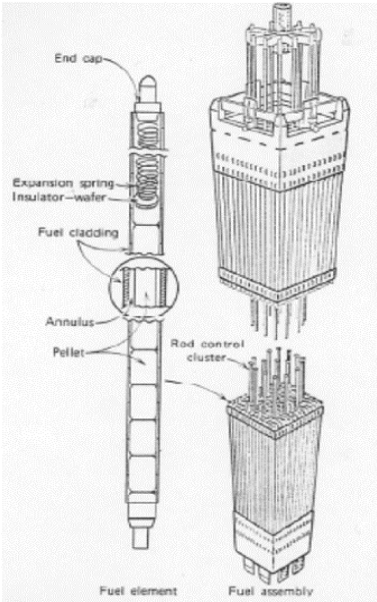


词汇	翻译	备注
CP1---Fermi	费米堆	第一座反应堆，1941
EBR-I	实验增殖堆 I	第一个利用原子能发电的发电厂，1951
Obninsk	奥布宁斯克核电站	世界上首个商用发电的核反应堆，1954
Shipping Port	希平港核电厂	世界第一座商用核电站，1955
Three-Mile-Island	三厘岛	1979
Chernobyl	切尔诺贝利	1986
Fukushima	福岛	2011
coal	煤炭	消耗量 1, 33%
oil	石油	消耗量 2, 28%
natural gas	天然气	消耗量 3, 24%
nuclear energy	核能	5%
cadmium	镉	
control rod	控制棒	
ax man	斧人	
Uranium	铀	
recorder	记录仪	
detector	探测器	
graphite	石墨	
zip rod	停堆棒	
core	堆芯	
pump	泵	
pressurizer	稳压器	
steam generator	蒸汽发生器	
containment	安全壳	
generator	发电机	
condenser	冷凝器	
core support	堆芯支撑板	

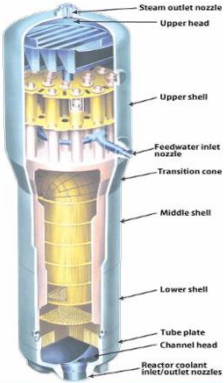
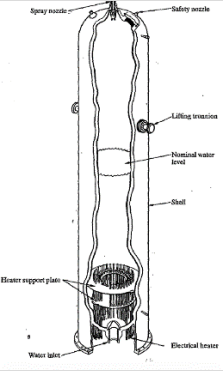
reactor pressure vessel	反应堆压力容器	
core shroud	堆芯屏蔽	
core equivalent diameter	堆芯等效直径	
end cap	端塞	
expansion spring	压紧弹簧	
insulator-wafer	芯块支撑板（隔热垫片）	
fuel cladding	燃料包壳	
annulus	间隙	
pellet	芯块	
rod control cluster	棒簇控制组件	
fuel element	燃料元件	
fuel assembly	燃料组件	
coolant	冷却剂	带走堆芯产生热量
self-sustained fission chain reaction	自持链式裂变反应	
kinetic	动能	
fission fragments	裂变碎片	
neutrons	中子	
gamma rays	伽马射线	
neutron distribution	中子分布	
power distribution	功率分布	
capture	捕获	
leakage	泄露	
effect multiplication factors	有效增殖系数	$k_{eff} = \frac{\text{某一代中子数}}{\text{直系上一代中子数}}$ $= \frac{\text{number of fissions in one generation}}{\text{number of fissions in preceding generation}}$

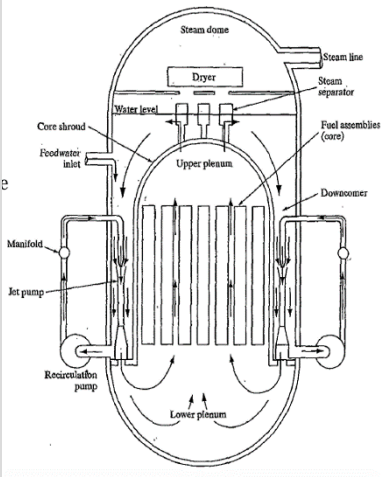
heat conduction	热传导	
heat convection	热对流	
heat radiation	热辐射	
thermo-hydraulics	热工水力	
pressure tap	压力表	
thermocouple	热电偶	
flow meter	流量计	
by-pass valve	旁通阀	
secondary loop	二回路	
primary loop	一回路	
critical	临界	$k = 1$
supercritical	超临界	$k > 1$
subcritical	亚临界	$k < 1$
fissile	易裂变	形容核素
fissionable	可裂变	
reproduction factor	增殖因子	$\eta$ = 每吸收一个中子放出中子数
nucleus	原子核	
nuclides	核素	
isotopic abundance	同位素富集度	
conversion	转化	
breeding	增殖	
conversion ratio / breeding ratio	转化比 / 增殖比	$c$ = 平均每消耗一个易裂变原子产生新的易裂变原子数 = The average number of fissile atoms produced in a reactor per fissile fuel atom consumed
breeder	增殖堆	$c > 1$
converter	转化堆	$c = 1$
burner	燃耗堆	$c < 1$
absorb	吸收	
thermal neutron	热中子	

thermal reactor	热中子堆	
fast reactor / fast breeder	快中子堆	
breeding gain	净增殖比	$G = \text{每消耗一个燃料原子增加的易裂变原子数} = c - 1$
liner doubling time	线性倍增时间	$t_{Dl} = \frac{m_0}{GwP_0}$
exponential doubling time	指数倍增时间	$t_{De} = \ln 2 t_{Dl}$
fuel consumption rate per unit power	单位功率燃料消耗率	$w$
Avogadro's number	阿伏伽德罗常数	$N_A$
fuel management	燃料管理	
chemically separated	化学分离	
fabricated	燃料制造	
burn up	燃耗	单位: $MWd$
specific burn up	特定燃耗	单位: $MWd / kg$
fraction burn up	比燃耗	$\beta = \frac{\text{易裂变原子数}}{\text{初始重原子核数}} = \frac{\text{numbers of fissions}}{\text{initial numbers of heavy atoms}}$
water vapor	水蒸气	
cooling tower	冷却塔	
transformer	变压器	
turbine	透平	
stream lines	蒸汽管线	
cooling water	冷却水	
cold water basin	冷却水箱	
cool water source	冷却水源	
electricity	电	
warm water inlet	热水进口	

non-nuclear components	非核部分	
bladed	汽轮机叶片	
axle	角度	
erosion	腐蚀	
lifetime	寿命	
droplets	液滴	
circumvent	方法	
superheating	过热	
remove	移除	
wet steam	湿蒸汽	
moisture separator	汽水分离器	
reheat	再热	
spent steam	废蒸汽	
hotter steam	更热的蒸汽	
overall efficiency	总效率	$eff = \frac{W}{Q_R} \approx 1 - \frac{Q_C}{Q_R}$
nuclear power plant	核电厂	
electrical energy output	输出电能	$W$
thermal energy output	输出热能	$Q_R$
megawatts	兆瓦	$MW$
heat losses	热损失	
heat exchangers	换热器	
piping	管道	
the heat rejected to the coolant in the condenser	冷凝器中冷却剂带走热量	$Q_C$ , $W = Q_R - Q_C$
availability factor	运行系数	给定周期内核电厂运行时间，一般 90%
capacity factor	能力系数	给定周期内核电厂满功率运行时间，一般 80-85%

moderator	慢化剂	热中子堆才有，水，重水，石墨
blanket	增殖包层	富集度高的同位素，用于转化堆和增殖堆的堆芯周围区域
reflector	反射层	节省中子，展平功率
Nuclear Steam Supply System (NSSS)	核蒸汽供应系统	
boiling water reactor	沸水堆	BWR
pressurized water reactor	压水堆	PWR
heavy-water moderated reactors	重水慢化堆	石墨慢化，氦气冷却
gas-cooled reactors	气冷堆	
helium	氦气	
light-water reactors	轻水堆	LWR，最常用，热中子堆，PWR，BWR 组成
reactor vessel	压力容器	<div> <p><b>Typical Pressurized Water Reactor</b></p> </div> <p>压水堆进口 290℃，出口 325℃，压力 15MPa，</p>
control rod drive mechanism	控制棒驱动机构	
upper support plate	上部支撑板	
internals support ledge	内部支撑构件	
support column	支撑柱	
upper core plate	堆芯上栅格板	
outlet nozzle	出口接管	
radial support	径向支撑	
baffle	挡板	
core support columns	堆芯支撑柱	
instrumentation thimble guides	仪表导向管	
lower core plate	堆芯下栅格板	
access port	检修孔	
rod travel housing	控制棒导管	

instrumentation ports	仪表端口	不会发生沸腾
thermal sleeve	热套	
lifting lug	吊耳	
closure head assembly	封头组件	
hold down spring	压紧弹簧板	
control rod drive shaft	控制棒驱动杆	
inlet nozzle	进口接管	
U-tube Steam Generator	U 型管蒸汽发生器	 <div>4 个蒸汽发生器，蒸汽参数 293℃，5MPa， 压水堆热效率 32-33%</div>
steam outlet nozzle	蒸汽出口管嘴	
feed water inlet nozzle	给水管嘴	
tube plate	管板	
reactor coolant inlet / outlet nozzles	反应堆冷却剂进 出口接管	
pressurizer	稳压器	
safety nozzle	安全阀	
spray nozzle	喷淋嘴	
nominal water level	标定水位	
shell	壳体	
heat support plate	加热器支撑板	
electrical heater	电加热器	
surge nozzle	升压喷嘴	
relief nozzle	降压喷嘴	
grid	格架	
bottom nozzle	下管座	
top nozzle	上管座	
melting point	熔点	
enrichment	富集度	
cylindrical	圆柱	

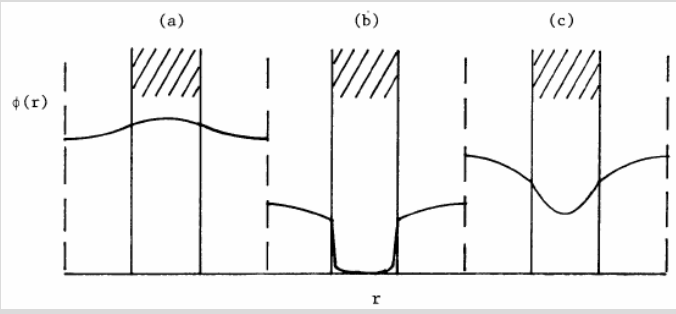
zircaloy-4	锆 Zr-4	
uranium dioxide	二氧化铀	
loading pattern	装料方式	三种不同富集度燃料分区布置，富集度最高的3.1%在外围 52 组件，中间以国际象棋棋盘交替布置 1.8%53 组件，2.4%52 组件
dryer	干燥器	 <div>蒸汽在反应堆内生成直接进入透平循环</div>
upper plenum	上腔室	
downcomer	下水管	
steam separator	蒸汽分离器	
manifold	集管	
jet pump	气泵	
recirculation pump	循环泵	
lower plenum	下腔室	
压水堆有二回路，沸水堆没有。 沸水堆焓升大，相同功率流量小	压水堆压力 15MPa，沸水堆 7MPa	沸水堆的控制棒从堆底引入①沸水堆堆芯上部蒸汽含量较多，中子慢化不足，下部有助于展平中子通量密度。②可以空出堆芯上方空间以安装汽水分离器和干燥器。
CANDU	坎杜堆	
On-line refueling	在线换料	
liquid-metal fast breeder reactor	液态金属增殖快堆	
loop type	回路式	
pool type	池式	
Generation IV reactors	第四代反应堆	
Very-High-Temperature Reactor	超高温气冷堆	VHTR
Molten-Salt Reactor	熔盐堆	MSR
Sodium-Cooled-Fast Reactor	钠冷快堆	SFR



Supercritical-Water-Cooled Reactor	超临界水堆	SCWR
Gas-Cooled-Fast Reactor	气冷快堆	GFR
Lead-Cooled-Fast Reactor	铅冷快堆	LFR
mono-energetic	相同能量	$D\nabla^2\phi - \Sigma_a\phi + s = \frac{1}{v} \frac{\partial\phi}{\partial t}$ $s = v \Sigma_f \phi$ $\frac{\partial\phi}{\partial t} = 0$ $D\nabla^2\phi - \Sigma_a\phi + \frac{1}{k} v \Sigma_f \phi = 0$ $B^2 = \frac{1}{D} \left( \frac{1}{k} v \Sigma_f - \Sigma_a \right)$ $\nabla^2\phi + B^2\phi = 0$
One-Group Reactor Equation	单群核反应堆方程	
neutron diffusion equation	中子扩散方程	
time-dependent	时间相关	
neutron source	中子源	
the slab reactor	板式反应器	
thickness	厚度	$\phi(x) = A \cos B_1 x \quad \text{with} \quad B_1 = \frac{\pi}{\tilde{a}}$ $\Rightarrow \phi(x) = \frac{\pi P}{2\tilde{a}E_R\Sigma_f \sin(\frac{\pi a}{2\tilde{a}})} \cos(\frac{\pi x}{\tilde{a}}) \approx \frac{\pi P}{2\tilde{a}E_R\Sigma_f} \cos(\frac{\pi x}{\tilde{a}})$ $\phi(r) \approx \frac{P}{4E_R\Sigma_f R^2} \frac{\sin(\pi r / R)}{r}$ $\phi(r) \approx \frac{0.738 P}{E_R\Sigma_f R^2} J_0\left(\frac{2.405 r}{R}\right)$ $\phi(r, z) \approx \frac{3.63 P}{VE_R\Sigma_f} J_0\left(\frac{2.405 r}{\tilde{R}}\right) \cos \frac{\pi z}{\tilde{H}}$ $\phi(x, y, z) \approx \frac{\pi^3 P}{8VE_R\Sigma_f} \cos \frac{\pi x}{a} \cos \frac{\pi y}{b} \cos \frac{\pi z}{c}$
symmetry	对称的	
b.c boundary condition	边界条件	
eigenvalues	特征值	
curvature	曲率	
material buckling	材料曲率	
sphere	球	
infinite cylinder	无限高圆柱	
finite cylinder	有限高圆柱	
parallelepiped	平行六面体	

		<div><div><div><div><div><div></div><div><div>Bucklings and Fluxes of Critical Bare Reactors (All Dimensions Are Extrapolated)*</div></div></div></div><table><tr><th>Geometry</th><th>Dimensions</th><th>Buckling</th><th>Flux</th></tr><tr><td>Infinite slab</td><td>Thickness <math>a</math></td><td><math>\left(\frac{\pi}{a}\right)^2</math></td><td><math>\cos\left(\frac{\pi x}{a}\right)</math></td></tr><tr><td>Rectangular parallelepiped</td><td><math>a \times b \times c</math></td><td><math>\left(\frac{\pi}{a}\right)^2 + \left(\frac{\pi}{b}\right)^2 + \left(\frac{\pi}{c}\right)^2</math></td><td><math>\cos\left(\frac{\pi x}{a}\right) \cos\left(\frac{\pi y}{b}\right) \cos\left(\frac{\pi z}{c}\right)</math></td></tr><tr><td>Infinite cylinder</td><td>Radius <math>R</math></td><td><math>\left(\frac{2.405}{R}\right)^2</math></td><td><math>J_0\left(\frac{2.405r}{R}\right)</math></td></tr><tr><td>Finite cylinder</td><td>Radius <math>R</math> Height <math>H</math></td><td><math>\left(\frac{2.405}{R}\right)^2 + \left(\frac{\pi}{H}\right)^2</math></td><td><math>J_0\left(\frac{2.405r}{R}\right) \cos\left(\frac{\pi z}{H}\right)</math></td></tr><tr><td>Sphere</td><td>Radius <math>R</math></td><td><math>\left(\frac{\pi}{R}\right)^2</math></td><td><math>\frac{1}{r} \sin\left(\frac{\pi r}{R}\right)</math></td></tr></table></div></div></div>	Geometry	Dimensions	Buckling	Flux	Infinite slab	Thickness $a$	$\left(\frac{\pi}{a}\right)^2$	$\cos\left(\frac{\pi x}{a}\right)$	Rectangular parallelepiped	$a \times b \times c$	$\left(\frac{\pi}{a}\right)^2 + \left(\frac{\pi}{b}\right)^2 + \left(\frac{\pi}{c}\right)^2$	$\cos\left(\frac{\pi x}{a}\right) \cos\left(\frac{\pi y}{b}\right) \cos\left(\frac{\pi z}{c}\right)$	Infinite cylinder	Radius $R$	$\left(\frac{2.405}{R}\right)^2$	$J_0\left(\frac{2.405r}{R}\right)$	Finite cylinder	Radius $R$ Height $H$	$\left(\frac{2.405}{R}\right)^2 + \left(\frac{\pi}{H}\right)^2$	$J_0\left(\frac{2.405r}{R}\right) \cos\left(\frac{\pi z}{H}\right)$	Sphere	Radius $R$	$\left(\frac{\pi}{R}\right)^2$	$\frac{1}{r} \sin\left(\frac{\pi r}{R}\right)$
Geometry	Dimensions	Buckling	Flux																							
Infinite slab	Thickness $a$	$\left(\frac{\pi}{a}\right)^2$	$\cos\left(\frac{\pi x}{a}\right)$																							
Rectangular parallelepiped	$a \times b \times c$	$\left(\frac{\pi}{a}\right)^2 + \left(\frac{\pi}{b}\right)^2 + \left(\frac{\pi}{c}\right)^2$	$\cos\left(\frac{\pi x}{a}\right) \cos\left(\frac{\pi y}{b}\right) \cos\left(\frac{\pi z}{c}\right)$																							
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Sphere	Radius $R$	$\left(\frac{\pi}{R}\right)^2$	$\frac{1}{r} \sin\left(\frac{\pi r}{R}\right)$																							
reactor	反应堆																									
flux	中子通量																									
eigenfunction	特征函数																									
joules	焦耳																									
sec	秒																									
geometric buckling	几何曲率	$P_L = \frac{\Sigma_a}{\Sigma_a + DB^2} = \frac{1}{1 + \frac{D}{\Sigma_a} B^2} = \frac{1}{1 + L^2 B^2}$																								
bare reactor	裸堆	$k_\infty = \frac{\nu \sum_f}{\sum_a}$																								
arbitrary geometry	任意几何条件	$L^2 = \frac{D}{\sum_a}$																								
non-leakage probability	逃脱泄露几率	$P_L$																								
the one-group critical equation	单群临界方程	$\frac{k_\infty}{1 + L^2 B^2} = k_\infty P_L = k = 1 \text{临界条件}$																								
in operation	在运																									
neutron cycle	中子循环																									
fast non-leakage fraction	快中子不泄露几率	$P_S$																								
resonance absorption fraction	逃脱共振吸收概率	$p$ =fraction of neutrons that passes through the resonance region without being absorbed																								
thermal non-leakage fraction	热中子不泄露几率	$P_t$																								

parasitic absorption fraction	燃料吸收系数	$f = \frac{\text{number of neutrons absorbed by fuels}}{\text{total number of neutrons be absorbed}}$
fast fission factor	快中子倍增系数	$\varepsilon = \frac{\text{numbers of fast neutrons}}{\text{number of fast neutrons produced by thermal fissions}}$ $= \frac{\text{快中子数}}{\text{热中子裂变产生的快中子数}}$
six quantities	六因子	$k = \varepsilon P_s p P_t f \eta$
eta factor / neutron yield factor	增殖因子	$\eta = \frac{\text{average number of fission neutrons released}}{\text{number of neutrons absorbed in the fuel}}$
four-factor equation	4 因子方程	$k_{\infty} = \varepsilon p f \eta$
neutron age	中子年龄	$\tau_T = \frac{D_1}{\sum_1}$ $k_{eff} = \frac{k_{\infty}}{(1 + L_T^2 B^2)(1 + B^2 \tau_T)} = \frac{k_{\infty}}{(1 + M_T^2 B^2)}$
migration area	迁移区域	$M_T^2 = L_T^2 + \tau_T$
core radius	堆芯半径	
homogeneous	匀质	
diffusion length	扩散长度	$L$
critical volume	临界体积	
multigroup calculations	多群计算	$\frac{1}{v_g} \frac{\partial}{\partial t} \phi_g(\mathbf{r}, t) = \underbrace{\nabla \cdot \mathbf{D}_g(\mathbf{r}) \nabla \phi_g(\mathbf{r}, t)}_{\text{leakage}} - \underbrace{\Sigma_{a g}(\mathbf{r}) \phi_g(\mathbf{r}, t)}_{\text{loss by absorption}} - \underbrace{\Sigma_{s g}(\mathbf{r}) \phi_g(\mathbf{r}, t)}_{\text{removal by scattering}} + \underbrace{\sum_{g'=1}^G \Sigma_{s g' g}(\mathbf{r}) \phi_{g'}(\mathbf{r}, t)}_{\text{scattering into group g}}$ $+ \underbrace{\lambda_g \sum_{g'=1}^G v_{g'} \Sigma_{f g'}(\mathbf{r}) \phi_{g'}(\mathbf{r}, t)}_{\substack{\text{fraction} \\ \text{appearing} \\ \text{in group g}} \text{ total fission production}} + \underbrace{S_g^{\text{ext}}}_{\text{external source}}$
energy spectrum	能谱	
cross sections	截面	
vary over decades	数十倍的变化	
void	空间	
scattering	散射	
coarse	粗糙的	

fission spectrum	裂变谱	
Spatial distribution of flux	中子空间分布	
heterogeneous reactors	非均匀反应堆	 <p>The neutron flux in a unit cell; (a) fast, (b) resonance, (c) thermal neutrons</p>
numerical computational methods	数值计算方法	
angular discretization	角度离散	
spatial discretization	空间离散	
energy discretization	能量离散	
time discretization	时间离散	
neutron transport equation	中子输运方程	$\frac{1}{v(E)} \frac{\partial}{\partial t} \psi(\vec{r}, E, \vec{\Omega}, t) + \vec{\Omega} \cdot \nabla \psi(\vec{r}, E, \vec{\Omega}, t) + \Sigma_t(\vec{r}, E, t) \psi(\vec{r}, E, \vec{\Omega}, t)$ $= \int dE' \int d\vec{\Omega}' \left[ \left( \frac{\chi(E)}{k_{eff}} v \Sigma_f(\vec{r}, E', t) + \Sigma_s(\vec{r}, \vec{\Omega} \rightarrow \vec{\Omega}', E' \rightarrow E, t) \right) \psi(\vec{r}, E', \vec{\Omega}', t) \right]$
deterministic methods	确定论方法	
stochastic	不确定论方法	
continuous energy formalism	连续能量形式	
monte carlo	蒙特卡洛	
multigroup formalism	多群形式	
collision probability	碰撞概率	
integral	整体的	
integrodifferential	微积分的	
finite difference	有限差分	
finite elements	有限元	

even parity	偶校验	
Whole-core analysis	全堆芯分析	
lattice calculation	格子计算	
supercell calculation	超晶胞计算	
The Numerical Reactor	数值反应堆	
Multi-physics coupling	多物理场耦合	
Multi-Scale Coupling	多尺度耦合	
Hi-Fidelity	高保真	
Massive parallel	大规模并行	
barrel	桶	
pads	垫	
explicitly	显式	
the pin cell pitch size	精细化尺寸	