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Sophia Zhu

Undergraduate Intern

Congjian Wang

Modeling/Simulation
Professional

Jisuk Kim

Computational Scientist

Safety Assurance of Software and ML Development for Nuclear Instrumentation and Controls

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Battelle Energy Alliance manages INL for the
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Introduction

- Nuclear power plants are safety-critical
- Instrumentation and control (I&C) equipment monitors and controls reactor processes
 - Shift to digital (DI&C)
- Challenges with software:
 - Complex — cannot test every case [1]
 - ML & AI introduce uncertainty [2]
- Focus on having a high-quality process of developing the software [1]

Approaches to Safety Assurance

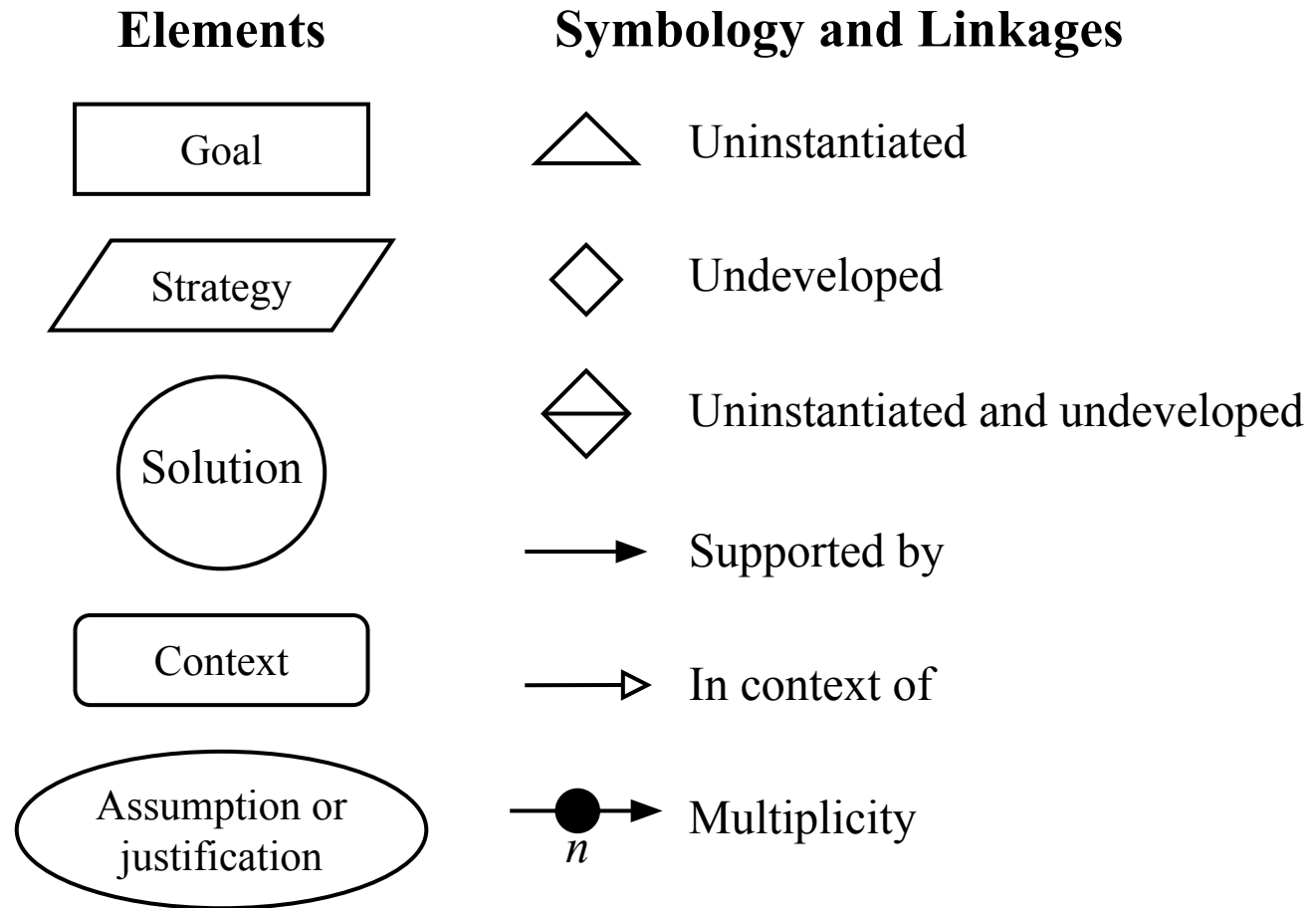
- Traditional U.S. NRC approach
 - Applicants submit hundreds of documents demonstrating safety of their reactor design
- Alternative: Safety assurance case (SAC)
 - Structured safety argument with supporting evidence [3]
 - Argument can be illustrated using diagram [3]

Research Objectives

- Identify software-related safety arguments through literature survey
 - Research papers
 - NRC regulations
 - IEEE and IEC standards
- Generate a SAC framework for DI&C software
 - Illustrate using Goal Structuring Notation (GSN)

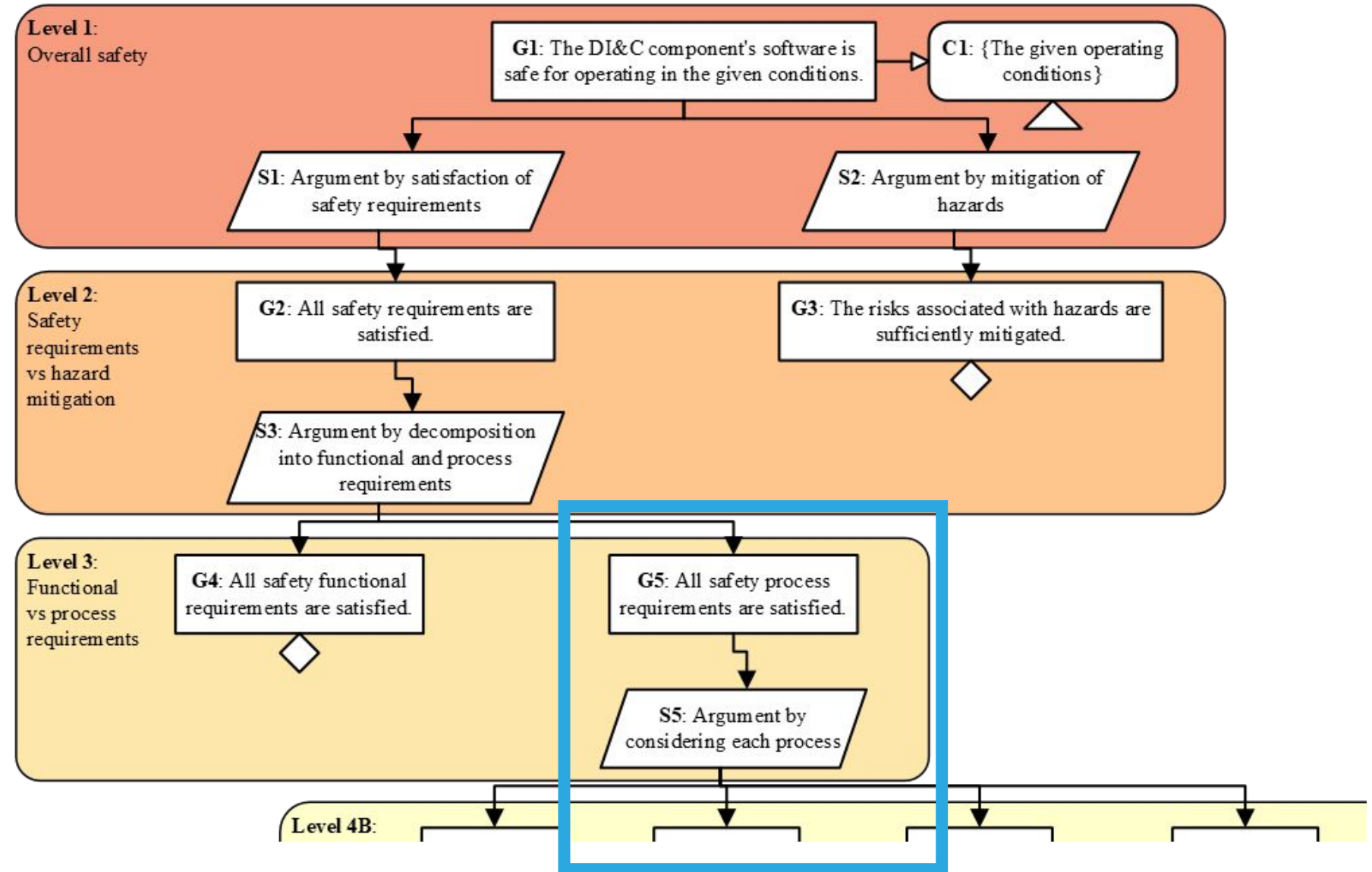
GSN Symbology

From the GSN
Community
Standard [4]



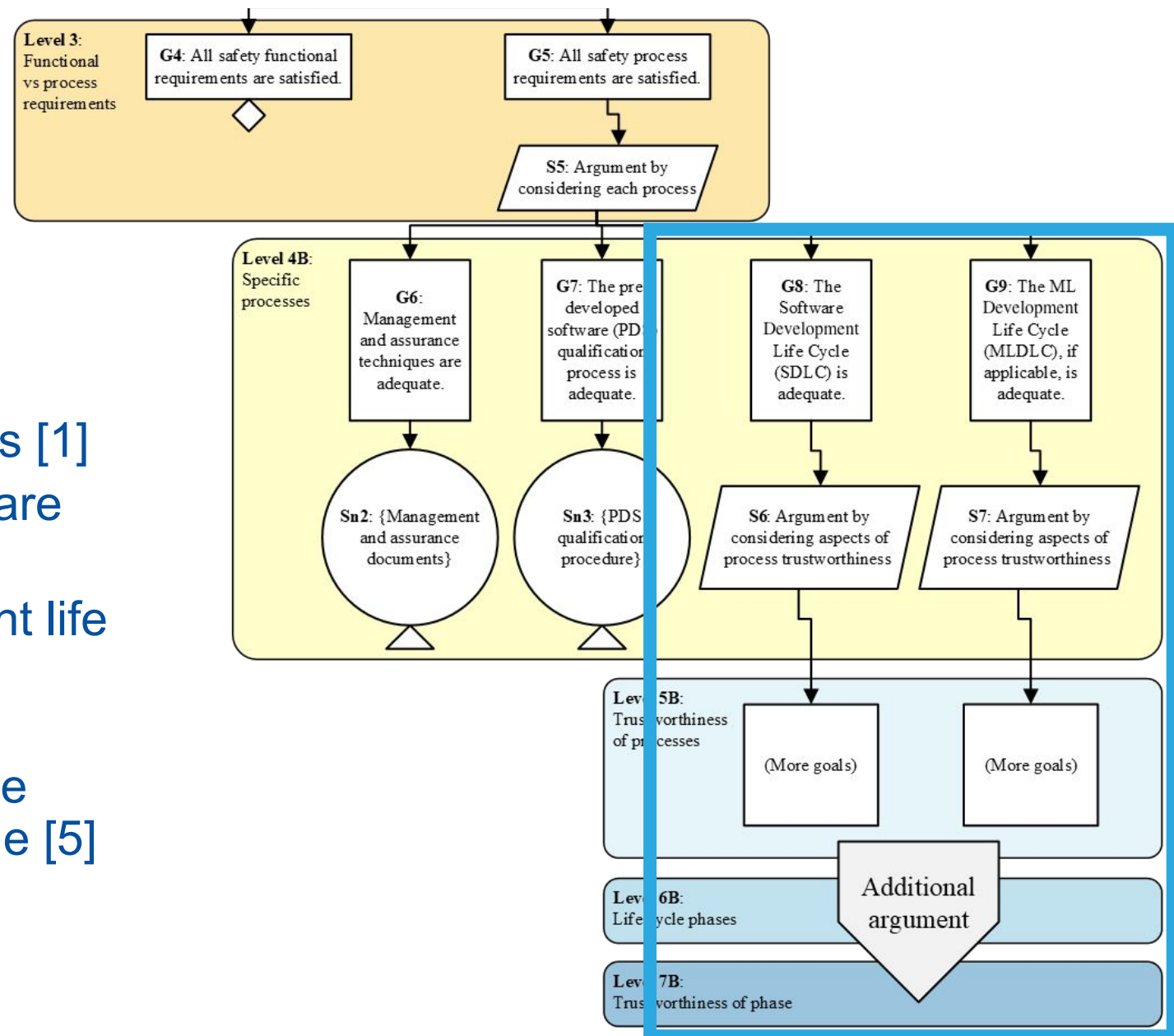
Upper Levels of Framework

- Every 2 rows forms a level
- Focus on G5: safety process requirements



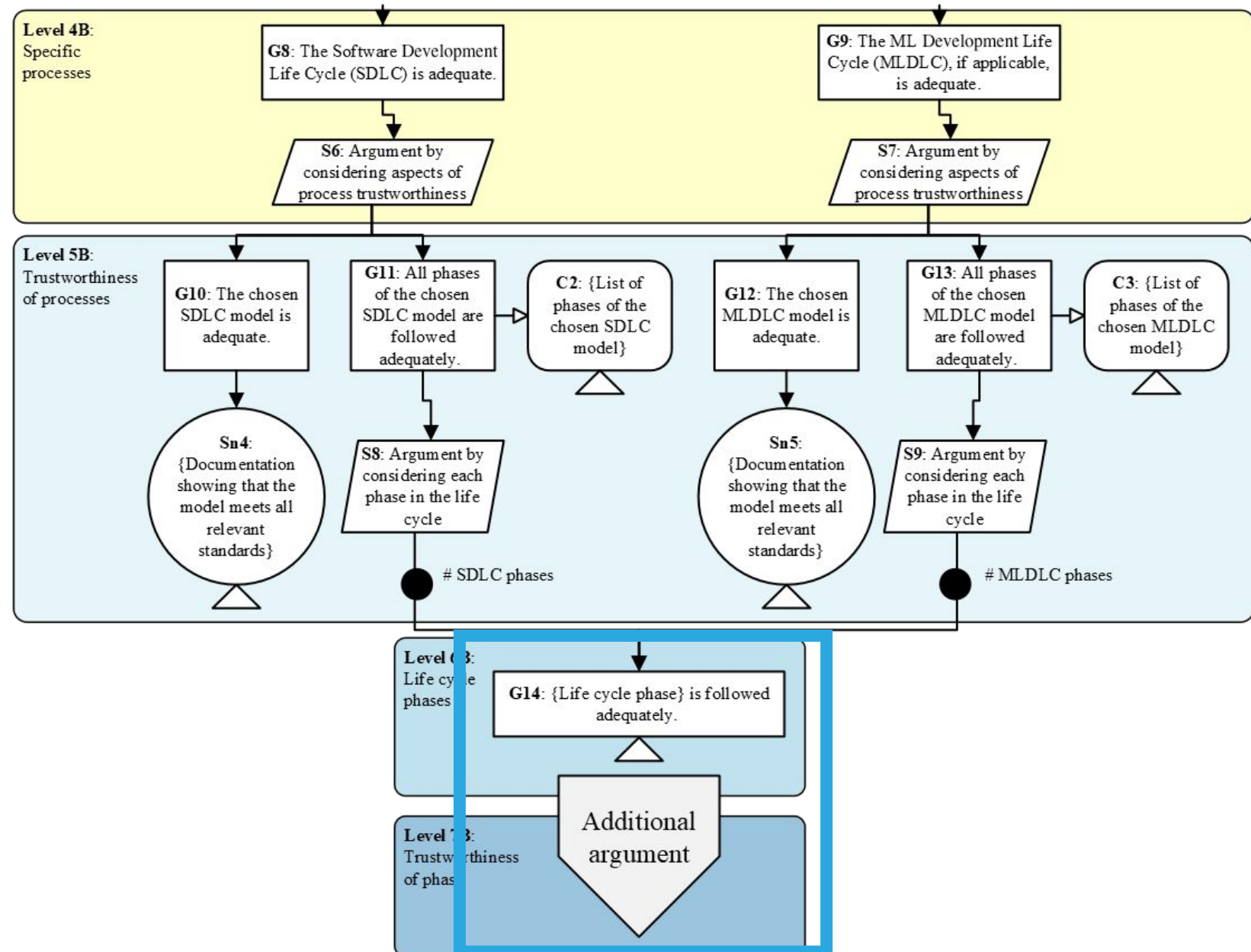
Lower Levels of Framework

- Identified four main processes
 - Management and assurance techniques [1]
 - Pre-developed software qualification [1]
 - Software development life cycle (SDLC) [1]
 - Machine learning development life cycle (MLDLC), if applicable [5]



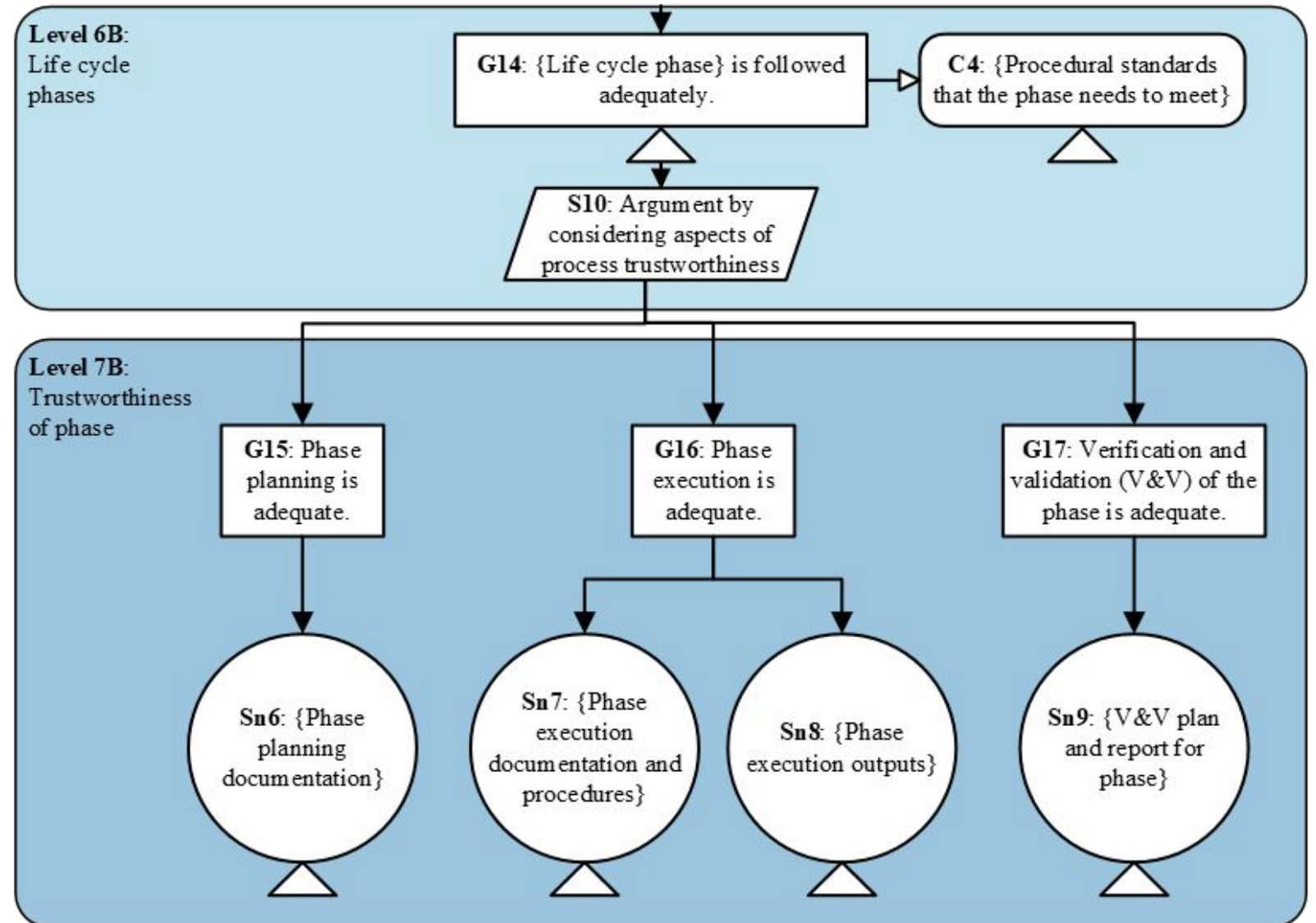
Development Life Cycles (Software and Machine Learning)

- Must select suitable development life cycle model and follow all phases adequately [6]



Life Cycle Phase

- Demonstrate each life cycle phase is adequately followed
- Involves proper planning, execution, and verification & validation (V&V) [7]



Conclusion and Future Work

- Our framework organizes safety assurance arguments for software development processes in DI&C components.
- Work in progress
 - Developing the rest of the SAC framework (functional requirements, hazard mitigation branches)
 - Conducting a case study on the APR 1400 reactor

References

- [1] Nuclear Regulatory Commission, “Appendix 7.0-A: Review Process for Digital Instrumentation and Control Systems,” in *Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition*, NUREG-0800, Aug. 2016. [Online]. Available: <https://www.nrc.gov/docs/ML1601/ML16019A085.pdf>
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- [3] M. Sivakumar et al., “The Last Decade in Review: Tracing the Evolution of Safety Assurance Cases through a Comprehensive Bibliometric Analysis.” arXiv, Nov. 13, 2023. doi: 10.48550/arXiv.2311.07495. [Online]. Available: <http://arxiv.org/abs/2311.07495>
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- [6] *IEEE Standard Criteria for Programmable Digital Devices in Safety Systems of Nuclear Power Generating Stations*, IEEE 7-4.3.2, Aug. 2016, doi: 10.1109/IEEESTD.2016.7552419.
- [7] R. Youngblood III, H. Everett, and H. Dezfuli, “Application of Objectives-Drive Assurance Cases to System Development in an Evolving Acquisition Model,” Oct. 2022. [Online]. Available: <https://www.osti.gov/biblio/2403507>



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