### 1a What are the major nuclear data libraries and which countries manage them?

ENDF (latest version ENDF/B -VIII) produced by the Cross Section Evaluation Working Group (US & Canada)

JEFF (latest version JEFF3.3) from the Joint Evaluated Fission and Fusion File Organization, consisting of members of the NEA Data Bank participating countries (Argentina, Austria, Belgium, Czech Rep., Denmark, Finland, France, Germany, Greece, Hungary, Italy, Japan, Korea, Mexico, Netherlands, Norway, Poland, Portugal, Romania, Russia, Slovak Rep., Slovenia, Spain, Sweden, Switzerland, Turkey, UK)

JENDL (latest version JENDL-4.0) is handled by the Japanese Nuclear Data Committee (Japan)

BROND (latest version BROND-3.1) and RUSFOND (2010) compiled by Russian Federation

CENDL (latest version CENDL-3.1) by China

#### 1b At what energy is the lowest isolated resonance of 235U, 238U, 239Pu, 240Pu, 241Pu, and 242Pu?

	U-235	U-238	Pu-239	Pu-240	Pu-241	Pu-242
ENDF	0.276 ev	6.674 ev	0.295 ev	1.056 ev	0.256 ev	2.676 ev
JEFF	0.275 ev	6.674 ev	0.301 ev	1.056 ev	0.257 ev	2.670 ev
JENDL	0.279 ev	6.673 ev	0.296 ev	1.055 ev	0.257 ev	2.680 ev

Source: <a href="https://www-nds.iaea.org/exfor/e4explorer.htm">https://www-nds.iaea.org/exfor/e4explorer.htm</a>

#### 1c Why do we care about that?

In correspondence of the resonance, we lose the typical 1/v behavior and typically absorption of a neutron is much more likely, therefore we observe a depression of the neutron flux at that energy. This is relevant because it represents the component of flux which is depressed in the first layers of a fuel rod (self-shielding) and the behavior of this resonance when T changes will be determinant for the Doppler Effect.

# 2a Briefly compare the diffusion equation, deterministic methods, and Monte Carlo methods in terms of complexity, accuracy, run time, and range of applicability.

	Diffusion Equation	Deterministic Method	MC method
Complexity	Least complex, solution by hand in some cases	Complex, multiple equations to be performed	Most complex: multiple generations of neutrons to be simulated
Accuracy	Least accurate, with discretized or homogeneous space, linearly anisotropic or isotropic direction and up to few energy groups	Discretized space, discretized direction and multiple energy groups	Most accurate, with continuous spatial, directional and energy distribution
Run Time	Fast running	Not fast running, but less time demanding than MC	Computationally expensive, parallelization often necessary
Range of Applicability	Simple geometries, not valid close to boundaries, sinks, sources	1D, 2D and 3D geometries. Not as limited as diffusion equation. Some approaches	All geometries, useful for studying physical objects such as reactors

become unco	onvenient and
difficult in mu	ulti-D problems
(e.g. spher-ha	arm)

## 2b Given what you've learned about deterministic methods so far, discuss their strengths and weaknesses

Deterministic methods have some important strengths:

- 1) Relatively fast running, especially if compared to MC methods
- 2) They have a Global solution in output, therefore they have the same quality everywhere
- 3) Inputs of codes based on deterministic methods can be simple

However, they bear some weaknesses:

- 1) The quality of the solution is governed by the discretization of the variable
- 2) The are memory-intensive, especially in case of large problems.
- 3) Their accuracy suffers from truncation error
- 4) Angle discretization causes ray effect at angles different from the co-location points
- 5) They are constrained by their mesh
- 6) It is not simple to parallelize their execution

# 3a What is the main challenge associated with the way that we derived the multigroup transport equation?

The main challenge implied in the way we derived the multigroup transport equation stands in how we computed the group constants. As a matter of fact, we defined them by weighting the nuclear parameters on the angular flux energy distribution, which is an unknown variable of the transport equation.

$$\Sigma_g = \frac{\int_{E_g}^{E_{g-1}} dE \ \Sigma(\boldsymbol{r}, E) \psi(\boldsymbol{r}, \boldsymbol{\Omega}, E)}{\int_{E_g}^{E_{g-1}} dE \ \psi(\boldsymbol{r}, \boldsymbol{\Omega}, E)}$$

Therefore, we will have to use general guess fluxes for averaging the cross sections and building the group constants.

### 3b Starting from the following general system of multigroup equations

$$\begin{split} \hat{\Omega} \cdot \nabla \psi^g(\vec{r}, \hat{\Omega}) + \Sigma_t^g(\vec{r}) \psi^g(\vec{r}, \hat{\Omega}) = \\ \sum_{g'=0}^{G-1} \Sigma_s^{g' \to g}(\vec{r}, \hat{\Omega}' \cdot \hat{\Omega}) \psi^{g'}(\vec{r}, \hat{\Omega}') + \frac{\chi_g}{4\pi} \sum_{g'=0}^{G-1} \nu_{g'} \Sigma_f^{g'}(\vec{r}) \phi^{g'}(\vec{r}) + \frac{Q^g(\vec{r})}{4\pi} \,, \end{split}$$

where  $\phi$  is scalar flux, write a set of five coupled equations for a five-group problem. Assume neutrons can only downscatter from fast groups (0 and 1) to thermal groups (2, 3, and 4). Assume that thermal groups can upscatter into other thermal groups and can downscatter. Assume both an external source and a fission source.

See attached document (page 1)

## 4. Show that the Legendre polynomials below are orthonormal on the interval $x \in [-1, 1]$

See attached (page 2)

#### 5. Consider the integral

$$\int_{4\pi} d\hat{\Omega} |\hat{\Omega}|.$$

- a. Use the S4 LQN quadrature set to execute this integral. The LQN quadrature set is given below in Figure 1. Recall that  $\mu i = \eta i = \xi i$  for a given level i.
- b. Repeat the integration with S6. What do you observe?
- c. Write a short code to execute this integration (and higher orders if you'd like). Try a few different functions. Turn in the code and the evaluation of these functions. Include comments on what you observe.
- (a) See attached (page 3)
- (b) See attached. (page 3)
  We observe that the error has slightly risen when passing from s4 to s6
- (c) See attached .m code.

**Evaluations:** 

```
1) f=1;

>> fun1 = @(mu,csi,nu) 1;

>> Sn_Quad(fun1,4)

ans = 12.566369357722113

>> Sn_Quad(fun1,6)

ans = 12.566373127633305

>> Sn_Quad(fun1,12)

ans = 12.566373127633312

>> Sn_Quad(fun1,16)

ans = 12.566361817899761
```

We see that the variability of the result is fairly reduced, as we had experienced in the hand-written calculation.

## 2) f=abs(mu+3\*csi-2\*nu);

```
>> fun2 = @(mu,csi,nu) abs(mu+3*csi-2*nu);

>> Sn_Quad(fun2,4)

ans = 23.683206088790772

>> Sn_Quad(fun2,6)

ans = 23.637338603274593

>> Sn_Quad(fun2,8)

ans = 23.440320351372318

>> Sn_Quad(fun2,16)

ans = 23.523158887315734

>> Sn_Quad(fun2,12)

ans = 23.515749271959240
```

With a less isotropic function, we observe a larger variability of results, but for larger n (n=12 and n=16), the results are pretty similar

### 3) f= mu.^2 +csi.^3;

```
>> fun3= @(mu,csi,nu) mu.^2 +csi.^3;

>> Sn_Quad(fun3,4)

ans = 4.188789785907369

>> Sn_Quad(fun3,6)

ans = 4.188791042544431

>> Sn_Quad(fun3,8)
```

```
ans = 4.188789785907371
```

>> Sn\_Quad(fun3,12)

ans = 4.188791042544434

>> Sn\_Quad(fun3,16)

ans = 4.188787272633244

fun3 does not have any type of symmetry (nor even, nor odd). Again, the different Sn integrations lead to very close results, with variability just at the fifth decimal figure

$$\begin{split} &\hat{\Omega} \cdot \nabla A^{o} + \Xi_{c}^{a} \cdot A^{o} = \Xi_{s}^{0 * o} (\bar{r}, \hat{\Omega}^{i}, \hat{\Omega}) A^{o} (\bar{r}, \hat{\Omega}^{i}) + \frac{\chi^{o}}{4\pi} \int_{g-o}^{4} A_{i}^{b} \Sigma_{g}^{a}(\bar{r}) \beta^{g}(\bar{r}) + \frac{Q^{o}}{4\pi} \\ &\hat{\Omega} \cdot \nabla A^{o} + \Sigma_{c}^{a} A^{o} = \sum_{s=0}^{4} \sum_{s=0}^{4} A_{s}^{a} (\bar{r}, \hat{\Omega}^{i}, \hat{\Omega}) A^{g} (\bar{r}, \hat{\Omega}^{i}) + \frac{\chi^{o}}{4\pi} \int_{g-o}^{4} \Sigma_{s}^{a}(\bar{r}) \beta^{g}(\bar{r}) + \frac{Q^{o}}{4\pi} \\ &\hat{\Omega} \cdot \nabla A^{o} + \Sigma_{c}^{a} A^{o} = \sum_{g=0}^{4} \sum_{s=0}^{4} \sum_{s=0$$

$$(2, 6) = \int_{1}^{1} \frac{1}{\sqrt{2}} \cdot \frac{1}{\sqrt{2}} dx = \frac{1}{2} [x]_{-1}^{1} = 1$$

$$.\langle P_0, P_3 \rangle = \int_{-1}^{1} \frac{1}{\sqrt{2}} \cdot \underbrace{\int_{1}^{3} \times dx}_{12} = \underbrace{\int_{2}^{2} \left[ \frac{x^2}{2} \right]_{-1}^{1}}_{12} = 0$$

$$\cdot \langle P_0, P_2 \rangle = \int_{-1}^{1} \frac{1}{\sqrt{2}} \sqrt{\frac{5}{2}} \left( \frac{3x^2 - 1}{2} \right) = \frac{\sqrt{5}}{4} \left[ x^3 - x \right]_{-1}^{1} = 0$$

$$\langle P_1, P_1 \rangle = \int_{-1}^{1} \sqrt{\frac{3}{2}} \times \sqrt{\frac{3}{2}} \times dx = \frac{3}{2} \left[ \frac{x^3}{3} \right]_{-1}^{1} = \frac{1}{2} + \frac{1}{2} = 1$$

$$= \sqrt{\frac{1}{2}} \sqrt{\frac{3}{2}} \sqrt{\frac{3}{$$

$$= \frac{5}{8} \left[ \frac{9}{5} \times^{5} - \frac{2}{8} \times^{3} + \times \right]_{-1}^{1} = \frac{5}{8} \left( \frac{9}{5} + \frac{1}{4} \times \frac{9}{5} + \frac{1}{4} \times \frac{1}{5} \right) = \frac{5}{8} \cdot \left( \frac{18}{5} - 2 \right)$$

$$=\frac{5}{8} \cdot \frac{8}{5} = 1$$

