

## Your Presentation

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**TEXAS A&M ENGINEERING  
EXPERIMENT STATION**

**NUCLEAR SECURITY  
SCIENCE & POLICY INSTITUTE**

# Outline

## Motivation

## Background

- The PUREX Process

- Distribution Coefficients

- Decontamination Factors

## Previous Work

- Experiment

- Recovery of Pu and U

- Experimental Decontamination Factors

## Future Work

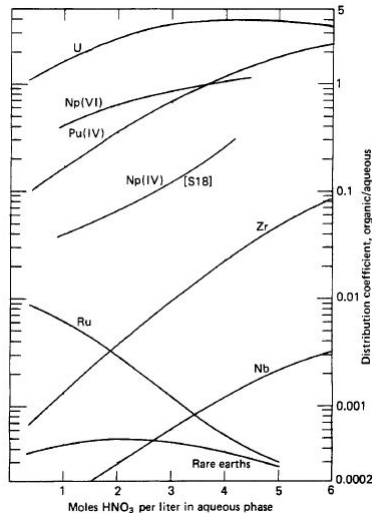
# Big Picture

- ❖ Weapons-grade Pu can be extracted from reactor discharged fuel with a burnup of about 1 (GWD/tU)
- ❖ Pu isotopes produced in irradiated fuel can vary
- ❖ Two examples of reactors which can intentionally discharge low burned fuel for extracting weapon-grade Pu are:
  - Fast Breeder Reactor
  - CANDU Reactor



# Smaller Picture

- ❖ Attribution for unpurified Pu has been previously studied [2, 6, 3]
- ❖ Interdicted Pu would likely have been processed
- ❖ Lack of literature on decontamination factors and distribution coefficients for useful forensic elements (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm)



Adapted from Stoller<sup>[7]</sup>

# Background

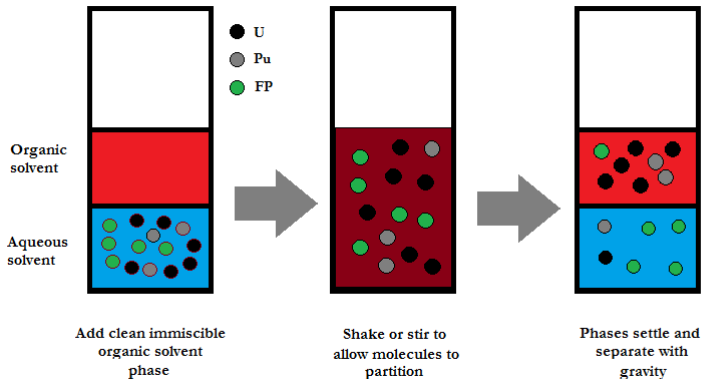
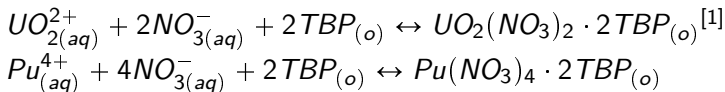
# What is PUREX - A type of laundry detergent?

## ❖ Plutonium Uranium Redox EXtraction

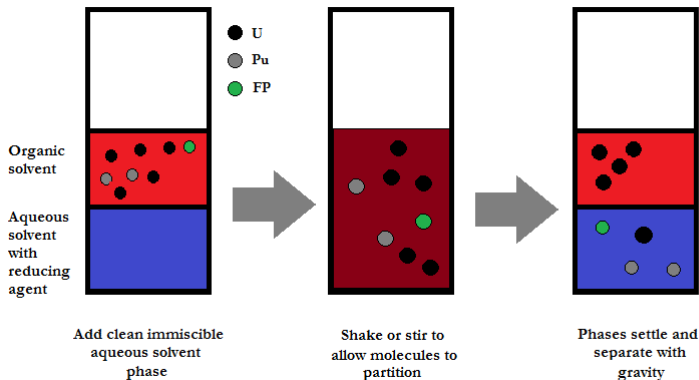
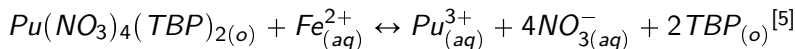
- Liquid-liquid solvent extraction
- Many stages:
  1. Preparation for Dissolution
  2. Dissolution
  3. Preparation of Dissolved Feed
  4. Primary Decontamination - Extraction to organic\*
  5. Scrubbing
  6. Plutonium Partition - Back-Extraction to aqueous\*
  7. Plutonium Purification

\* - Discussing Next

# Extraction



# Back-Extraction





# Distribution Coefficients - The Missing link

- ❖ Distribution Coefficient (D): The ratio between the organic and aqueous phases (aka: D-values)

$$D = \frac{C_o}{C_{aq}}$$

- ❖ Specific element to element
- ❖ Vary widely with:<sup>[7]</sup>
  - Composition of phases
  - Solution saturation
  - Temperature of the solvent
- ❖ The fraction of mass,  $f_o$  deposited in the organic phase, assuming a volume ratio between the aqueous and organic phases,  $V_R$ , is:

$$f_o = (1 + D^{-1}V_R^{-1})^{-1}$$

# Decontamination Factors - The Pot of gold

- ❖ After several cycles of Pu extraction/scrubbing/back-extraction are completed, the effectiveness of a PUREX cycle is described by the decontamination factor (DF):

$$DF_j = \frac{\left| \frac{c_j}{c_{Pu}} \right|_{initial}}{\left| \frac{c_j}{c_{Pu}} \right|_{final}}$$

- ❖ DFs are characteristic of different process cycles
- ❖ Larger values ( $10^7$ ) for industrial scale PUREX (compared to benchtop)<sup>[7, 1]</sup>

# Previous Work

# Irradiation

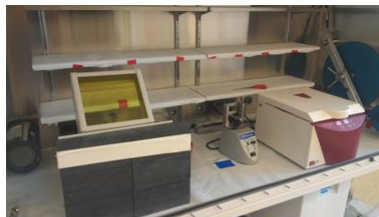
- ❖  $12.9 \pm 0.1$  mg of  $\text{DUO}_2$  was irradiated
  - High Flux Isotope Reactor at Oak Ridge National Laboratory
- ❖ Burnup was  $4.43 \pm 0.31$  GWd/tHM<sup>[8]</sup>
- ❖  $0.196 \pm$  mg of total Pu was produced as measured by ICP-MS



# Dissolution of the spent fuel pellet



# Glovebox



# Experiments

## ❖ Single stage extraction and back-extraction

- Purpose: quantify product recovery, D-values and DF values for single stage extraction and back extraction
- Conditions:

| Starting Solution | Extraction Solution              | Back extraction solution                        |
|-------------------|----------------------------------|---|
| 4 M nitric acid   | 30% vol.% TBP, 70 vol.% kerosene | 0.024 M ferrous sulfamate in 0.75 M nitric acid |

## ❖ Multi-contact extraction and back-extraction

- Purpose: Maximize recovery of Pu with 4 extractions, 3 back extractions
- Conditions:

| Starting Solution | Extraction Solution              | Back extraction solution                     |
|-------------------|----------------------------------|--|
| 4 M nitric acid   | 30% vol.% TBP, 70 vol.% kerosene | 0.024 M ferrous sulfamate in 4 M nitric acid |

## Previous Experiment Results

### Recoveries of U and Pu

|                       | Pu Recovery        | U Recovery          |
|-----------------------|--------------------|---------------------|
| Single stage          | $(83.4 \pm 9.5)\%$ | $(11.2 \pm 1.3)\%$  |
| Multi-contact Cycle 1 | $(99.7 \pm 4.2)\%$ | $(6.8 \pm 0.3)\%$   |
| Multi-contact Cycle 2 | $(93.0 \pm 4.6)\%$ | $(6.6 \pm 0.3)\%$   |
| Overall Experiment 2  | $(92.7 \pm 6.0)\%$ | $(0.45 \pm 0.03)\%$ |



# Previous Experiment Results

## Decontamination Factors

| Element (Z) | SS   | Error | MC Cycle 1 | Error | Isotopes Used             |
|-------------|------|-------|------------|-------|---------------------------|
| Rb(37)      | 39.0 | 5.9   | 11.8       | 0.8   | <sup>85</sup> Rb          |
| Sr(38)      | 283  | 43    | 84.6       | 5.9   | <sup>90</sup> Sr          |
| Mo(42)      | 5.7  | 0.8   | 1.9        | 0.2   | <sup>97,98,100</sup> Mo   |
| Ru(44)      | 59.2 | 6.4   | 16.6       | 2.5   | <sup>101,102,104</sup> Ru |
| Pd(46)      | 65   | 14    | 8.9        | 1.2   | <sup>110</sup> Pd         |
| Cd(48)      | 74   | 17    | 22.1       | 2.5   | <sup>112</sup> Cd         |
| Cs(55)      | 177  | 28    | 52.9       | 3.9   | <sup>133</sup> Cs         |
| Ce(58)      | 43   | 16    | 11.5       | 4.9   | <sup>140,142</sup> Ce     |
| Nd(60)      | 19.2 | 2.1   | 5.9        | 0.4   | <sup>143</sup> Nd         |
| Pm(61)      | 12.8 | 1.9   | 3.9        | 0.3   | <sup>147</sup> Pm         |
| Sm(62)      | 11.5 | 1.5   | 3.6        | 0.3   | <sup>151</sup> Sm         |
| Eu(63)      | 10.0 | 1.4   | 3.6        | 0.3   | <sup>154</sup> Eu         |
| U(92)       | 7.4  | 1.2   | 14.7       | 0.9   | <sup>238</sup> U          |

# Conclusions

- ❖ Two PUREX experiments were conducted
  - Single stage: Determined DC values for Pu, U and several FP
  - Multi-contact: Utilized Experiment 1 to recover over 92% of Pu while leaving less than 1% of the U
- ❖ DF values were measured for 12 FP elements
- ❖ DF values were lower than those typically found in industrial scale PUREX plants due to multiple extraction and back-extraction steps without an intermittent scrubbing step.
- ❖ This work provide DF data that will be built upon for nuclear forensic investigations of interdicted Pu.

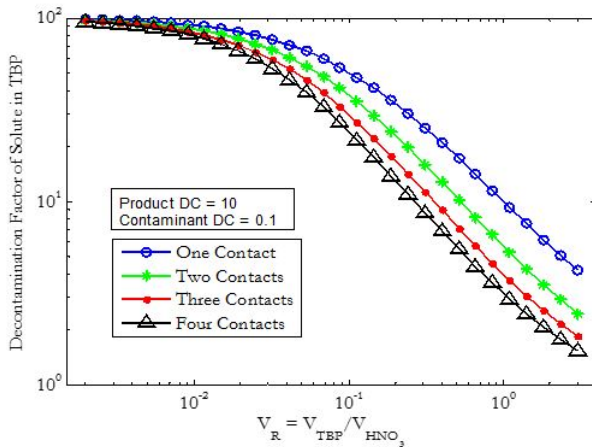
# Future Work

# Future Work

- ❖ Modify Multi-contact extraction, to recover a larger fraction of Pu
- ❖ Investigation of how D-values for (Cs, Sb, Eu, Rb, Sr, Nd, Pm, and Sm) change as a function of nitric acid concentration
- ❖ Determine statistical uncertainty of D and DF values.
  - Repeat above experiments 3-5 times
- ❖ Connect D-values with process information to DF values

# Questions?

# Previous Experiment Results



Decontamination Factors for multi-contact extraction.

# References I

- [1] M Benedict, H Levi, and T Pigford. Nuclear chemical engineering. *Nucl. Sci. Eng.:(United States)*, 82(4), 1982.
- [2] Sunil S Chirayath, Jeremy M Osborn, and Taylor M Coles. Trace fission product ratios for nuclear forensics attribution of weapons-grade plutonium from fast and thermal reactors. *Science & Global Security*, 23(1):48–67, 2015.
- [3] Alexander Glaser. Isotopic signatures of weapon-grade plutonium from dedicated natural uranium-fueled production reactors and their relevance for nuclear forensic analysis. *Nuclear Science and Engineering*, 163(1):26–33, 2009.
- [4] Kenneth D Kok. *Nuclear engineering handbook*, volume 60. CRC Press, 2009.
- [5] RJM Konings, LR Morss, J Fuger, LR Morss, NM Edelstein, and J Fuger. The chemistry of the actinide and transactinide elements. *Springer, Dordrecht*, 4:2113–224, 2006.
- [6] Mark Robert Scott. *Nuclear forensics: attributing the source of spent fuel used in an RDD event*. PhD thesis, Texas A&M University, 2005.
- [7] Sidney M Stoller, Walter Henry Zinn, Stuart MacLain, and Atomic Energy Commission USA. *Reactor handbook. 2. Fuel reprocessing*. Interscience Publ., 1961.

## References II

- [8] Mathew Wayne Swinney. *Experimental and Computational Assessment of Trace Nuclide Ratios in Weapons Grade Plutonium for Nuclear Forensics Analysis*. PhD thesis, 2015.



# Mass Spec

