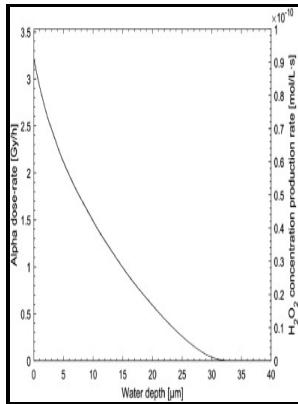


Internal Sources of Hydrogen in Unirradiated uo2 Fuel Elements.

s.n - A pragmatic approach to modelling thermal conductivity of irradiated UO2 fuel: Review and recommendations



Description: -

-Internal Sources of Hydrogen in Unirradiated uo2 Fuel Elements.

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Atomic Energy of Canada Limited. AECL -- 5312

Atomic Energy of Canada Limited. AECL -- 4063 Internal Sources of Hydrogen in Unirradiated uo2 Fuel Elements.

Notes: 1

This edition was published in 1972



Filesize: 6.106 MB

Tags: #Long

Reactor and Power Plant Materials

One safety issue that is a long-term problem brought on by the aging of nuclear facilities is pressurized thermal shock PTS. Asbestos had been used so widely and for so long that it must have seemed beyond credibility when evidence first started appearing that it might be harmful. Another bug donated by my assistant Nick.

Spent fuel UO 2 matrix alteration in aqueous media under oxidizing conditions, Radiochimica Acta

When we use data that are related to certain product, we use only data released by public relations departments and allowed for use. Some things you don't expect to see again. Technical Report TR-97—13 1—60 SKB, 1997.

Reactor and Power Plant Materials

In the event of loss of reactor coolant, the power of the rod decreases, the fuel temperature is only a few tens of degrees Celsius higher than the cladding temperature.

Fuel Elements: Design and Considerations

The γ -induced formation and decomprn. Generally speaking there are two common choices for this, and three other less-common but also considered fissile fuels.

Reactor and Power Plant Materials

Halothane is C₂HBrClF₃, basically a chlorofluorocarbon of the ozone-depleting variety. Special Reference: CEA, Nuclear Energy Division.

Reactor and Power Plant Materials

. An instrumentation tube is a vacant tube only for incore instrumentation such as the incore neutron flux monitoring system. This is a baby rattle that I cut on an improvised lathe in my grandfather's workshop in his cabin in the Swiss alps over 20 years ago.

Long

Included are discussions on the influence of various oxidants including radiolytic ones, pH, temp. The results support the idea that there is an upper temperature limit above which DHC will not occur in zirconium cladding. Reactor vessel surveillance program utilizes capsules located on the vessel wall directly opposite the center of the core.

UO₂ properties affecting performance

This is well in line with the very low solubility previously reported.

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