

# Exploring Effects of Homogenization on an OpenMC Depletion Analysis of a TRISO Fueled, Helium Cooled Microreactor

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April 24, 2024



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① Virtual Test Bed Gas-Cooled Microreactor

② OpenMC Model

③ Results and Discussion

④ Next Steps



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① Virtual Test Bed Gas-Cooled Microreactor

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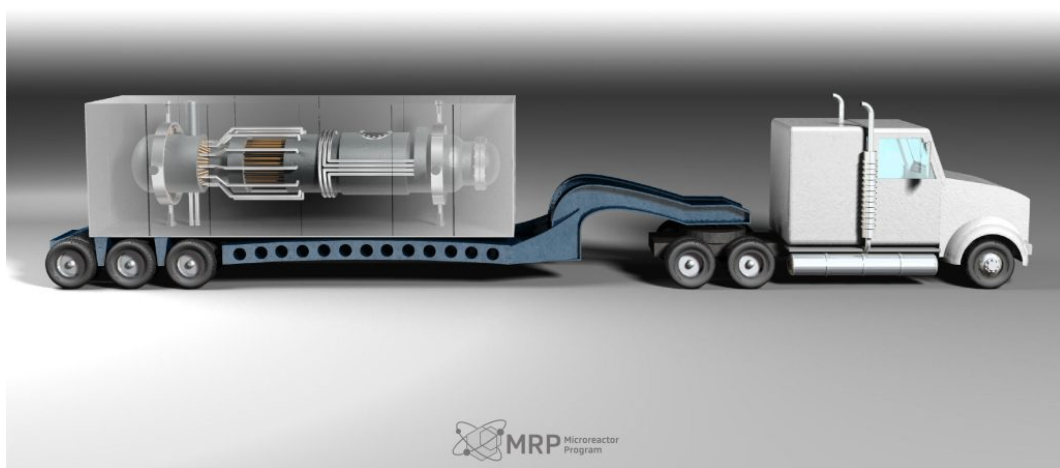
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NEAMS

MOOSE





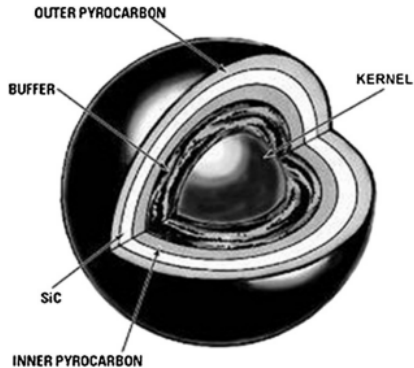


Image from [3]

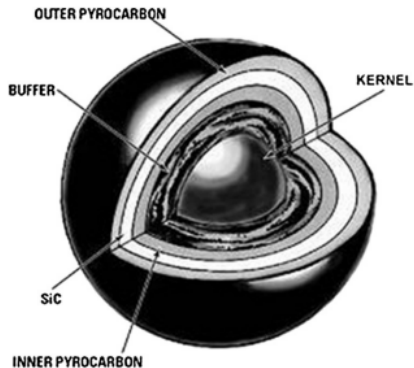


Image from [3]

- TRISO fuel synergizes with HTGRs
  - Melting temperature significantly higher than operational temperatures [3]
  - Designed to contain fission products [3]
  - Typically packed into graphite compacts or into spherical pebbles for PBRs



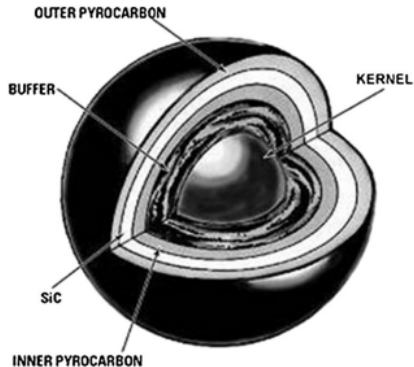
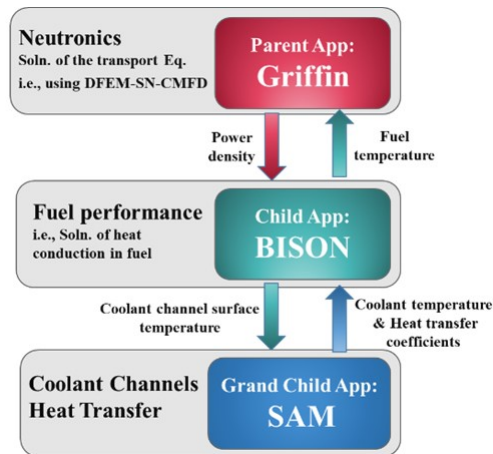


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- TRISO modeling challenges
  - Five surfaces per TRISO
  - Causes very many surface crossings per history
  - High memory requirement for fully explicit representation

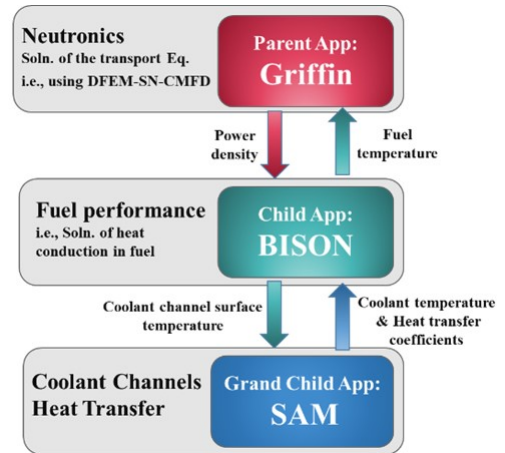


- Existing simulations using MOOSE tools
  - Multiphysics models, including accident and load-following transients: Griffin-BISON-SAM [4, 5, 6]
  - Balance of plant 1D thermal hydraulic simulation [7]



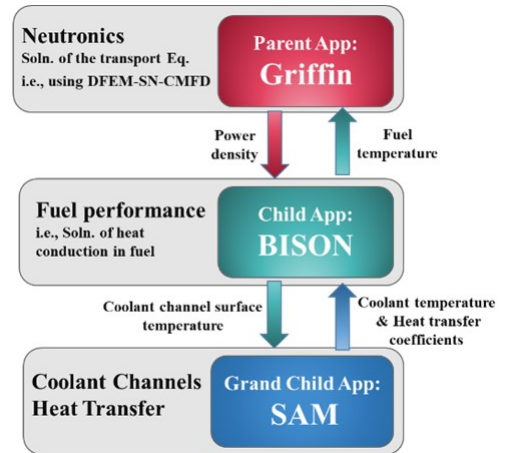
MultiApp hierarchy of preliminary GCMR models.  
Image from [5].

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  - Plans to add this work's model to the VTB this summer



MultiApp hierarchy of preliminary GCMR models. Image from [5].

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  - Plans to add this work's model to the VTB this summer
- For a full core model, it will be prohibitively expensive to model every TRISO explicitly
  - $O(10^{13})$  in this reactor



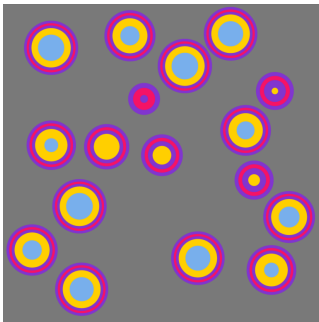
MultiApp hierarchy of preliminary GCMR models. Image from [5].



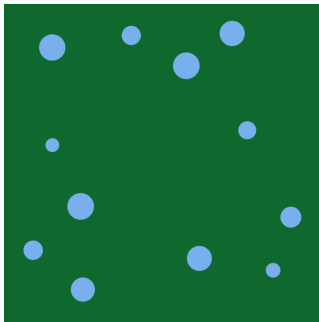


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- Two homogenization strategies: “kernel only” and “full volume”



explicit



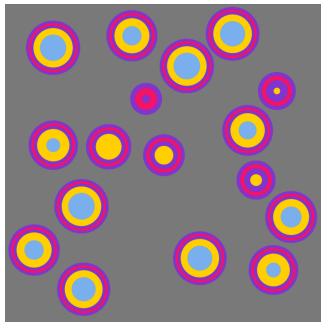
kernel only



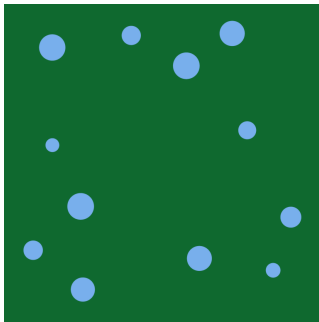
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- Burnup simulations at 100%, 50%, and 10% of full power (225 KWt).
- Compare each  $k_{inf}$  as a basis for deciding how proceed with a full core model.



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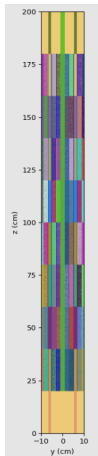
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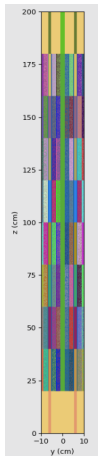
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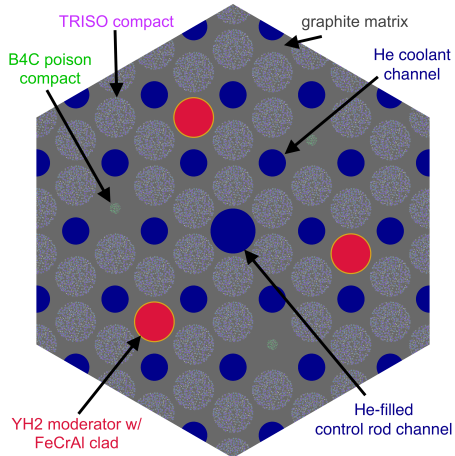




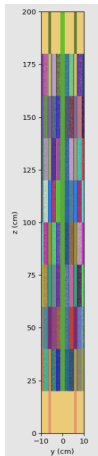
YZ slice of reactor



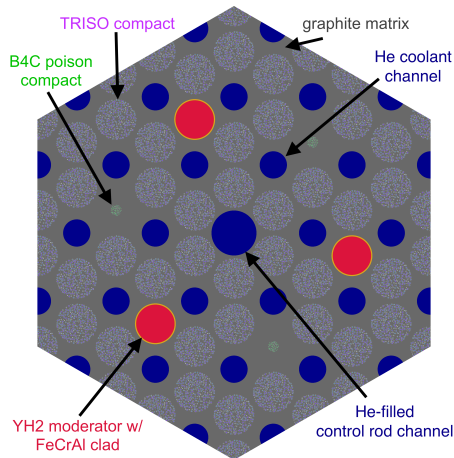
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XY slice of reactor



YZ slice of reactor



XY slice of reactor

- spatial depletion scheme
- 3-way radial symmetry cloning scheme
- 8 axial layers in the core
- 2 axial layers per reflector





- Predictor-Corrector time integration scheme
  - requires two transport solves per time step
  - Constant Extrapolation/Constant Midpoint scheme from Isotalo et al. [9]
  - 25 inactive and 75 active batches with 10000 particles per batch
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  - based on the CASL project [10], which uses a thermal spectrum to compute the burnup matrix
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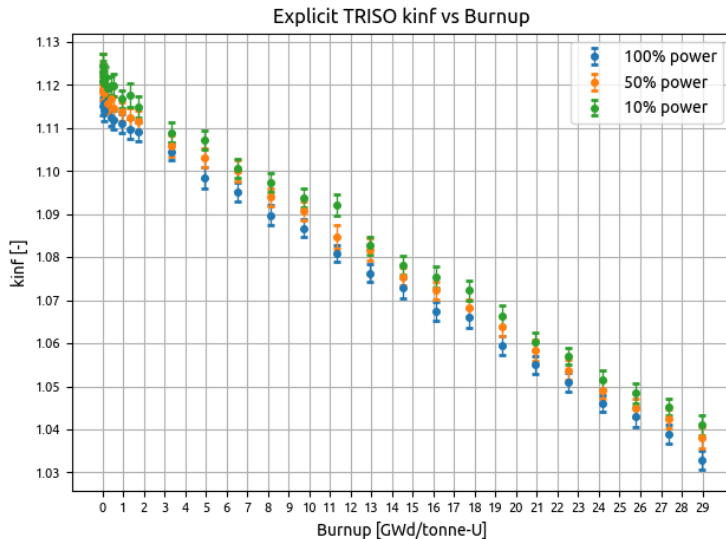
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- Time Steps
  - Full power time steps:  $[1]*5 + [5]*3 + [15]*3 + [60]*17$  (days)
  - Burnup-consistent time steps for other powers



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# Fully-explicit $k_{inf}$ versus burnup with $2\sigma$ error bars up to $\sim 29$ GWd/tonne-U







- Denote the first set of eigenvalues  $k_1$  and the second set  $k_2$ , the  $\Delta\rho$  between them is given by

$$\Delta\rho \equiv \rho_1 - \rho_2 = \frac{k_1 - 1}{k_1} - \frac{k_2 - 1}{k_2} = \frac{1}{k_2} - \frac{1}{k_1} \quad (1)$$

## Comparing Eigenvalues

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- Average  $\Delta\rho$  compared with the explicit reference at every power with  $2\sigma$  uncertainties

$\overline{\Delta\rho}$	explicit - homogenized	explicit - kernel only
100% power	$1533 \pm 55$ pcm	$-158 \pm 55$ pcm
50% power	$1495 \pm 56$ pcm	$-193 \pm 55$ pcm
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- Kernel only  $\Delta\rho$  on average performs about 1 order of magnitude better than full homogenization.





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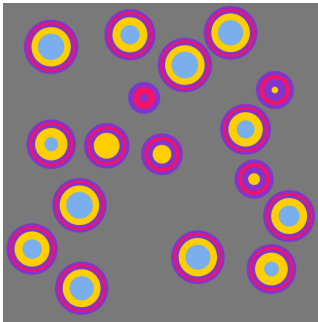
- While the kernel only eigenvalue computation outperforms the fully homogenized in terms of accuracy, it would be desirable to lower  $\Delta\rho$  below 100 pcm.



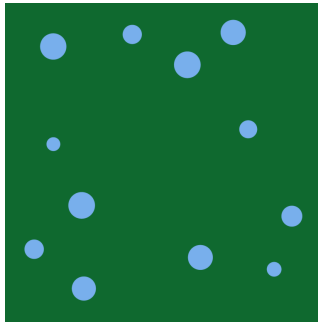
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- Si is less efficient at thermalizing neutrons and absorbs more neutrons than C

# Two-layer TRISO Homogenization

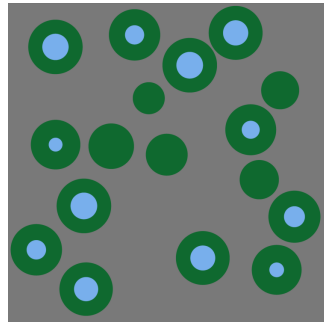
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- The kernel only model moves SiC further from the fuel than it exists in the explicit model.
- Si is less efficient at thermalizing neutrons and absorbs more neutrons than C
- A next iteration on fuel homogenization would be to create a **two layer** homogenization



explicit



kernel only



two layer TRISO

- The first author was supported in part by the US Nuclear Regulatory Commission's Graduate Fellowship Program administered by the University of Wisconsin-Madison.
- The OpenMC team!
- The Center for High Throughput Computing at the University of Wisconsin - Madison
- Co-authors: Patrick Shriwise, Benjamin Lindley, and Paul P.H. Wilson.

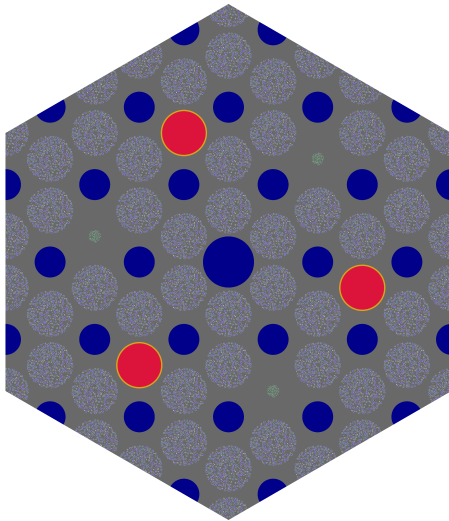


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- [2] Joel Hiller. *Microreactors*. Idaho National Lab, Aug. 2023. URL: <https://inl.gov/trending-topics/microreactors/> (visited on 04/09/2024).
- [3] X.W. Zhou and C.H. Tang. "Current status and future development of coated fuel particles for high temperature gas-cooled reactors". In: *Progress in Nuclear Energy* 53.2 (2011), pp. 182–188. ISSN: 0149-1970. DOI: <https://doi.org/10.1016/j.pnucene.2010.10.003>. URL: <https://www.sciencedirect.com/science/article/pii/S0149197010001587>.
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- [5] Ahmed Amin Abdelhameed et al. "High-Fidelity Multiphysics Modeling of Load Following for 3-D Gas-Cooled Microreactor Assembly using NEAMS Codes". In: *Proceedings of the American Nuclear Society Winter Conference*. 2022.

- [6] N. Stauff et al. *High-fidelity multiphysics load following and accidental transient modeling of microreactors using NEAMS tools*. Tech. rep. Argonne National Laboratory, Sept. 2023.
- [7] Edward M. Duchnowski et al. “Pre-conceptual high temperature gas-cooled microreactor design utilizing two-phase composite moderators. Part I: Microreactor design and reactor performance”. In: *Progress in Nuclear Energy* (2022).
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- [10] Kang Seog Kim. “Specification for the VERA Depletion Benchmark Suite”. In: (Dec. 2015). URL: [10.2172/1256820](https://doi.org/10.2172/1256820).
- [11] *Depletion Chains*. URL: <https://openmc.org/depletion-chains/>.



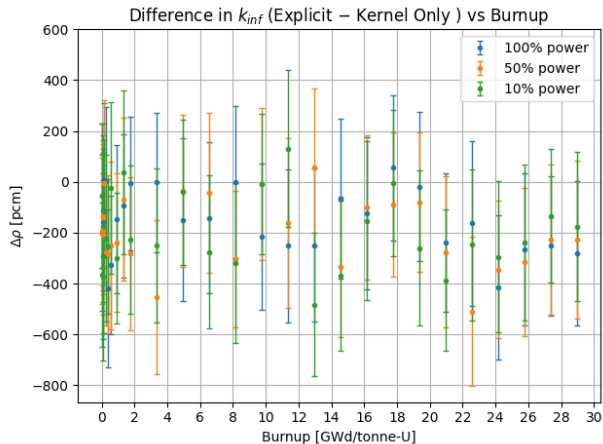
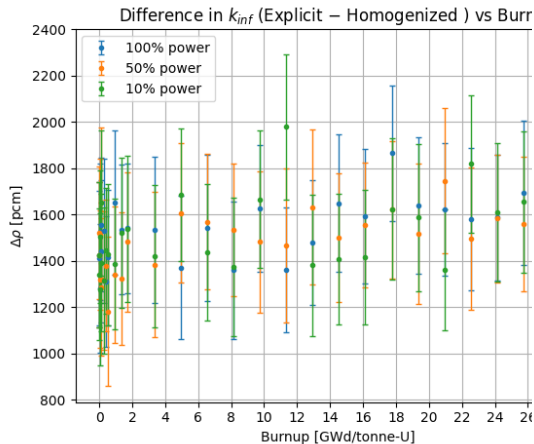
- OpenMC website: <https://openmc.org>
- OpenMC repository:  
<https://github.com/openmc-dev/openmc>
- OpenMC plotter:  
<https://github.com/openmc-dev/plotter>
- VTB:  
[https://mooseframework.inl.gov/virtual\\_test\\_bed](https://mooseframework.inl.gov/virtual_test_bed)
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- Add me on LinkedIn ([lewisgross1296](#)) and GitHub ([lewisgross1296](#))!



geometric parameters		
fuel compact radius	poison compact radius	moderator compact radius
0.90 cm	0.25 cm	0.843 cm
control compact radius	coolant compact radius	FeCrAl thickness
0.99 cm	0.60 cm	0.05 cm
Cr coating thickness	reflector heights	core height
0.007 cm	20 cm	160 cm

operation and design parameters		
fuel packing fraction	poison packing fraction	enrichment
40%	25%	19.95%
inlet temperature	outlet temperature	outlet pressure
873.15 K	1133.65 K	7 MPa

# $\Delta\rho$ with $2\sigma$ error as a function of burnup up to $\sim 29$ GWd/tonne-U



**Table:** All units are atom per cubic centimeter. Since the first five time steps are used to converge xenon, the numbers below are the average of the fifth to the last value for xenon number density.

representation	explicit	kernel only	homogenized
100% power	$2.43127 \times 10^{16}$	$2.41845 \times 10^{16}$	$1.19125 \times 10^{15}$
50% power	$1.31047 \times 10^{16}$	$1.30864 \times 10^{16}$	$6.45668 \times 10^{14}$
10% power	$2.82398 \times 10^{15}$	$2.81919 \times 10^{15}$	$1.38810 \times 10^{14}$

- These equilibrium values explain the observed trend at fresh fuel – and for much of the simulation – that lower power with the same total burnup has more excess reactivity.
- The higher the power, the more Xenon-135 is produced during the initial jump to steady state, contributing to a larger negative reactivity insertion.

# Plutonium 241 at Each Power for the Fully Explicit Case

