

## Experimental Characterization of Pu Separation by PUREX Process on a Low-Burnup, Pseudo-Fast-Neutron Irradiated DUO<sub>2</sub> for Product Decontamination Factors and Nuclear Forensics

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**TEXAS A&M ENGINEERING  
EXPERIMENT STATION**

**NUCLEAR SECURITY  
SCIENCE & POLICY INSTITUTE**

# Outline

## Introduction

- Motivation

- Contexts

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- Experiment

- Recovery of Pu and U

- Experimental Decontamination Factors

## Future Work

# Motivation

## ❖ Current Events

- Joint Comprehensive Plan of Action
- Non-safeguarded reactors
- Islamic State of Iraq and Syria

## ❖ Past Events

- September 11, 2001

## ❖ Limited scope of IAEA safeguards

## ❖ “the awful arithmetic of the atomic bomb” [1]

## ❖ Need for improved forensic capabilities [2, 3, 4]

# Definitions

- ❖ Special Nuclear Material (SNM)
  - Plutonium,  $^{233}\text{U}$ , or  $^{235}\text{U}$
- ❖ Nuclear Forensics
  - The investigative activity that surrounds the search for attributes of undetermined radioactive specimens for the purpose of attribution.
- ❖ SNM origin attributes/indicators
  - Indicators or clues for SNM origin attribution. Examples include burnup, fluence rate, initial fuel enrichment, fuel age, and fast-to-thermal irradiation ratios
- ❖ Decontamination Factors (DF)
  - A measure of the effectiveness with which a product is decontaminated from a contaminant

$$DF_j = \frac{\frac{c_j}{c_{Pu}}|_{\text{initial}}}{\frac{c_j}{c_{Pu}}|_{\text{final}}}$$

# National Context

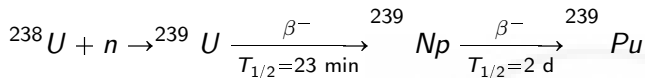
“The United States has developed a nuclear forensics capability that has been demonstrated in real-world incidents of **interdicted materials** and in exercises of actions required after a nuclear detonation. The committee, however, has concerns about the program and finds that without strong leadership, careful planning, and additional funds, these capabilities will decline” [2]

Major areas of concern include:

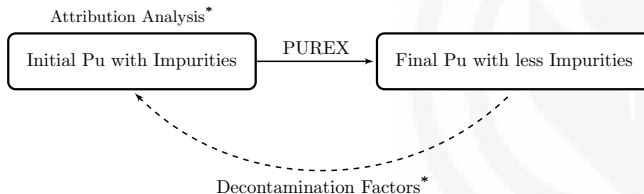
- ❖ Organization
- ❖ Sustainability
- ❖ **Workforce and Infrastructure**
- ❖ **Procedures and Tools**

# Forensic Context

- ❖ Nature of inverse problems
- ❖ Plutonium purification necessary for weapons production

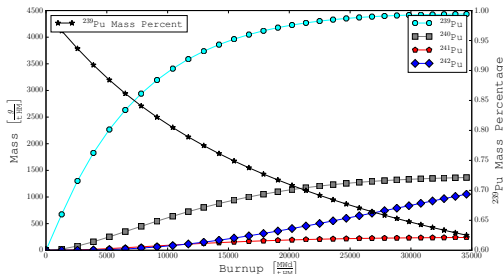


- ❖ Attribution for unpurified Pu has been previously studied [5, 6, 7]



# Nuclear Context

- ❖ Weapons-grade Pu can be extracted from reactor discharged fuel with a burnup of about 1 (GWD/tU)
- ❖ Pu isotopes produced in irradiated fuel can vary
- ❖ Two examples of reactors which can intentionally discharge low burned fuel for extracting weapon-grade Pu are:
  - Fast Breeder Reactor, CANDU Reactor



## Chemical Context

- ❖ Plutonium Uranium Redox EXtraction (PUREX)
  - Liquid-liquid solvent extraction
  - Many stages:
- ❖ Distribution Coefficient (D): The ratio between the organic and aqueous phases (aka: D-values)

$$D = \frac{C_o}{C_{aq}}$$

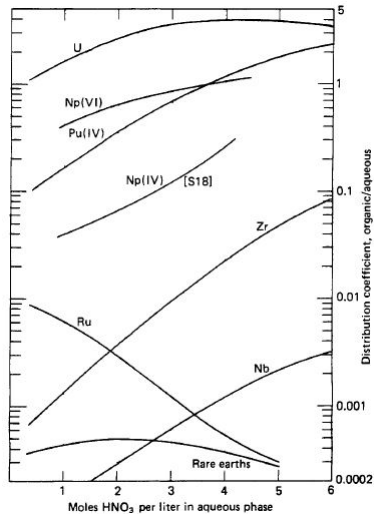
- Specific element to element
- Vary widely<sup>[8]</sup>
- The fraction of mass,  $f_o$  deposited in the organic phase, assuming a volume ratio between the aqueous and organic phases,  $V_R$ , is:

$$f_o = (1 + D^{-1}V_R^{-1})^{-1}$$



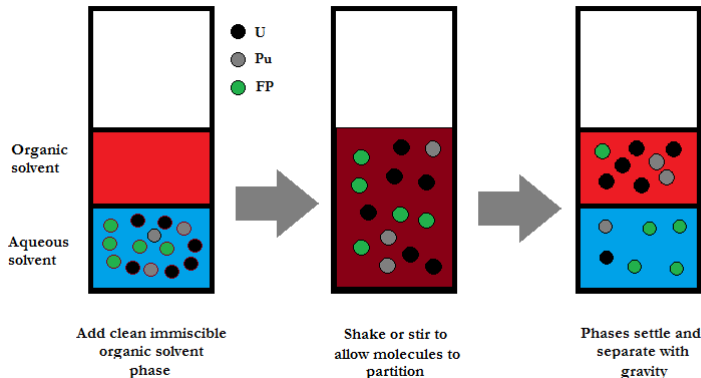
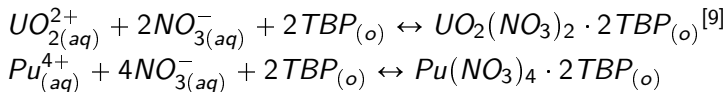
# Chemical Context

- ❖ Lack of literature on decontamination factors and distribution coefficients for useful forensic elements (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm)
- ❖ With a known process and D-values, DF values for individual elements can be determined

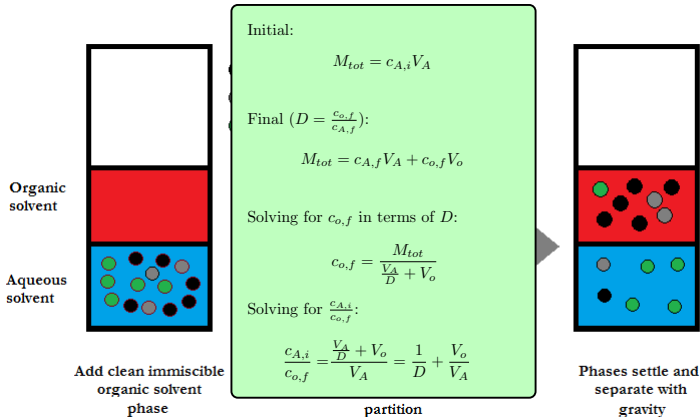
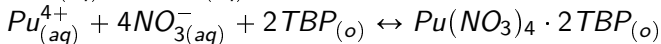
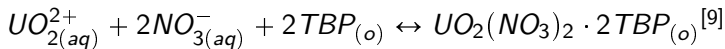


Adapted from Stoller<sup>[8]</sup>

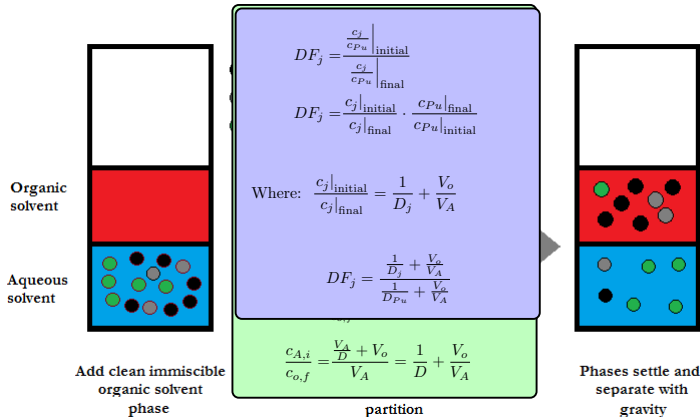
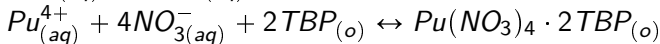
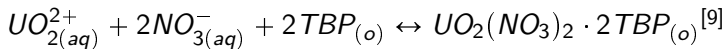
# Extraction



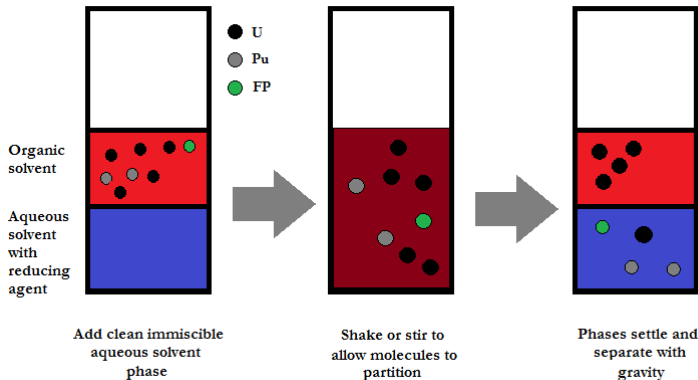
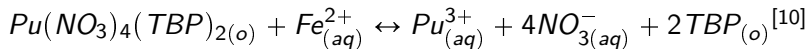
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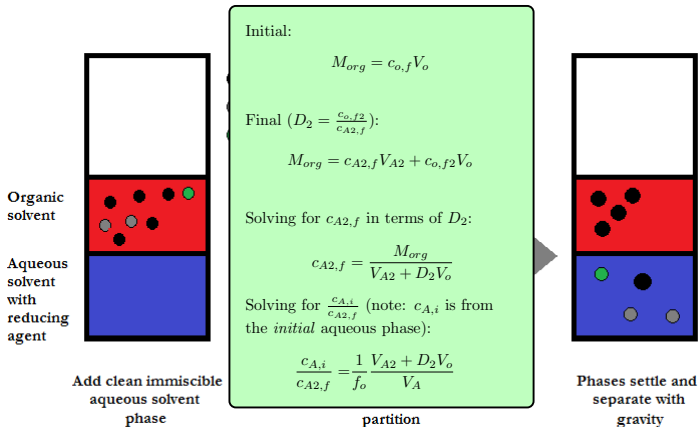
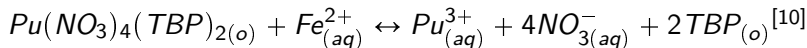
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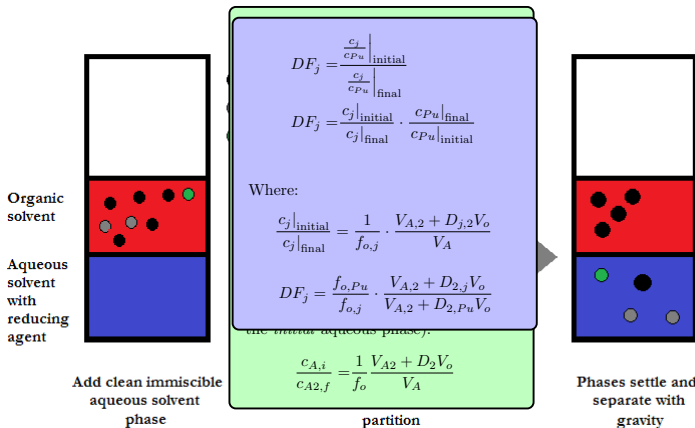
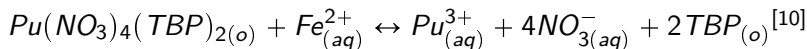
# Back-Extraction



# Back-Extraction



# Back-Extraction



# Decontamination Factors - The Pot of gold

- ❖ After several cycles of Pu extraction/scrubbing/back-extraction are completed, the effectiveness of a PUREX cycle is described by the decontamination factor (DF):

$$DF_j = \frac{\left| \frac{c_j}{c_{Pu}} \right|_{initial}}{\left| \frac{c_j}{c_{Pu}} \right|_{final}}$$

- ❖ DFs are characteristic of different process cycles
- ❖ Larger values ( $10^7$ ) for industrial scale PUREX (compared to benchtop)<sup>[8, 9]</sup>



## Previous Work

# Irradiation

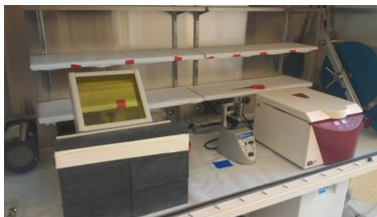
- ❖  $12.9 \pm 0.1$  mg of  $\text{DUO}_2$  was irradiated
  - High Flux Isotope Reactor at Oak Ridge National Laboratory
- ❖ Burnup was  $4.43 \pm 0.31$  GWd/tHM<sup>[11]</sup>
- ❖  $0.196 \pm$  mg of total Pu was produced as measured by ICP-MS



## Dissolution of the spent fuel pellet



# Glovebox



# Experiments

## ❖ Single stage extraction and back-extraction

- Purpose: quantify product recovery, D-values and DF values for single stage extraction and back extraction
- Conditions:

Starting Solution	Extraction Solution	Back extraction solution
4 M nitric acid	30% vol.% TBP, 70 vol.% kerosene	0.024 M ferrous sulfamate in 0.75 M nitric acid

## ❖ Multi-contact extraction and back-extraction

- Purpose: Maximize recovery of Pu with 4 extractions, 3 back extractions
- Conditions:

Starting Solution	Extraction Solution	Back extraction solution
4 M nitric acid	30% vol.% TBP, 70 vol.% kerosene	0.024 M ferrous sulfamate in 4 M nitric acid

## Previous Experiment Results

### Recoveries of U and Pu

	Pu Recovery	U Recovery
Single stage	$(83.4 \pm 9.5)\%$	$(11.2 \pm 1.3)\%$
Multi-contact Cycle 1	$(99.7 \pm 4.2)\%$	$(6.8 \pm 0.3)\%$
Multi-contact Cycle 2	$(93.0 \pm 4.6)\%$	$(6.6 \pm 0.3)\%$
Overall Experiment 2	$(92.7 \pm 6.0)\%$	$(0.45 \pm 0.03)\%$

# Previous Experiment Results

## Decontamination Factors

Element (Z)	SS	Error	MC Cycle 1	Error	Isotopes Used
Rb(37)	39.0	5.9	11.8	0.8	<sup>85</sup> Rb
Sr(38)	283	43	84.6	5.9	<sup>90</sup> Sr
Mo(42)	5.7	0.8	1.9	0.2	<sup>97,98,100</sup> Mo
Ru(44)	59.2	6.4	16.6	2.5	<sup>101,102,104</sup> Ru
Pd(46)	65	14	8.9	1.2	<sup>110</sup> Pd
Cd(48)	74	17	22.1	2.5	<sup>112</sup> Cd
Cs(55)	177	28	52.9	3.9	<sup>133</sup> Cs
Ce(58)	43	16	11.5	4.9	<sup>140,142</sup> Ce
Nd(60)	19.2	2.1	5.9	0.4	<sup>143</sup> Nd
Pm(61)	12.8	1.9	3.9	0.3	<sup>147</sup> Pm
Sm(62)	11.5	1.5	3.6	0.3	<sup>151</sup> Sm
Eu(63)	10.0	1.4	3.6	0.3	<sup>154</sup> Eu
U(92)	7.4	1.2	14.7	0.9	<sup>238</sup> U

# Conclusions

- ❖ Two PUREX experiments were conducted
  - Single stage: Determined DC values for Pu, U and several FP
  - Multi-contact: Utilized Experiment 1 to recover over 92% of Pu while leaving less than 1% of the U
- ❖ DF values were measured for 12 FP elements
- ❖ DF values were lower than those typically found in industrial scale PUREX plants due to multiple extraction and back-extraction steps without an intermittent scrubbing step.
- ❖ This work provide DF data that will be built upon for nuclear forensic investigations of interdicted Pu.



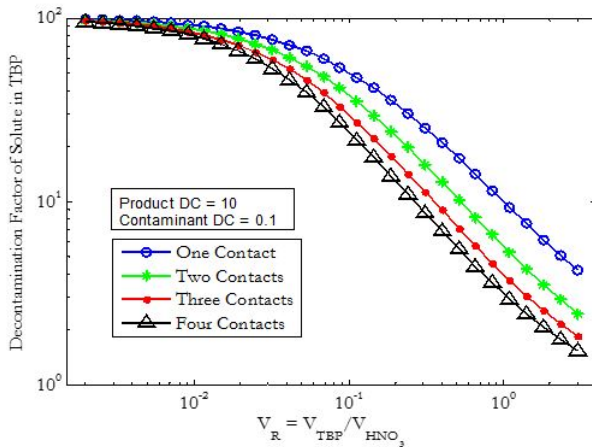
# Future Work

# Future Work

- ❖ Modify Multi-contact extraction, to recover a larger fraction of Pu
- ❖ Investigation of how D-values for (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm) change as a function of nitric acid concentration
- ❖ Determine statistical uncertainty of D and DF values.
  - Repeat above experiments 3-5 times
- ❖ Connect D-values with process information to DF values

Questions?

## Previous Experiment Results



Decontamination Factors for multi-contact extraction.

## References I

- [1] Dwight D Eisenhower. Atoms for peace speech. *Voices of Democracy*, 1953.
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- [8] Sidney M Stoller, Walter Henry Zinn, Stuart MacLain, and Atomic Energy Commission USA. *Reactor handbook. 2. Fuel reprocessing*. Interscience Publ., 1961.
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- [11] Mathew Wayne Swinney. *Experimental and Computational Assessment of Trace Nuclide Ratios in Weapons Grade Plutonium for Nuclear Forensics Analysis*. PhD thesis, 2015.
- [12] Kenneth D Kok. *Nuclear engineering handbook*, volume 60. CRC Press, 2009.
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# Mass Spec

