

## Experimental Characterization of Pu Separation by PUREX Process on a Low-Burnup, Pseudo-Fast-Neutron Irradiated $\text{DUO}_2$ for Product Decontamination Factors and Nuclear Forensics

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**TEXAS A&M ENGINEERING  
EXPERIMENT STATION**

**NUCLEAR SECURITY  
SCIENCE & POLICY INSTITUTE**

# Outline

## Introduction

- Motivation
- Background

## Background

- The PUREX Process
- Distribution Coefficients
- Decontamination Factors

## Previous Work

- Experiment
- Recovery of Pu and U
- Experimental Decontamination Factors

## Future Work

# Motivation

- ❖ Current Events
  - Joint Comprehensive Plan of Action
  - Non-safeguarded reactors
  - Islamic State of Iraq and Syria
- ❖ Past Events
  - September 11, 2001
- ❖ Limited scope of IAEA safeguards
- ❖ “the awful arithmetic of the atomic bomb”<sup>[1]</sup>
- ❖ Need for improved forensic capabilities<sup>[2, 3, 4]</sup>

Can talk a lot on this slide: North Korea, detonating a nuclear device  
October 2006

## Forensic Context

“The United States has developed a nuclear forensics capability that has been demonstrated in real-world incidents of **interdicted materials** and in exercises of actions required after a nuclear detonation. The committee, however, has concerns about the program and finds that without strong leadership, careful planning, and additional funds, these capabilities will decline” [2]

Major areas of concern include:

- ❖ Organization
- ❖ Sustainability
- ❖ **Workforce and Infrastructure**
- ❖ **Procedures and Tools**

According to a report from the committee on nuclear forensics released in 2010, the United States forensic capability has been demonstrated in real world scenarios, but is at risk unless certain developmental requirements are met. The committee listed 4 areas of concern, where improvement is needed.

- In terms of organization, nuclear forensics responsibility is shared by several agencies without central authority and with no consensus on strategic requirements to guide the program.
- For sustainability, our current capabilities are the fruit of the nuclear weapons program and our laboratory infrastructure, funding for both have been declining
- Skilled personnel in these areas are few, and key facilities are in need of replacement because they are old, outdated, or do not meet modern environmental, health, or safety standards.
- In recent years, nuclear forensic techniques methodologies have been on the rise, some from presented from our own department, but according to this source, a large fraction of techniques are remnants of the cold war era, with less restrictions. Also, forensic exercises usually take months to complete, a time scale which is too long

Problem must be met on an administrative level, mentioned here because this proposed project will help, in a small way, with two areas of concern.

# Motivation

- ❖ Weapons-grade Pu can be extracted from reactor discharged fuel with a burnup of about 1 (GWD/tU)
- ❖ Pu isotopes produced in irradiated fuel can vary
- ❖ Two examples of reactors which can intentionally discharge low burned fuel for extracting weapon-grade Pu are:
  - Fast Breeder Reactor
  - CANDU Reactor

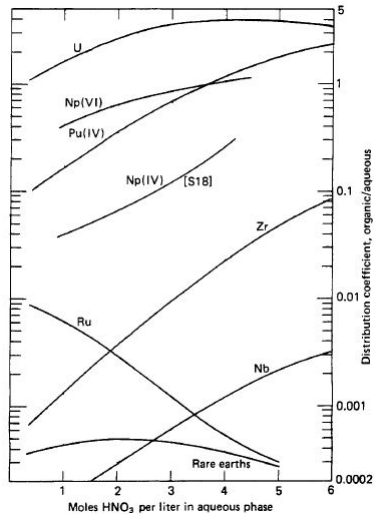




1. Madras Atomic Power Station Kalpakkam, India
2. Expected criticality in Jan 2017
3. Cost from 450 million euros to 750 euros
4. Sodium-cooled reactor design - U238 for breeding
5. 100 GWd/t for core, 40 year life, 1750 tonnes of sodium about 75% of olympic sized swimming pool.
6. liquid sodium has a density a little less than water
7. MOX fuel (UO<sub>2</sub> and PuO<sub>2</sub>) fuel
8. Fuel discharged at 100GWd/t, but I just mentioned that we are worried about 1GWd/t, mistake?

## Smaller Picture

- ❖ Attribution for unpurified Pu has been previously studied [5, 6, 7]
- ❖ Interdicted Pu would likely have been processed
- ❖ Lack of literature on decontamination factors and distribution coefficients for useful forensic elements (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm)



Adapted from Stoller<sup>[8]</sup>

# Background

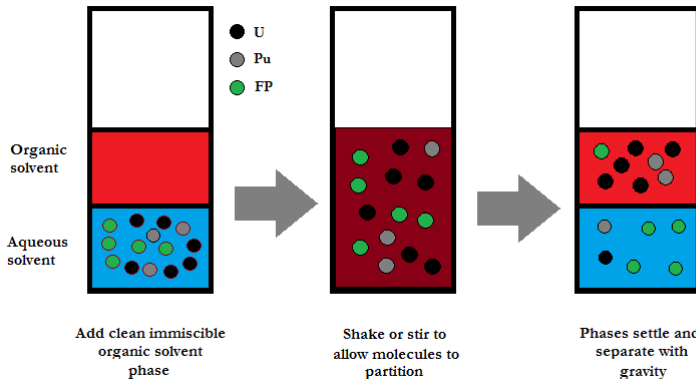
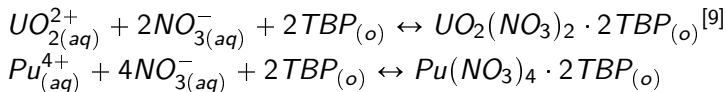
# What is PUREX - A type of laundry detergent?

## ❖ Plutonium Uranium Redox EXtraction

- Liquid-liquid solvent extraction
- Many stages:
  1. Preparation for Dissolution
  2. Dissolution
  3. Preparation of Dissolved Feed
  4. Primary Decontamination - Extraction to organic\*
  5. Scrubbing
  6. Plutonium Partition - Back-Extraction to aqueous\*
  7. Plutonium Purification

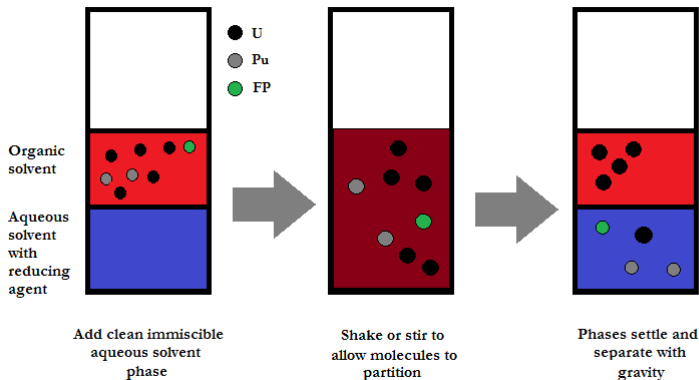
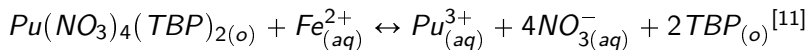
\* - Discussing Next

# Extraction



1. Most of the fission products are left in the aqueous solution at valence III and V states<sup>[10]</sup>

# Back-Extraction



1. The fission products that contribute mostly to the radioactive contamination of product in PUREX are zirconium, niobium, and ruthenium - with multiple oxidation states.



# Distribution Coefficients - The Missing link

- ❖ Distribution Coefficient (D): The ratio between the organic and aqueous phases (aka: D-values)

$$D = \frac{C_o}{C_{aq}}$$

- ❖ Specific element to element
- ❖ Vary widely with:<sup>[8]</sup>
  - Composition of phases
  - Solution saturation
  - Temperature of the solvent
- ❖ The fraction of mass,  $f_o$  deposited in the organic phase, assuming a volume ratio between the aqueous and organic phases,  $V_R$ , is:

$$f_o = (1 + D^{-1}V_R^{-1})^{-1}$$

1. Distribution coefficients can be reported in terms of volume basis (weight per unit volume), or a mass basis (mass of solute per unit mass of solute free solvent)- usually reported on volume basis
2. Note not a function of density, even though the two solutions have different densities, when solving for this value it cancels out
3. Solved this way to show, volume matters, and to give me a more intuitive sense of where things are going

# Decontamination Factors - The Pot of gold

- ❖ After several cycles of Pu extraction/scrubbing/back-extraction are completed, the effectiveness of a PUREX cycle is described by the decontamination factor (DF):

$$DF_j = \frac{\left| \frac{c_j}{c_{Pu}} \right|_{initial}}{\left| \frac{c_j}{c_{Pu}} \right|_{final}}$$

- ❖ DFs are characteristic of different process cycles
- ❖ Larger values ( $10^7$ ) for industrial scale PUREX (compared to benchtop)<sup>[8, 9]</sup>

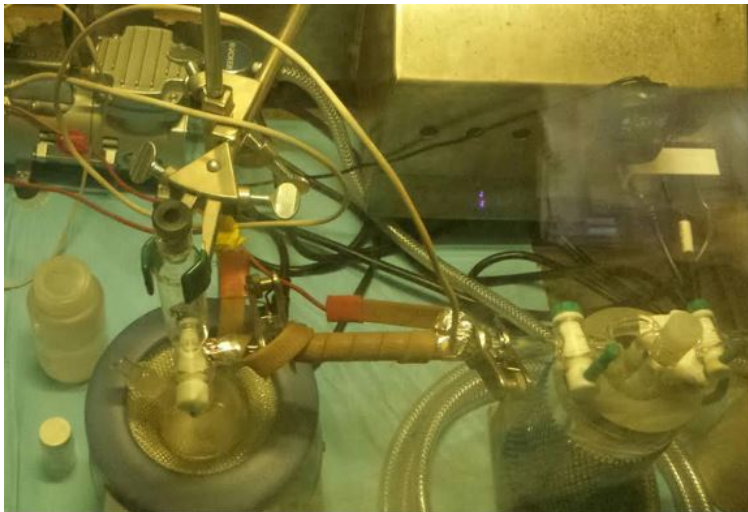
## Previous Work

# Irradiation

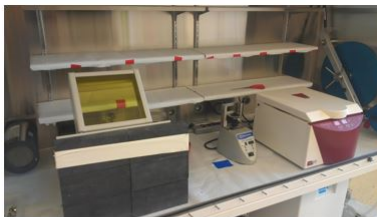
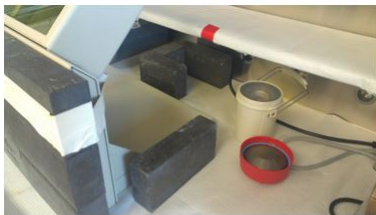
- ❖  $12.9 \pm 0.1$  mg of  $\text{DUO}_2$  was irradiated
  - High Flux Isotope Reactor at Oak Ridge National Laboratory
- ❖ Burnup was  $4.43 \pm 0.31$  GWd/tHM<sup>[12]</sup>
- ❖  $0.196 \pm$  mg of total Pu was produced as measured by ICP-MS



## Dissolution of the spent fuel pellet



# Glovebox



# Experiments

## ❖ Single stage extraction and back-extraction

- Purpose: quantify product recovery, D-values and DF values for single stage extraction and back extraction
- Conditions:

| Starting Solution | Extraction Solution              | Back extraction solution                        |
|-------------------|----------------------------------|---|
| 4 M nitric acid   | 30% vol.% TBP, 70 vol.% kerosene | 0.024 M ferrous sulfamate in 0.75 M nitric acid |

## ❖ Multi-contact extraction and back-extraction

- Purpose: Maximize recovery of Pu with 4 extractions, 3 back extractions
- Conditions:

| Starting Solution | Extraction Solution              | Back extraction solution                     |
|-------------------|----------------------------------|--|
| 4 M nitric acid   | 30% vol.% TBP, 70 vol.% kerosene | 0.024 M ferrous sulfamate in 4 M nitric acid |



## Previous Experiment Results

### Recoveries of U and Pu

|                       | Pu Recovery        | U Recovery          |
|-----------------------|--------------------|---------------------|
| Single stage          | $(83.4 \pm 9.5)\%$ | $(11.2 \pm 1.3)\%$  |
| Multi-contact Cycle 1 | $(99.7 \pm 4.2)\%$ | $(6.8 \pm 0.3)\%$   |
| Multi-contact Cycle 2 | $(93.0 \pm 4.6)\%$ | $(6.6 \pm 0.3)\%$   |
| Overall Experiment 2  | $(92.7 \pm 6.0)\%$ | $(0.45 \pm 0.03)\%$ |

# Previous Experiment Results

## Decontamination Factors

| Element (Z) | SS   | Error | MC Cycle 1 | Error | Isotopes Used             |
|-------------|------|-------|------------|-------|---------------------------|
| Rb(37)      | 39.0 | 5.9   | 11.8       | 0.8   | <sup>85</sup> Rb          |
| Sr(38)      | 283  | 43    | 84.6       | 5.9   | <sup>90</sup> Sr          |
| Mo(42)      | 5.7  | 0.8   | 1.9        | 0.2   | <sup>97,98,100</sup> Mo   |
| Ru(44)      | 59.2 | 6.4   | 16.6       | 2.5   | <sup>101,102,104</sup> Ru |
| Pd(46)      | 65   | 14    | 8.9        | 1.2   | <sup>110</sup> Pd         |
| Cd(48)      | 74   | 17    | 22.1       | 2.5   | <sup>112</sup> Cd         |
| Cs(55)      | 177  | 28    | 52.9       | 3.9   | <sup>133</sup> Cs         |
| Ce(58)      | 43   | 16    | 11.5       | 4.9   | <sup>140,142</sup> Ce     |
| Nd(60)      | 19.2 | 2.1   | 5.9        | 0.4   | <sup>143</sup> Nd         |
| Pm(61)      | 12.8 | 1.9   | 3.9        | 0.3   | <sup>147</sup> Pm         |
| Sm(62)      | 11.5 | 1.5   | 3.6        | 0.3   | <sup>151</sup> Sm         |
| Eu(63)      | 10.0 | 1.4   | 3.6        | 0.3   | <sup>154</sup> Eu         |
| U(92)       | 7.4  | 1.2   | 14.7       | 0.9   | <sup>238</sup> U          |

# Conclusions

- ❖ Two PUREX experiments were conducted
  - Single stage: Determined DC values for Pu, U and several FP
  - Multi-contact: Utilized Experiment 1 to recover over 92% of Pu while leaving less than 1% of the U
- ❖ DF values were measured for 12 FP elements
- ❖ DF values were lower than those typically found in industrial scale PUREX plants due to multiple extraction and back-extraction steps without an intermittent scrubbing step.
- ❖ This work provide DF data that will be built upon for nuclear forensic investigations of interdicted Pu.

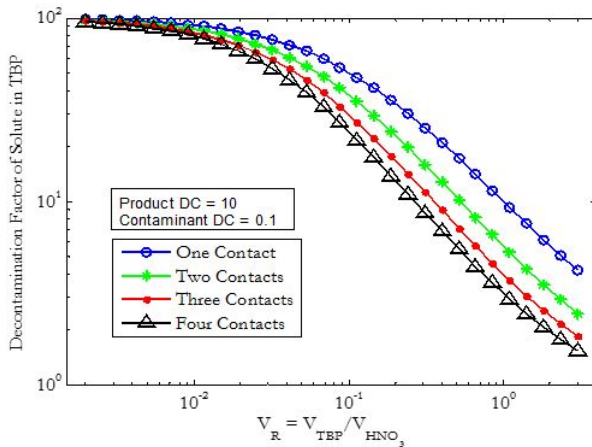
## Future Work

## Future Work

- ❖ Modify Multi-contact extraction, to recover a larger fraction of Pu
- ❖ Investigation of how D-values for (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm) change as a function of nitric acid concentration
- ❖ Determine statistical uncertainty of D and DF values.
  - Repeat above experiments 3-5 times
- ❖ Connect D-values with process information to DF values

Questions?

## Previous Experiment Results



Decontamination Factors for multi-contact extraction.

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- [1] Dwight D Eisenhower. Atoms for peace speech. *Voices of Democracy*, 1953.
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- [4] Kenton J Moody, Ian D Hutcheon, and Patrick M Grant. *Nuclear forensic analysis*. CRC Press, 2014.
- [5] Sunil S Chirayath, Jeremy M Osborn, and Taylor M Coles. Trace fission product ratios for nuclear forensics attribution of weapons-grade plutonium from fast and thermal reactors. *Science & Global Security*, 23(1):48–67, 2015.
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- [7] Alexander Glaser. Isotopic signatures of weapon-grade plutonium from dedicated natural uranium-fueled production reactors and their relevance for nuclear forensic analysis. *Nuclear Science and Engineering*, 163(1):26–33, 2009.



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- [8] Sidney M Stoller, Walter Henry Zinn, Stuart MacLain, and Atomic Energy Commission USA. *Reactor handbook. 2. Fuel reprocessing*. Interscience Publ., 1961.
- [9] M Benedict, H Levi, and T Pigford. Nuclear chemical engineering. *Nucl. Sci. Eng.:(United States)*, 82(4), 1982.
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- [11] RJM Konings, LR Morss, J Fuger, LR Morss, NM Edelstein, and J Fuger. The chemistry of the actinide and transactinide elements. *Springer, Dordrecht*, 4:2113–224, 2006.
- [12] Mathew Wayne Swinney. *Experimental and Computational Assessment of Trace Nuclide Ratios in Weapons Grade Plutonium for Nuclear Forensics Analysis*. PhD thesis, 2015.

# Mass Spec

