# NE 155 Introduction to Numerical Simulations in Radiation Transport

**Lecture 35: Geometry and Collisions** 

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#### **OUTLINE**

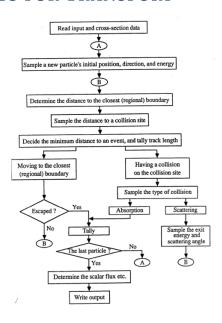
- Overview of Monte Carlo for Neutron Transport
- 2 Determining next event location
  - Sampling flight path
  - Distance to boundary
  - Next event selection
- 3 Collision Physics
  - Sampling target nuclide
  - Sampling reaction type
  - Sampling exit direction
  - Sampling exiting particles

Notes derived from Jasmina Vujic and Paul Wilson

## **LEARNING OBJECTIVES**

- 1 Be able to provide MC transport algorithm
- 2 Understand basic tracking of particles through a geometry
  - Understand the steps necessary for tracking particles
  - Understand the use of mean free path
  - Sample the distance to the next physics event
  - Determine next event
- 3 Understand what sampling needs to happen after a collision

#### MONTE CARLO FOR TRANSPORT



## POSSIBLE FUTURES FOR A PARTICLE

After we've gotten to Circle B, we have a neutral particle:

- At point  $(x_p, y_p, z_p)$
- Moving in direction (u, v, w)
- With energy E

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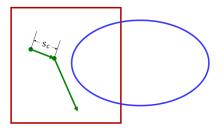


Figure 1: Collision

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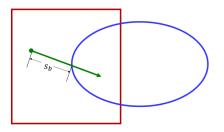


Figure 1: ../figs/Surface Crossing

## SAMPLING DISTANCE TO COLLISION

#### Collisions are probabilistic

- Note that  $\Sigma_t$ , the total macroscopic cross section, will be a function of space if we have multiple materials
- Along a particular path, the *probability of a collision at distance s* from the start:

$$p_c(s) = \Sigma_t(s)e^{-\Sigma_t(s)s}ds$$

$$P_c(n) = \int_0^s \Sigma_t(s)e^{-\Sigma_t(s)s'}ds' = -e^{-\Sigma_t(s)s'}|_0^s = 1 - e^{-\Sigma_t(s)s}$$

• The cross section,  $\Sigma_t(s)$ , is piecewise constant, but changing

## SAMPLING DISTANCE TO COLLISION

• Variable transformation: measure distance in units of *mean free path*:

$$n = \Sigma_t(s)s$$
,  $dn = \Sigma_t(s)ds$ 

We'll start with the PDF and integrate to get the CDF

$$p_c(n)dn = e^{-n}dn$$
  
 $P_c(n) = \int_0^n e^{-n'}dn' = -e^{-n'}|_0^n = 1 - e^{-n}$ 

• Importantly, this is now independent of the material

## SAMPLING DISTANCE TO COLLISION

Randomly sample to determine number of mean free paths until next collision,  $n_c$ 

- $g(n_c)dn_c = e^{-n_c}dn_c$
- $G(n_c)dn_c = 1 e^{-n_c}$
- Directly invert to get:  $n_c = -\ln(1-\xi)$  (note  $1-\xi$  is equivalent to  $\xi$ )
- In the absence of material boundaries ( $\Sigma \neq f(s)$ ), the distance to a collision,  $s_c$ , is

$$s_c = \frac{n_c}{\Sigma_t}$$

## CALCULATING DISTANCE TO BOUNDARY

- Usually have more than one material
- Distance to boundary is deterministic
- Algebra to determine distance between point and surface,  $s_b$
- Convert it to units of mean free path for the current cell's material,

$$n_b = s_b \Sigma_t$$

## **GEOMETRY REPRESENTATIONS**

- Combinatorial Surfaces
  - Define surfaces
  - Boolean operations combine surfaces to create cells
- Combinatorial Solids
  - Choose solid objects
  - Boolean operations combine objects to create regions
- B-Rep (Vertex-Edge)
  - Each object is a single set of vertices and edges connecting them

We're skipping how to find  $n_b$ , just know that we can find it

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  - Sample new energy
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- · Some of these may depend on each other
- Repeat
  - Sample new  $n_c$  following collision
  - Calculate new  $n_b$  in new direction

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- Move particle along ray
  - Update  $n_c = n_c n_b$
- **DO NOT SAMPLE** for new *n<sub>c</sub>*

## **OPTION B: CELL BOUNDARY**

#### $n_b < n_c$ :

- Boundary is closer than collision
- Boundary crossing occurs
- Move particle along ray
  - Update  $n_c = n_c n_b$
- **DO NOT SAMPLE** for new *n<sub>c</sub>*
- Calculate new  $n_b$  in new cell
  - New set of boundaries
  - New value of  $\Sigma_t$

## So You HAD A COLLISION?

• Sample **target nuclide** for a mixture with *J* nuclides

$$\Sigma_t = \sum_{j=1}^J N_j \sigma_{t,j}$$

• Discrete PDF to determine which nuclide is hit

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• Sample reaction type for an nuclide with R types of reactions

$$\Sigma_{t,j} = \sum_{x=1}^{R} \Sigma_{x,j}$$

• Discrete PDF to determine which reaction occurs

$$p_x = \frac{\Sigma_{x,j}}{\Sigma_{t,j}}$$

## **OUTCOME OF REACTION**

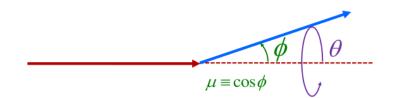
- Particle maybe absorbed
- Particle may continue its history in a different direction and with a different energy
- Energy-angle distributions are tabulated in different formats
  - Scattering laws have analytic forms with parameters in data tables (Direct inversion or rejection sampling)
  - Tabulated data that describes a piecewise analytic interpolation (Hybrid sampling; we skipped this)

#### USING A SCATTERING ANGLE

Scattering angles are defined relative to the original direction (considered as the z-axis)

- Polar angle,  $\phi$ , determined by sampling from data
- Azimuthal angle,  $\theta$ , determined by sampling isotropically
- The new direction is  $(\sin(\phi)\cos(\theta),\sin(\phi)\sin(\theta),\cos(\theta))$

$$= \left(\sqrt{1-\mu^2}\cos(\theta), \sqrt{1-\mu^2}\sin(\theta), \mu\right)$$



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We've developed a general sense of using MC for neutron transport

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- Repeat analysis for collisions/crossing until particle terminates

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We've developed a general sense of using MC for neutron transport

- Basic Algorithm
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- *After a collisions* we need to determine many things associated with the collisions (target, reaction, direction, energy)
- Repeat analysis for collisions/crossing until particle terminates
- Next time: tallying results