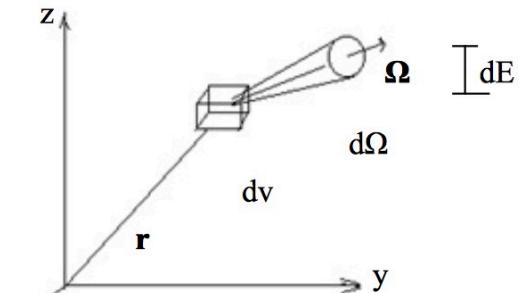


Rattlesnake: A MOOSE/LibMesh based Multiscale Neutronics Application

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The Radiation Transport Equation (RTE)

- Describes transport of uncharged particles (neutrons, photons) in a host medium
- Phase space 7-dimensional! Space (3), energy (1), direction of motion (3), time (1)
- Solve for angular flux ψ (speed * neutron density), typically # DoFs high



$$\frac{1}{v} \frac{\partial \psi}{\partial t} + \hat{\Omega} \cdot \nabla \psi + \sigma_t(\vec{r}, t)\psi(\vec{r}, E, \hat{\Omega}, t) = \int_0^\infty \int_{4\pi} \sigma_s(\vec{r}, t, E', \hat{\Omega}' \rightarrow E, \hat{\Omega})\psi(\vec{r}, E', \hat{\Omega}', t)d\hat{\Omega}'dE'$$

- Linear integro-differential equation, first order, hyperbolic
- Nonlinearity comes from coupling: cross sections depend on temperatures, density changes etc.

Alternative Forms of the RTE

1. Diffusion approximation:
 1. Approximation of transport equation => linearly anisotropic
 2. Diffusion type equation for each energy group
2. SAAF equations:
 1. 2nd order form of the transport equation
 2. Elliptical set of equation (as opposed to hyperbolic)
3. Operator Notation:

$$\frac{1}{v} \frac{\partial \psi}{\partial t} + \boxed{L}\psi = \boxed{S}\psi + \boxed{F}\psi$$

Streaming & Collision Scattering Fission
 Lower triangular

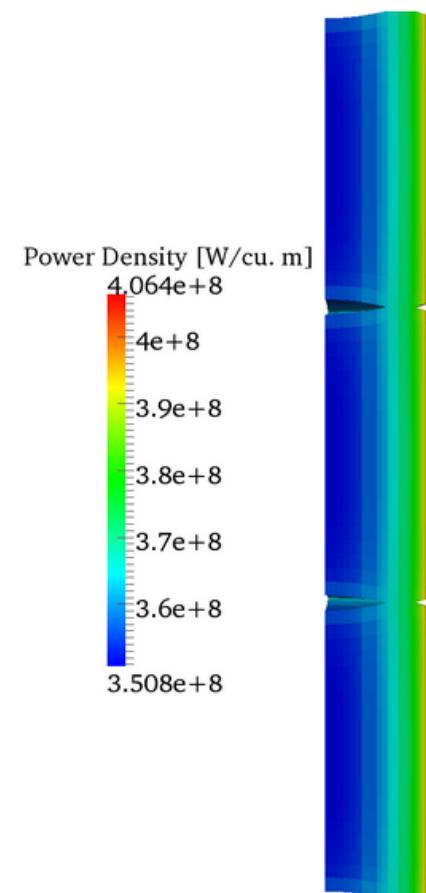
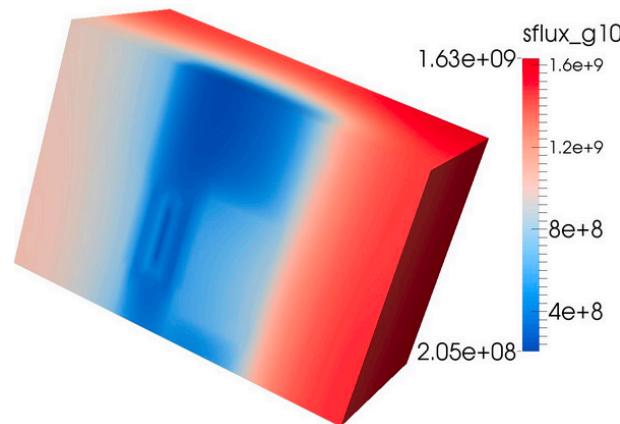
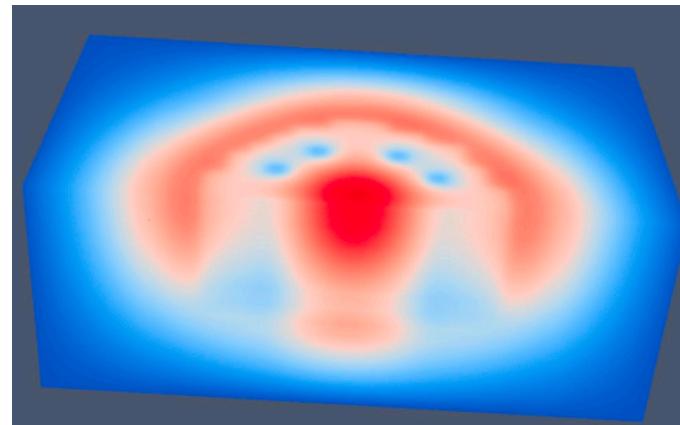
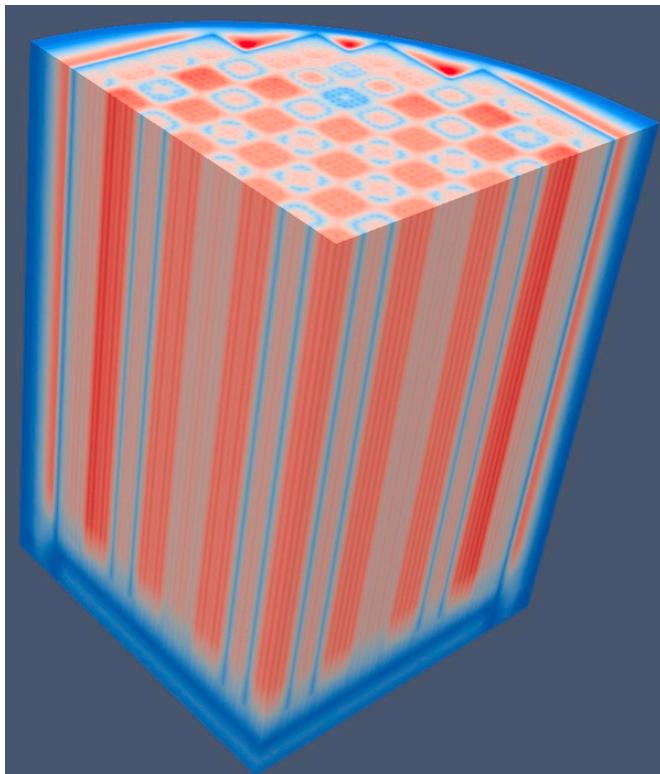
Rattlesnake Applications

PWR: BEAVRs benchmark

RMS: 4% (OPENMC: 5.4%)

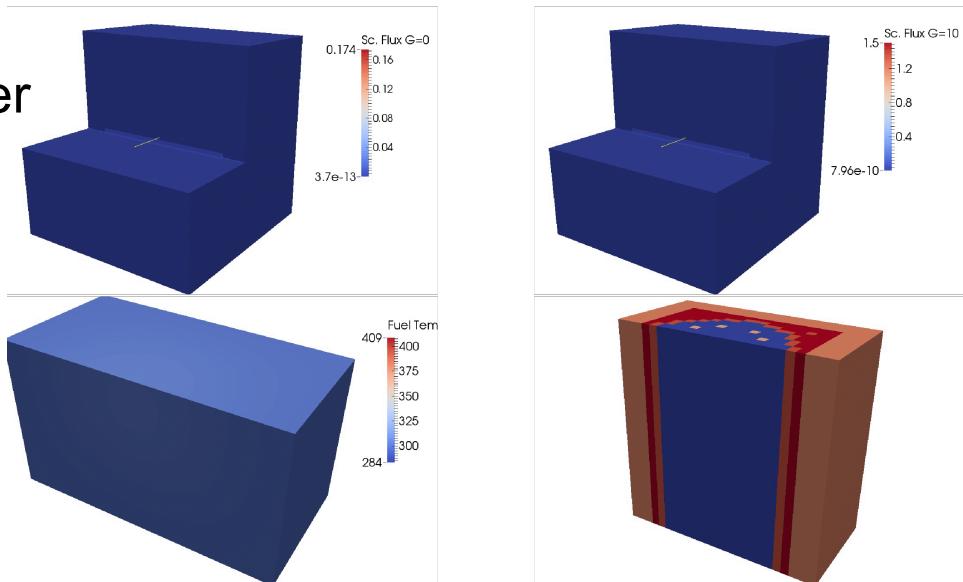
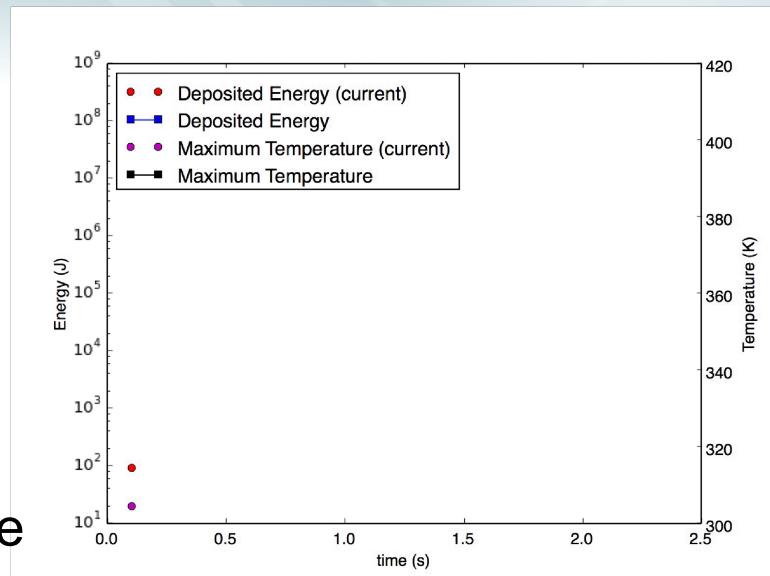
TREAT transient analysis & experiment modeling

Drive Fuel Performance



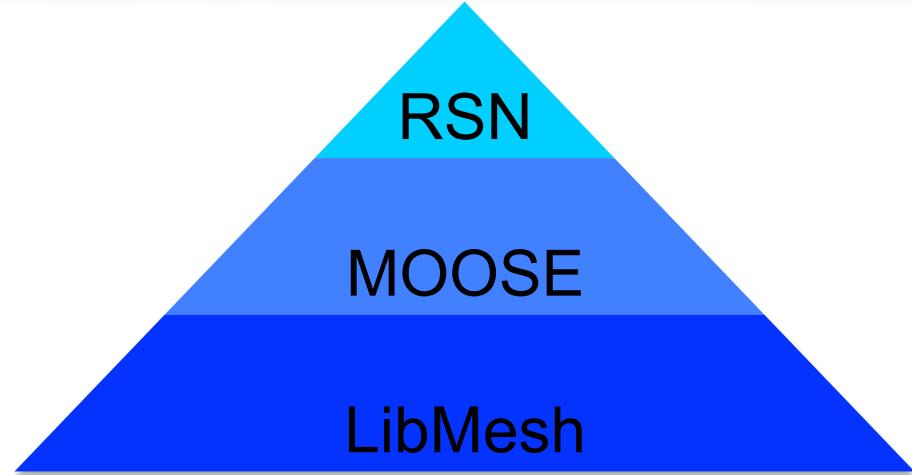
Transient-15

- TREAT: transient reactor for fuel testing
- Graphite with tiny fuel particles
- Experiment contains tested fuel
- Insert reactivity => power increase (10-20 GW)
- Thermal shift kill reactor power



Why is Rattlesnake different?

*Oscillating Godiva
Experiment*



Rattlesnake	Standard transport code
Built on LibMesh	Custom spatial discretization
Natural coupling with MOOSE apps	Customized via (fragile) interface
Full Newton, Picard, loose coupling	Loose coupling or Picard
Material properties evaluated at Qps	Constant element properties
2 nd order mixed mesh fully supported	1 st order mesh, one or few element types
Inherits timestepping, AMR etc.	Usually focused on transport only
Framework performance penalty	Streamlined

Flexible Nonlinear Diffusion Acceleration (NDA)

- Transport Update

$$Q = S\psi^{p,n+1} + F\psi^{p,n+1}$$

$$\psi^{p+1,n+1} = L^{-1}Q$$

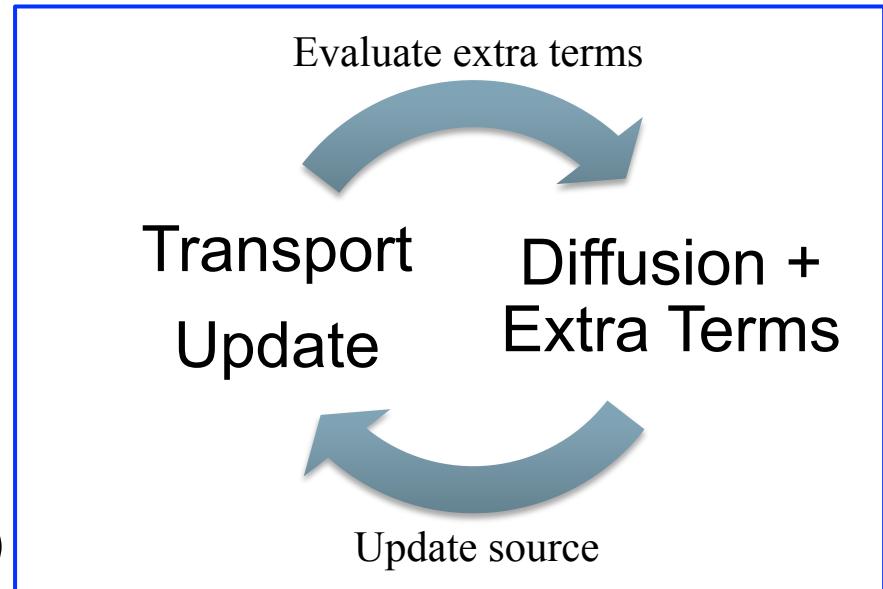
- L^{-1} computed via

- transport sweep (1st order)
- BoomerAMG V-cycles (SAAF)

- Flexible 1st order NDA method

- Transport: DGFEM discretization
- Diffusion: interior penalty DGFEM (advantage: linear/quadratic expansion)

- Allows transport & diffusion shape functions & mesh to be different

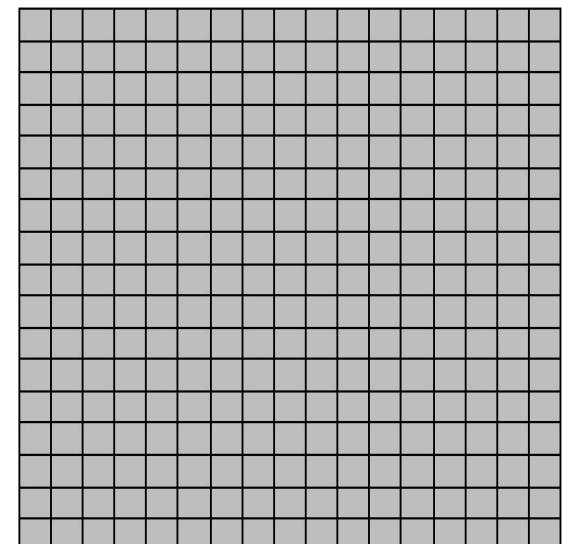
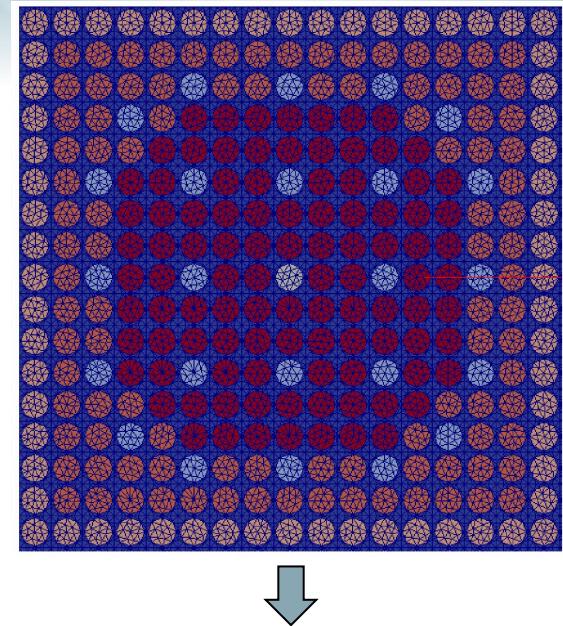
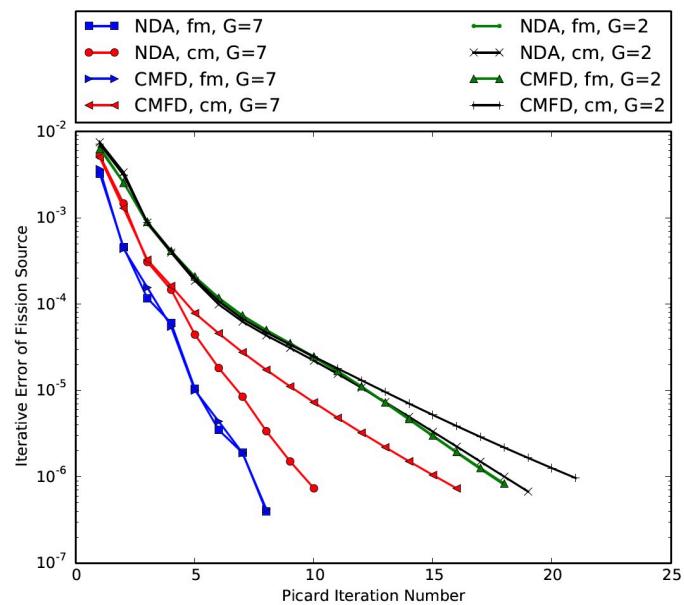


Picard iteration scheme

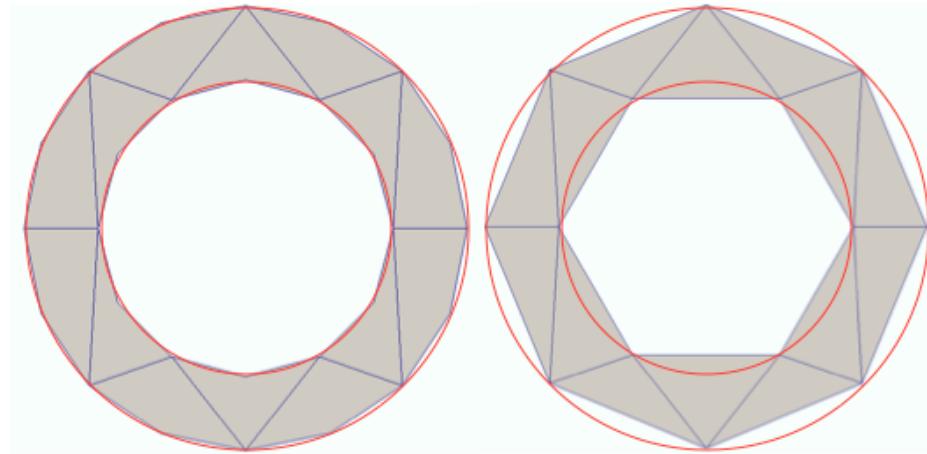
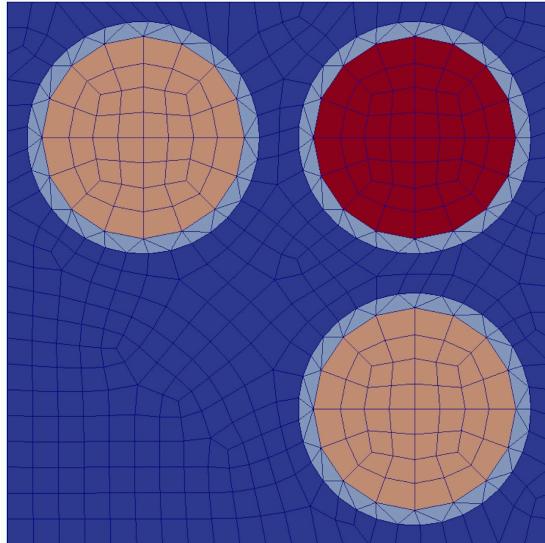
Flexible NDA – 2 (Example)

- Flexibility from FEM Projection & Prolongation
- C5G7 with pin-cell homogenized diffusion
- Results: Flexible NDA better than standard for pin-cell homogenized diffusion

	NDA	CMFD
Fine	8	8
Coarse	10	16



Second Order Sweeps - 1



2nd order

1st order

- 2nd order mesh: edges are not straight lines but 2nd order curves
- Highly beneficial for typical light water reactor geometries (pin/cladding)
- Ideas:
 - GMRES solution on 2nd mesh; precondition with sweep on 1st mesh
 - Use NDA [diffusion & transport source], sweep on 1st mesh
 - Both approaches are novel (to our knowledge)

Second Order Sweeps - 2

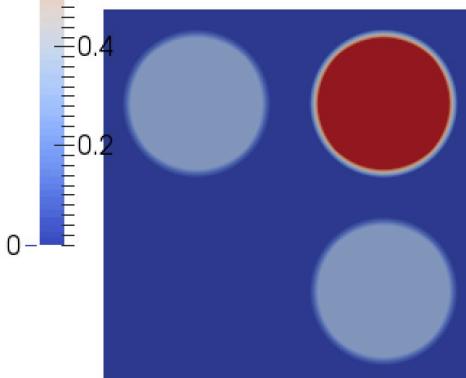
- Results using 1st order mesh, 2nd order mesh, 2nd mesh / sweep 1st mesh
- Multiplication factor $k_{\text{eff}} = \frac{\text{Neutron Production}}{\text{Neutron Losses}}$

keff	Direct (sweep prec.)	NDA	Reference
1 st order	0.710411	0.710411	0.733041
2 nd order	0.733685	0.733683	-
2nd order/ 1st sweep	0.733685	0.733683	-

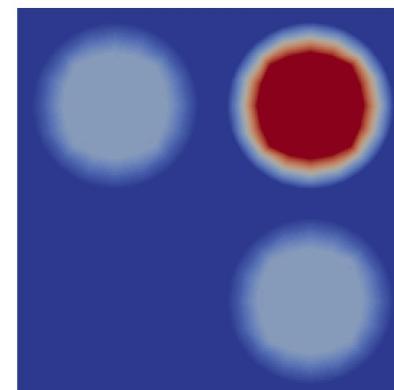
fission_source
0.884

Fission Source Distribution

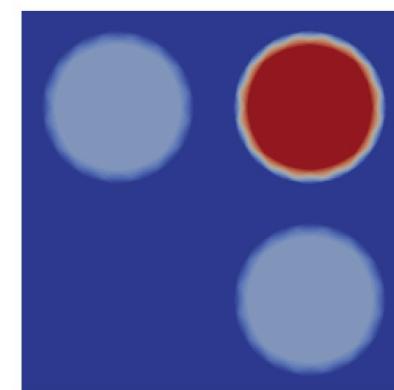
1st order reference



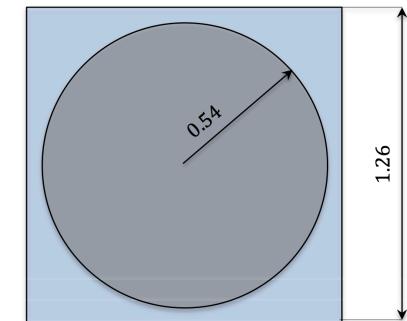
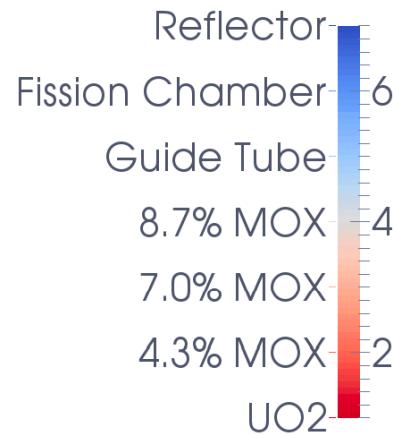
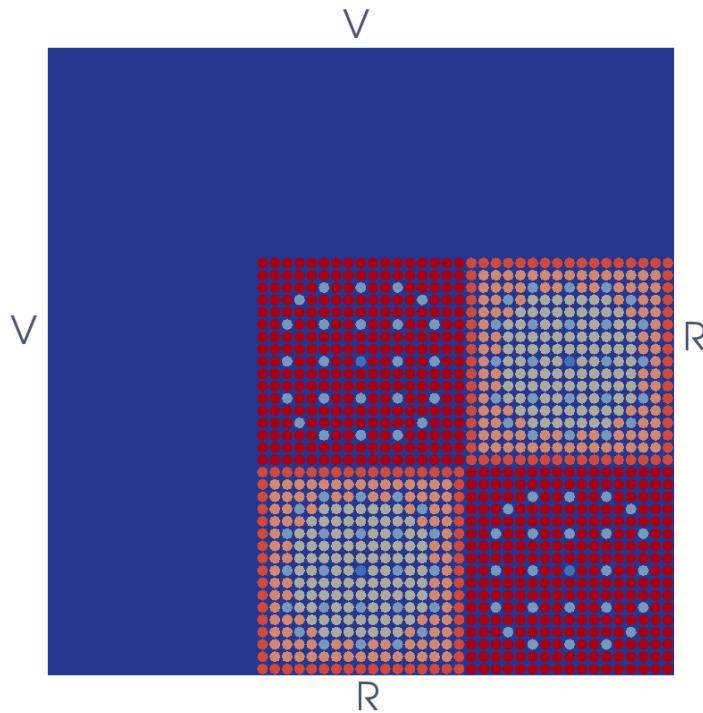
1st order



2nd order



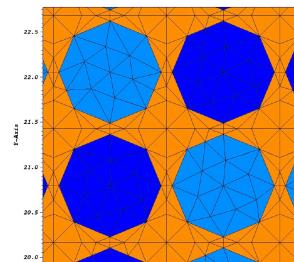
C5G7 Solutions with SAAF-S_N



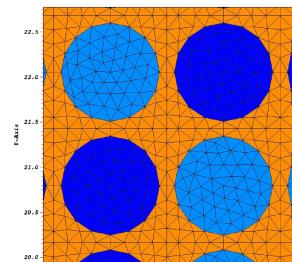
- C5G7 is a 2D neutronics benchmark, 2x2 assemblies minicore
- Rattlesnake SAAF-SN solution with up to 4.5×10^{10} DoFs (45 billion)
- Largest all-in-one PETSc system?

C5G7 Solutions with SAAF- S_N - Discretization

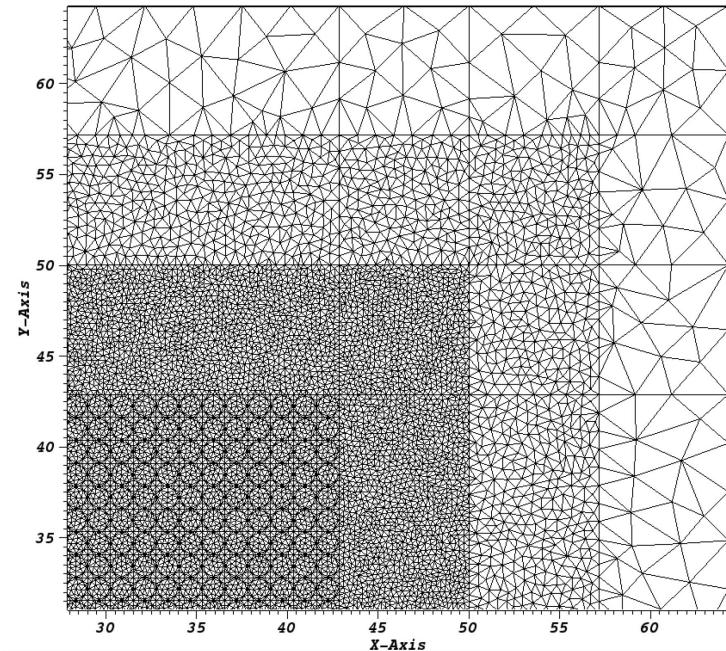
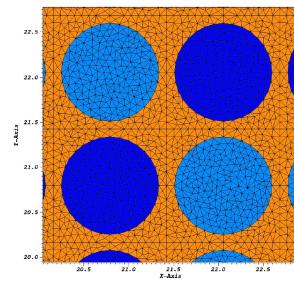
Mesh	#elem
8	87,108
16	265,577
32	1,033,668
64	4,216,587



(a) Mesh08

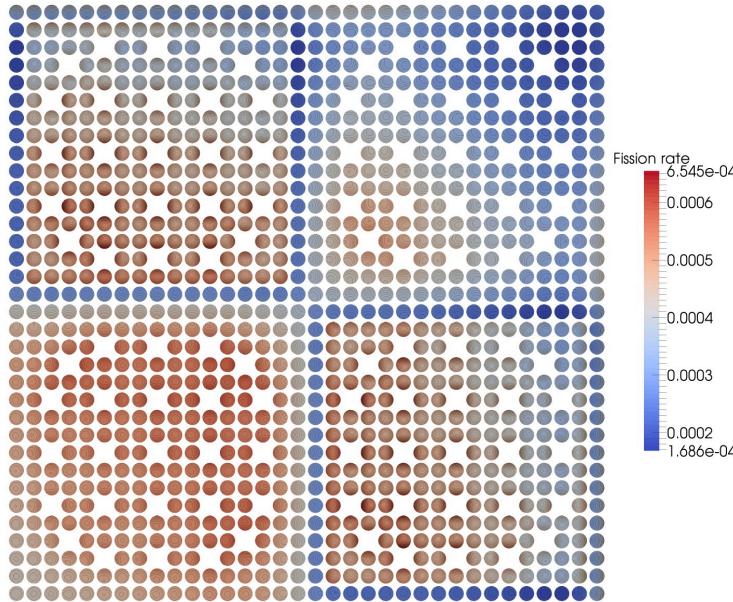


(b) Mesh16



- Angular Discretization (# direction): NA=8-64, Np=2-5, (1280 directions)
- SAAF a coupled system of elliptical equations
- K-eigenvalue calculations (forced steady-state) solved via NDA (L^{-1} via multiple V-cycles & BoomerAMG)

C5G7 Solutions with SAAF- S_N - Results



*Fission rate, Second order elements,
 Mesh 32, $N_A = 64, N_p = 4$*

Notable Results	
Error k_{eff} (%)	<10 ⁻³ %
Avg. Pin Power (%)	0.004
RMS Pin Power (%)	< 0.001
Mean Rel. Pin power (%)	0.005
# Picard NDA iterations	~7
Runtime (45 billion DoFs) (2376 CPUs)	5.6 h

- Reference: MCNP5 Monte Carlo calculation
- Nailed the eigenvalue to < 1pcm
- Nailed power distribution to < 0.001 %
- Efficient execution using < 10h & < 10 (NDA) Picard iterations

Conclusions

- Rattlesnake is a LibMesh/MOOSE based radiation transport solver
- Designed for multiphysics applications & unique coupling capabilities
- Strong focus on transient capabilities
- Unique examples in this talk
 - Effective & flexible NDA method & application to C5G7
 - Second order sweep capabilities (novel approach)
 - Largest (?) PETSc system ever solver in SAAF-S_N C5G7 solution
- Unique challenges: solve highly specialized, huge systems in a general framework!