

Variable Eddington Factor for Mixed Hybrid Finite Element/Linear Discontinuous Galerkin Source Iteration

S. Olivier, J.E. Morel

Department of Nuclear Engineering
Texas A&M University
College Station, TX 77843

Abstract

Abstract goes here

Keywords

P_N equations, S_N equations, asymptotic decay lengths.

Running Head

Asymptotic S_{N+1} Equations

Corresponding Author

Jim E. Morel, Phone: (979)845-6072, FAX: (979)845-6075, E-mail: *morel@tamu.edu*.

1 Introduction

2 Variable Eddington Method

2.1 Source Iteration

The steady-state, mono-energetic, isotropically-scattering, fixed-source Linear Boltzmann Equation in slab geometry is:

$$\mu \frac{\partial \psi}{\partial x}(x, \mu) + \Sigma_t(x) \psi(x, \mu) = \frac{\Sigma_s(x)}{2} \int_{-1}^1 \psi(x, \mu') d\mu' + \frac{Q(x)}{2}, \quad (1)$$

where $\mu = \cos \theta$ is the cosine of the angle of flight θ relative to the x -axis, $\Sigma_t(x)$ and $\Sigma_s(x)$ the total and scattering macroscopic cross sections, $Q(x)$ the isotropic fixed-source and $\psi(x, \mu)$ the angular flux [?]. The factors of 1/2 are consistent with the following definition of the scalar flux:

$$\phi(x) = \int_{-1}^1 \psi(x, \mu) d\mu. \quad (2)$$

Equation 1 is an integro-differential equation due to the placement of the unknown, $\psi(x, \mu)$, under both a derivative and an integral.

The Discrete Ordinates (S_N) angular discretization sets μ to discrete values stipulated by an N -point Gauss quadrature rule. The scalar flux is then

$$\phi(x) = \int_{-1}^1 \psi(x, \mu) d\mu \xrightarrow{S_N} \sum_{n=1}^N w_n \psi_n(x), \quad (3)$$

where $\psi_n(x) = \psi(x, \mu_n)$ and the w_n are the quadrature weights corresponding to each μ_n [?]. To remain consistent with Eq. 2, the quadrature weights sum to 2. The S_N equations are

then

$$\mu_n \frac{d\psi_n}{dx}(x) + \Sigma_t(x)\psi_n(x) = \frac{\Sigma_s(x)}{2}\phi(x) + \frac{Q(x)}{2}, \quad (4)$$

where $n = 1, 2, \dots, N$ and $\phi(x)$ is defined by Eq. 3. This is now a system of N coupled, ordinary differential equations.

The Source Iteration (SI) solution method decouples the S_N equations by lagging the right side of Eq. 5. In other words,

$$\mu_n \frac{d\psi_n^{\ell+1}}{dx}(x) + \Sigma_t(x)\psi_n^{\ell+1}(x) = \frac{\Sigma_s(x)}{2}\phi^\ell(x) + \frac{Q(x)}{2}, 1 \leq n \leq N, \quad (5)$$

where the superscripts indicate the iteration index. Equation 5 represents N independent, first-order, ordinary differential equations each of which are easily solved by the well-known sweeping process.

The iteration process begins with an initial guess for the scalar flux, $\phi^0(x)$. Equation 5 is solved, using $\phi^0(x)$ on the right side, for $\psi_n^1(x)$. $\phi^1(x)$ is then computed using Eq. 3 and is used to update the right side of Eq. 5. This process is repeated until

$$\frac{\|\phi^{\ell+1}(x) - \phi^\ell(x)\|}{\|\phi^{\ell+1}(x)\|} < \epsilon, \quad (6)$$

where ϵ is a sufficiently small tolerance.

If $\phi^0(x) = 0$, then $\phi^\ell(x)$ is the scalar flux of particles that have undergone at most $\ell - 1$ collisions [?]. Thus, the number of iterations until convergence is directly linked to the number of collisions in a particle's lifetime. Typically, SI becomes increasingly slow to converge as the ratio of Σ_s to Σ_t approaches unity and the amount of particle leakage from the system goes to zero. SI is slowest in large, optically thick systems with small losses to

absorption. In full radiation transport simulations each iteration could involve solving for hundreds of millions of unknowns. To minimize computational expense, acceleration schemes must be developed to rapidly increase the rate of convergence of SI.

2.2 Eddington Acceleration

The zeroth and first angular moments of Eq. 1 are

$$\frac{d}{dx}J(x) + \Sigma_a(x)\phi(x) = Q(x), \quad (7a)$$

$$\frac{d}{dx}\langle\mu^2\rangle(x)\phi(x) + \Sigma_t(x)J(x) = 0, \quad (7b)$$

where $J(x) = \int_{-1}^1 \mu \psi(x, \mu) d\mu$ is the current and

$$\langle\mu^2\rangle(x) = \frac{\int_{-1}^1 \mu^2 \psi(x, \mu) d\mu}{\int_{-1}^1 \psi(x, \mu) d\mu} \quad (8)$$

the Eddington factor. In S_N , the Eddington factor is

$$\langle\mu^2\rangle(x) = \frac{\sum_{n=1}^N \mu_n^2 w_n \psi_n(x)}{\sum_{n=1}^N w_n \psi_n(x)}. \quad (9)$$

Note that no approximations have been made to arrive at Eqs. 7a and 7b. The Eddington factor is the true angular flux weighted average of μ^2 and therefore Eqs. 7a and 7b are just as accurate as Eq. 1.

This formulation is beneficial because Eq. 7a is a conservative balance equation and—if $\langle\mu^2\rangle(x)$ is known—the moment equations’ system of two first-order, ordinary differential equations can be solved directly with well-established methods. However, computing $\langle\mu^2\rangle(x)$

requires knowledge of the angular flux.

In Eddington Acceleration, S_N is used to compute the Eddington factor needed to solve the moment equations. Source iteration is then:

1. Given the previous estimate for the scalar flux, $\phi^\ell(x)$, solve Eq. 5 for $\psi_n^{\ell+1/2}(x)$.
2. Compute $\langle \mu^2 \rangle^{\ell+1/2}(x)$ with Eq. 9.
3. Solve the moment equations for $\phi^{\ell+1}(x)$ using $\langle \mu^2 \rangle^{\ell+1/2}(x)$.
4. Update the scalar flux estimate on the right side of Eq. 5 with $\phi^{\ell+1}(x)$ and repeat the iteration process until the scalar flux converges.

Acceleration occurs because the angular shape of the angular flux, and thus the Eddington factor, converges much faster than the scalar flux. In addition, the moment equations model the contributions of all scattering events at once, reducing the dependence on source iterations to introduce scattering information. The solution from the moment equations is then an approximation for the full flux and not the $\ell - 1$ collided flux as it was without acceleration.

In addition to acceleration, this scheme allows the S_N equations and moment equations to be solved with arbitrarily different spatial discretization methods. S_N can be spatially discretized using normal methods, such as Linear Discontinuous Galerkin or Diamond Difference, while the moment equations can be solved with MHFEM.

3 Computational Results

3.1 Diffusion Limit

3.2 Method of Manufactured Solutions

3.3 Solution Convergence

4 Conclusions and Future Work

References

- [1] Pomraning, G. C. (1964). A generalized P_N approximation for neutron transport problems. *Nukleonik* 6:348.
- [2] Ganguley, K., Allen, E. J., Coskun, E., and Nielsen, S. (1993). On the discrete-ordinates method via Case's solution, *J. Comp. Phys.*, 107:66.
- [3] Morel, J. E. (1989). A hybrid collocation-Galerkin- S_N method for solving the Boltzmann transport equation, *Nucl. Sci. and Eng.*, 101:72.
- [4] Digital Library of Mathematical Functions.2011-08-29.National Institute of Standards and Technology from URL <http://dlmf.nist.gov/14.7E11>.
- [5] Larsen, E. W., McGhee, J. M., Morel, J. E. (1992). The simplified P_N equations as an asymptotic limit of the transport equation. *Trans. Am. Nucl. Soc.* 66:231.

- [6] Larsen, E. W., Morel, J. E., McGhee, J. M. (1996). Asymptotic derivation of the multigroup P_1 and simplified P_N equations with anisotropic scattering. *Nucl. Sci. Eng.* 123:328.
- [7] Larsen, E. W., Asymptotic diffusion and simplified Pn approximations for diffusive and deep penetration problems. part 1: theory. (2011) *TTSP* 39:110.