## VIRGINIA ELECTRIC AND POWER COMPANY RICHMOND, VIRGINIA 23261

10CFR50.73

NIRR

**Virginia Electric and Power Company** North Anna Power Station 1022 Halev Drive Mineral, Virginia 23117

November 30, 2023

Attention: Document Control Desk U. S. Nuclear Regulatory Commission

Washington, DC 20555-0001

Serial No.: 23-321 NAPS:

RAP

Docket Nos.: 50-339 License Nos.: NPF-7

## Dear Sir or Madam:

Pursuant to 10CFR50.73, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Power Station Unit 2.

Report No. 50-339/2023-001-00

This report has been reviewed by the Facility Safety Review Committee and will be forwarded to the Management Safety Review Committee for its review.

Sincerely,

Lisa Hilbert

Site Vice President

North Anna Power Station

## Enclosure

Commitments contained in this letter: None

cc: United States Nuclear Regulatory Commission Region II Marquis One Tower 245 Peachtree Center Ave., NE, Suite 1200

Atlanta, Georgia 30303-1257

NRC Senior Resident Inspector North Anna Power Station

#### NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OMB: NO. 3150-0104 (10-01-2023) Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons ed into the licensing process and fed back to industry. Send comments regarding burden LICENSEE EVENT REPORT (LER) estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and the OMB reviewer (See Page 2 for required number of digits/characters for each block) at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory (See NUREG-1022, R.3 for instruction and guidance for completing this form Commission, 725 17th Street NW, Washington, DC 20503; email: oira submission@omb.eop.cov. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the dohttp://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/) requesting or requiring the collection displays a currently valid OMB control number. 1. Facility Name 2. Docket Number 3. Page 050 North Anna Power Station 00339 1 OF 3 052 Reactor Coolant Pressure Boundary Leak Due to Poor Weld Workmanship 5 Event Date 6. LER Number 7. Report Date 8. Other Facilities Involved Sequential Revision Facility Name **Docket Number** Month Year Month Year Day Number 050 **Facility Name Docket Number** 10 03 2023 2023 001 00 11 30 2023 052 9. Operating Mode 10. Power Level 6 000 11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply) 10 CFR Part 20 10 CFR Part 50 20.2203(a)(2)(vi) 50.73(a)(2)(II)(A) 50.73(a)(2)(vIII)(A) 73.1200(a) 20.2201(b) 20.2203(a)(3)(i) 50.36(c)(1)(I)(A) 50.73(a)(2)(II)(B) 50.73(a)(2)(viii)(B) 73.1200(b) 20.2201(d) 20.2203(a)(3)(ii) 50.36(c)(1)(ii)(A) 50.73(a)(2)(III) 50.73(a)(2)(ix)(A) 73.1200(c) 20.2203(a)(1) 20.2203(a)(4) 50.36(c)(2) 50.73(a)(2)(lv)(A) 50.73(a)(2)(x) 73.1200(d) 10 CFR Part 21 20.2203(a)(2)(i) 10 CFR Part 73 50.46(a)(3)(ii) 50.73(a)(2)(v)(A) 73.1200(e) 20.2203(a)(2)(ii) 21.2(c) 50.69(g) 73.77(a)(1) 50.73(a)(2)(v)(B) 73.1200(f) 20.2203(a)(2)(iii) 50.73(a)(2)(I)(A) 50.73(a)(2)(v)(C) 73.77(a)(2)(i) 73.1200(g) 20.2203(a)(2)(iv) 50.73(a)(2)(i)(B) 50.73(a)(2)(v)(D) 73.77(a)(2)(ii) 73.1200(h) 20.2203(a)(2)(v) 50.73(a)(2)(I)(C) 50.73(a)(2)(vii) OTHER (Specify here, in abstract, or NRC 366A). 12. Licensee Contact for this LER

Lisa Hilbert, Site Vice President

Phone Number (Include area code) (540) 894-2101

## 13. Complete One Line for each Component Failure Described In this Report

Cause	System	Component	Manufacturer	Reportable to IRIS	Cause	System	Component	Manufactu	rer Repo	rtable to IRIS
Α	AB	TBG		Y						
	14. Supplemental Report Expected					15. Expected Submission Date		Month	Day	Year
✓ No	Y	es (If yes, comp	lete 15. Expec	ted Submission Date)		expected Submis	ssion Date			

<sup>16.</sup> Abstract (Limit to 1326 spaces, i.e., approximately 13 single-spaced typewritten lines)

On September 11, 2023, at 1558 hours with Unit 2 in Mode 5 at 140 degrees F and 30 psig for a refueling outage, a boric acid leak was discovered on tubing associated with a Pressurizer level transmitter. The leak was not quantifiable as it consisted of a small amount of dry boric acid. Non-destructive examination (NDE) was performed on the leak to determine if it was a through wall leak. On October 3, 2023, at 1154 with Unit 2 in Mode 6 at 100 degrees F and atmospheric pressure, the NDE determined the leak was a through wall leak. This failure constitutes welding or material defects in the primary coolant system that cannot be found acceptable under ASME Section XI. Therefore, an 8-hour report was made for a degraded condition under 10 CFR 50.72(b)(3)(ii)(A).

The direct cause of the weld failure was due to inadequate welding process control by the welder. Unit 1 was not impacted by this event. The health and safety of the public were not affected by this event.

#### NRC FORM 366A (10-01-2023)

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## LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

(See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503; email: oira submission@omb.eop.gov. The NRC may not conduct o sponsor, and a person is not required to respond to, a collection of information unless the docume

requesting or requiring the collection displays a currently valid OMB control number.

EXPIRES: 03/31/2024

APPROVED BY OMB: NO. 3150-0104

1. FACILITY NAME		050	2. DOCKET NUMBER	3. LER NUMBER					
North Anna Power Station			00339	YEAR	SEQUENTIAL NUMBER			REV NO.	
		052		2023	-	001	-	00	

#### NARRATIVE

1.0 Description of Event

On September 11, 2023, at 1558 hours with Unit 2 in Mode 5 at 140 degrees F and 30 psig for a refueling outage, a boric acid leak was discovered on tubing associated with a Pressurizer (EIIS Component PZR, System AB) level transmitter (EIIS Component LT, System AB). The leak was not quantifiable as it consisted of a small amount of dry boric acid. Non-destructive examination (NDE) was performed on the leak to determine if it was a through wall leak. On October 3, 2023, at 1154 with Unit 2 in Mode 6 at 100 degrees F and atmospheric pressure, the NDE determined the leak was a through wall leak. This failure constituted welding or material defects in the primary coolant system that cannot be found acceptable under ASME Section XI. Therefore, an 8-hour report was made for a degraded condition under 10 CFR 50.72(b) (3)(ii)(A). This weld was part of an instrument tubing design change that was implemented in 1998.

2.0 Significant Safety Consequences and Implications

No significant safety consequences resulted from this event. The leak was discovered while Unit 2 was shut down for a refueling outage. The leak was not quantifiable based on the small amount of boric acid noted and, therefore, well within the capability of one charging pump (EIIS Component P, System CB). The health and safety of the public were not affected by this event.

3.0 Cause of the Event

The direct cause of the weld failure was due to inadequate welding process control by the welder. The specific socket welds in question exhibited poor workmanship by having a large degree of melt-through and suck-back on the inside surface, and multiple arc strikes and excessive grinding on the outside surface. During the metallurgical failure analysis, the sample was sectioned through the area of suspected leakage, which revealed a lack of fusion defect between the weldment and base metal. The degree of lack of fusion confirmed in the laboratory analysis was substantial enough to provide the eventual leak path after a 25-year service period.

4.0 Immediate Corrective Action

Both the leaking socket welded coupling and a non-leaking downstream socket welded coupling were replaced. NDE surface examinations were performed on the replacement socket welds, and the area was also examined during an external leakage check.

5.0 Additional Corrective Actions

Additional Liquid Penetrant (LP) exams were performed on tubing welds from the pressurizer steam space to a different Pressurizer level transmitter. No weld flaws or boric acid residue was identified during these examinations. For the next refueling outages for each unit, work orders have been created to inspect tubing socket welds that were fabricated in a similar timeframe, under similar field conditions, using the same welding and inspection procedures, and had an overlap of qualified welders performing the work.

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## NRC FORM 366A (10-01-2023)

## U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

EXPIRES: 03/31/2024



# LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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1. FACILITY NAME	■ 050	2. DOCKET NUMBER	YEAR	3. LER NUMBER SEQUENTIAL	REV			
North Anna Power Station		00339	TEAR TO	NUMBER	NO.			
	052		2023	- 001	- 00			
NARRATIVE								
6.0 Actions to Prevent Recurrence	The state of the s			And the second s				
Nuclear Welding Program Authorized B	Ry training will be undate	d to include this operation	na evnerie	nce (OE) of wor	kmanehin			
	by training will be update	d to include this operation	ig experie	ice (OL) of wor	Killalisliip			
issues when welding tubing.					- 1			
7.0 Similar Events								
No similar events have been noted at N	North Anna for instrumen	t tube welding failures.						
8.0 Additional Information								
Unit 1 was unaffected by this event.								
					1/19			
					11/2			
					1.79			
					73			

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