

¹ SCIANTIX: an open-source multi-scale code for fission gas behaviour modelling designed for nuclear fuel performance codes

⁴ **Davide Pizzocri¹, Giovanni Zullo¹, Elisa Cappellari¹, Giovanni Nicodemo¹,**
⁵ **Aya Zayat¹, and Lelio Luzzi¹**

⁶ 1 Politecnico di Milano, Department of Energy, Nuclear Engineering Division, Milano, Italy

DOI: [10.xxxxxx/draft](https://doi.org/10.xxxxxx/draft)

Software

- [Review](#) ↗
- [Repository](#) ↗
- [Archive](#) ↗

Editor: ↗

Submitted: 29 November 2025

Published: unpublished

License

Authors of papers retain copyright and release the work under a Creative Commons Attribution 4.0 International License ([CC BY 4.0](https://creativecommons.org/licenses/by/4.0/))

⁷ Summary

⁸ SCIANTIX is an open-source multi-scale code for modelling fission gas behaviour in oxide
⁹ nuclear fuel, together with the relevant microstructural phenomena. The code behaves as a
¹⁰ 0D meso-scale solver implementing physics-based rate-theory models, designed to be either a
stand-alone tool for separate-effect calculations or a module embedded in engineering thermo-
mechanical fuel performance codes (FPCs). Compared with empirical approaches used in FPCs,
¹¹ SCIANTIX solves evolutionary equations for stable and radioactive intra-granular gas diffusion,
¹² trapping, re-solution, grain-boundary bubble evolution, diffusional and burst release, helium
¹³ behaviour and high-burnup structure (HBS) porosity. The code adopts verified numerical
¹⁴ solvers and includes a regression suite to ensure reproducibility.

Statement of need

¹⁸ Engineering FPCs typically rely on empirical correlations for modelling fission gas release and
¹⁹ gaseous swelling. The underlying physical processes are commonly described using classical
²⁰ rate-theory formulations ([Forsberg & Massih, 1985a, 1985b](#)), which are often simplified in
engineering codes for robustness and computational speed. These implementations are usually
²¹ calibrated for specific datasets or reactor conditions and are rarely available as open-source
²² software, limiting reproducibility and research efforts for new fuel designs or irradiation scenarios.
²³

²⁴ SCIANTIX addresses these limitations by providing an open-source physics-based meso-scale
²⁵ module ([Pizzocri et al., 2020](#)), together with a modular C++ architecture that enables
²⁶ extensions and direct coupling to external multi-physics solvers ([Giovanni Zullo et al., 2023](#)).
²⁷ The code is supported by numerical verification through the Method of Manufactured Solutions
²⁸ (MMS) for all employed solvers, and by a regression testing suite covering intra- and inter-
²⁹ granular swelling, HBS porosity, helium behaviour and radioactive gas release. A stable API
³⁰ facilitates its integration into engineering-scale codes, and is already used for online coupling
³¹ with TRANSURANUS, FRAPCON/FRAPTRAN and OFFBEAT ([Giovanni Zullo et al., 2024](#)).

³² Software description

³³ Implementation

³⁴ SCIANTIX (>2.0) adopts an object-oriented structure, in which matrices, gas, systems, models
³⁵ and solvers are implemented as independent classes. Separation between solvers and models
³⁶ supports independent MMS verification and separate-effect model validation. The spectral
³⁷ diffusion solver provides a meshless approach with controlled numerical error for Booth-type

38 diffusion problems. First-order L-stable implicit time integrators ensure numerical stability. A
39 segregated operator-splitting scheme maintains CPU-time compatibility in online coupling with
40 engineering-scale thermo-mechanical codes.

41 **Functionality**

42 SCIANTIX models intra-granular diffusion, trapping and irradiation-induced re-solution; nu-
43 cleation and growth of intra-granular bubbles; grain-boundary bubble growth, coalescence,
44 saturation and fission gas release; HBS formation and porosity evolution; helium diffusion,
45 solubility and thermal re-solution; the release of short-lived radioactive fission gases through
46 diffusion–decay with first-precursor enhancement.

47 **Verification and Validation**

48 The numerical solvers are verified using the Method of Manufactured Solutions (Oberkampf et
49 al., 2004; G. Zullo et al., 2022), with verification tests available in the repository. Separate-
50 effect validation reproduces the published results (Pizzocri et al., 2020; Giovanni Zullo et al.,
51 2023) without parameter tuning and using published models and parameters.

52 **Research efforts and ongoing developments**

53 SCIANTIX is continuously developed within international research projects (e.g., R2CA, PA-
54 TRICIA, OperaHPC, TRANSPARANT). Current development covers digital-twin workflows,
55 in which SCIANTIX provides fast calculations of helium behaviour for real-time monitor-
56 ing; reduced-order models enabling accelerated surrogate evaluations of complex multi-scale
57 phenomena; machine-learning assisted developments such as Gaussian Process regression
58 for automatic correlation updates; and extensions towards volatile fission products including
59 thermo-chemical evaluations.

60 **Acknowledgements**

61 The authors acknowledge contributions from collaborators providing feedback and support
62 during development.

63 **References**

- 64 Forsberg, K., & Massih, A. R. (1985a). Diffusion theory of fission gas migration in irradiated
65 nuclear fuel UO₂. *Journal of Nuclear Materials*, 135(2-3). [https://doi.org/10.1016/0022-3115\(85\)90071-6](https://doi.org/10.1016/0022-3115(85)90071-6)
- 66 Forsberg, K., & Massih, A. R. (1985b). Fission gas release under time-varying conditions.
67 *Journal of Nuclear Materials*, 127(2-3). [https://doi.org/10.1016/0022-3115\(85\)90348-4](https://doi.org/10.1016/0022-3115(85)90348-4)
- 68 Oberkampf, W. L., Trucano, T. G., & Hirsch, C. (2004). Verification, validation, and predictive
69 capability in computational engineering and physics. *Applied Mechanics Reviews*, 57(5),
70 345–384. <https://doi.org/10.1115/1.1767847>
- 71 Pizzocri, D., Barani, T., & Luzzi, L. (2020). SCIANTIX: A new open source multi-scale code
72 for fission gas behaviour modelling designed for nuclear fuel performance codes. *Journal of
73 Nuclear Materials*, 532, 152042. <https://doi.org/10.1016/j.jnucmat.2020.152042>
- 74 Zullo, G., Pizzocri, D., & Luzzi, L. (2022). On the use of spectral algorithms for the prediction
75 of short-lived volatile fission product release: Methodology for bounding numerical error.
76 *Nuclear Engineering and Technology*, 54(4), 1195–1205. <https://doi.org/10.1016/j.net.2021.10.028>

⁷⁹ Zullo, Giovanni, Pizzocri, D., & Luzzi, L. (2023). The SCIANTIX code for fission gas behaviour:
⁸⁰ Status, upgrades, separate-effect validation, and future developments. *Journal of Nuclear
⁸¹ Materials*, 587, 154744. <https://doi.org/10.1016/j.jnucmat.2023.154744>

⁸² Zullo, Giovanni, Pizzocri, D., Scolaro, A., Van Uffelen, P., Feria, F., Herranz, L. E., & Luzzi,
⁸³ L. (2024). Integral-scale validation of the SCIANTIX code for light water reactor fuel rods.
⁸⁴ *Journal of Nuclear Materials*, 601, 155305. <https://doi.org/10.1016/j.jnucmat.2024.155305>

DRAFT