

# PhD Defense:

## Finite Elements with Discontiguous Support for Energy Discretization in Particle Transport

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# Outline

## ① Introduction and Motivation

## ② Method

- A finite element in energy
- Researching and inventing our way out of problems

## ③ Results

- A neutron time-of-flight problem
- A reactor problem with comparisons to MCNP

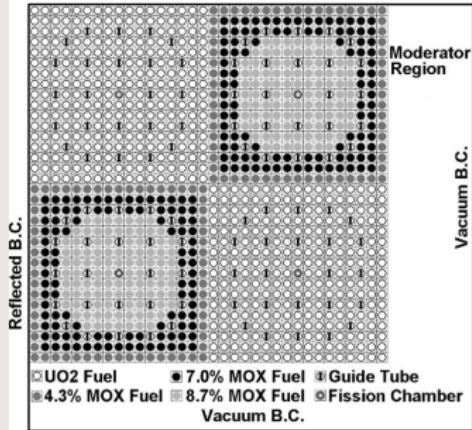
## ④ Conclusions

- . . . and continuations

We desire accurate and inexpensive quantities of interest (QOI) for neutron problems, which requires accurate and efficient simulation of neutron transport

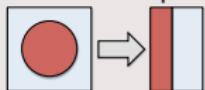
## Reactor applications

## QOI: Pin powers, criticality eigenvalue Reflected B.C.



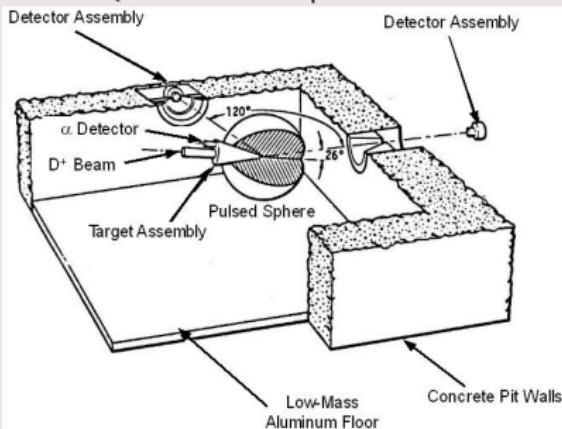
M.A. Smith et al., *Prog. in Nuclear Energy*, Vol. 45, 2004

## Simplified reactor problems



Time-dependent neutronics problems

### QOI: Detector response in time



R.J. Procassini, M.S. McKinley, LLNL-PROC-453212, SNA+MC2010

## Simplified TOF problem



Problem: How to discretize the energy variable accurately without requiring an  $\mathcal{O}(10,000)$  energy unknown count?

This research tackles a long-standing issue in neutron transport through a series of “problem-research-solution” cycles

Our solution: Divide in “solution space” instead of energy space using our **Finite-Element-with-Discontiguous Support (FEDS)** method

The linear Boltzmann transport equation describes the interaction of radiation quanta with their environment

### The neutron transport equation

(radiative transfer equation is similar; no material motion or neutron-neutron interactions are considered;  $q_{\text{coll}}$  term includes fission)

$$\left[ \frac{1}{v(E)} \partial_t + \boldsymbol{\Omega} \cdot \nabla + \Sigma_t(\mathbf{r}, E, t) \right] \psi(\mathbf{r}, E, \boldsymbol{\Omega}, t) = q_{\text{coll}}(\mathbf{r}, E, \boldsymbol{\Omega}, t) + q_{\text{ext}}(\mathbf{r}, E, \boldsymbol{\Omega}, t)$$

$$q_{\text{coll}}(\mathbf{r}, E, \boldsymbol{\Omega}, t) = \sum_i N_i(\mathbf{r}, t) \int_{4\pi} d\Omega' \int_0^\infty dE' \sigma_{s,i}(E' \rightarrow E, \boldsymbol{\Omega}' \cdot \boldsymbol{\Omega}, T(\mathbf{r}, t)) \psi(\mathbf{r}, E', \boldsymbol{\Omega}', t)$$

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$$q_{\text{coll}}(\mathbf{r}, E, \Omega, t) = \sum_i N_i(\mathbf{r}, t) \int \frac{d\Omega'}{4\pi} \int_0^\infty dE' \sigma_{s,i}(E' \rightarrow E, \Omega' \cdot \Omega, T(\mathbf{r}, t)) \psi(\mathbf{r}, E', \Omega', t)$$

$$\text{Total cross section} = \Sigma_t(\mathbf{r}, E, t) = \sum_i N_i(\mathbf{r}, t) \sigma_{t,i}(E, T(\mathbf{r}, t))$$

$$\text{Solution ("Scalar flux")} = \phi(\mathbf{r}, E, t) = \int_{4\pi} d\Omega \psi(\mathbf{r}, E, \Omega, t)$$

$$\text{Quantities of interest (QOI)} = R_{i,n} \propto \int_{\Delta t_n} dt \int_{V_i} d^3r \int_0^\infty dE \phi(\mathbf{r}, E, t) \Sigma_x(\mathbf{r}, E, t)$$

$$\text{or} \propto \int_{\Delta t_n} dt \int_{S_i} d^2 r \int_0^\infty dE \int_{\hat{n} \cdot \Omega > 0} d\Omega (\hat{n} \cdot \Omega) \psi(\mathbf{r}, E, \Omega, t)$$

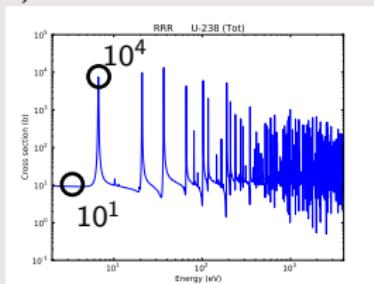
Problem: The solution is strongly impacted by resonances in the material cross sections (XS)

## Reactor problems

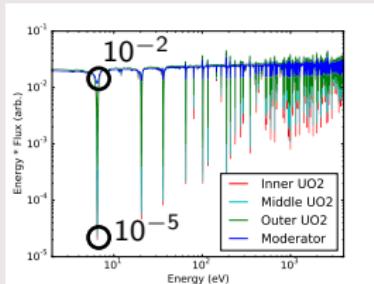
$$[E\phi]_{\text{outside of fuel}} \approx 1$$

$$[E\phi]_{\text{inside of fuel}} \simeq 1/\Sigma_t(E)$$

$\sigma_t(E)$  for U-238 in the resonance range



### Flux in the resonance range

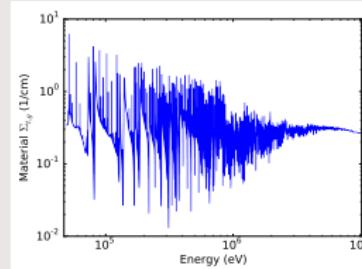


## Time-of-flight (TOF) problems

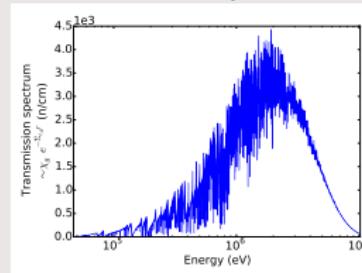
$$\text{Absorption} \propto \chi(E) \left(1 - e^{-\Sigma_t(E)X}\right)$$

$$\text{Transmission} \propto \chi(E) e^{-\Sigma_t(E)X}$$

$\Sigma_t(E)$  for natural iron



## Transmission spectrum



MG solution: Discretize in energy by grouping together particles with similar energies

This multigroup (MG) approach averages material parameters within each group

## The neutron transport equation

$$\left[ \frac{1}{v(E)} \partial_t + \boldsymbol{\Omega} \cdot \nabla + \Sigma_t(\mathbf{r}, E, t) \right] \psi(\mathbf{r}, E, \boldsymbol{\Omega}, t) = q_{\text{scat}}(\mathbf{r}, E, \boldsymbol{\Omega}, t) + q_{\text{ext}}(\mathbf{r}, E, \boldsymbol{\Omega}, t)$$

## The MG neutron transport equation

$$\left[ \frac{1}{v_g} \partial_t + \boldsymbol{\Omega} \cdot \nabla + \Sigma_{t,i,g}(t) \right] \psi_g(\mathbf{r}, \boldsymbol{\Omega}, t) = q_{\text{scat},g}(\mathbf{r}, \boldsymbol{\Omega}, t) + q_{\text{ext},g}(\mathbf{r}, \boldsymbol{\Omega}, t)$$

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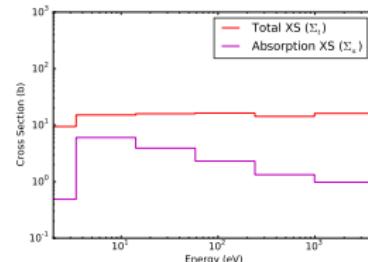
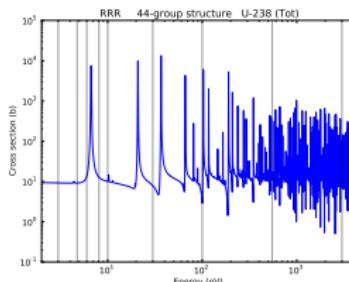
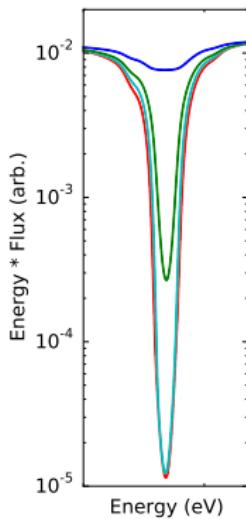
## The MG neutron transport equation

$$\left[ \frac{1}{v_g} \partial_t + \boldsymbol{\Omega} \cdot \nabla + \Sigma_{t,i,g}(t) \right] \psi_g(\mathbf{r}, \boldsymbol{\Omega}, t) = q_{\text{scat},g}(\mathbf{r}, \boldsymbol{\Omega}, t) + q_{\text{ext},g}(\mathbf{r}, \boldsymbol{\Omega}, t)$$

$$\Sigma_{t,i,g}(t) \equiv \frac{\int_{\Delta E_g} dE f_i(E) \Sigma_t(\mathbf{r} \in V_i, E, t)}{\int_{\Delta E_g} dE f_i(E)}$$

$$\psi_g(\mathbf{r}, \boldsymbol{\Omega}, t) \simeq \int_{\Delta E_g} dE \psi(\mathbf{r}, E, \boldsymbol{\Omega}, t) \quad \text{if} \quad f_i(E) \simeq \psi(\mathbf{r}, E, \boldsymbol{\Omega}, t) \quad \text{or} \quad \Delta E_g \text{ is small}$$

Problem: We cannot afford to resolve cross sections in energy space and the solution is not separable in energy



Usually impractical to make  $\Delta E_g$  small wrt to resonance widths (top)

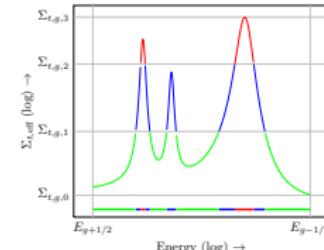
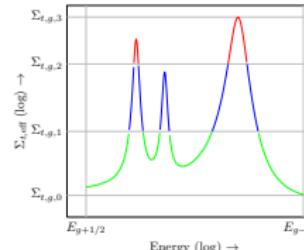
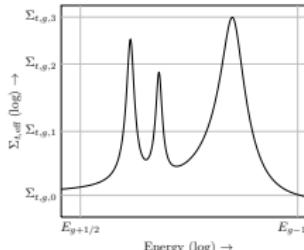
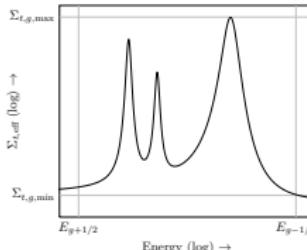
- There are many resonances:  $\mathcal{O}(1,000) - \mathcal{O}(10,000)$
- We must resolve a multiscale 7-D phase space ( $\mathbf{r}, E, \Omega, t$ )

$f_i(E) \neq \psi(\mathbf{r}, E, \Omega, t)$  due to self-shielding effects (left)

- The spectrum changes within a material region and in angle
- We can preserve bulk reaction rates if  

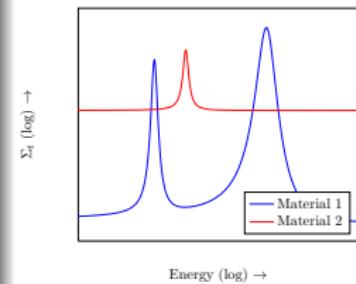
$$\int_{V_i} d^3r \int_{4\pi} d\Omega \int_{\Delta E_g} dE [f_i(E) - \psi(\mathbf{r}, E, \Omega, t)] \simeq \text{small}$$
- Above does not work for quantities such as leakage

# MB solution: Resonance resolution at low unknown count requires discretizing cross section space



## Problem: Solution depends on more than one total XS

- The MB family discretizes total XS space through banding (top)
- $[\Omega \cdot \nabla + \Sigma_t(\mathbf{r}, E, t)] \psi(\mathbf{r}, E, \Omega, t) = q(\mathbf{r}, E, \Omega, t)$ 
  - Solution depends on  $\Sigma_t$  not any  $\sigma_t$
  - Solution depends on the source,  $q$ , which is not smooth in energy (scattering resonances)
  - Solution depends on upstream  $\Sigma_t$  and  $q$
- Banding does not work when multiple resonant materials / nuclides are simultaneously considered (right)



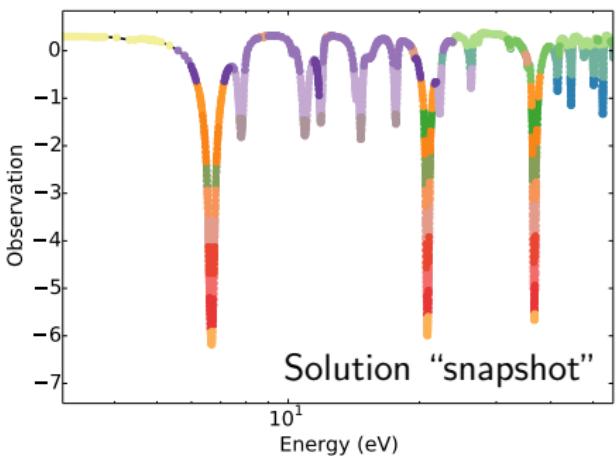
# Outline

## Up Next

- A high-level overview of FEDs
- Details of FEDS

FEDS solution: Our new method overcomes the drawbacks of MG and MB

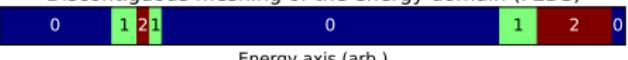
- We divide in solution space, not energy or XS space (top)
  - Our method is defined in energy space (bottom)
  - We naturally handle multiple resonant inputs  
(minimize the spectral variance in each energy unknown over all snapshots)
  - Our method often achieves  $\mathcal{O}(10^{-4})$  relative accuracy with  $\mathcal{O}(200)$  energy unknowns
  - Less sensitive to weighting spectrum,  $f_i(E)$ , for XS condensation than is MG



### Contiguous meshing of the energy domain (MG)



#### Discontiguous meshing of the energy domain (FEDS)



Our method is the Finite-Element with Discontiguous Support (FEDS) method

FEDS is a Petrov-Galerkin finite element method in energy

This is our *only* approximation

$$\psi_{\text{exact}}(\mathbf{r}, E, \Omega, t) \simeq \psi_{\text{FEDS}}(\mathbf{r}, E, \Omega, t) \equiv \sum_k b_k(\mathbf{r}, E) \psi_k(\mathbf{r}, \Omega, t),$$

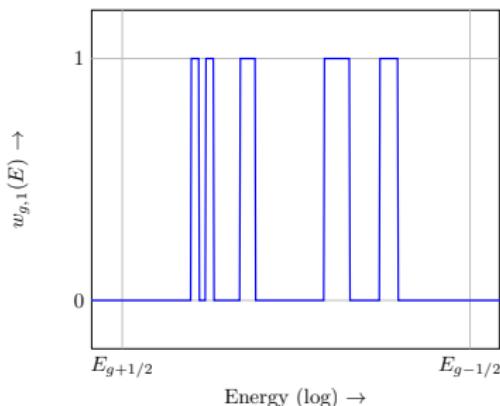
## Our weight functions (left)

$$w_k(E) = \begin{cases} 1 & \text{if } E \in \mathbb{E}_k, \\ 0 & \text{otherwise,} \end{cases}$$

## Our basis functions

$$b_k(\mathbf{r} \in V_i, E) = \begin{cases} C_{i,k} f_i(E) & E \in \mathbb{E}_k, \\ 0 & \text{otherwise,} \end{cases}$$

where the  $f_i(E)$  are approximate solution spectra (usually from simpler calculations) and  $C_{i,k} = \frac{1}{\int_{E_k} dE f_i(E)}$ .



We derive a weak form of the transport equation

We begin with the continuous-energy neutron transport equation

$$\boldsymbol{\Omega} \cdot \nabla \psi(\mathbf{r}, E, \boldsymbol{\Omega}) + \Sigma_t(\mathbf{r}, E) \psi(\mathbf{r}, E, \boldsymbol{\Omega}) = \frac{1}{4\pi} \int_0^\infty dE' \Sigma_s(\mathbf{r}, E' \rightarrow E) \phi(\mathbf{r}, E') + \frac{\chi(\mathbf{r}, E)}{4\pi k_{\text{eff}}} \int_0^\infty dE' \nu \Sigma_f(\mathbf{r}, E') \phi(\mathbf{r}, E')$$

We test the transport equation against the weight functions and expand the fluxes into their basis function representations

- 1 -

After algebraic manipulation we get what looks like a standard MG formulation

$$\Omega \cdot \nabla \psi_k(\mathbf{r}, \Omega) + \Sigma_{t,k,i} \psi_k(\mathbf{r}, \Omega) = \frac{1}{4\pi} \sum_{k'} \Sigma_{s,k' \rightarrow k,i} \phi_{k'}(\mathbf{r}) + \frac{\chi_{k,i}}{4\pi k_{\text{eff}}} \sum_{k'} \nu \Sigma_{f,k',i} \phi_{k'}(\mathbf{r})$$

# The FE definitions give us expressions for the cross sections and unknowns

Cross sections are now averaged over discontinuous energy domains instead of continuous ones

$$\Sigma_{t,k,i} \equiv \int_0^\infty dE \, b_k(\mathbf{r}, E) \bar{\Sigma}_{t,i}(E),$$

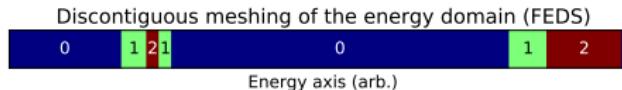
$$\chi_{k,i} \equiv \int_0^\infty dE \, w_k(E) \bar{\chi}_i(E),$$

$$\Sigma_{s,k' \rightarrow k,i} \equiv \int_0^\infty dE' \, b_{k'}(\mathbf{r}, E') \int_0^\infty dE \, w_k(E) \bar{\Sigma}_{s,i}(E' \rightarrow E),$$

$$\psi_k(\mathbf{r}, \Omega, t) \equiv \int_0^\infty dE \, w_k(E) \psi(\mathbf{r}, E, \Omega, t),$$

$w_k(E)$  and  $b_k(\mathbf{r}, E)$  are nonzero only for  $E \in \mathbb{E}_k$ ,

The  $\mathbb{E}_k$  are discontiguous energy ranges (e.g., see below)



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# Problem: How to discretize the energy variable accurately without requiring an $\mathcal{O}(10,000)$ energy unknown count?

This research tackles a long-standing issue in neutron transport through a series of problem-research-solution cycles

Our solution: Divide in solution space instead of energy space using our **Finite-Element-with-Discontiguous Support (FEDS) method**

- I How to determine the energy mesh ( $\mathbb{E}_k$ )?
- II How to determine the condensing spectrum ( $f_i(E)$ )?
- III How to assess the method?

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## I How to determine the energy mesh ( $\mathbb{E}_k$ )?

- ① How do we divide in solution space when there is more than one spectrum?  
**Minimize a projection error to the coarse grid**
- ② How do we form accurate spectra for the minimization problem?  
**Take advantage of decades of transport and MG work**
- ③ How do we solve this minimization problem efficiently?  
**Use clustering algorithms from machine learning**
- ④ How do we solve our discrete transport equation in energy?  
**Employ traditional MG solvers**

## II How to determine the condensing spectrum ( $f_i(E)$ )?

## III How to assess the method?

# Problem: How to discretize the energy variable accurately without requiring an $\mathcal{O}(10,000)$ energy unknown count?

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- I How to determine the energy mesh ( $\mathbb{E}_k \Leftrightarrow w_k(E)$ , the FE weight functions)?
- II How to determine the condensing spectrum ( $f_i(E)$ )?
  - ① How do we treat **resonance-scale** energy behavior of the solution accurately?  
Take advantage of decades of MG work
  - ② How do we treat **long-range** energy behavior of the solution accurately?  
Nested energy mesh with an outer level of coarse groups
  - ③ How do we apportion energy unknowns among the coarse groups?  
Apportion based on relative variance within a coarse group
- III How to assess the method?

*Reactor problem*

$$\phi_\infty(E) \simeq \frac{1}{E} \frac{1}{\Sigma_t(E)}$$

*TOF problem*

$$\phi_{\text{transmission}}(E) \simeq \chi(E) e^{-\Sigma_t(E)X}$$

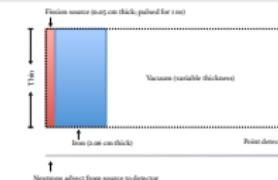
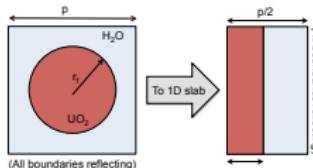
$$t_{\text{arrival}} \propto \frac{1}{v(E)}$$

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- II How to determine the condensing spectrum  
( $f_i(E) \Leftrightarrow b_k(\mathbf{r} \in V_i, E)$ , the FE basis functions)?
- III How to assess the method?
  - Compare to an **optimal MG competitor** whose energy mesh comes from a contiguous partitioning solution of the minimization problem
  - Compare to **continuous-energy Monte Carlo** for simple reactor problems (left)
  - Compare to an analytic solution for a **time-dependent problem** where the  $1/v$  term becomes important (right)



# We travel through a series of problem-research-solution cycles

## I Determining the energy mesh

- ① Dividing in solution space with more than one spectrum
- ② Forming accurate spectra for the minimization problem
- ③ Solving this minimization problem efficiently
- ④ Solving our discrete transport equation in energy

## II Determining the condensing spectrum

## III Testing the method

# Problem: How do we divide in solution space for multiple resonant snapshots

## Research: Applied mathematics

Before performing the minimization

- ① Determine a hyperfine group structure that resolves all desired resonances  
 $\{E_{g\pm 1/2}\}, g = 1, \dots, G$
- ② Obtain  $P$  representative high-resolution snapshots of the solution on the hyperfine structure  
 $\phi_{g,p}, p = 1, \dots, P$
- ③ Choose a final number of energy elements,  $N_e$   
Elements indexed by  $e = 1, \dots, N_e$

Solution: The minimization problem

- ① Guess  $\mathbb{S}_e$ , the set of hyperfine groups that belong to element  $e$   
Each group,  $g$ , belongs to exactly one element,  $e$
- ② Average the spectra in energy over each element for each material  
 $\bar{\phi}_{e,p} = \text{mean}_{g \in \mathbb{S}_e}(\phi_{g,p})$
- ③ Compute a metric of variance over all elements and snapshots  
 $F = \sum_p \sum_e \sum_{g \in \mathbb{S}_e} |\phi_{g,p} - \bar{\phi}_{e,p}|^2$
- ④ Choose element definitions that minimize  $F$   
 $\mathbb{S}_e = \text{argmin } F$

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Discontiguous meshing of the energy domain (FEDS)



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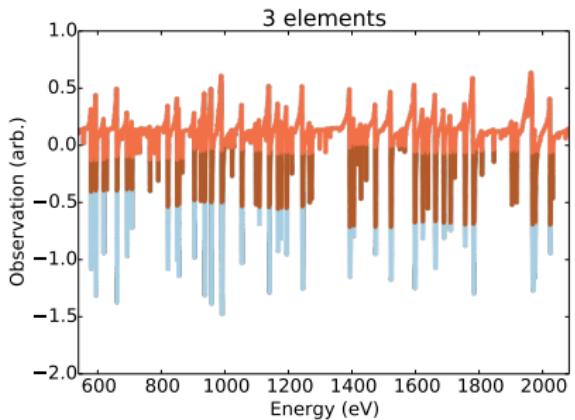
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- ④ Choose element definitions that minimize  $F$   
 $\mathbb{S}_e = \text{argmin } F$

Problem: How do we obtain accurate snapshots for the minimization problem  
Research: Integral form of the transport equation, physics of downscattering



## Solution: Infinite-medium calculations

Snapshots are spectra that come from solving:

$$(\Sigma_e + \Sigma_{t,g}) \phi_{g,p} = q_{\text{scat},g,p}(\phi_{g',p}) + q_{\text{ext},g,p}$$

This is one simple way to generate high-resolution snapshots

FEDS is insensitive to this choice

Minimization problem uses these  $\phi_{g,p}$ :

$$\mathbb{S}_e = \operatorname{argmin} \sum_p \sum_e \sum_{g \in \mathbb{S}_e} |\phi_{g,p} - \bar{\phi}_{e,p}|^2$$

Mesh is encoded in the  $\mathbb{S}_e$  (left)

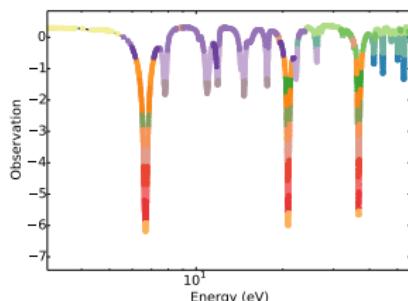
# Problem: How do we solve the minimization problem

Research: Clustering algorithms from machine learning

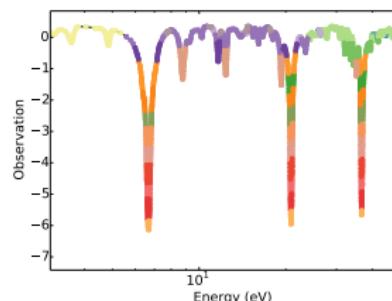
Solution: Clustering algorithms solve the minimization problem approximately

- Minimization problem:  $\mathbb{S}_e = \operatorname{argmin} \sum_p \sum_e \sum_{g \in \mathbb{S}_e} |\phi_{g,p} - \bar{\phi}_{e,p}|^2, e = 1, \dots, N_e$
- We use **hierarchical agglomeration** (accurate, not stochastic, medium cost)
  - Starts with all hyper-fine groups in their own cluster ( $\mathbb{S}_e^0 = \{g\}, e = 1, \dots, G$ )
  - In each stage, it combines the two closest clusters into one new cluster ( $\mathbb{S}_{\text{new}}^{n+1} = \mathbb{S}_{e_1}^n \cup \mathbb{S}_{e_2}^n$ )
- Creates elements with minimal variance over all the snapshots
- Efficient implementation in `scikit-learn`, a Python module
- Other notable algorithms include Birch, K-Means, and Mini-Batch-K-Means

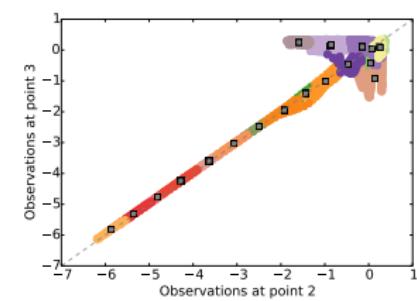
Snapshot of MOX spectrum:



Snapshot of UO<sub>2</sub> spectrum:



Clustering algorithm view:

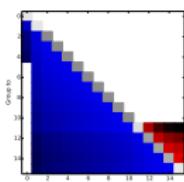


# Problem: How do we solve the FEDS neutron transport equation

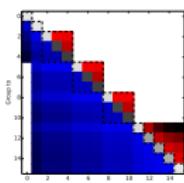
Research: Finite element methods, iterative methods, sweeps

Scattering matrices

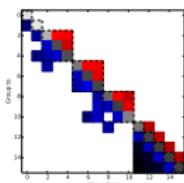
MG H-1:



FEDS H-1:

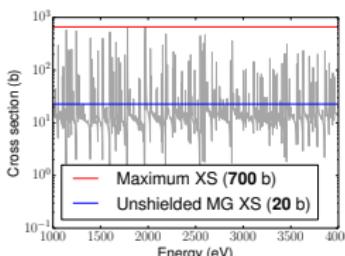


FEDS Pu-239:

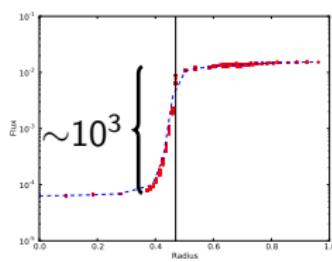


Unresolved MG does not

preserve large XS:



Large XS may introduce  
boundary layers:



Solution: FEDS can use existing MG transport solvers

- Code thinks it is solving MG
- Caveat 1: Iterative methods  
**FEDS introduces effective upscattering (left)**
- Caveat 2: Spatial discretizations  
**FEDS resolves large XS that introduce boundary layers (middle)**

# We travel through a series of problem-research-solution cycles

- I Determining the energy mesh
- II Determining the condensing spectrum
  - ① Treating resonance-scale energy behavior of the solution
  - ② Treating long-range energy behavior of the solution
  - ③ Apportioning energy unknowns among the coarse groups
- III Testing the method

# Problem: How do we choose accurate and insensitive basis functions to capture local resonance effects

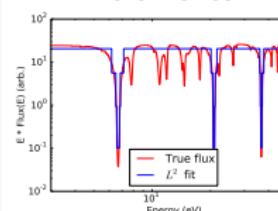
Research: Physics of streaming and downscattering, equivalence theory

## Solution, Part 1: FEDS

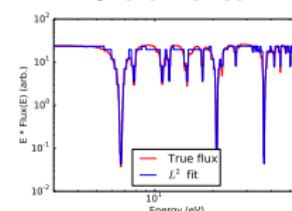
- Minimization problem yields elements that can adapt to the behavior of the solution-like spectra
- As energy unknowns are increased, fit improves (right)

$L^2$  fit of the spectra by the elements:

4 elements



20 elements



## Solution, Part 2: Reduced-geometry, high-energy-resolution calculations

- Basis function shape should match material-average solution shape (not be flat, as above)
- We use infinite-medium-equivalent calculations for the basis functions:
- $$\left[ \Sigma_{e,i} + \Sigma_{t,i}(E) \right] f_i(E) = \sum_{k \in \text{matl. } i} \left[ \frac{N_k}{1-\alpha_k} \int_E^{E/\alpha_k} dE' \frac{f_i(E') \sigma_{s,k}(E')}{E'} \right] + q_{\text{ext},i}(E)$$
- $\Sigma_{e,i} \sim \text{SA}_i / (4V_i)$  captures geometry information of real problem

# Problem: How do we choose accurate and insensitive basis functions to capture long-range energy behavior

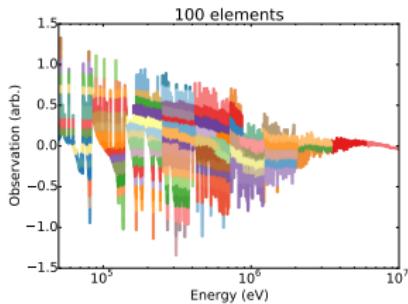
Research: Physics of streaming and downscattering, MB methods

## Solution: Use coarse groups

- Large energy elements are inaccurate
- Restrict each element to live in one coarse group
- Balance coarse groups and,  $r$ , the average number of elements per coarse group (below)
- $r = 1$  is equal-lethargy-spaced MG

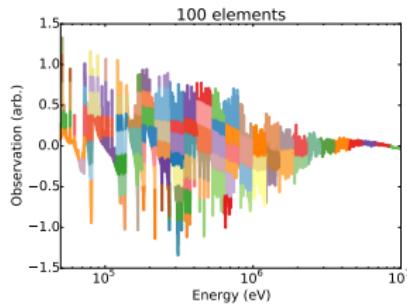
10 coarse groups

$$(r = 10)$$



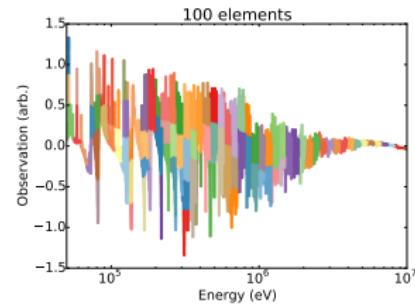
25 coarse groups

$$(r = 4)$$



50 coarse groups

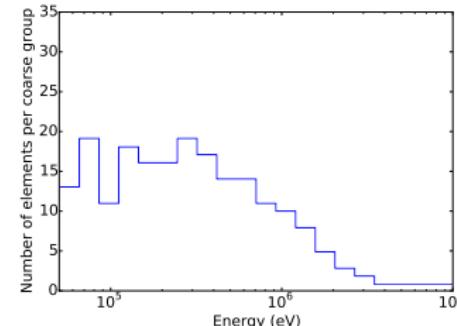
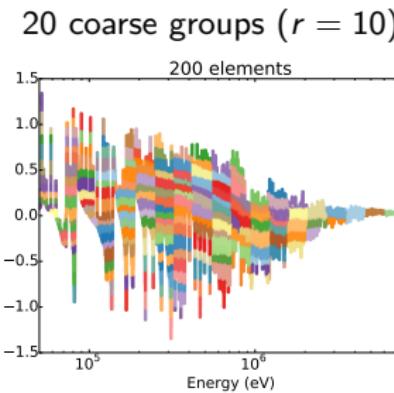
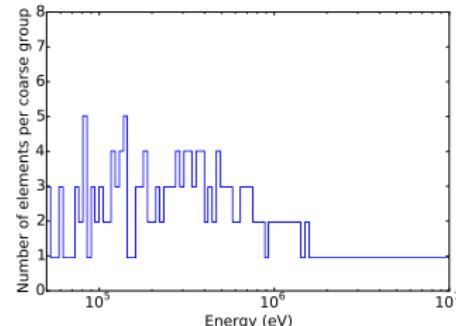
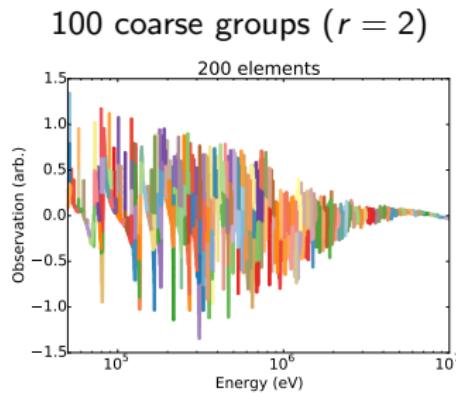
$$(r = 2)$$



# Problem: How do we automatically apportion energy unknowns among the coarse groups?

Research: Applied mathematics, the minimization problem

Solution: Proportionally to the relative standard deviation of the spectra within a coarse group



# We travel through a series of problem-research-solution cycles

- I Determining the energy mesh
- II Determining the condensing spectrum
- III Assessing the method
  - Comparing to a MG competitor
  - Comparing to an analytic solution for a streaming-dominated time-dependent problem where the  $1/v$  term becomes important
  - Comparing to a continuous-energy Monte Carlo solution for reactor problems

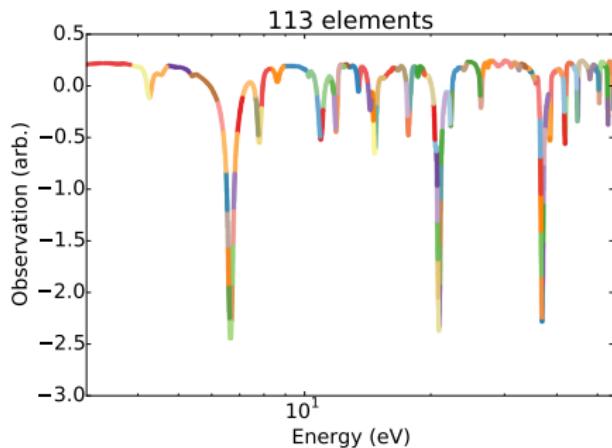
# Problem: How do we compare against an optimal MG method?

Research: The minimization problem

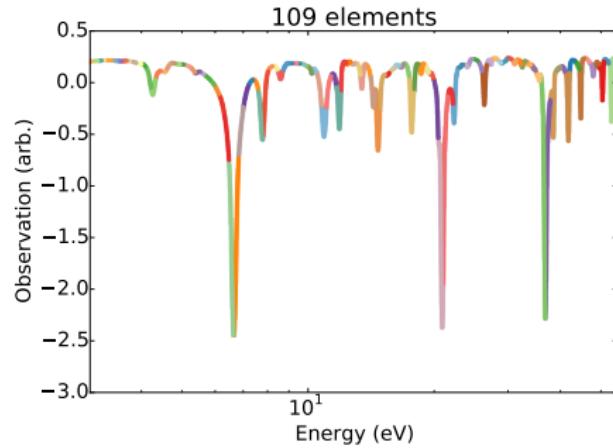
Solution: Solve the minimization problem with contiguous clustering

Low-energy RRR (resolved)

Adaptive MG



Standard MG



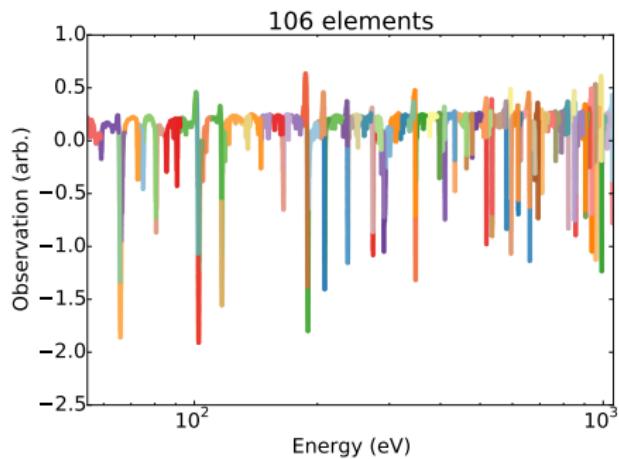
# Problem: How do we compare against an optimal MG method?

Research: The minimization problem

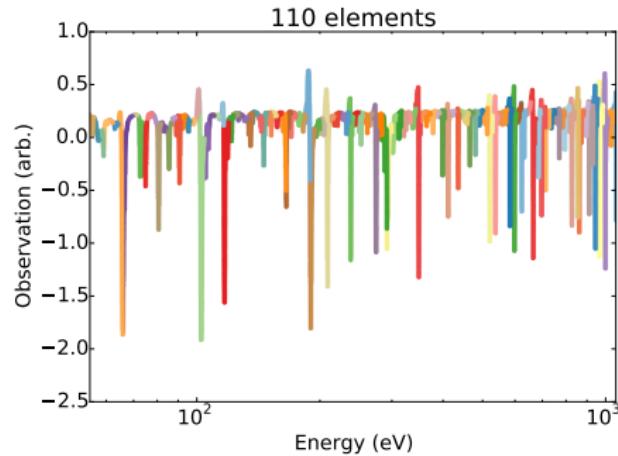
Solution: Solve the minimization problem with contiguous clustering

## Medium-energy RRR (unresolved)

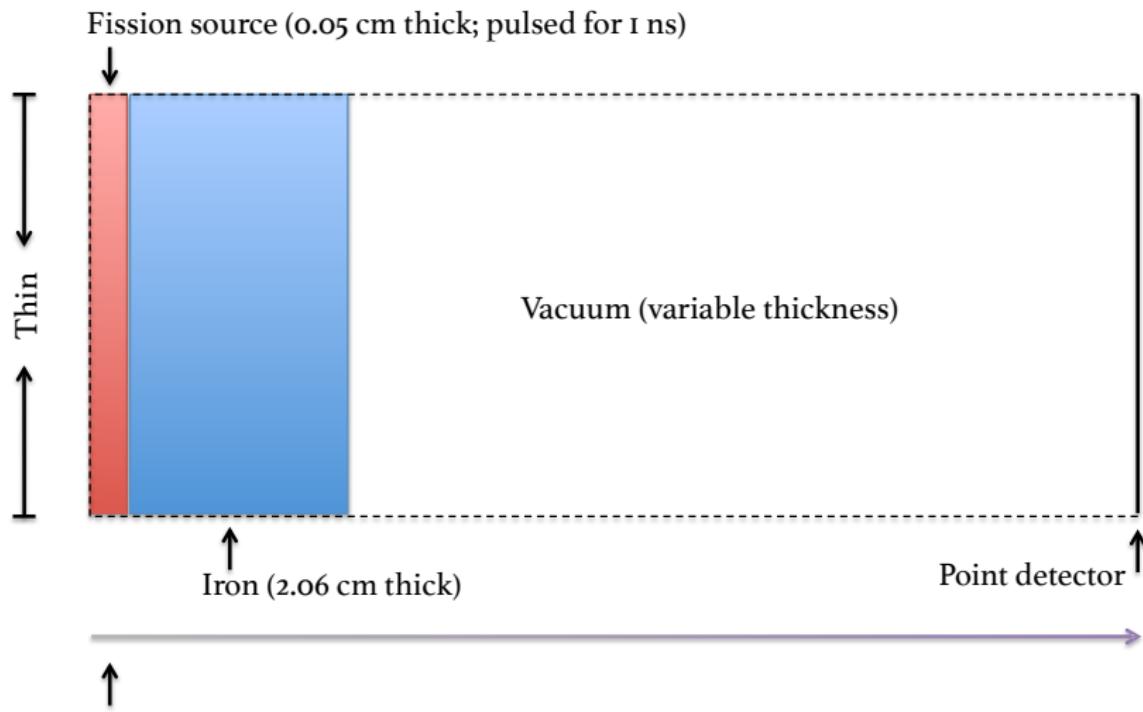
Adaptive MG



Standard MG

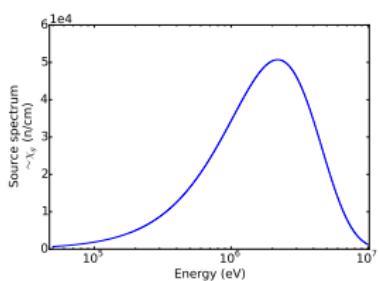


We investigated a time-dependent neutron pulsed-source problem with analytic solution in space-time each energy element  
Simplified such that all neutrons move in  $+\hat{x}$  and do not scatter



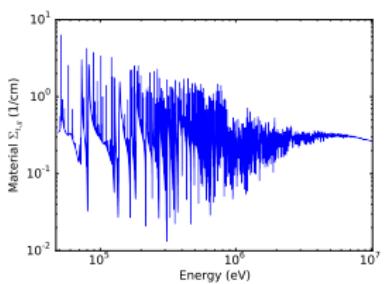
We investigated a time-dependent neutron pulsed-source problem with analytic solution in space-time each energy element

Fission spectrum



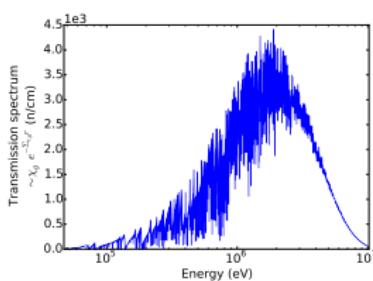
+

Iron cross section



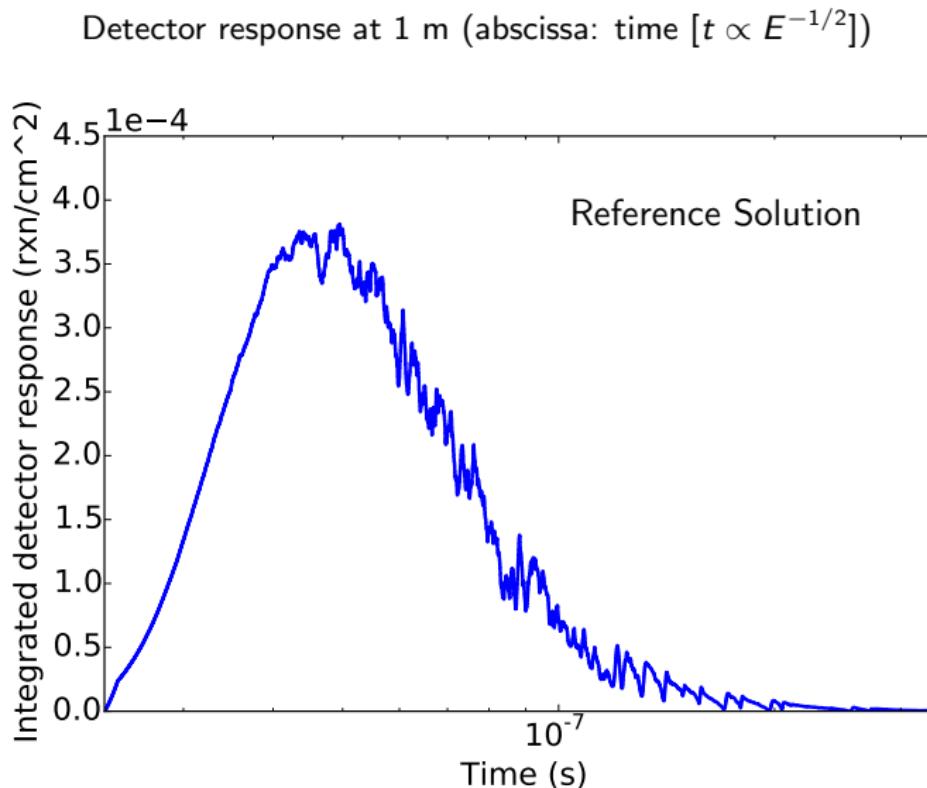
=

Transmission spectrum



(Abscissae: energy)

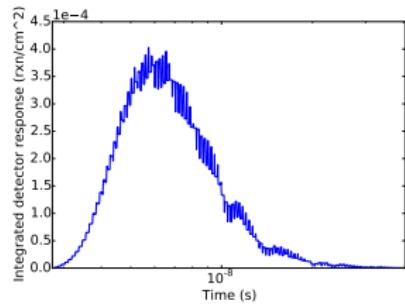
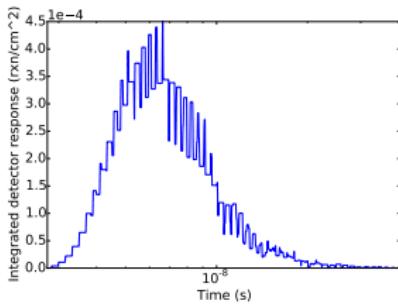
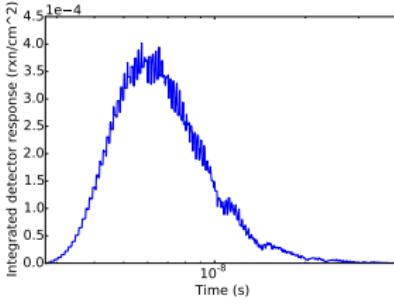
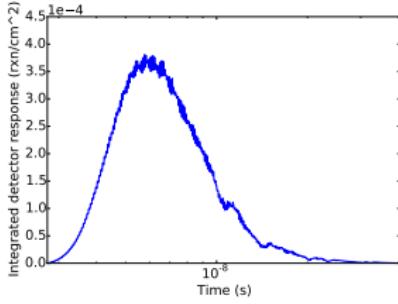
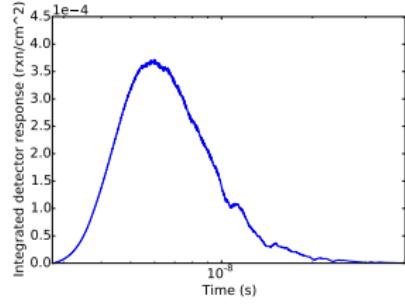
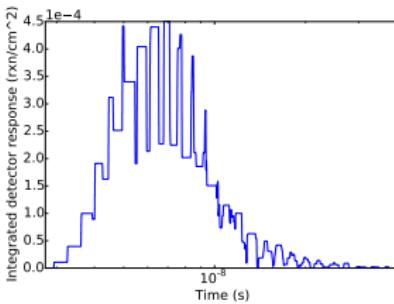
We investigated a time-dependent neutron pulsed-source problem with analytic solution in space-time each energy element



## TOF results — Detector response at 10 cm with 10,000 time bins

**Top** row: 100 total energy elements**Bottom** row: 400 total energy elements

$$r = (\text{number of total energy elements}) / (\text{number of coarse groups})$$

Standard MG ( $r = 1$ )FEDS ( $r = 2$ )FEDS ( $r = 4$ )

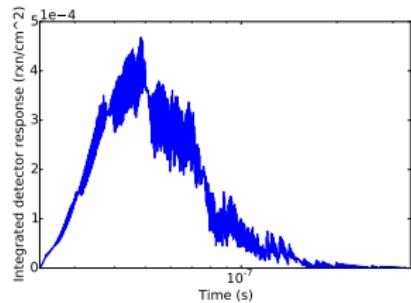
# TOF results — Detector response at 1 m with 10,000 time bins

Top row: **400** total energy elements

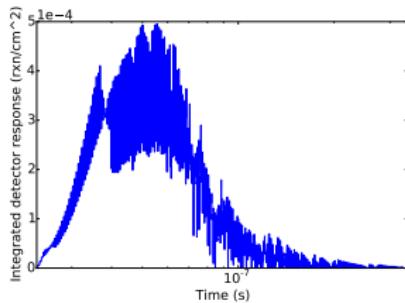
Bottom row: **1600** total energy elements

$$r = (\text{number of total energy elements}) / (\text{number of coarse groups})$$

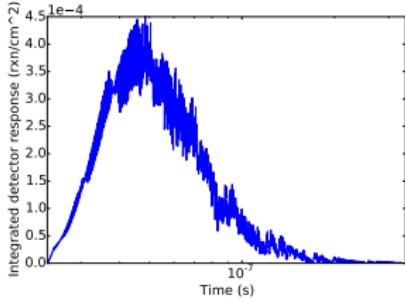
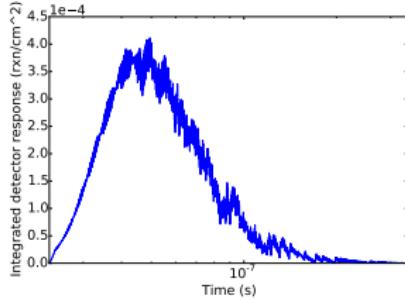
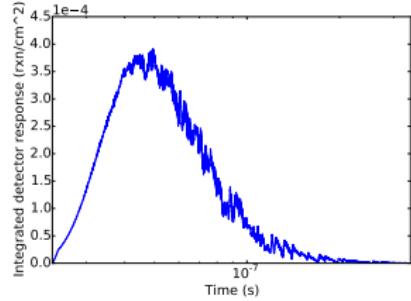
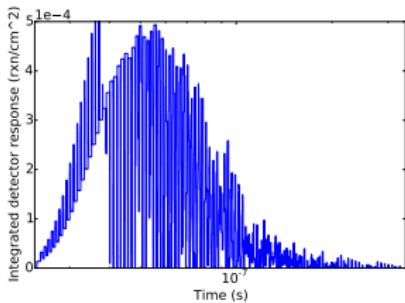
Standard MG ( $r = 1$ )



FEDS ( $r = 2$ )



FEDS ( $r = 4$ )



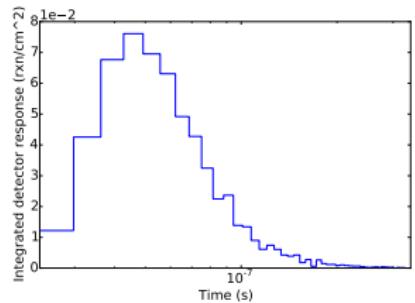
## TOF results — Detector response at 1 m with 50 time bins

Top row: **100** total energy elements

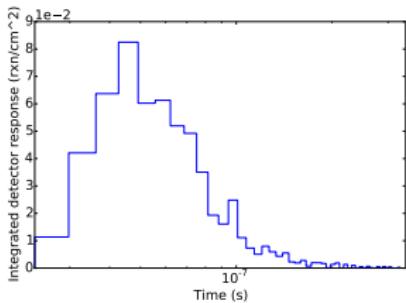
Bottom row: **400** total energy elements

$$r = (\text{number of total energy elements}) / (\text{number of coarse groups})$$

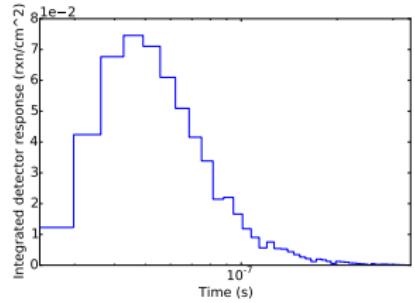
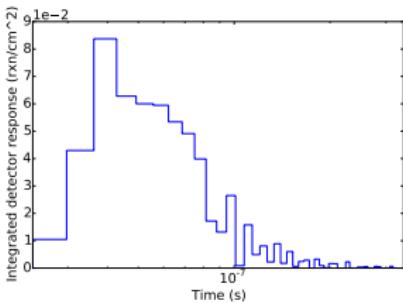
### Standard MG ( $r = 1$ )



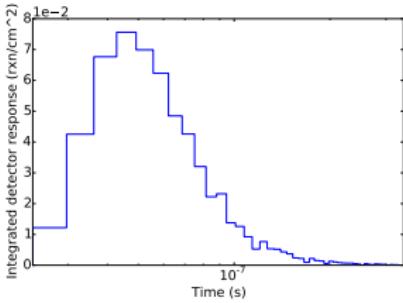
## FEDS ( $r = 2$ )



## FEDS ( $r = 4$ )



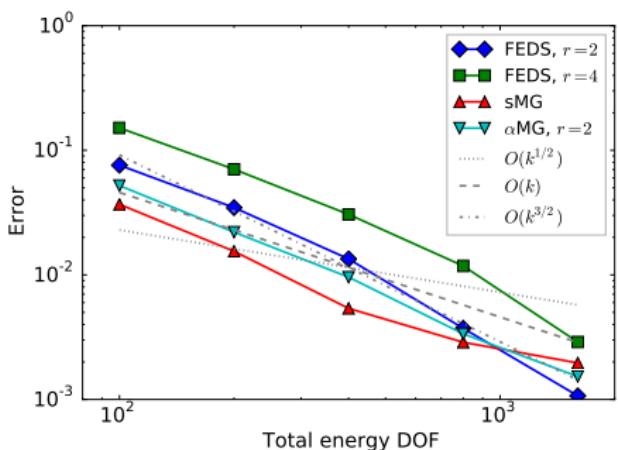
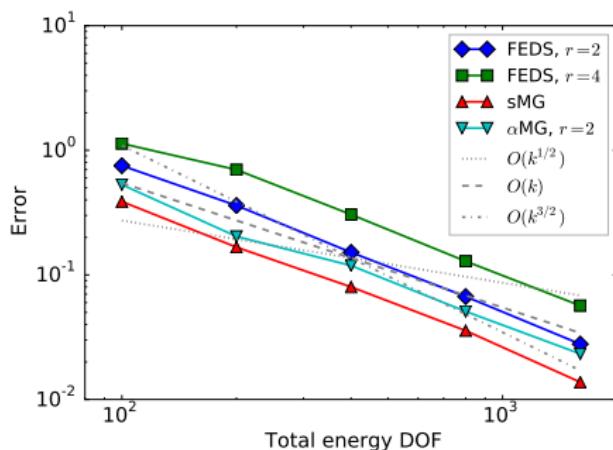
Time (s)	Integrated detector response ( $\times 10^{-2}$ )
0.0	1.2
0.05	4.0
0.1	6.8
0.2	6.8
0.3	6.5
0.4	5.5
0.5	4.5
0.6	3.5
0.7	2.5
0.8	1.8
0.9	1.2
1.0	0.8
1.2	0.2
1.5	0.05



## TOF results — Convergence rates for fixed ratios

## Detector Response Error

$$\text{Error} \sim \sqrt{\frac{\sum_b |\text{ref}_b - \text{feds}_b|^2}{\sum_b |\text{ref}_b|^2}}$$

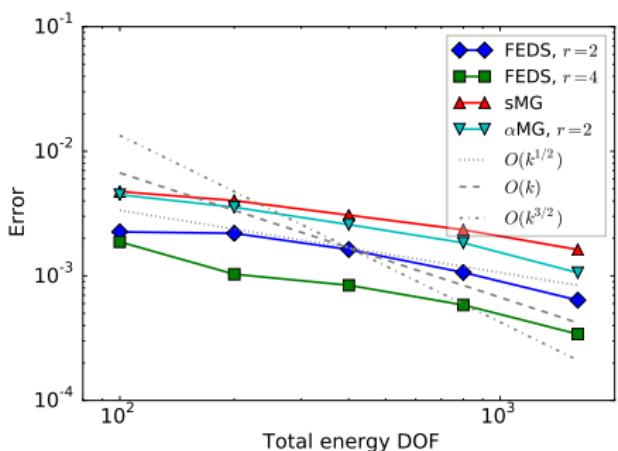
 $r = (\text{number of total energy elements}) / (\text{number of coarse groups})$ **10 cm, 10,000 time bins****1 m, 10,000 time bins**

# TOF results — Convergence rates for fixed ratios

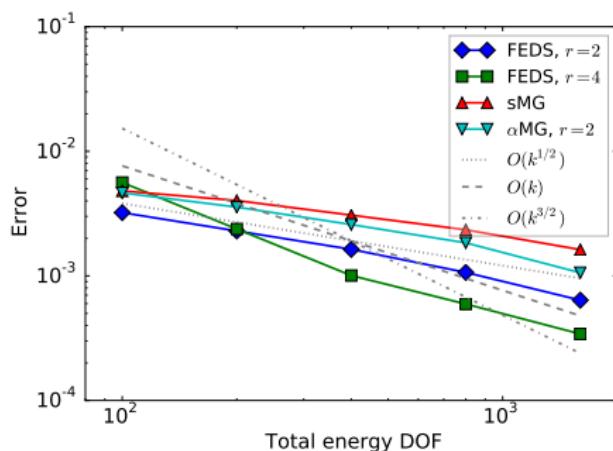
## Cumulative Detector Response Error

$$\text{Error} \sim \sqrt{\frac{\sum_b \left| \sum_{b' \leq b} (\text{ref}_{b'} - \text{feds}_{b'}) \right|^2}{\sum_b \left| \sum_{b' \leq b} \text{ref}_{b'} \right|^2}}$$

$r = (\text{number of total energy elements}) / (\text{number of coarse groups})$



**10 cm, 10,000 time bins**

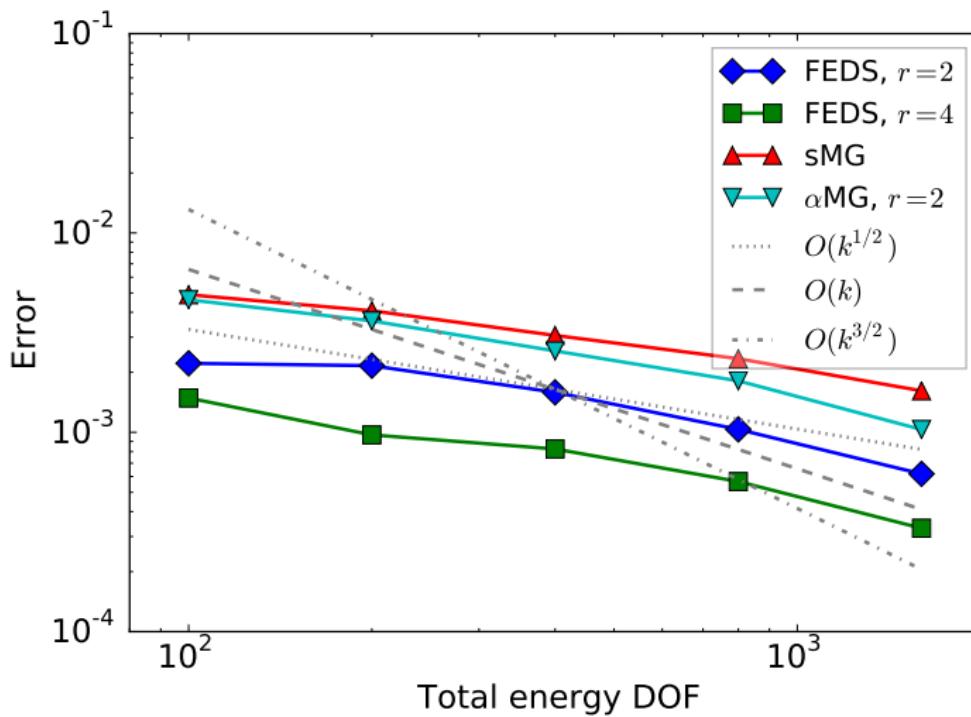


**1 m, 10,000 time bins**

## TOF results — Convergence rates for fixed ratios

**Absorption Fraction in Iron Error**

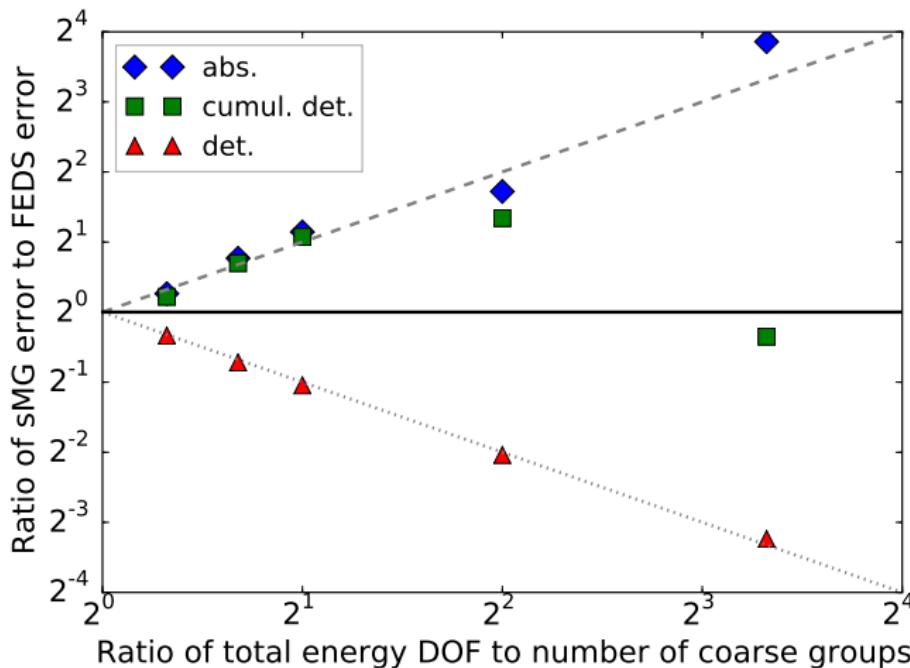
$$r = (\text{number of total energy elements}) / (\text{number of coarse groups})$$



## TOF results — Convergence rates for varying ratios

## FEDS

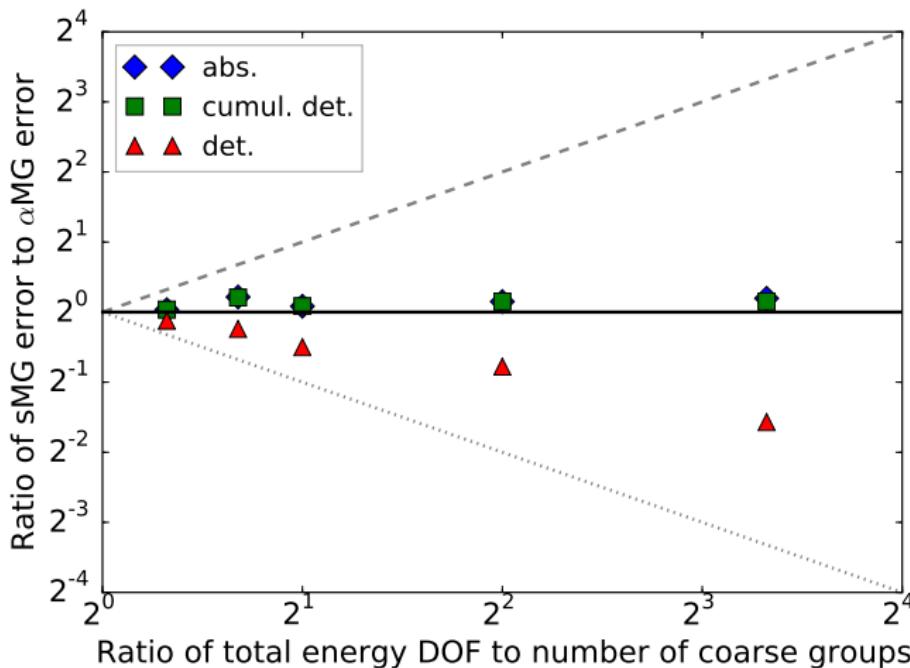
10 cm, 10,000 time bins, 100 total energy unknowns  
Ratio of standard MG error to FEDS error (higher is better)



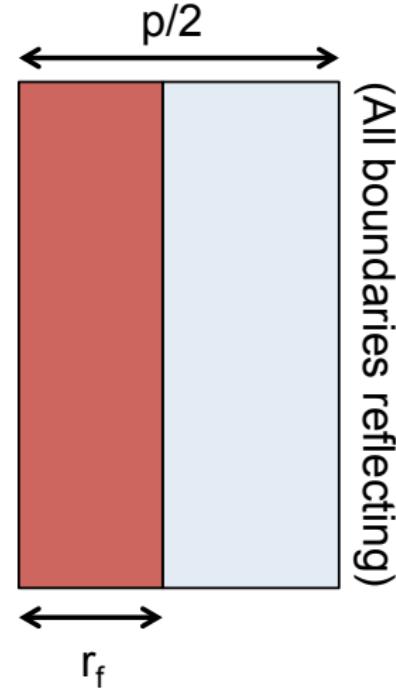
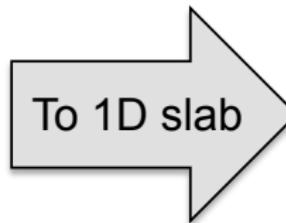
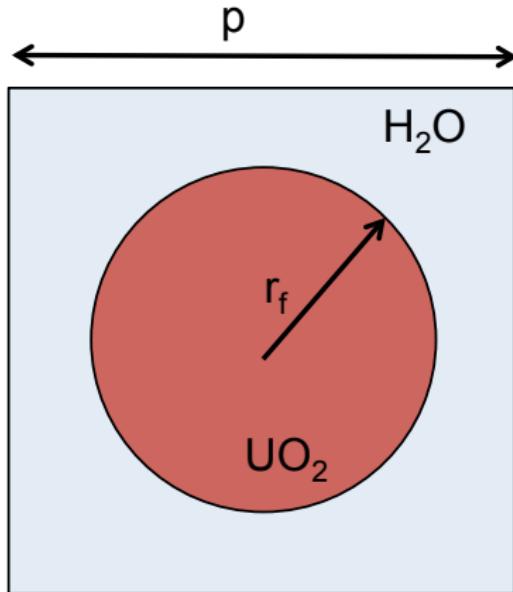
## TOF results — Convergence rates for varying ratios

## Adaptive MG

10 cm, 10,000 time bins, 100 total energy unknowns  
Ratio of standard MG error to FEDS error (higher is better)

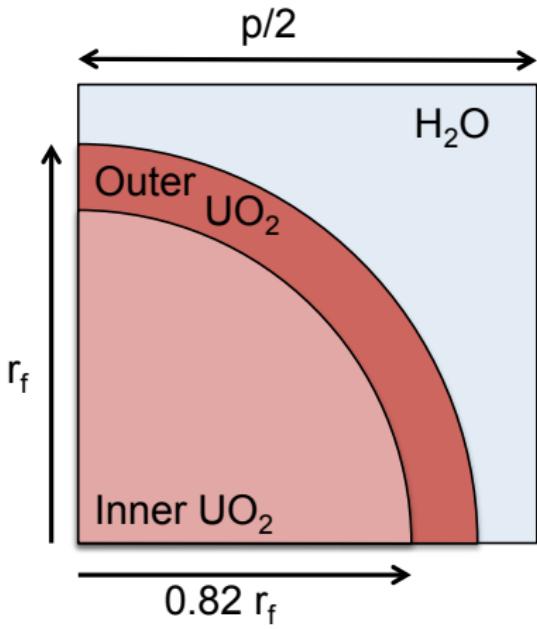


We investigated a slab pin-cell problem and made comparisons to continuous-energy MCNP with consistent cross sections

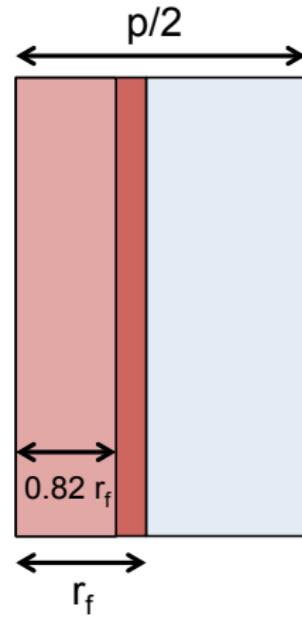
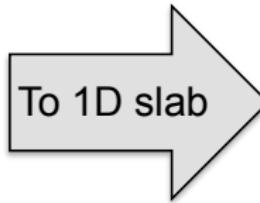


(All boundaries reflecting)

We investigated a slab pin-cell problem and made comparisons to continuous-energy MCNP with consistent cross sections



(All boundaries reflecting)



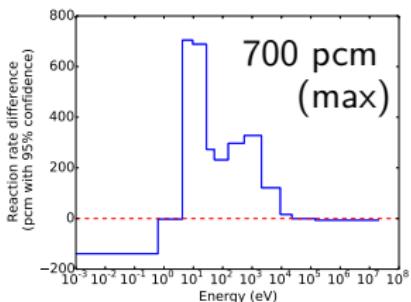
(All boundaries reflecting)

## Slab vs MCNP results — U-238 absorption rate in the inner fuel errors

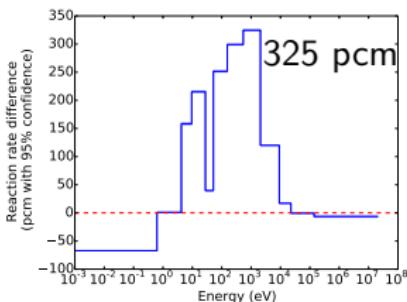
Top row: MG (SHEM boundaries)

Bottom row: FEDS

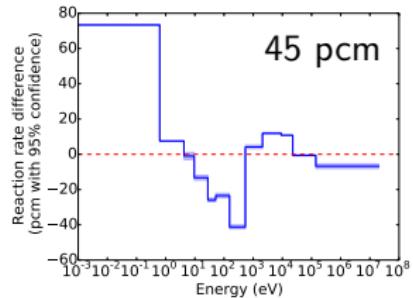
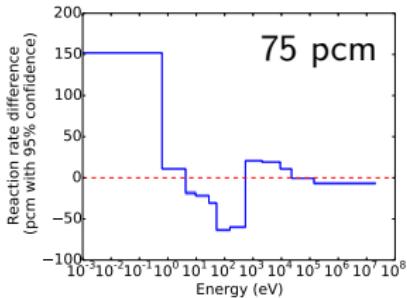
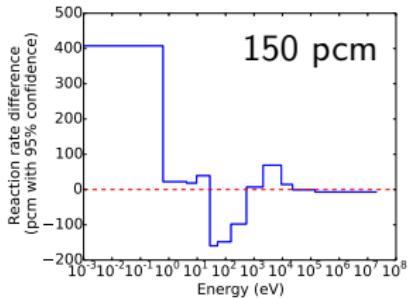
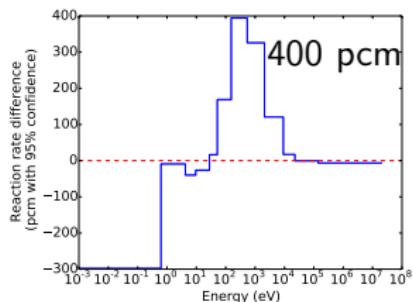
30 elements in the RRR



108 elements in the RRR



225 elements in the RRR

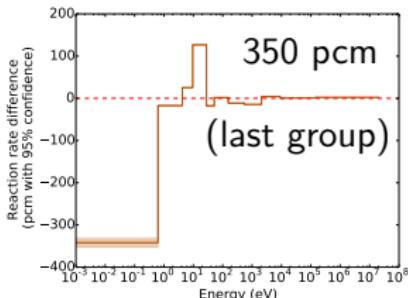


# Slab vs MCNP results — U-235 fission rate errors

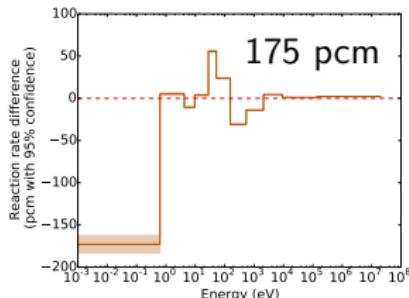
Top row: MG (SHEM boundaries)

Bottom row: FEDS

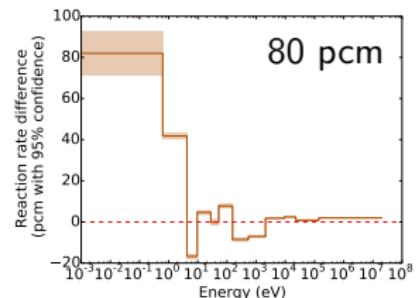
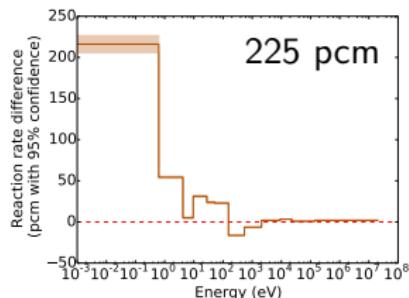
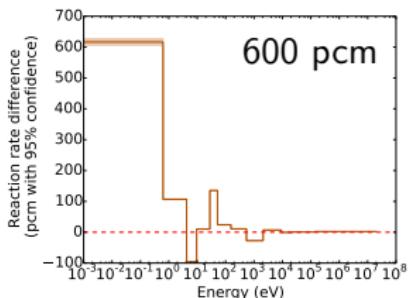
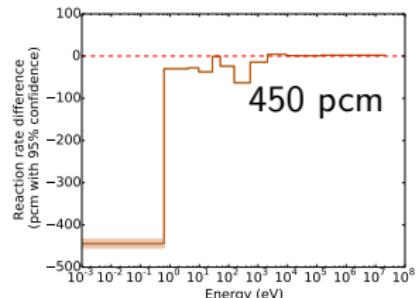
30 elements in the RRR



108 elements in the RRR



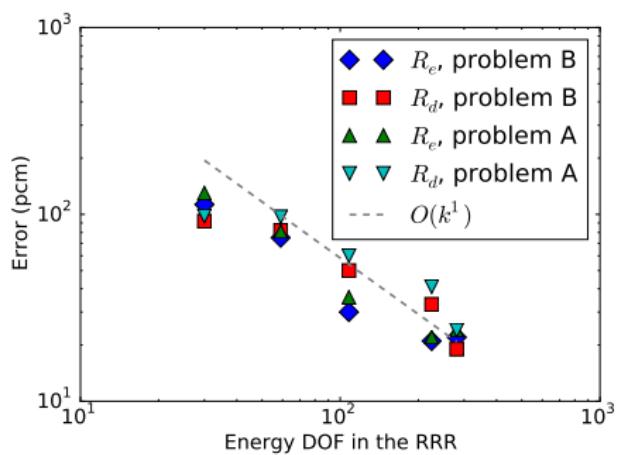
225 elements in the RRR



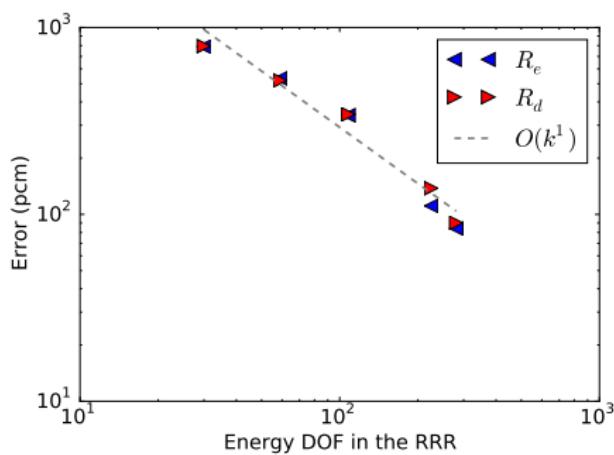
## Slab vs MCNP results — Convergence rates

## FEDS Errors

Inner U-238 absorption (154.2 – 539.2 eV)

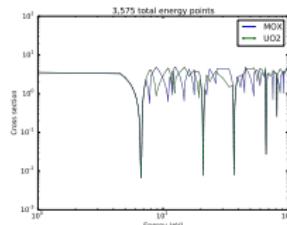


U-235 fission

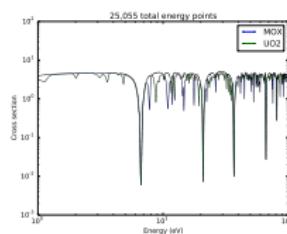


# Problem: No method is perfect

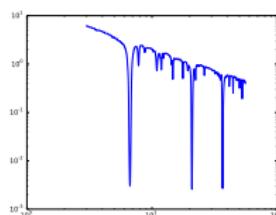
## Unresolved spectra



## Resolved spectra



## Unnormalized spectra



## Solution: How to break FEDS

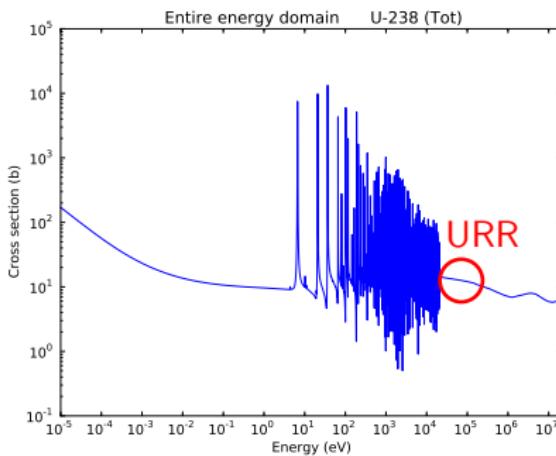
- ① Use microscopic cross sections instead of spectra as inputs to the minimization problem
- ② Do not resolve the resonances in the spectra (left, top)
- ③ Fail to renormalize the spectra (left, bottom)
- ④ Use an expensive clustering algorithm
- ⑤ Fail to use coarse groups (rely on energy penalties alone for dividing in energy space)
- ⑥ Use too many elements at high energies, which yields too many subelements for NJOY
- ⑦ Use the wrong escape cross section for the basis function
- ⑧ Use a MG solver that cannot handle upscattering
- ⑨ Fail to increase spatial / angular resolution when going from MG to FEDS

# Problem: I would like to graduate

Research: Common practice

## Solution: Future Work

- Applying FEDS to 2-D and 3-D pin-cell problems and comparing to MCNP with consistent continuous-energy cross sections (with Milan Hanuš)
- Applying FEDS to the fully resolved C5 problem (Carolyn McGraw)
- Applying FEDS to traveling-wave reactor problems (Pablo Vaquer)
- Passing the torch (Jijie Lou)



## Possible extensions of FEDS

- To the unresolved resonance region (URR, left)
- To thermal radiation problems (to preserve Planck and Rosseland opacity limits simultaneously with  $S_n$  transport or IMC)
- As a basis for estimating discretization error in energy

I look forward to your feedback and questions regarding my FEDS method  
And to possible future collaboration

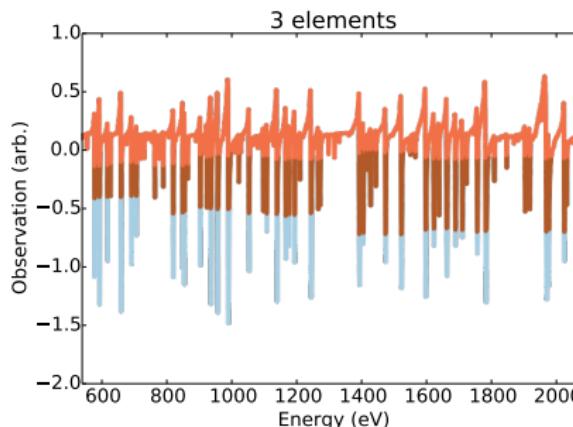
## Questions?

A special acknowledgment to the Department of Energy Computational Science Graduate Fellowship program (DOE CSGF - grant number DE-FG02-97ER25308), which provides strong support to its fellows and their professional development.



**Problem:** How do we form accurate spectra for the minimization problem

**Research:** Integral form of the transport equation, equivalence theory, physics of downscattering



Snapshot are spectra that come from solving:

$$(\Sigma_e + \Sigma_{t,g}) \phi_{g,p} = q_{\text{scat},g,p} + q_{\text{ext},g,p}$$

Minimization problem:

$$\mathbb{S}_e = \operatorname{argmin}_p \sum_e \sum_{g \in \mathbb{S}_e} |\phi_{g,p} - \bar{\phi}_{e,p}|^2$$

**Solution:** Infinite-medium calculations

Parametrize particle advection with  $s$ :

$$\begin{aligned} (\mathbf{r}, t) &= (\mathbf{r}_{\text{bdr}}, t_{\text{bdr}}) + s (\boldsymbol{\Omega}, 1/\nu) \\ \Rightarrow (1/\nu) \partial_t + \boldsymbol{\Omega} \cdot \nabla &\rightarrow \partial_s \end{aligned}$$

Transport equation becomes:

$$[\partial_s + \Sigma_t(s, E)] \psi(s, E) = q_{\text{(total)}}(s, E)$$

$\psi(s, E)$  is the convex sum of upstream  
 $q(s, E)/\Sigma_t(s, E) \sim q_p(E)/\Sigma_{t,p}(E)$

Our spectra:  $\phi_{g,p} = q_{\infty,p}(E_g)/\Sigma_{t,p}(E_g)$

Best case:  $\phi_{g,p} = \phi(\mathbf{r}_p, E_g)$

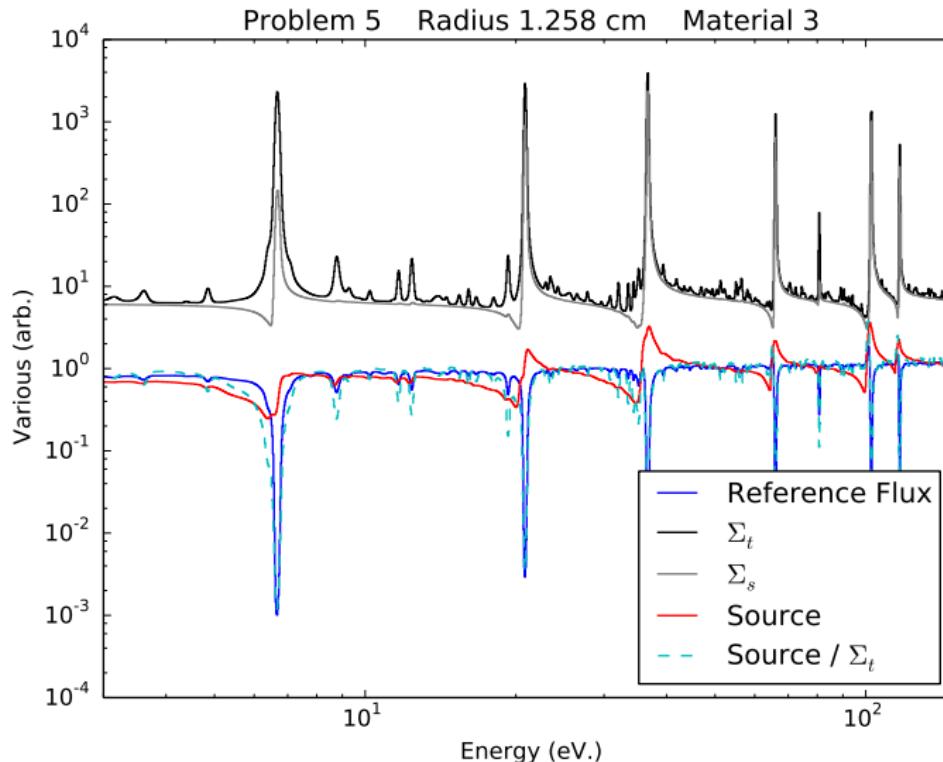
Worst case:  $\phi_{g,p}$  are solution indicators

Desire:

$\phi_{g,p}$  spans behavior of  $\psi(\mathbf{r}, E, \boldsymbol{\Omega}, t)$  by spanning behavior of  $q_p(E)/\Sigma_{t,p}(E)$

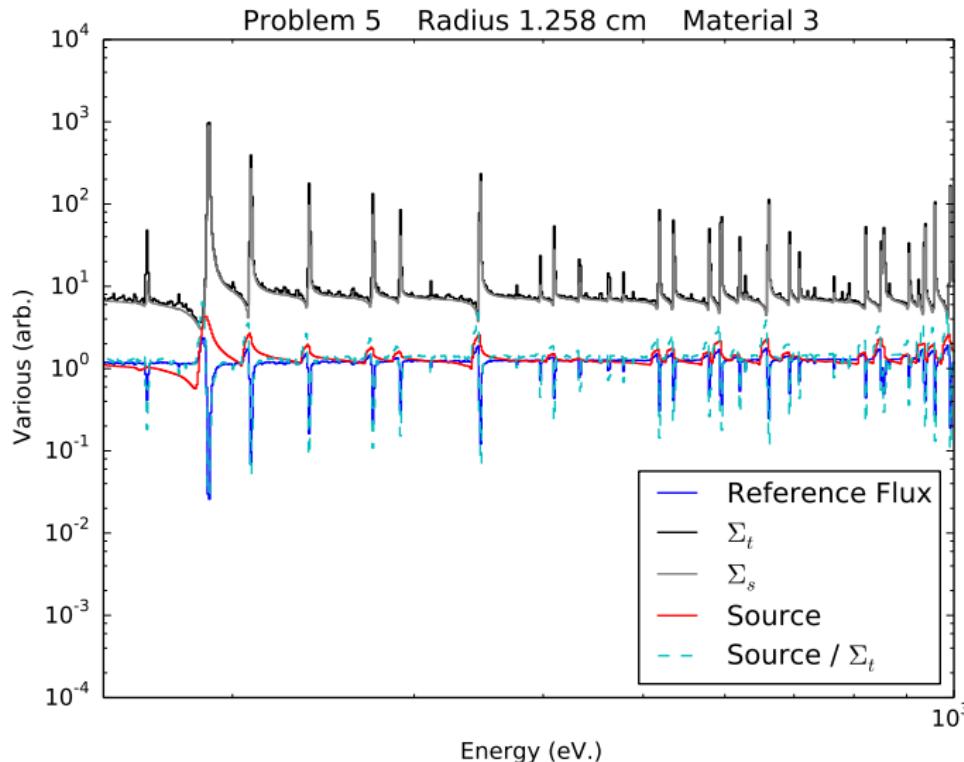
Research: The scalar flux is often well approximated by  $q(E)/\Sigma_t(E)$

Low-energy resolved RR



Research: The scalar flux is often well approximated by  $q(E)/\Sigma_t(E)$

Medium-energy resolved RR

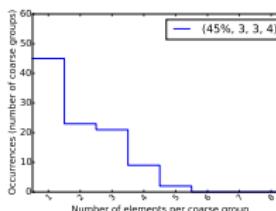
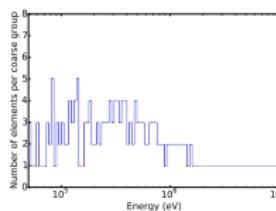
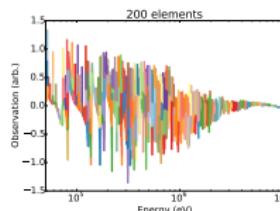


# Problem: How do we automatically apportion energy unknowns among the coarse groups?

Research: Applied mathematics, the minimization problem

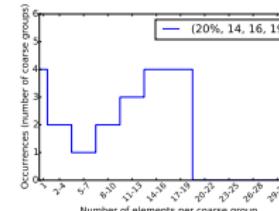
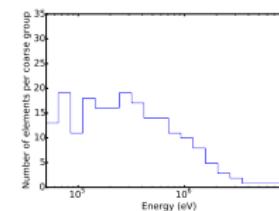
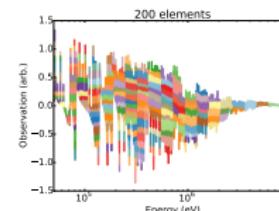
100 coarse groups

$$(r = 2)$$



20 coarse groups

$$(r = 10)$$



Solution: Apportion based on relative standard deviation within a coarse group

- Reuse the spectra to calculate the standard deviation in each coarse group:
- $S_c = \sqrt{\frac{1}{N_g P} \sum_{g=g_{c-1/2}}^{g_{c+1/2}} \sum_{p=1}^P |O_{g,p} - \bar{O}_{c,p}|^2}$
- Give coarse group  $c$  approximately  $\frac{S_c}{S_{\text{tot}}} N_e$  elements
- Assigns more unknowns to groups with higher variance (left)

# Problem: How do we compare against an optimal MG method?

Research: The minimization problem

Solution: Solve the minimization problem with contiguous clustering

- Goal: minimize solution variance within a group
- Spectra indicate which energies have large solution variance
- Define group boundaries s.t. **each fine group has the same squared error**:

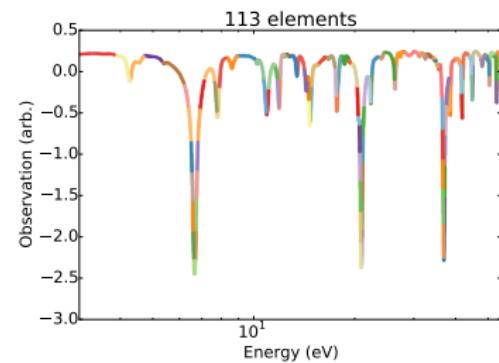
$$\bullet S_c = \sum_{g' = g_{c-1/2}}^{g_{c+1/2}} \max_p |O_{g',p} - O_{g'-1,p}|^2$$

- Able to group hierarchically (coarse groups / fine groups per coarse group)

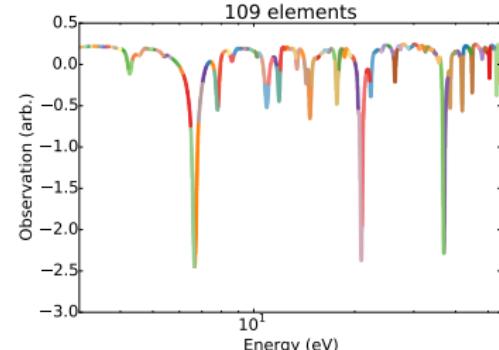
Low-energy RRR

(DOF count sufficient to resolve resonances)

Adaptive MG:



Standard MG:



# Problem: How can we predict convergence order?

Research: First mean value theorem for integration, Taylor series

Solution: We should expect first-order convergence for most cross section definitions

- Define  $\sigma_{t,g} = \frac{\int_{\Delta E_g} dE f(E) \sigma_t(E)}{\int_{\Delta E_g} dE f(E)}$ , with  $\sigma_t(E)$  continuous and  $f(E) > 0$ , bounded

- First mean value theorem for integration:

$$\exists E_* \text{ st. } \sigma_t(E_*) = \sigma_{t,g}, \text{ with } E_{g-1/2} \leq E_* \leq E_{g+1/2}$$

- Collocate the transport equation at  $E_*$ :

$$[(1/v(E_*)) \partial_t + \Omega \cdot \nabla + \sigma_t(E_*)] \psi(E_*) = q(E_*)$$

- Make  $\psi_g = \Delta E_g \psi(E_*)$  by choosing  $q_g = \Delta E_g q(E_*)$ ,  $v_g = v(E_*)$ , etc.

- Assume smoothness and Taylor-expand  $\psi(E)$ :

$$\psi(E) = \psi(E_*) + (E - E_*) \partial_E \psi(E_*) + \frac{1}{2}(E - E_*)^2 \partial_E^2 \psi(E_*) + \dots$$

- Integrate over a group and collect terms

$$\int_{\Delta E_g} dE \psi(E) = \psi_g + (E_g - E_*) [\Delta E_g \partial_E \psi(E_*)] +$$

$$\frac{1}{2}(E_g - E_*)^2 [\Delta E_g \partial_E^2 \psi(E_*)] + \frac{1}{24} (\Delta E_g)^2 [\Delta E_g \partial_E^2 \psi(E_*)] + \dots$$

- $|E_g - E_*| \leq \frac{1}{2} \Delta E_g$

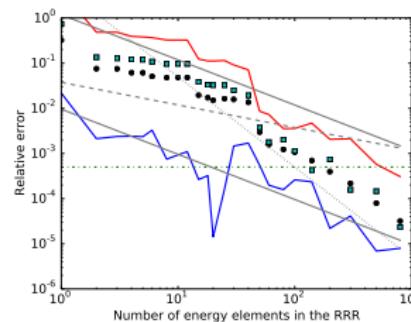
- Unless  $E_* = E_g = \frac{1}{2}(E_{g-1/2} + E_{g+1/2})$ , method is first-order

- MB has mapping to  $\sigma_t$  space and integrals of  $\int_{\sigma_{b-1/2}}^{\sigma_{b+1/2}} d\sigma J(\sigma)(\cdot)$ ;  $J = dE/d\sigma$

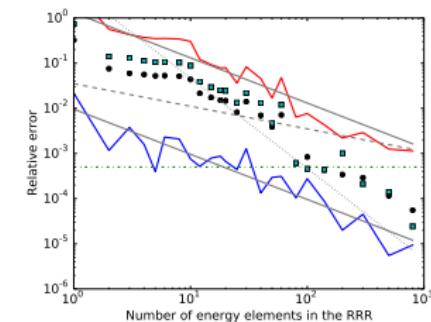
## Addendum: Selected results for cylindrical pin-cell problem

Problem 3 (multi-temperature, MOX), low-energy RRR (fewer resonances),  
case 1 ( $1/E$  condensing spectrum), resolved MG reference solution

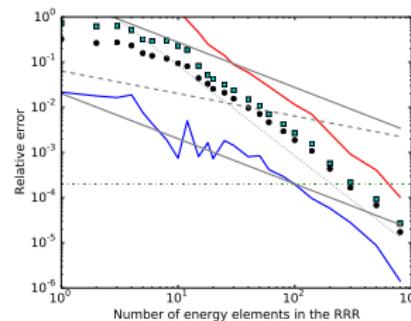
Hierarchical agglomeration



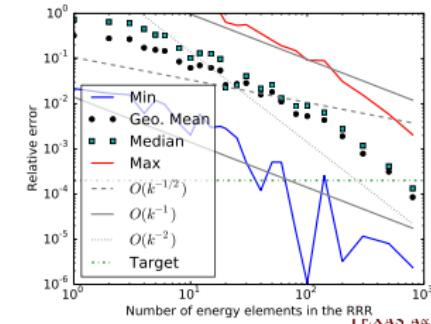
K-means



Adaptive MG



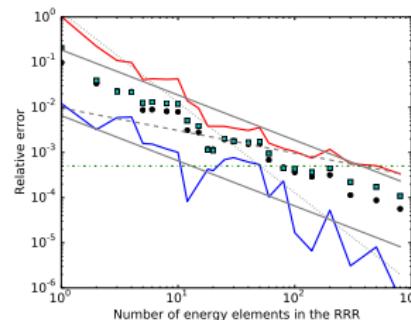
Standard MG



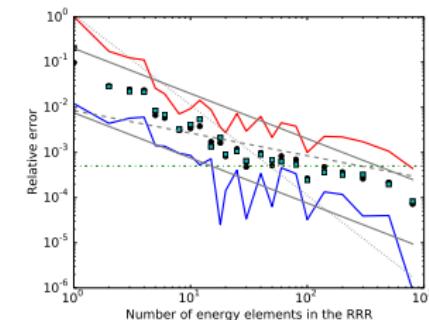
# Addendum: Selected results for cylindrical pin-cell problem

Problem 5 (MOX, moderator, UO<sub>2</sub>), medium-energy RRR (more resonances), case 1 (1/E condensing spectrum), resolved MG reference solution

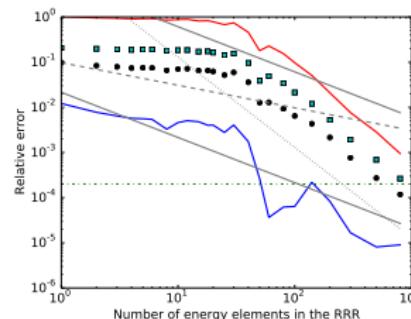
Hierarchical agglomeration



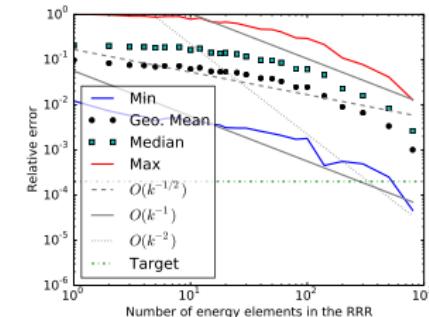
K-means



Adaptive MG



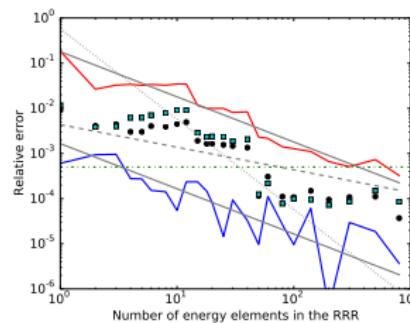
Standard MG



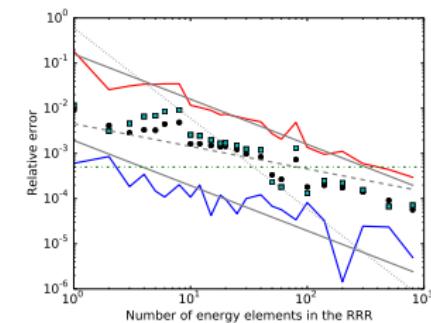
# Addendum: Selected results for cylindrical pin-cell problem

Problem 3 (multi-temperature, MOX), low-energy RRR (fewer resonances),  
case 2 (inf.-med.-equiv. condensing spectrum), resolved MG reference solution

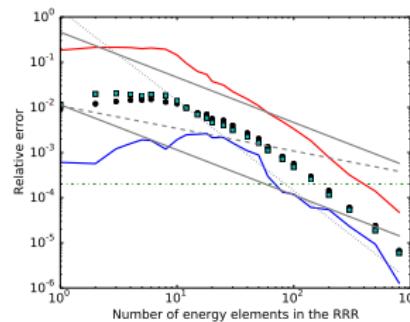
Hierarchical agglomeration



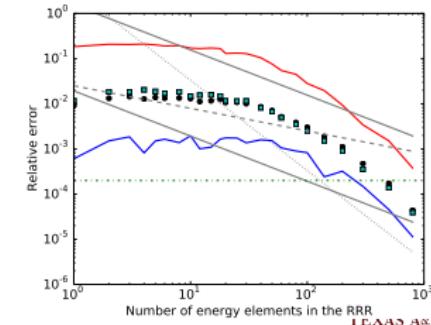
K-means



Adaptive MG



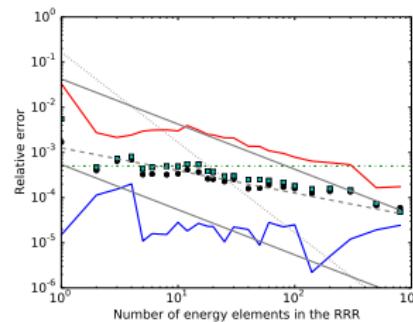
Standard MG



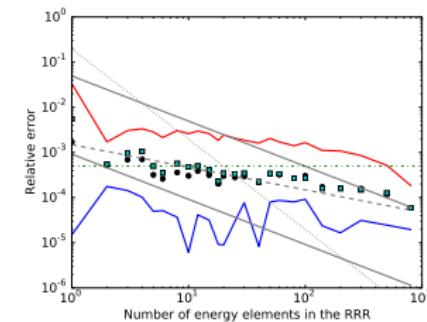
## Addendum: Selected results for cylindrical pin-cell problem

Problem 5 (MOX, moderator, UO<sub>2</sub>), medium-energy RRR (more resonances), case 2 (inf.-med.-equiv. condensing spectrum), resolved MG reference solution

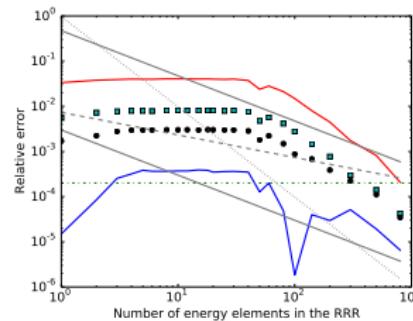
Hierarchical agglomeration



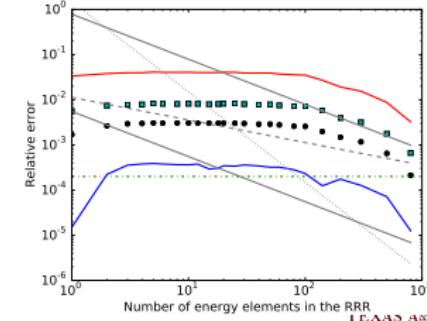
K-means



Adaptive MG



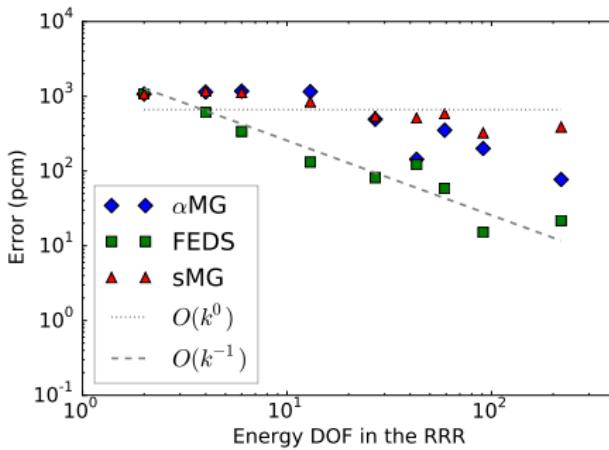
Standard MG



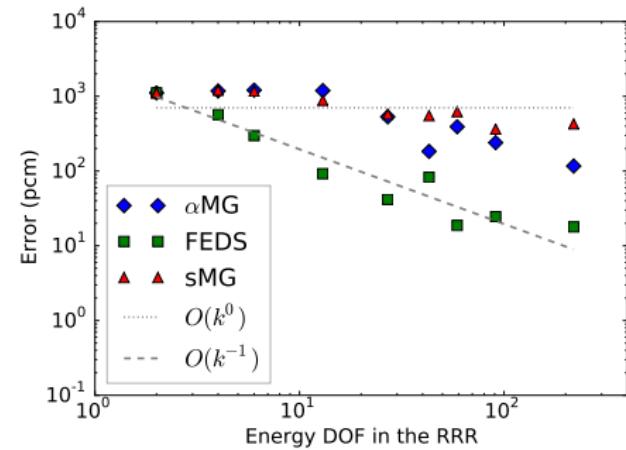
# Addendum: Selected results for C5G $^\infty$ problem

## Errors in criticality eigenvalue

Errors using MG reference



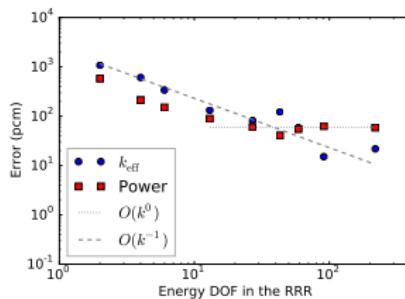
Errors using Wynn-epsilon-accelerated FEDS reference



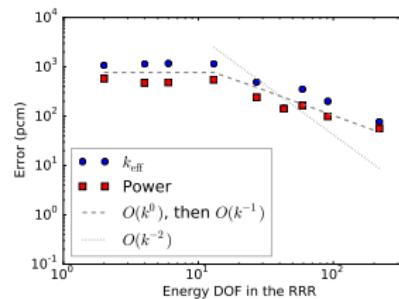
# Addendum: Selected results for C5G $^\infty$ problem

## Criticality eigenvalue error and average of pin-power-QOI errors

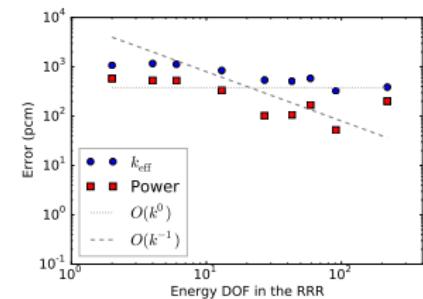
FEDS with MG reference



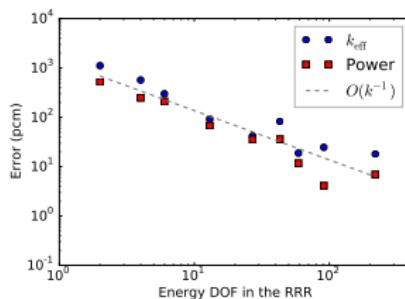
$\alpha$ MG with MG reference



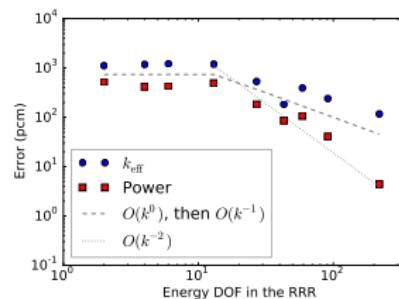
sMG with MG reference



FEDS with FEDS reference



$\alpha$ MG with FEDS reference



sMG with FEDS reference

