

Codes and standards and applications for design and analysis of pressure vessel and piping components - presented at the 1988 ASME Pressure Vessels and Piping Conference, Pittsburgh, Pennsylvania, June 19-23, 1988

American Society of Mechanical Engineers - Low

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vol. 136.

PVP (Series) ;

vol. 136

PVP ; Codes and standards and applications for design and analysis of pressure vessel and piping components - presented at the 1988 ASME Pressure Vessels and Piping Conference, Pittsburgh, Pennsylvania, June 19-23, 1988

Notes: Includes bibliographies.

This edition was published in 1988



Filesize: 70.63 MB

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Design & FEA of Reactor Pressure Vessel

At temperatures below 150 C 302 F the reduction in life due to reactor water environmental effects is less than a factor of 2, and the existing ASME Code Section III fatigue design curves for air can be used. Within the Operations, Applications and Components Technical Committee of the Pressure Vessel and Piping Division he has been a paper author, reviewer, topic organizer, technical program representative, and editor of the OAC Newsletter. Academically, he received the NSF Career Award, the SAE Ralph Teetor Award for engineering education excellence and faculty fellowships at NASS, ONR and Boeing.

Design & FEA of Reactor Pressure Vessel

He received the Outstanding Engineering Professor Award from the University of Alabama in Huntsville and the University of Alabama Foundation Award for Outstanding Research. Aravas is also an educator with numerous awards for excellence in teaching of graduate and undergraduate courses at the Universities of Pennsylvania and Illinois.

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He is currently the chair of the Emerging Technologies Division of the American Society for Composites, an associate fellow in the American Institute of Aeronautics and Astronautics, and serves as a scientific advisor for Golf Digest. Han has authored over 80 peer-reviewed journal papers. Wu Christine Wu's theoretical research on nonlinear dynamic systems has been a significant contribution to engineering applications.

Low

Ross has chaired and co-chaired conference sessions on computer architecture, served on the ASME nominating committee, and is District A History and Heritage Chair. She is the current president of CSME and she has tirelessly promoted and supported women developing careers in engineering.

Design & FEA of Reactor Pressure Vessel

He is well known in ASME Codes and Standards as an active member of the Section XI Standards Committee as well as chair and member of a number of Working Groups.

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As an illustration, the crack growth rates given in Section XI of the ASME Code are used to include reactor water effects on A533 reactor pressure vessel steel in the fatigue design curves of Section III. The University of Illinois at UrbanaChampaign, and the University of Michigan, as well as senior staff at Intel, AMD, Tesla, Freescale, and other high-tech companies, he has co-authored more than 120 archival journal articles, 24 patents, two books, and eight book chapters. Porowski, ASME Pressure Vessels Piping Conference, San Francisco, California, ASME 79:PVP Vol.

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