# 2D $S_N$ with Diffusion Acceleration

MATH 676 - Milestone 1 Presentation

Logan H. Harbour

Department of Nuclear Engineering Texas A&M University



### One-group Linear Boltzmann Equation

Begin with the one-group  $S_N$  transport equation for a single direction d (neglecting boundary conditions for simplicity), as

$$\mathbf{\Omega}_d \cdot \nabla \psi_d(\mathbf{x}) + (\sigma_a(\mathbf{x}) + \sigma_s(\mathbf{x})) \psi_d(\mathbf{x}) - \frac{\sigma_s(\mathbf{x})}{2\pi} \sum_{d=1}^{N_{\Omega}} \omega_d \psi_d(\mathbf{x}) = q(\mathbf{x}), \quad (1)$$

where  $\sigma_a$  represents a probability of particle absorption and  $\sigma_s$  represents a probability of radiation scattering. Let  $\mathbb{T}_h$  be the set of all cells of the triangulation in a discontinuous approximation space. The DG weak form with test function  $v_d$  is

$$\sum_{K \in \mathbb{T}_{h}} \left[ \left( -\mathbf{\Omega}_{d} \cdot \nabla v_{d}, \psi_{d} \right)_{K} + \left( \psi_{d}^{+} \mathbf{\Omega}_{d} \cdot \mathbf{n}, v_{d} \right)_{\delta K} + \left( \sigma_{t} \psi_{d}, v_{d} \right)_{K} \right. \\
\left. - \left( \sigma_{s} \phi, v_{d} \right)_{K} = \left( q, v_{d} \right)_{K} \right], \quad (2)$$

where  $\phi$  is the scalar flux,  $\phi = \frac{1}{2\pi} \sum_{d}^{N_{\Omega}} \omega_{d} \psi_{d}$ , and  $\psi_{d}^{+}$  is the upwind value of  $\psi_{d}$  (the value from the side of the face in which  $\Omega \cdot \mathbf{n} \geq 0$ ).

#### Issue: Source Iteration

We commonly solve the transport equation by source iteration, a form of Richardson iteration. Cast Eq. (1) with iterative index  $\ell$  as

$$\mathbf{\Omega}_d \cdot \nabla \psi_d^{(\ell+1)} + \sigma_t \psi_d^{(\ell+1)} = \sigma_s \phi^{(\ell)} + q, \qquad (3)$$

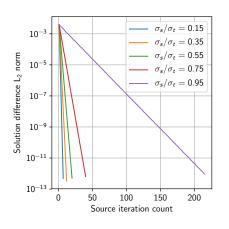
where  $\ell$  is the iterative index,  $\psi_d^{(0)} = \phi^{(0)} = \vec{0}$ . After solving each direction, d, for an iteration  $\ell$  in Eq. (3), update the scalar flux with

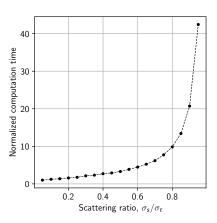
$$\phi^{(\ell+1)} = \frac{1}{2\pi} \sum_{d=1}^{N_{\Omega}} w_d \psi_d^{(\ell+1)}.$$

 $\psi^{(\ell+1)}$  is the particles that have scattered at most  $\ell$  times. As  $\sigma_s/\sigma_t \to 1$ , particles scatter more before they are absorbed  $\to$  **the number of source iterations becomes significant!** This problem becomes the goal of this work: introduce a diffusion problem as a preconditioner for Eq. (3).

### **Example: Lots of Scattering**

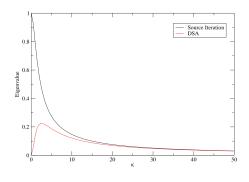
Introduce  $\mathcal{D}=[0,10]^2$ ,  $N_{\Omega}=20$ , q=1,  $\sigma_a+\sigma_s=\sigma_t=100$ , and  $64^2$  elements. Increase the scattering ratio,  $\sigma_s/\sigma_t$  and observe results.





#### Diffusion Acceleration

- The source iteration process will converge quickly whenever particles scatter just a few times on average before being absorbed or escaping.
- It can converge very slowly in diffusive problem, as particles scatter an arbitrary number of times on average before being absorbed or escaping.



## Project Progress and Future Goals

#### Completed works

- A one-group, 2D neutron transport code using the  $S_N$  approximation has been developed using linear discontinuous finite elements in Deal.ii.
- Verified using known constant source solutions and MMS.
- Primarily uses the MeshWorker interface as discussed in step-12.
- Utilizes downstream\_renumbering to precondition the within-direction solve.