# DEVELOPMENT OF A LOW ENERGY NEUTRON SOURCE FOR BUBBLE CHAMBER CALIBRATIONS

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20 Abstract

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The use of bubble chambers for direct dark matter detection requires high sensitivity to energy levels in the range of 1-100 keV and strict measures to reduce background radiation. Neutrons can be used to simulate WIMP elastic scattering interactions with the target volume in order to ensure high detection efficiency. We aim to develop a low energy neutron source that will allow us to properly calibrate bubble chambers to ensure their ability to detect such events. We propose a solution consisting of a neutron source composed of a radioisotope capable of emitting gamma radiation at the required energy thresholds and a target capable of ejecting photoneutrons when struck by the gamma radiation. We have chosen seven main candidates for a gamma source, taking note of important properties such as half-life, availability, cost, and many others. We have also calculated the theoretical energies of the neutrons emitted by each source and the rate at which each source would emit neutrons. We utilize the GEANT4 simulation software to explore various scenarios and determine effective neutron emission rates and the energies upon interaction with the C<sub>3</sub>F<sub>8</sub>. Results yielded from the Drexel Bubble Chamber will be useful for other members of the PICO collaboration and other direct detection experiments. [include other results as they come along]

### 37 1 Introduction

#### $_{ ext{ iny 38}}$ 1.1 Dark Matter & Bubble Chambers

The nature of cosmological dark matter has been an elusive mystery for many decades despite occupying nearly a quarter of the universe's content<sup>[1]</sup>. The modern theory of Weakly Interacting Massive Particles (WIMPs) requires highly specialized detection methods. WIMPs as the leading candidate address many critical issues in the ΛCDM Model such as galaxy rotation curves, the gravitational lensing of light in underdense regions, and the flatness of the universe<sup>[2]</sup>. Bubble chambers have been used since the early 1950's as particle detectors<sup>[11]</sup> and have more recently been adapted for the detection of WIMPs. The PICO

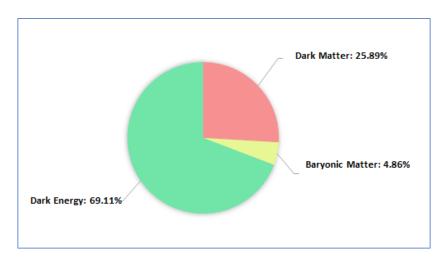


Figure 1: Fractional chart displaying the content of the universe. Dark matter composes a significant portion; however, it still has yet to be detected.

Collaboration uses the concept of bubble chambers with superheated liquids that provide some of the strongest constraints on the WIMP nucleon cross-sections<sup>[7]</sup>.

The ability to achieve low background is critical to direct detection methods of WIMPs.

Several design decisions have been included to maximize detection efficiency and ensure

accuracy of events. Fluorine based liquids are especially appealing as a target volume due

to their insensitivity to gamma and beta particles<sup>[8]</sup>. The use of piezoelectric sensors allows

for discrimination of alphas by acoustic analysis. Muons are eliminated through the use of

a water tank containing PMT's. Reconstruction of events in the target volume allows for

discrimination of neutron events due to bubble multiplicity.

Previous bubble chambers<sup>[6]</sup> in the PICO collaboration utilized an up-side down design
where the target volume was placed below a buffer fluid as seen in Figure 2a. This chamber
faced many issues in the form of particulate falling down into the target volume and other
issues at the water-target fluid interface. The Drexel Bubble Chamber is a prototype rightside up design that aims to remove these issues by flipping the design. In order to isolate
the target volume, we use a sharp temperature gradient between the active and non-active
regions of the chamber as seen in Figure 2b.

The bubble chambers require calibrations with several types of sources to ensure that

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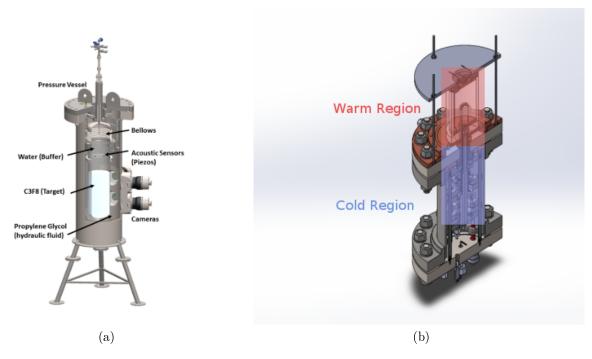


Figure 2: (a) Design of PICO-60 bubble chamber. This design places the target volume below a water buffer layer. (b) CAD design of the Drexel Bubble Chamber. This chamber utilizes a right-side up design to minimize particulate from falling into the target volume, removes the need for a buffer fluid, and allows easy access to the target volume.

each particle is detected and identified correctly. Most importantly, we must be sure that
the conditions of the chamber allow for detection of WIMPs. As result, the detector must
be sensitive to elastic scattering interactions from nuclear recoils in energy ranges of 1-100
keV. The best method for determining the efficiency of nuclear recoils is through low energy
neutron calibrations.

### <sup>58</sup> 1.2 Low Energy Neutrons

Neutrons can be generated through many different methods such as spontaneous fission,  $\alpha$  interactions on a low-atomic-weight target,  $\gamma$  interactions with a target such as  $^9\mathrm{Be}$  or  $^2\mathrm{D}$ , or through neutron generators. We desire to generate neutrons at relatively low ( $\leq$   $^2\mathrm{D}$ 00 keV) energy scales, and as a result, the only viable method for a neutron source is  $^2\mathrm{D}$ 10 through  $\gamma$ 20 interactions upon a target. Although the other methods may satisfy many of the requirements, they are only capable of producing neutrons on the MeV energy scale, which
is a vital component to this project. We require low energy neutrons to properly calibrate
bubble chambers for WIMP events which will occur at similar energy levels. Without neutron
calibration, it is impossible to tell if the chambers are able to efficiently and accurately detect
potential WIMP events.

Neutron sources for chamber calibrations have been considered before within the PICO 79 collaboration $^{[3]-[5]}$ ; however, a low energy neutron source has yet to be developed. The com-80 ponents of our neutron source will include a gamma source, target material, and appropriate 81 shielding to prevent gammas and neutrons from leaking in undesired directions. Starting 82 with an isotope that undergoes some form of decay, it will then emit gammas at a monoen-83 ergetic level. If the gamma contains more energy than the threshold for neutron production, 84 it will deposit enough energy upon contact and emit a neutron. The neutron will then come into contact with the bubble chamber and deposit its energy on to superheated C<sub>3</sub>F<sub>8</sub>. From there, our chamber will detect an event trigger, and we will be able to perform calibrations through image analysis, piezoelectric acoustic data, and other simulations.

$${}^{9}Be + \gamma \longrightarrow {}^{8}Be + n$$

$${}^{2}D + \gamma \longrightarrow {}^{1}D + n$$

$$(1)$$

Equation 1 shows the reactions of the targets with an incident gamma and the resulting isotopes and particles. From this equation, we are able to determine the threshold energy of the gammas needed to emit one neutron from each nucleus with simple conservation of energy. Since the number of electrons does not change during this process, we are free to emit them from the calculations since they will cancel. A similar calculation for  ${}^{2}D$  is omitted, but yields 2.23 MeV. For the  ${}^{9}Be$  target, we have

$$m_{^{9}Be} * c^{2} + Q = m_{^{8}Be} * c^{2} + m_{n} * c^{2}$$
  
 $\Rightarrow Q = (m_{^{8}Be} + m_{n} - m_{^{9}Be}) * c^{2} = 1.67 \ MeV$ 
(2)

The key difficulty in designing a low energy neutron source is finding an isotope that
fits all of our requirements. The isotope must have a long enough half-life such that it can
survive through the extensive safety procedures at Snolab without losing a majority of the
material. It should also produce gammas close to the threshold energy of the target such
that the emitted neutron will be in our desired energy range. It is important to have large
branching ratios for these gamma energies so that we obtain neutrons primarily at the energy
we desire. Finally, the isotope should be feasible to buy in large enough quantities without
either costing too much or requiring too much time to be delivered.

## <sup>103</sup> 2 Methods

#### <sup>04</sup> 2.1 Identifying the Proper Gamma Source

Our first approach to tackling this problem is to create an exhaustive list of all potential candidates for gamma sources and note the specific properties that we require them to have.

Included in the list will be each candidates gamma energy spectrum, branching ratios, halflife, and other information on the feasibility of acquiring the isotope. Another important property to keep track of is the neutron production rate. If the rate is too low, then we will not be able to perform accurate calibrations; however, we also want to avoid too many neutrons from being produced, as this could cause unwanted leakage that could cause issues elsewhere.

By searching the Table of Isotopes, a list has been developed as seen below in Table
1. The seven isotopes listed are the most likely in terms of their branching ratios, gamma
energies, half-lives, and feasibility of being obtained. Most gamma sources do not emit >2
MeV, so finding a source to use with a <sup>2</sup>D target is unlikely outside of <sup>226</sup>Ra.

Isotope	Target	Half-Life (Years)	Main Gamma Energy (keV)	Branching Ratios (%)
<sup>26</sup> Al	$^9{ m Be}$	$7.17\mathrm{E}\!+\!05$	1808	99.76
$^{207}\mathrm{Bi}$	$^9{ m Be}$	31.55	1770	6.87
$^{58}\mathrm{Co}$	$^9{ m Be}$	0.19	1674	0.52
<sup>150</sup> Eu	<sup>9</sup> Be	36.9	1690	0.15
<sup>124</sup> Sb	<sup>9</sup> Be	0.16	1690	47.79
<sup>88</sup> Y	<sup>9</sup> Be	0.29	1836	99.2
$^{226}$ Ra $(^{214}$ Bi $)$	$^2\mathrm{D}$	1590	2447	1.57

Table 1: Lists of potential gamma radioisotope sources with their corresponding targets, half-lives, relevant gamma energies, and branching ratios gathered from the Table of Isotopes.

#### 117 2.2 Calculations

After gathering the list of potential gamma sources, our next step is to calculate and gather as much information about each of the isotopes to better inform our decision. This data is listed below in Table 2. The expected theoretical neutron energies for each system is calculated through [9]:

$$E_n = \frac{A-1}{A} \left[ E_{\gamma} - Q - \frac{E_{\gamma}^2}{1862(A-1)} \right] + \delta; \tag{3}$$

Here, A is the mass number,  $E_{\gamma}$  is the energy of the incident gamma, Q is the threshold energy for the target, and  $\delta$  is an energy spread function defined by:

$$\delta \approx E_{\gamma} \left[ \frac{2(A-1)(E_{\gamma}-Q)}{931A^{3}} \right]^{1/2} cos(\theta)$$

$$\delta_{max} = 2E_{\gamma} \left[ \frac{2(A-1)(E_{\gamma}-Q)}{931A^{3}} \right]^{1/2}$$
(4)

The angle  $\theta$  is defined as the angle between the incident gamma and the emitted neutron.

As seen in Figure 3, the  $\theta$  dependence of  $\delta$  can have significant effects, and it is difficult to determine the value of  $\theta$ , so we will place the gamma source such that the gammas are incident in an isotropic manner, yielding  $\delta_{max}$  instead.

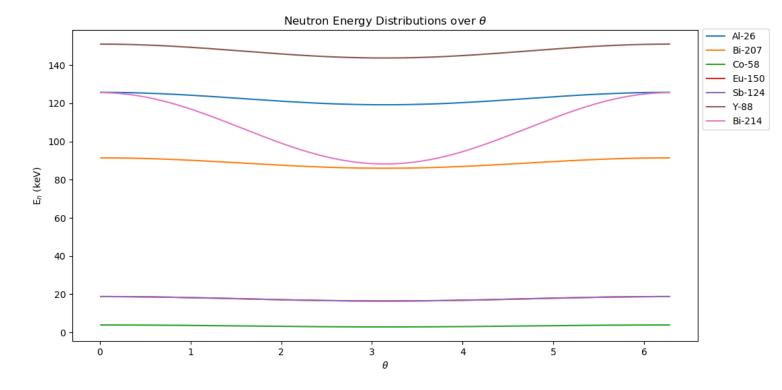


Figure 3: Expected neutron energies over a range of incident gamma angles from 0 to  $2\pi$ . This is shown for each of our potential sources. Some like <sup>214</sup>Bi have large variability while most only vary by a few keV.

A much more involved process is estimating the rate at which neutrons will be emitted from each system. We first start with basic assumptions that will allow for convenience when doing the calculations: we will have 1g of material for both the gamma source and target, and the distance between the source and target will be 1cm. From the half-life of the gamma source, the decay constant, and subsequently, the activity can be calculated:

$$t_{1/2} = \frac{ln(2)}{\lambda} \implies \lambda = \frac{ln(2)}{t_{1/2}}$$

$$A = \lambda N$$

Then the number of neutrons can be estimated through the most relevant factors: activity, branching ratio, the amount of target, the gamma-neutron cross section, and the surface area

Isotope	Theoretical Neutron Energy (keV)	n/s/g
<sup>26</sup> Al	128.99	539
$^{207}\mathrm{Bi}$	94.14	$1.52\mathrm{E}{+05}$
<sup>58</sup> Co	4.42	$2.27 \mathrm{E}\!+\!07$
<sup>150</sup> Eu	19.93	$2.51\mathrm{E}\!+\!04$
$^{124}\mathrm{S}\mathrm{b}$	19.93	$2.07\mathrm{E}\!+\!09$
88Y	154.62	$3.34\mathrm{E}\!+\!08$
$^{226}$ Ra( $^{214}$ Bi)	144.25	$9.00\mathrm{E}\!+\!03$

Table 2: Gamma sources with their corresponding expected neutron energies, neutron rates, and feasibility of obtaining each.

of a sphere. The gamma-neutron cross section can be found by reading them off plots from JANIS for each specific incident gamma energy. Combining these factors leads to equation 5:

$$\#n \approx A * \%Branching * \frac{m_{target}}{m_{target nucleus}} * \sigma * \frac{1}{SA_{sphere}}$$
 (5)

Two more calculations were done to determine how much deuterium is found per amount of heavy water since that is the most available form and how large would a sphere of that amount of heavy water be. The first is simply calculated by taking the ratios of the masses, so if we desire 1g of Deuterium as was used for the calculations above, we would need 10g of heavy water. The size of a sphere would then be found by equating the mass to the density times the volume of a sphere. This yields a radius of 1.29 cm, which is on the scale of the size that we would like the system to be at.

### 5 2.3 Choosing a Source

The availability of the seven potential gamma sources is listed below in Table 3. The first isotope to be explored is <sup>26</sup>Al, which looks to be a strong candidate at first; however, it is incredibly difficult and expensive to obtain. Only one supplier, Oak Ridge National Laboratory, is able to provide it at a cost of \$381 per nCi. This is incredibly expensive

Isotope	Availability
<sup>26</sup> Al	Too Expensive
$^{207}\mathrm{Bi}$	Available at 0.01 mCi
<sup>58</sup> Co	Expensive and high lead time
<sup>150</sup> Eu	No suppliers found
$^{124}\mathrm{Sb}$	Available at 0.1 mCi
88Y	Available at 0.1 mCi
$^{226}$ Ra( $^{214}$ Bi)	Available at 0.1 $\mu \mathrm{Ci}$

Table 3: Gamma sources with their corresponding expected neutron energies, neutron rates, and feasibility of obtaining each.

considering we require amounts on the scale of mCi for a neutron rate large enough. <sup>150</sup>Eu is extremely is rare, making it infeasible to use for a gamma source. Similarly, <sup>58</sup>Co is also expensive and rare, requiring too long of a lead time to acquire.

Of the remaining sources, <sup>226</sup>Ra is the weakest due to the available strength of the source and high theoretical neutron energy. This leaves the two most popular gamma sources, <sup>88</sup>Y and <sup>124</sup>Sb, and <sup>207</sup>Bi. Looking at Table 1, it's clear that <sup>124</sup>Sb will never make it through the Snolab safety procedures and would be replaced far too often, resulting in high expenses. <sup>88</sup>Y, although a strong contender, has been studied before and does not result in low neutron energies that we would like. As result, we are able to conclude that the best source for our purposes will be <sup>207</sup>Bi.

### 160 2.4 Improved Neutron Rates

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The calculations done above for the neutron rates were naive in a few ways. First, we assumed that we would have 1g of gamma source material, when it would be more proper to use the the quoted strength of the gamma sources instead. The target was approximated as a sphere, when it will actually be a cylinder. In addition, the target was assumed to be homogeneous and the mass was severely underestimated. In order to correct for this, we improve upon the calculations below.

The easiest issue to correct is that of the amount of gamma source material. The strengths

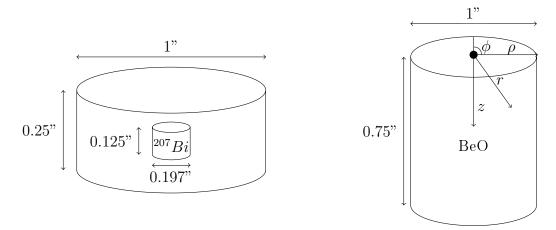


Figure 4: (a) Dimensional drawing of the <sup>207</sup>Bi source. The source consists of an active volume located inside a non-active disk. (b) Dimensional drawing of the gamma target, BeO. Integrating over this volume allows for more accurate calculations.

quoted in Table 1, when multiplied by a scaling factor of 3.7E5, will replace A in Equation 5. Next, the actual target that will be used is BeO, which given a density of  $3.02^g/cm^3$  and the dimensions in Figure 4b, yields a target mass of 29.15g. An additional factor of 9/25 will be picked up in Equation 5 due to the inhomogeneity of the target.

A better estimation of the neutron rate can be found from taking the average value of the function:

$$\bar{f} = \frac{1}{V} \iiint_D f(\rho, \phi, z) dV \tag{6}$$

Applying Equation 6 to our function in Equation 5 yields:

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$$\#n \approx A * \%Branching * (\frac{m_{target}}{m_{Be}} * \frac{9}{25}) * \sigma * \frac{1}{V} \int_{0}^{1.905} \int_{0}^{2\pi} \int_{0}^{1.27} \frac{\rho}{4 * \pi * r^{2}} d\rho d\phi dz \qquad (7)$$

Here,  $\rho = \sqrt{(r^2 + z^2)}$ , V is the volume of the cylinder in Figure 4b, and the additional mass factor has been included. The updated rates can be seen below in Table 3, and as a sanity check, we can compare our calculated rates with other results. For 1 Ci of <sup>88</sup>Y, the expected value is  $10E4 \text{ N/s/Ci}^{[8]}$ . For 0.1 mCi, we would expect a rate on the order of 10

Isotope	Strength	n/s	
$^{207}\mathrm{Bi}$	$0.01~\mathrm{mCi}$	0.30	
$^{124}\mathrm{S}\mathrm{b}$	$0.1~\mathrm{mCi}$	129.86	
<sup>88</sup> Y	0.1 mCi	25.89	

Table 4: Updated neutron rates for the three most likely gamma sources to be used.

n/s, which agrees with our calculation.

## 3 Analysis

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#### 3.1 Simulations

#### maybe this should be in methods too?

GEANT4 is a simulation software that allows for the tracking of particles as they pass through matter, recording properties of each event such as the volume collided with, energies of the particle, scattering types, and more. GEANT4 is highly versatile in its ability to simulate particle events for a wide array of purposes ranging from astroparticle physics to accelerator physics. The software allows for highly detailed and specialized construction of detectors, enabling for high accuracy simulations of many different scenarios.

Our first test with GEANT4 is to gather neutron deposition energies at varying distances. 189 The three potential locations that the neutron source can be placed at are far outside the 190 chamber, up against the mineral oil, or up against the C<sub>3</sub>F<sub>8</sub>. For each of these locations, 191 we use three different emission distributions: isotropic, arced, and linear. The results of 192 the simulations can be seen below in Figure 5. It is clear that a linear source placed as 193 close as possible results in the greatest number of hits and the highest average energy upon 194 recoil. The differences in those parameters between distances is inflated by the mineral oil. 195 Mineral oil is a strong shield against neutrons, greatly reducing both the energies and counts 196 of neutrons that reach the target volume. As the beam becomes less focused, from linear to 197 arc to isotropic, the energy and number of hits decreases as expected. 198

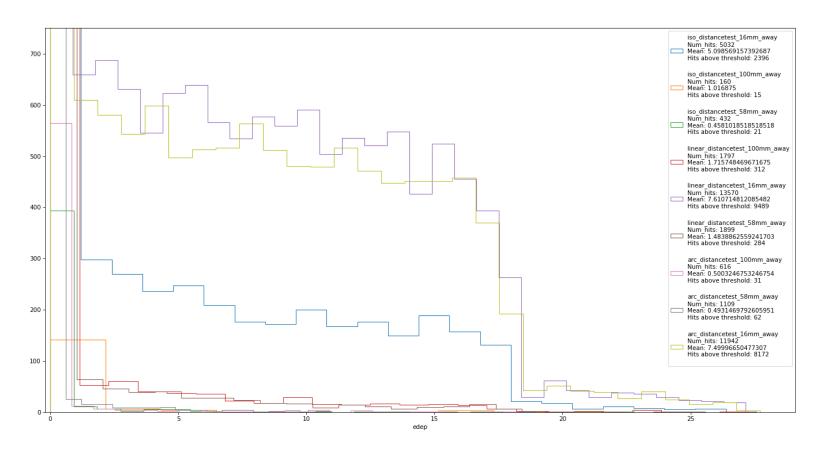


Figure 5: GEANT4 simulations overplotted to show the number of events in the target volume and the average energy of those events. These simulations were at varying distances from the center of the target volume. At 100mm, the source was placed far from the detector; at 58mm, the source was placed right outside the mineral oil; and at 16mm, the source was placed inside the mineral, right up against the  $C_3F_8$ .

Our second test is to determine whether placing a particle guide inside the mineral oil to allow for a clearer path for the neutrons to traverse towards the C<sub>3</sub>F<sub>8</sub>. We test with three different materials of air, copper, and lead with the same three emission distributions as the first test. Looking at Figure 6, we see, as expected, that air performs the best at allowing neutrons to travel through it. Surprisingly, the biggest factor in average energy and hit counts is how far away and how much mineral oil the neutron encounters. A linear beam

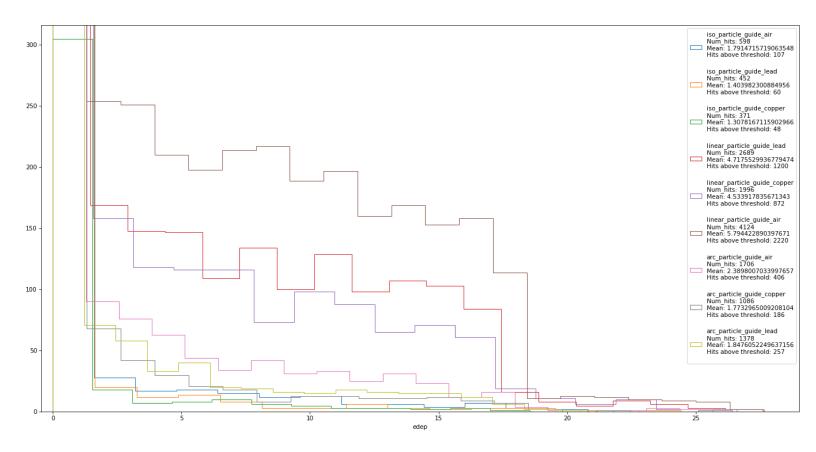


Figure 6: GEANT4 simulations overplotted to show the number of events in the target volume and the average energy of those events. These simulations were run with a particle guide made of air, lead, and copper introduced inside the mineral oil volume. Each material was also tested with three distribution types: isotropic, arced, and linear.

of neutrons placed 58mm away with a particle guide of air is only slightly better than an isotropic source placed 16mm away.

#### 08 3.2 Results

fill this in once actual data is collected

# 210 4 Conclusion

fill this in once data collection finished

## 212 4.1 Future Plans

[fill this in once at end of spring term]

- make sure everything in references is properly cited
- 215 [how to cite [3] and [4] not available to public]
- include typo in Alan's title for his slides?

### 217 References

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## Appendix A: GEANT4 Detector Construction

#### maybe move this text to simulations section

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The current model of the Drexel Bubble Chamber is simulated using a simplistic version of the detector. Within GEANT, the world is composed of volume of air surrounding the detector, which has four components. The outer most layer is a cylindrical container of acrylic, and the next layer within is a similarly shaped object used for the mineral oil. The next object located within the mineral oil is the quartz container for the inner most volume, the C<sub>3</sub>F<sub>8</sub>. The C<sub>3</sub>F<sub>8</sub> volume is set as the sensitive detector so that when neutrons scatter off the volume, GEANT will record the data. might not need all this images

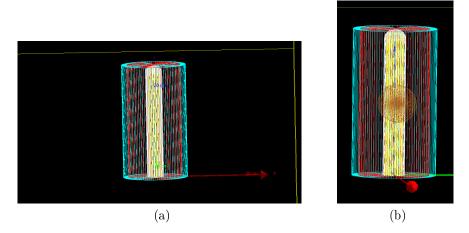


Figure 7: (a) GEANT4 assembly of the relevant Drexel Bubble Chamber volume. The components are colored as: acrylic: cyan, mineral oil: red, quartz: white, C<sub>3</sub>F<sub>8</sub>: yellow. The yellow outer lines define the world of the system, which is given air as a material. (b) The brown material is the inclusion of a particle guide to aid in the neutrons traversing the mineral oil.

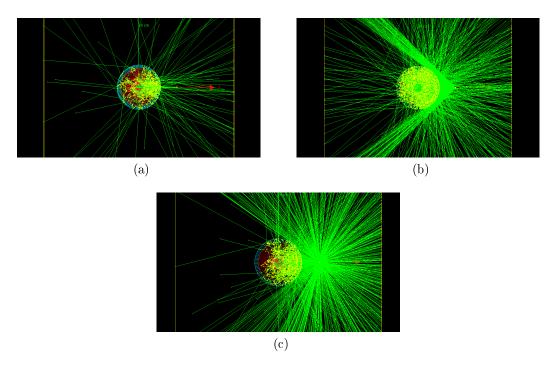


Figure 8: (a) Linear beam of neutrons shot at the chamber. (b) Isotropic beam limited to an arc aimed at the chamber. (c) Completely isotropic source.

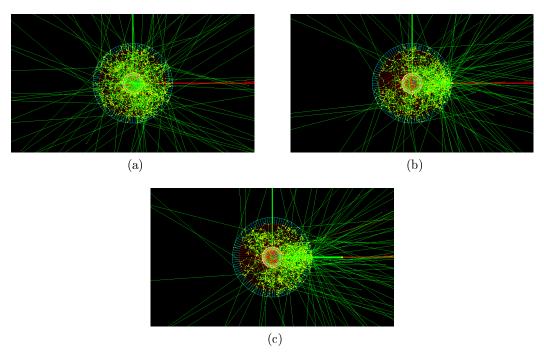


Figure 9: (a) Neutron source located at 16mm away, inside the mineral oil. (b) Neutron source located 58mm away right outside the mineral oil. (c) Neutron source located far away from the chamber.

# <sup>249</sup> Appendix B: $\gamma$ -neutron Cross Section Plots

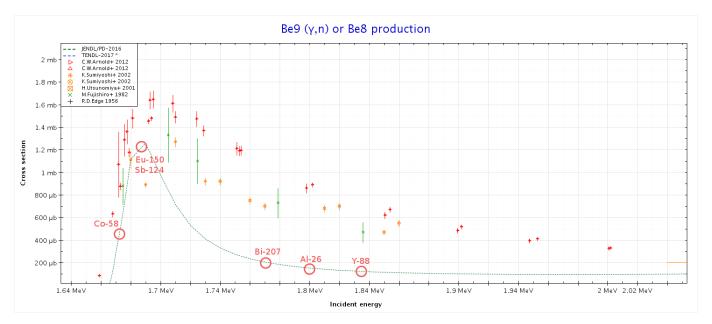


Figure 10: [maybe zoom in/remove legend for better visibility] Cross sections for gamma-neutron interactions with a <sup>9</sup>Be target as a function of incident gamma energies. Highlighted are the energies of the emitted gammas from each potential source. [10]

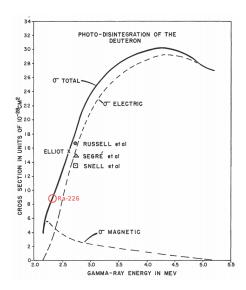


Figure 11: [try to find better plot of this] Cross sections for gamma-neutron interactions with a Deuterium target as a function of incident gamma energies. The energy for <sup>226</sup>Ra is highlighted.<sup>[9]</sup>

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