

The laboratory microscope glass slide is found to contain 0.83 ppm of uranium and a uniform distribution of induced fission tracks. It can serve as a cheap and reliable glass dosimeter for routine neutron flux density measurements in a reactor at high neutron fluences with a calibration constant of 1.1×10^{11} n/track.

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Intercalibration of Glass Dosimeters for Neutron Fluence Determination

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Fission track technique can be used for neutron fluence measurement from a reactor using a glass dosimeter. Calibration constants of various glass dosimeters are determined using Fleischer's 0.4 ppm glass as reference standard. The calibration constant for our laboratory glass slide is $\sim 1.1 \times 10^{11}$ n/track.

1. Introduction

GLASS dosimeters doped with known concentrations of uranium are being widely used in various laboratories all over the world for neutron fluence measurement in nuclear reactors. The glass standards being used are mostly supplied by Corning Museum of Glass, Corning, New York; General Electric Company, Schenectady, New York and National Bureau of Standards, Washington, DC, U.S.A.

FLEISCHER *et al.*⁽¹⁾ were the first to standardize a microscope glass slide for neutron flux density determination of a reactor by using both fission track counts and radiochemical techniques. The U content of this glass is 0.4 ppm and the thermal neutron fluence is calculated by using the relation:⁽¹⁾

$$\phi = k\rho \quad (1)$$

where ρ is the number of tracks cm^{-2} counted over the internal surface of glass dosimeter and k , the calibration constant, which is estimated to be 2.26×10^{11} neutron/track.

SCHREURS *et al.*⁽²⁾ made use of Corning glass with 41 ppm of U content as the reference standard (U_1) and prepared a number of sub-reference standards (U_2 – U_7) by using fission track calibration. The value of k , the calibration constant for U_1 glass is $\sim 4 \times 10^9$ n/track.

CARPENTER and REIMER⁽³⁾ of National Bureau of Standards have done an excellent work in preparing fission track glass dosimeter SRM's 961, 962, 963 and 964. These are available in the form of wafers at four different U concentrations of 461.5 ppm, 37.88 ppm, 0.823 ppm and 0.0721 ppm, respectively. They are prepared from the same lot of material as SRM's 610 through 616. In preparation of these dosimeters, the uranium used for the dopant is depleted uranium but they are certified for more than 60 trace elements and at present considered to be the best standards available.

The purpose of this study is twofold—to calibrate a glass

2. Experimental

Sample preparation and irradiation

Glass samples were prepared from a series of glasses, i.e. Fleischer's GEC glass standards with 0.4 ppm and 20 ppm U content, NBS standards SRM's 612, 614 and 962, and a laboratory soda lime microscope glass slide to be calibrated. All the samples were washed with alcohol and wrapped in polythene jackets before packing into Al irradiation capsules. The irradiations were carried out in the thermal column of 40 MW CIRUS reactor at BARC Trombay, Bombay at a conventional neutron fluence of 10^{15} (nvt) using IC-1 self serve position. The neutron flux density in this position is $\sim 5 \times 10^{12}$ $\text{n cm}^{-2} \text{s}^{-1}$ and is predominantly thermalized.

Dose measurement

Irradiated glass samples were fractured, ground with cerium oxide and polished with alumina powder. Fleischer's glass (0.4 ppm) was etched in 40% HF at 30°C for 30 s. The track counting was done on internal surface for 4π geometry under total magnification of 300 \times using Olympus microscope. The neutron fluence calculated by using equation (1) is 0.994×10^{15} (nvt) which is amazingly close to the nominal value of 10^{15} (nvt).

The above procedure was repeated for SRM 962 and other glass specimen. Thermal neutron fluence was calculated for SRM 962 by comparison of its induced track density with that of irradiated wafer supplied by NBS using the mean value of neutron flux density for Cu and Au foil irradiations in RT-3 position.⁽³⁾ The irradiation fluence is estimated to be 1.046×10^{15} (nvt). Thus the irradiation fluences calculated by using Fleischer's glass standard and NBS-SRM 962 for CIRUS reactor differ by 5% only. The counting statistical errors are nearly the same in two cases.

The data for all the irradiated glass samples is summarized in Table 1.

Etching efficiency

The most troublesome parameter in neutron glass dosimetry is etching efficiency factor E .^(4,5) Glasses are easily etched by HF acid. The etching efficiency depends upon concentration of the etchant, its temperature and time of etching. An optimum combination of these parameters has to be found by a hit and miss method for proper revelation of tracks.

In our study Fleischer's reference glass standard was repeatedly etched in 40% HF at 30°C for varying intervals of time starting from 5 s. Induced fission tracks were counted after each etching. It was observed that the track density attains a plateau value for an etching interval of 30 s.

This procedure was repeated for other glass dosimeters and it was observed that optimum etching conditions depend upon the chemical composition of the material. The numerical value of E for optimum etching conditions approaches unity.

Geometry factor

A correction known as geometry factor (G)^(6,7) has to be applied in case of track measurements carried over external surfaces of glass dosimeters, i.e. under conditions of 2π geometry. This factor varies with the composition of the material and hence with its etching parameters. Under