



Characterization of Pu Separation by PUREX

Paul Mendoza

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Outline

Motivation

Background

- The PUREX Process

- Distribution Coefficients

- Decontamination Factors

- Experiment

Completed Work

- Recovery of Pu and U

- Experimental Decontamination Factors

Future Work

Big Picture

- ❖ Pu isotopes produced in irradiated fuel can vary depending on
 - Burnup (irradiation history)
 - Reactor neutron spectrum (core design)
- ❖ Weapons-grade Pu can be extracted from reactor discharged fuel with a burnup of about 1 (GWD/tU)
- ❖ Two examples of non-safeguarded reactors which can intentionally discharge low burned fuel for extracting weapon-grade Pu are:
 - Indian Prototype Fast Breeder Reactor (PFBR-500MWe)
 - Indian Pressureized Heavy Water Reactor (PHWR-CANDU type 220 MWe)
- ❖ In accord with the Indo-US 123 agreement, these reactors were not required to be kept under IAEA safeguards

Smaller Picture

- ❖ Attribution for unpurified Pu has been previously studied [2, 6, 3]
- ❖ Intercepted Pu would likely have been processed via PUREX
- ❖ Due to a lack of literature on decontamination factors and distribution coefficients for useful forensic elements (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm) this research was pursued.

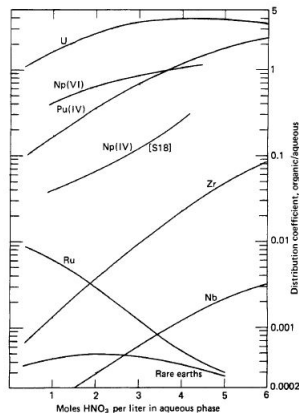


Figure 1: Effect of nitric acid concentration on distribution coefficients [7]

Background

What is PUREX - A type of laundry detergent?

❖ Plutonium Uranium Redox EXtraction

- Liquid-liquid solvent extraction
- Many stages:
 1. Preparation for Dissolution
 2. Dissolution
 3. Preparation of Dissolved Feed
 4. Primary Decontamination - Extraction to organic*
 5. Scrubbing
 6. Plutonium Partition - Back-Extraction to aqueous*
 7. Plutonium Purification

* - Discussing Next

Extraction

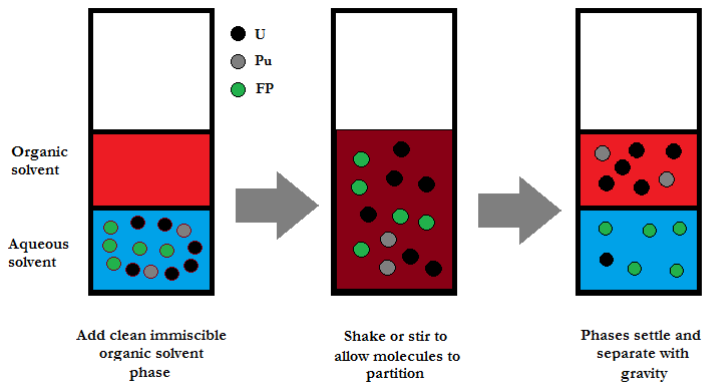


Figure 2: Extraction graphic

Back-Extraction

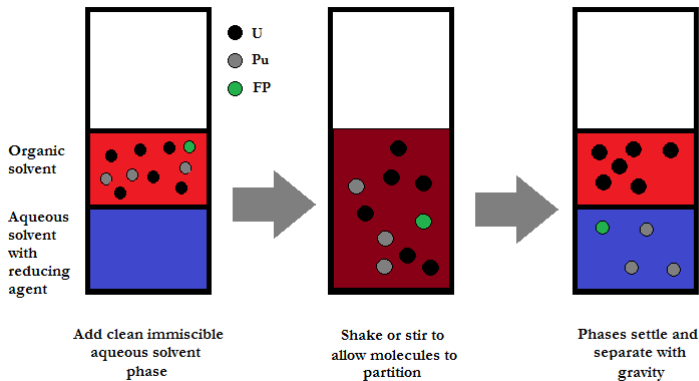


Figure 3: Back-extraction graphic

Extraction and Back-extraction

❖ Extraction



Most of the fission products are left in the aqueous

solution at valence III and V states^[4]

❖ Back-extraction

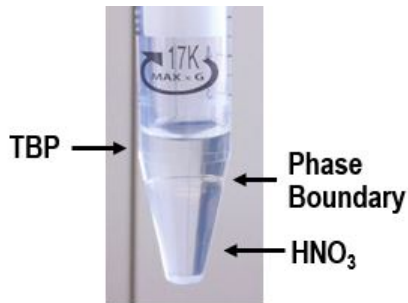
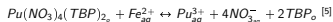


Figure 4: TBP and HNO₃

Distribution Coefficients - The Missing link

- ❖ Distribution Coefficient (DC): The ratio between the organic and aqueous phases

$$DR = \frac{C_o}{C_{aq}}$$

- ❖ Specific element to element
- ❖ Vary widely with:^[7]
 - Concentrations of solutions
 - Saturation of U and Pu in the system
 - Temperature of the solvents
- ❖ The fraction of mass, f_o deposited in the organic phase, assuming a volume ratio between the aqueous and organic phases, V_R , is:

$$f_o = (1 + DC^{-1} V_R^{-1})^{-1}$$

Decontamination Factors - The Pot of gold

- ❖ After several cycles of Pu extraction/scrubbing/back-extraction are completed, the effectiveness of a PUREX cycle is described by the decontamination factor (DF):

$$DF_j = \frac{\left| \frac{c_j}{c_{Pu}} \right|_{initial}}{\left| \frac{c_j}{c_{Pu}} \right|_{final}}$$

- ❖ DFs are characteristic of different process cycles
- ❖ Larger values (10^7) for industrial scale PUREX (compared to benchtop)^[7, 1]

Complexity of reprocessing schemes

Worry about dependancies of of DC Worry about non equilibrium flow rates, blah blah blah. Show mixer settler columns, show batch. What I want to do, get some distribution coefficients develop a reasonable process for isolating a large fraction of Pu, then, based on process, calculate what the decontamination factor should be, and then actually measure it

Irradiation

❖ $12.9 \pm$



Experiments

❖ Experiment 1

- Purpose: quantify product recovery and DF values for single stage extraction and back extraction
- Conditions: Latex Table

❖ Experiment 2

- Purpose: recover a large fraction of Pu (4 extractions, 3 back extractions)
- Conditions Latex Table

Completed Work

Previous Experiment Results

Show the first table here



Previous Experiment Results

- ❖ Calculate Compton edge for each peak

$$E_c = E_e - |_{(\theta=\pi)} = E_\gamma \left(\frac{2E_\gamma}{m_e c^2 + 2E_\gamma} \right) \quad (1)$$

Compton edges for gamma-ray sources:

Element	E_γ (keV)	Compton Edge (keV)
^{133}Ba	356	207.25
^{137}Cs	662	477.65
^{54}Mn	835	639.36
^{22}Na (P.P.)	511	340.67
^{60}Co	1173	963.42
^{22}Na	1274	1061.18
^{60}Co	1332	1118.10

Previous Experiment Results

Show pretty plot



Future Work

Future Work

- ❖ Continue improving stilbene modeling parameters and capabilities in DRiFT
 - PSD using waveforms
 - Low-energy photon detection improvements

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- ❖ Begin modeling multiple stilbene detectors in different angle configurations
- ❖ Model full NEUANCE detector array
- ❖ Study detector cross-talk
- ❖ Compare MCNP6/DRiFT simulations using CGMF/FREYA against experimental measurements
 - ^{252}Cf : spontaneous fission
 - ^{239}Pu and ^{235}U : neutron-induced fission

Questions?

References I

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